

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

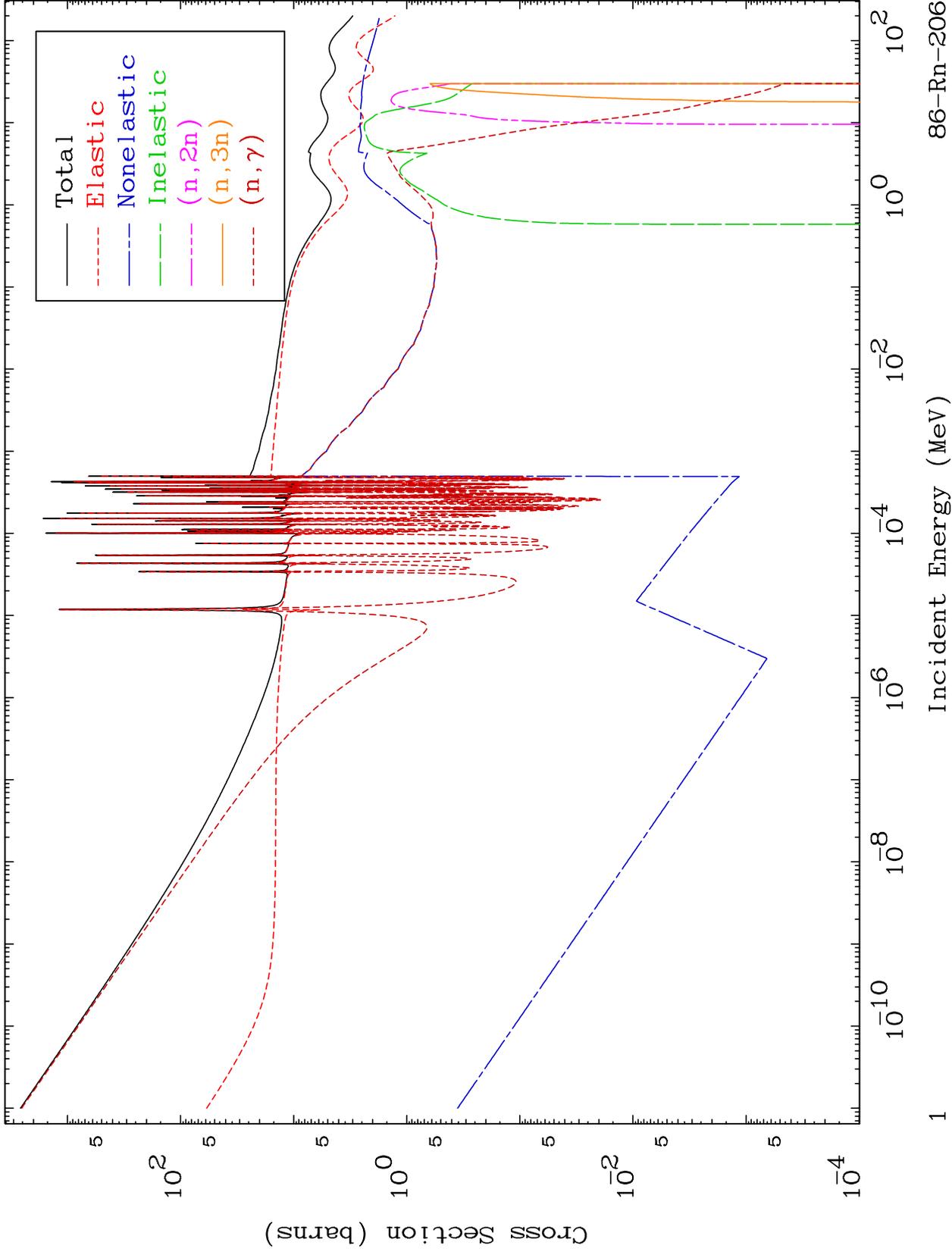
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Press Mouse Button to Start

MAT 8610

Neutron Major  
293 Kelvin Cross Sections

86-Rn-206



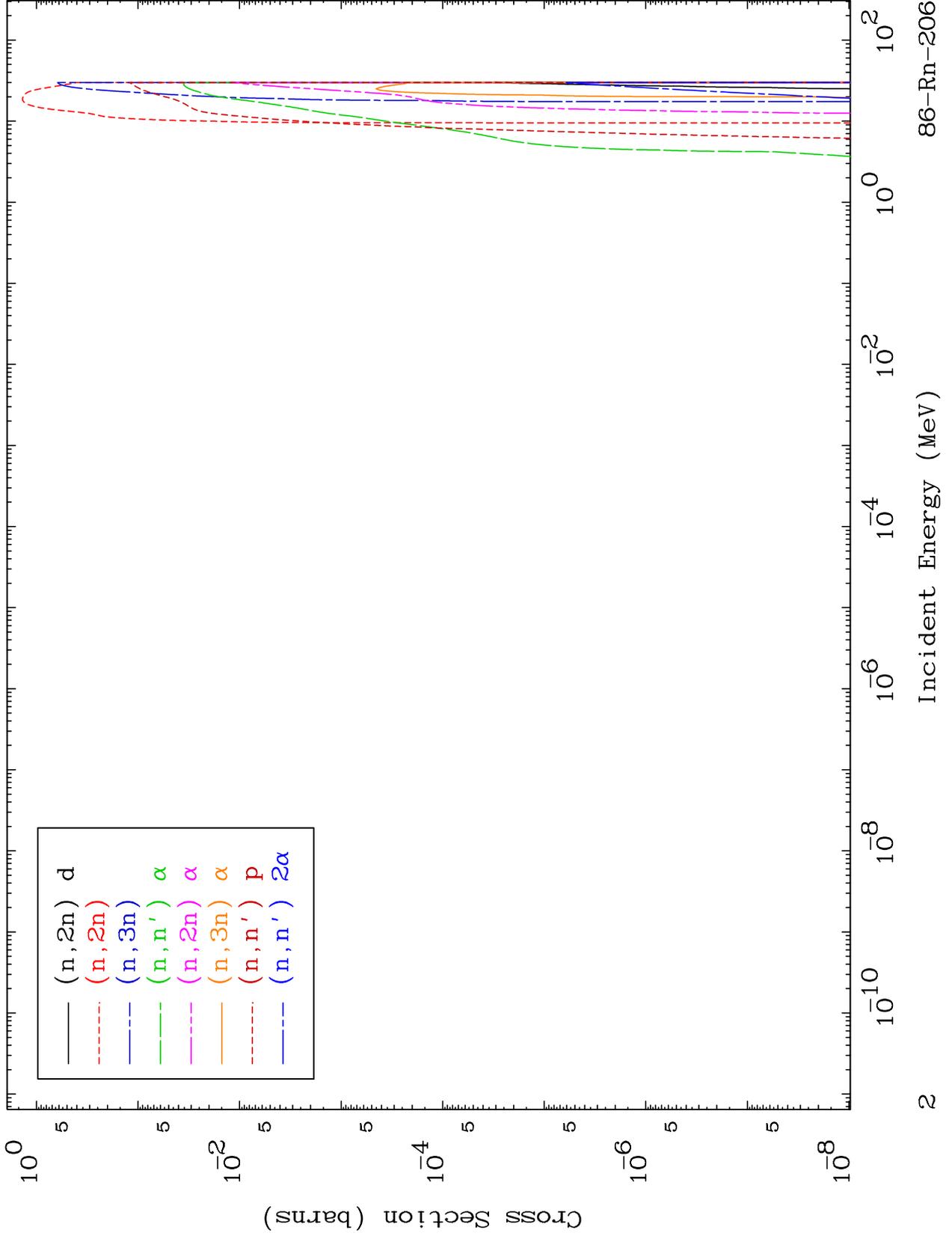
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86-Rn-206

MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

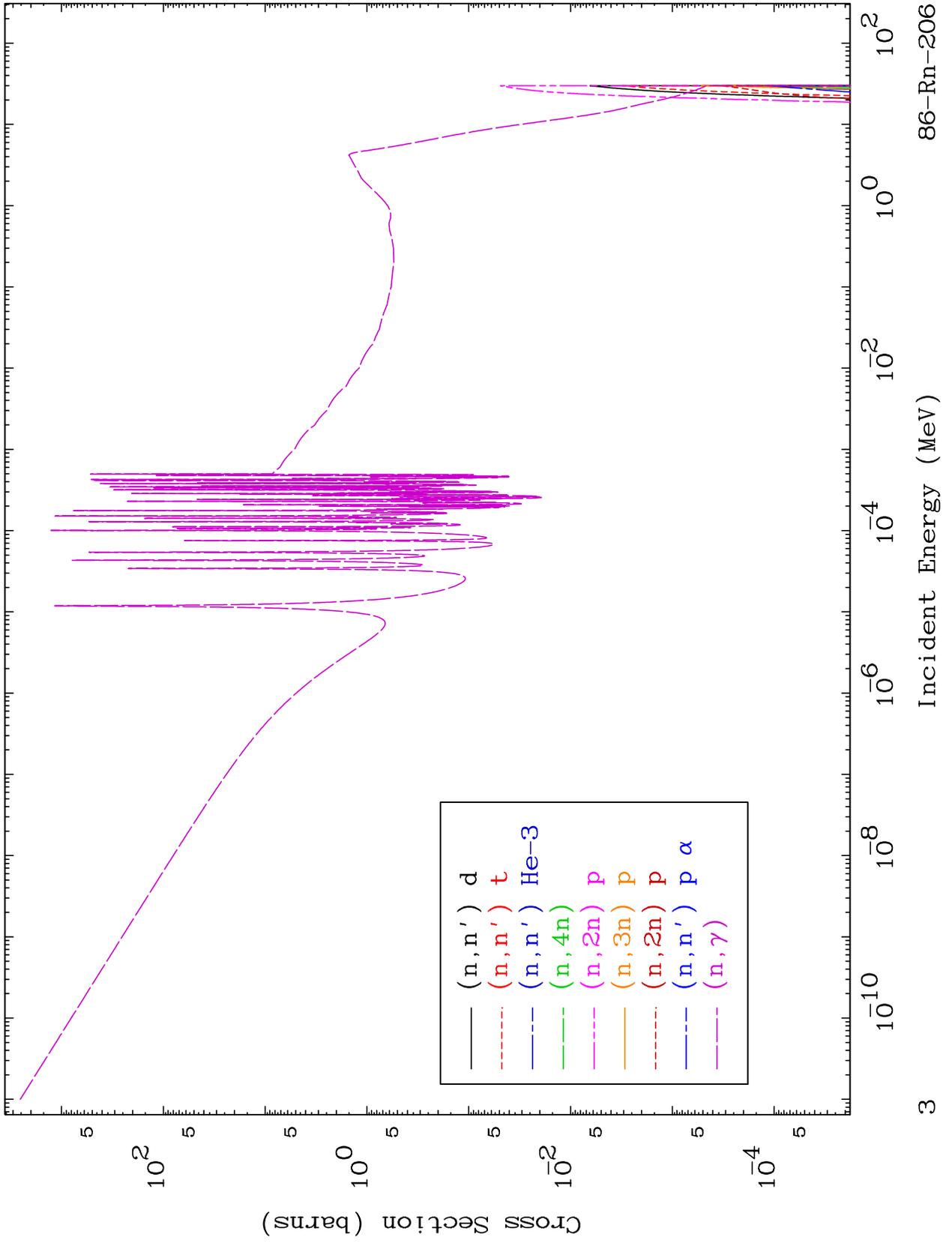
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

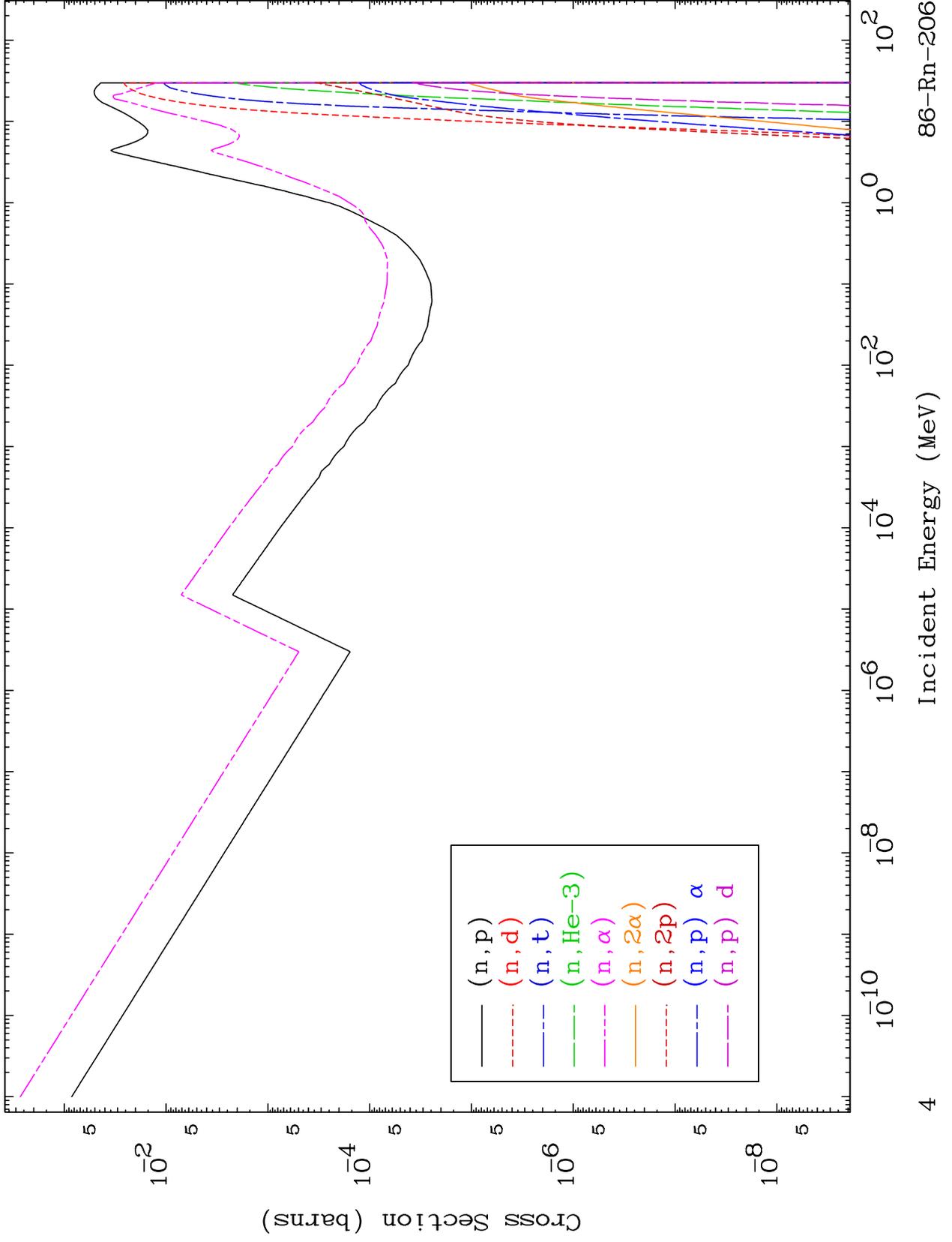
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

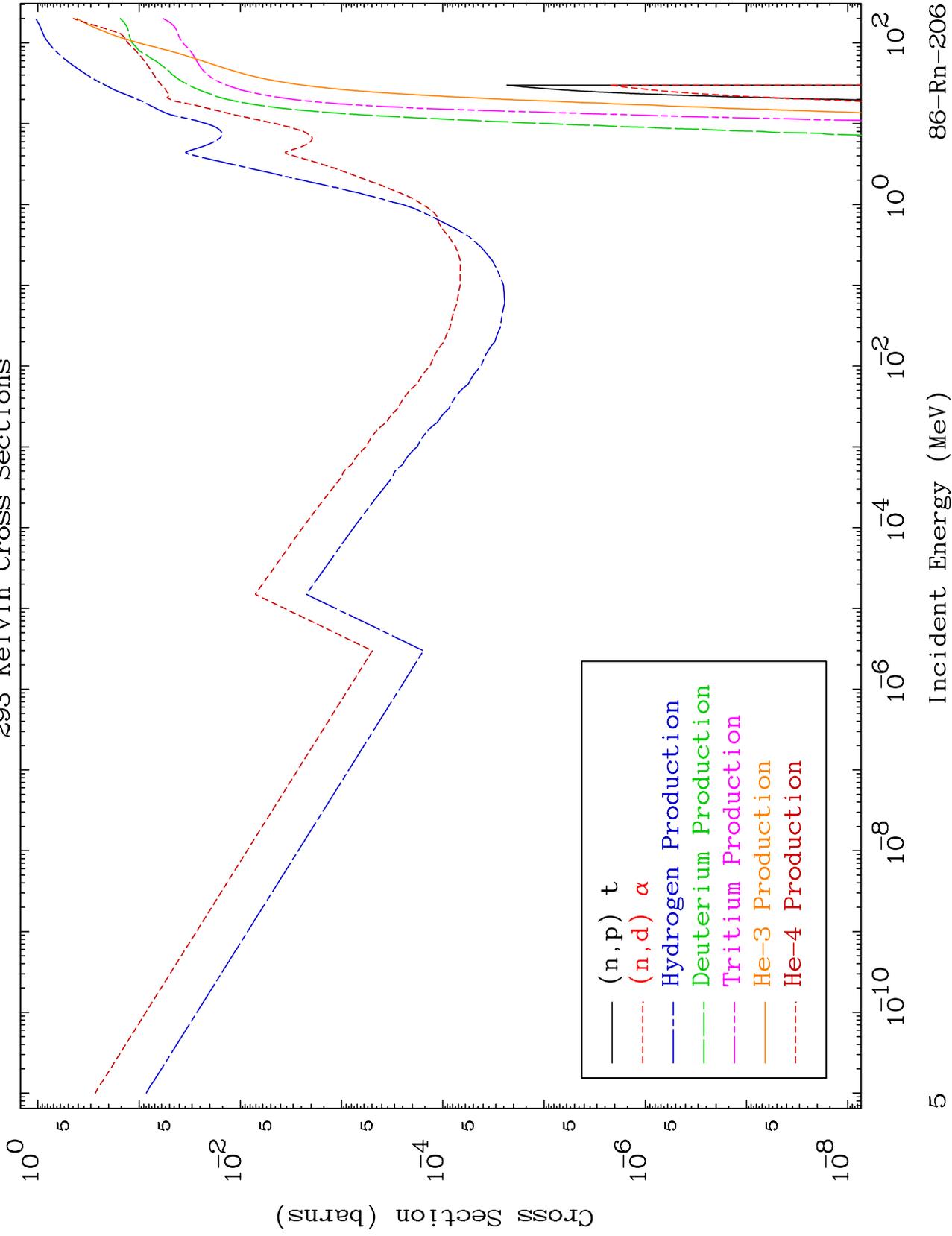
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

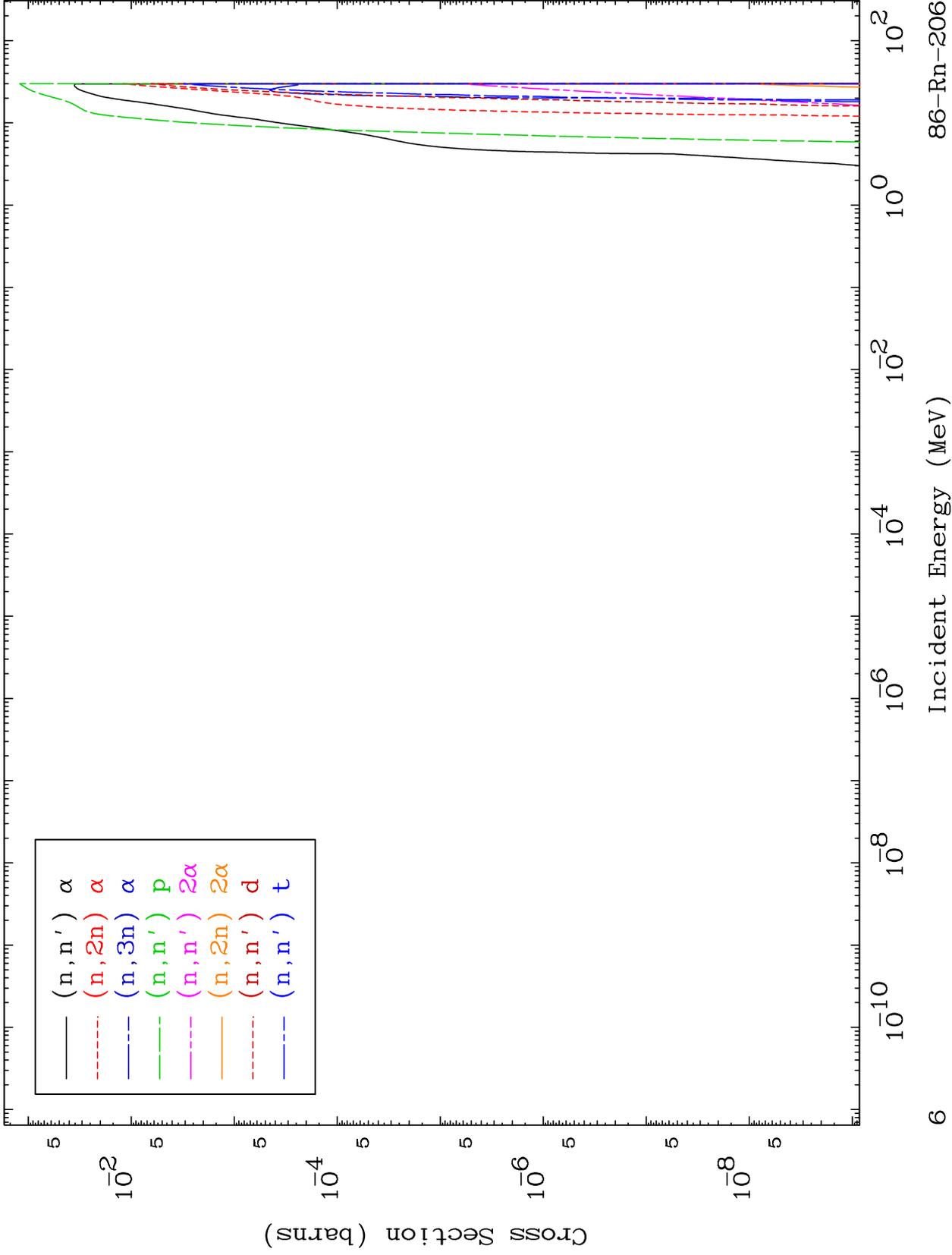
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MAT 8610

Charged Particle  
293 Kelvin Cross Sections

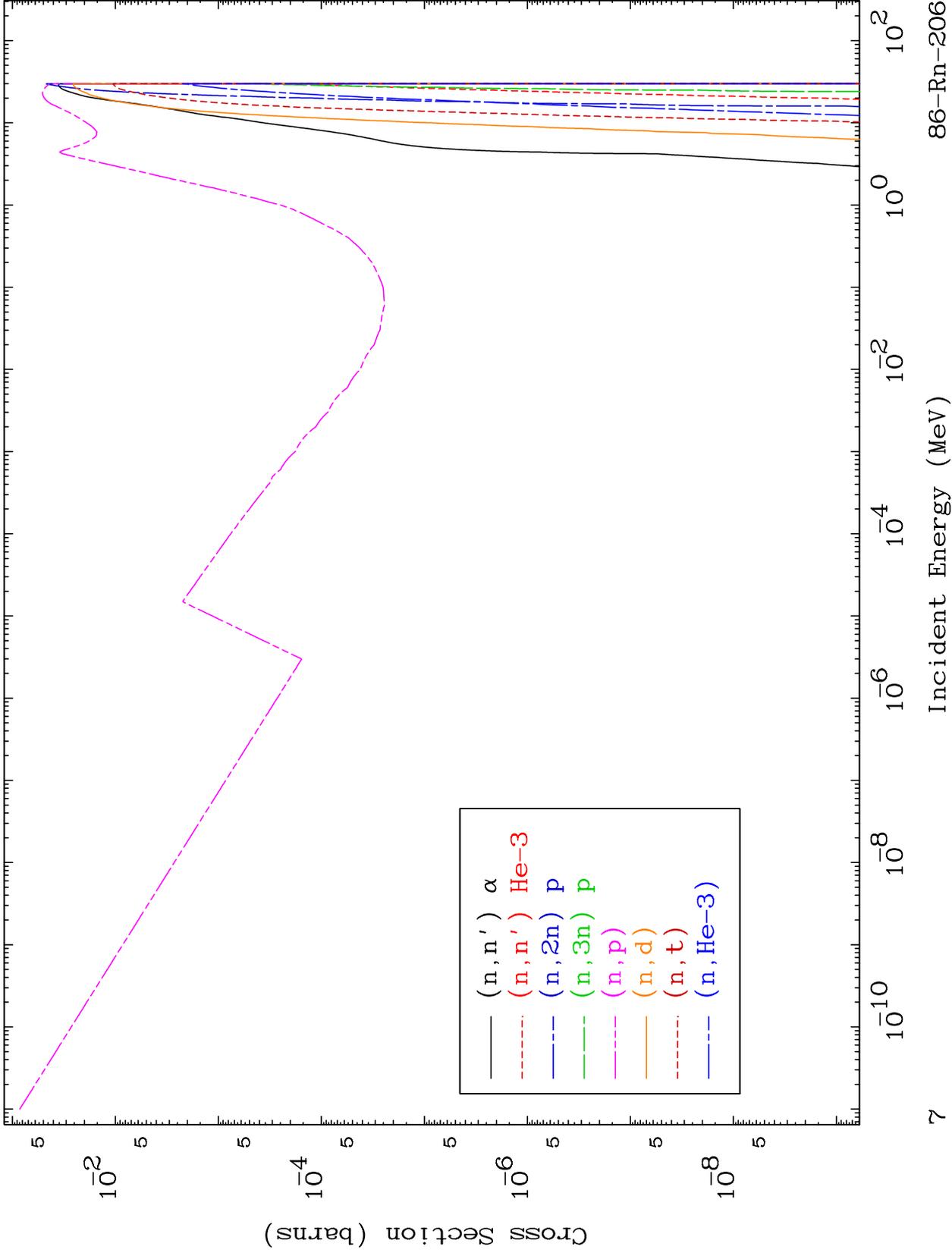
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MAT 8610

Charged Particle  
293 Kelvin Cross Sections

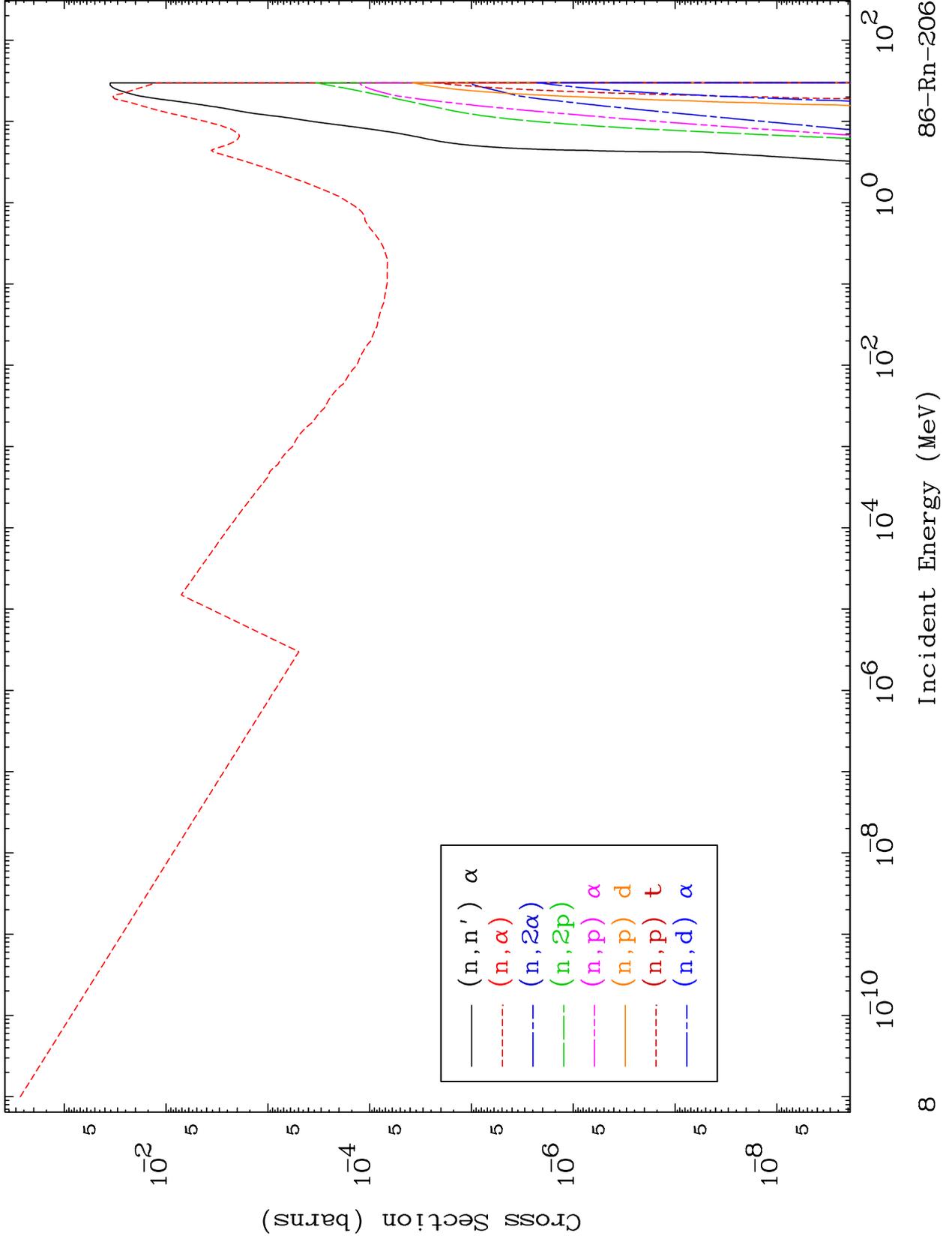
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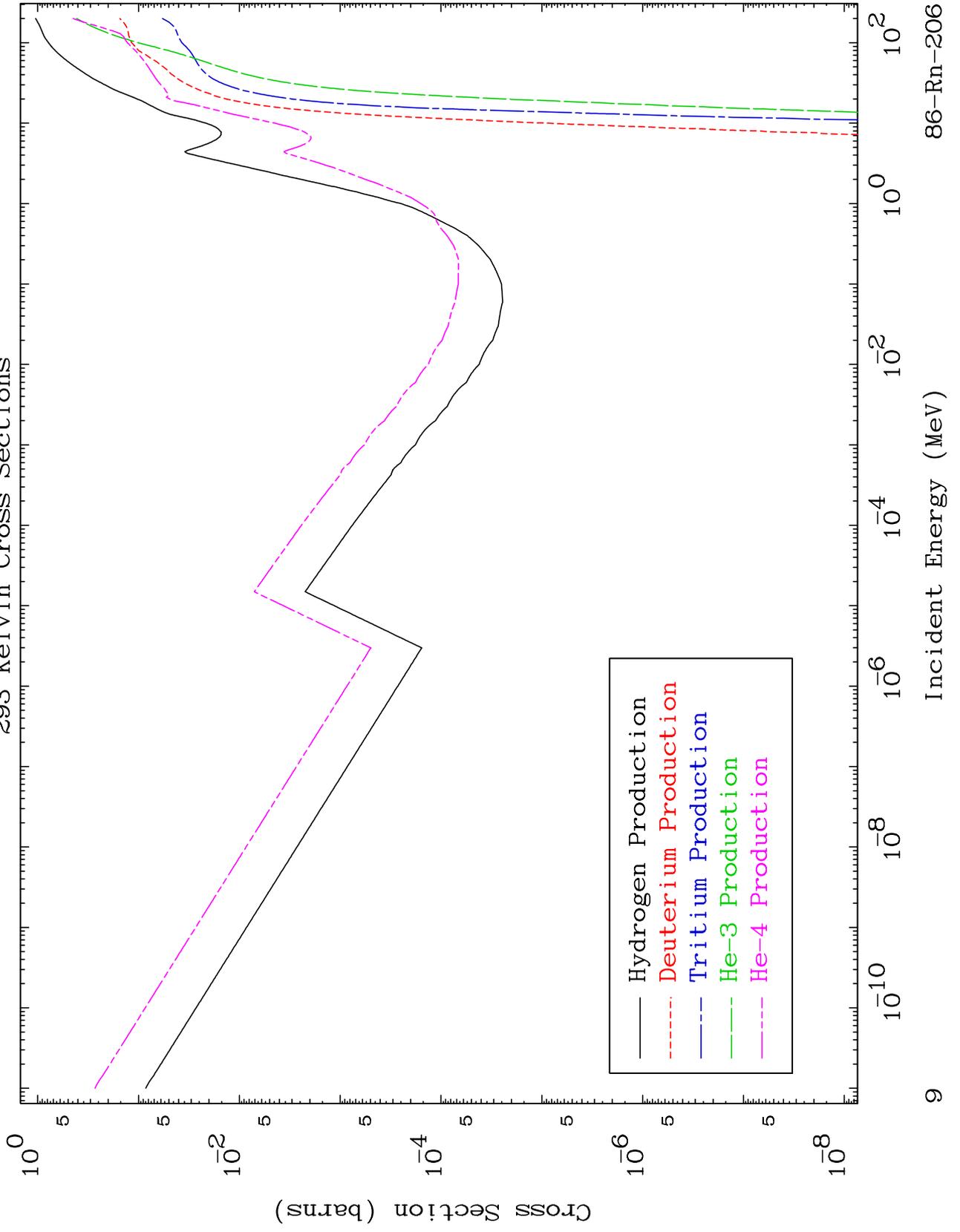


MAT 8610

Charged Particle  
293 Kelvin Cross Sections

86-Rn-206



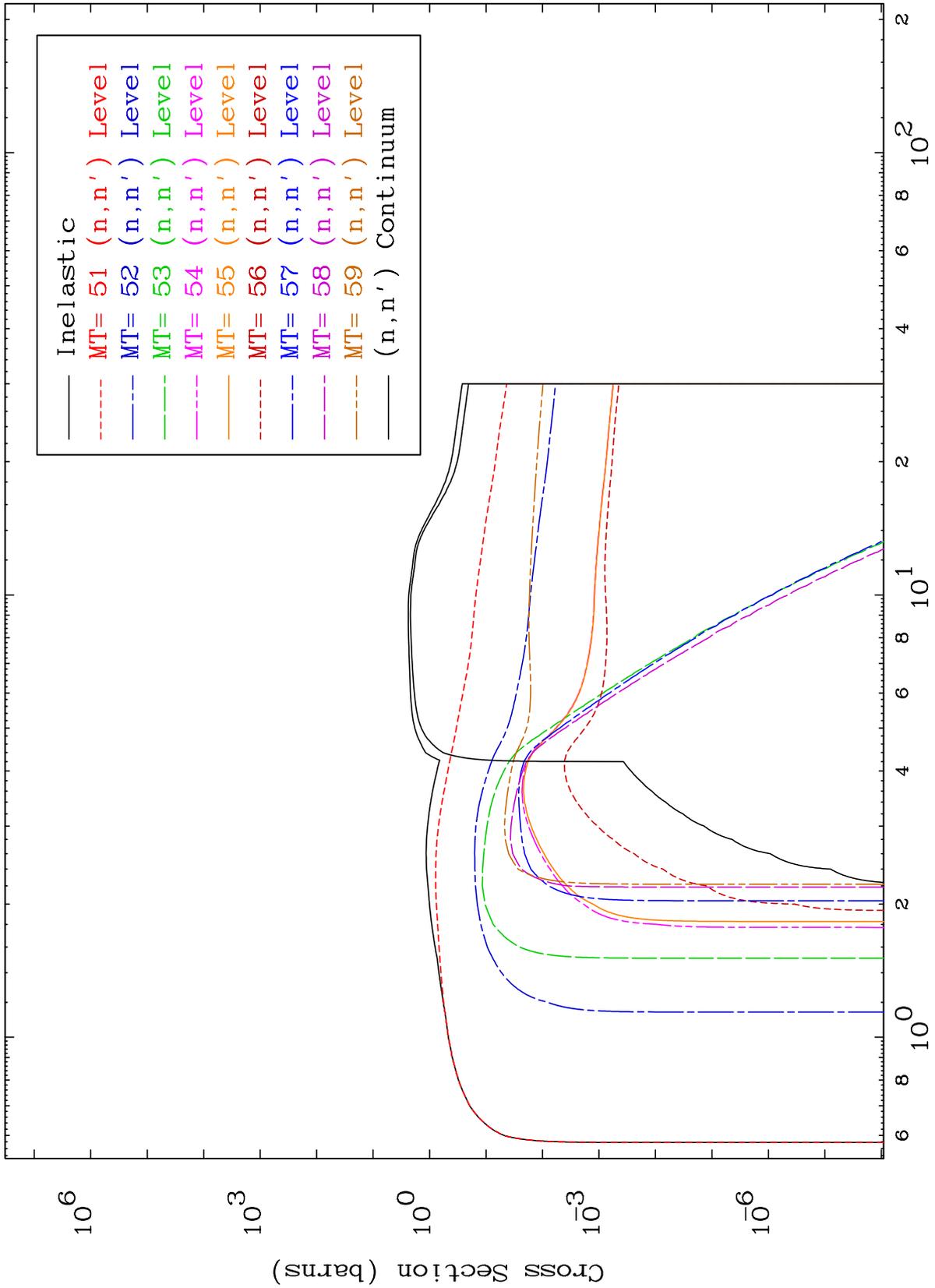


MAT 8610

(n,n') Levels

86-Rn-206

293 Kelvin Cross Sections



10

Incident Energy (MeV)

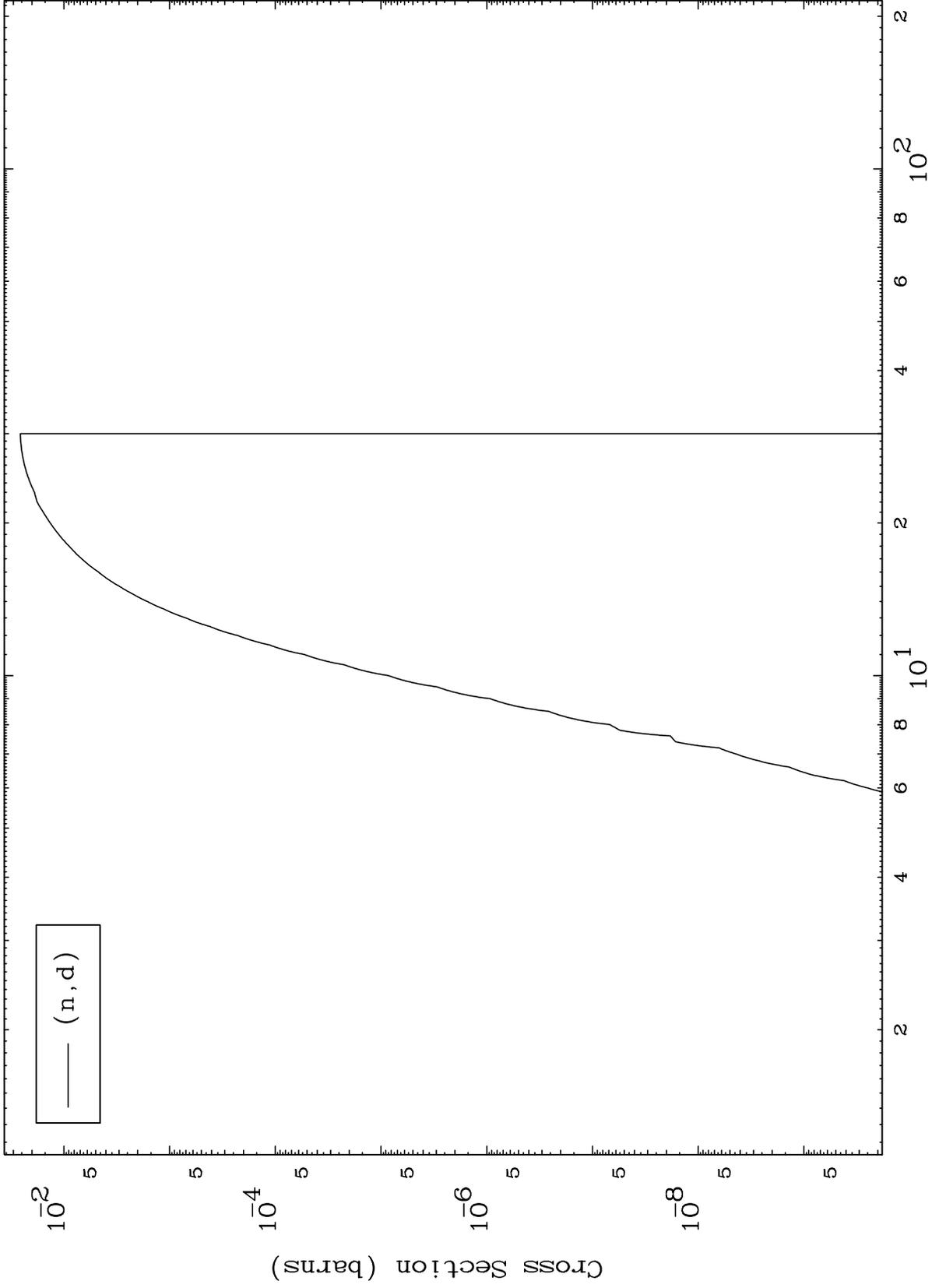
86-Rn-206



MAT 8610

(n,d) Levels  
293 Kelvin Cross Sections

86-Rn-206



12

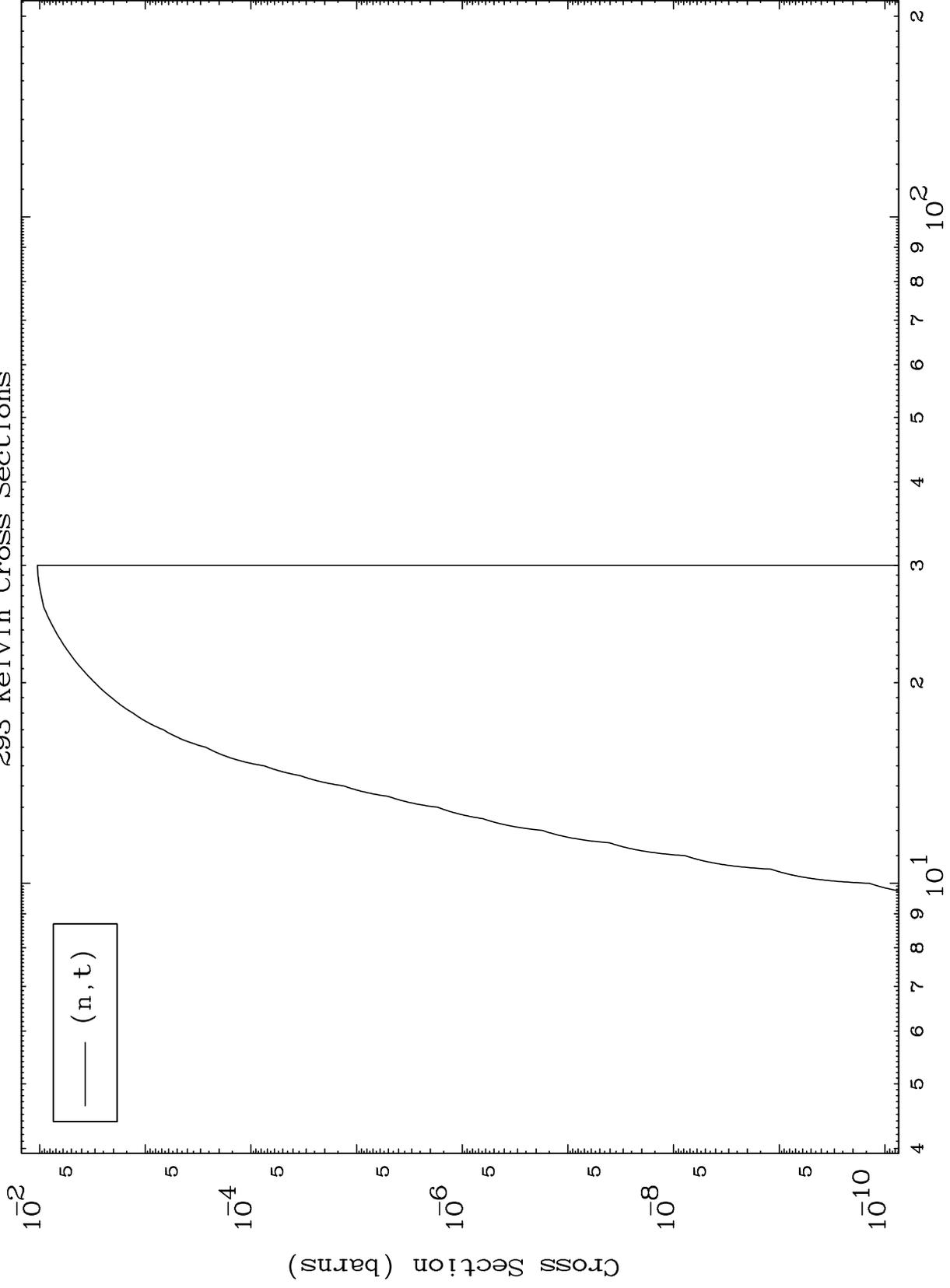
Incident Energy (MeV)

86-Rn-206

MAT 8610

(n,t) Levels  
293 Kelvin Cross Sections

86-Rn-206



13

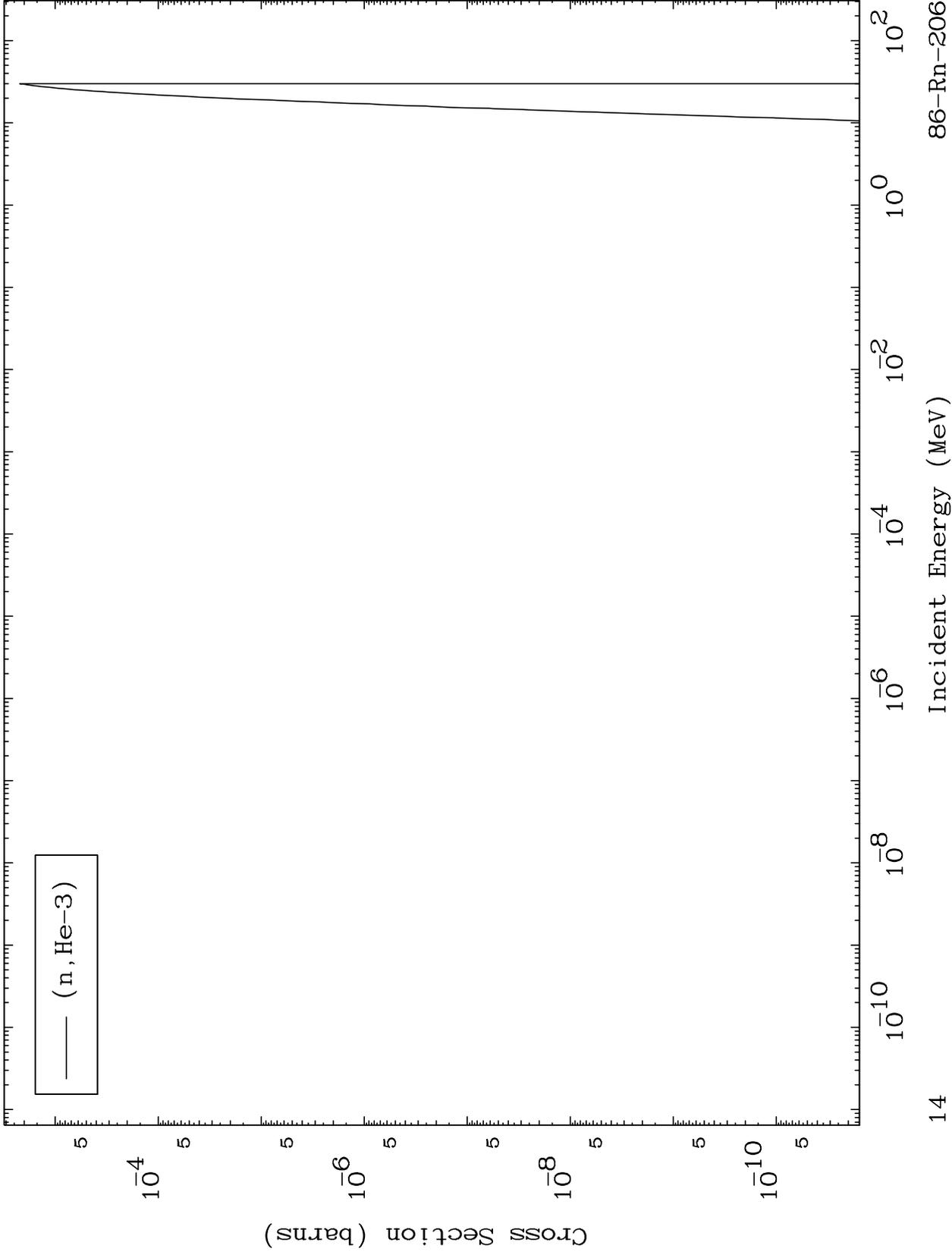
Incident Energy (MeV)

86-Rn-206

MAT 8610

(n,He3) Levels  
293 Kelvin Cross Sections

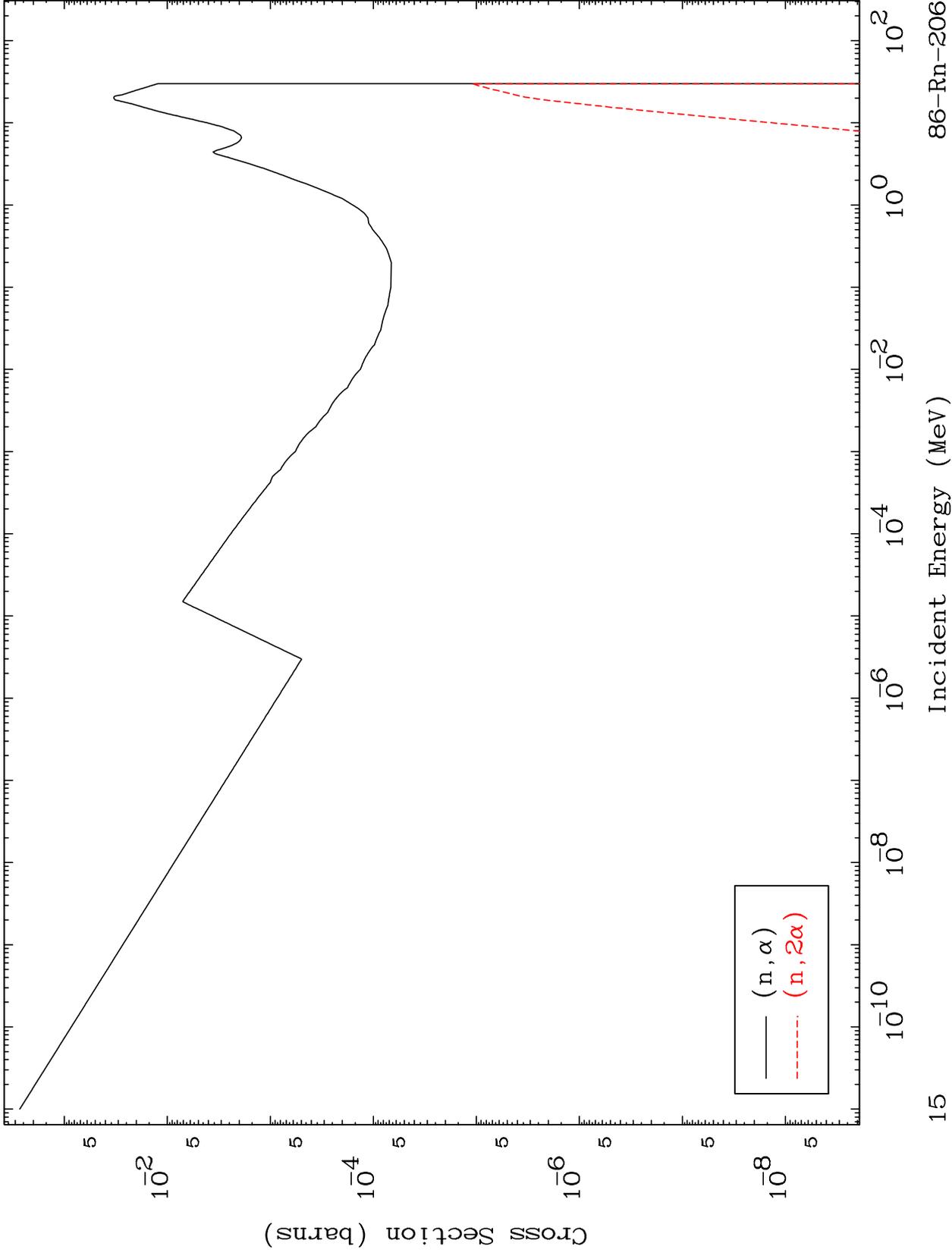
86-Rn-206



MAT 8610

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

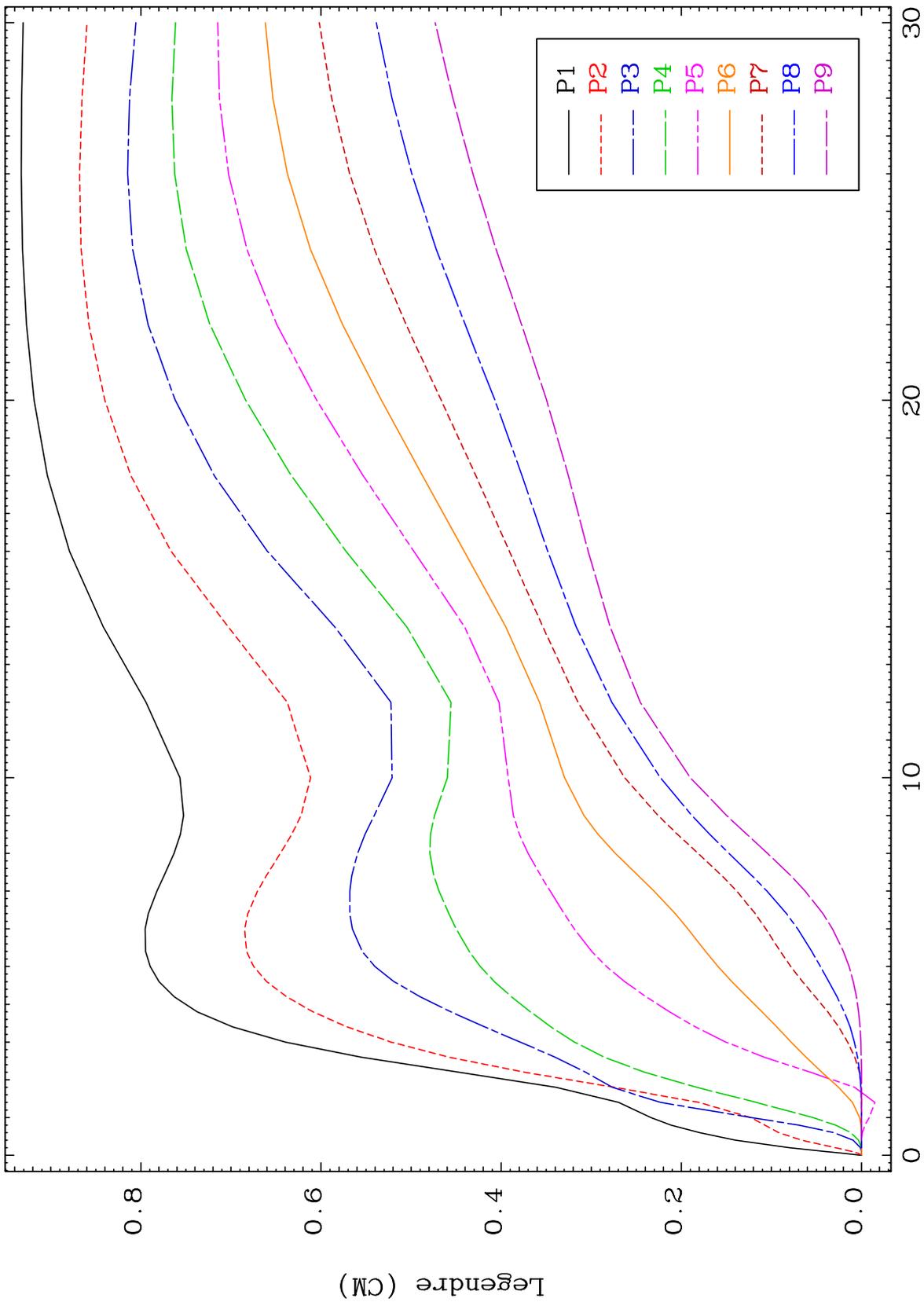
86-Rn-206



MAT 8610

86-Rn-206

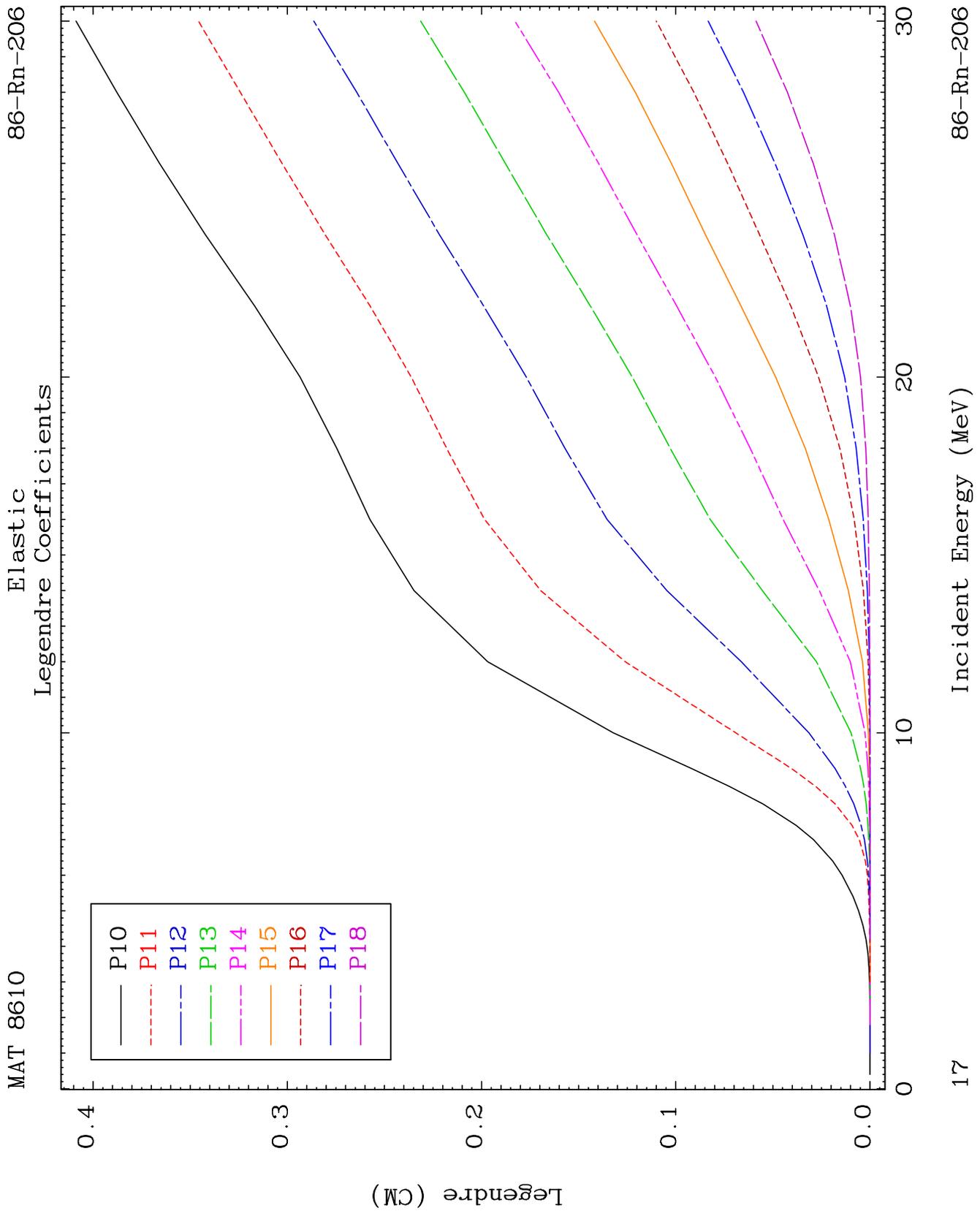
Elastic Legendre Coefficients



86-Rn-206

Incident Energy (MeV)

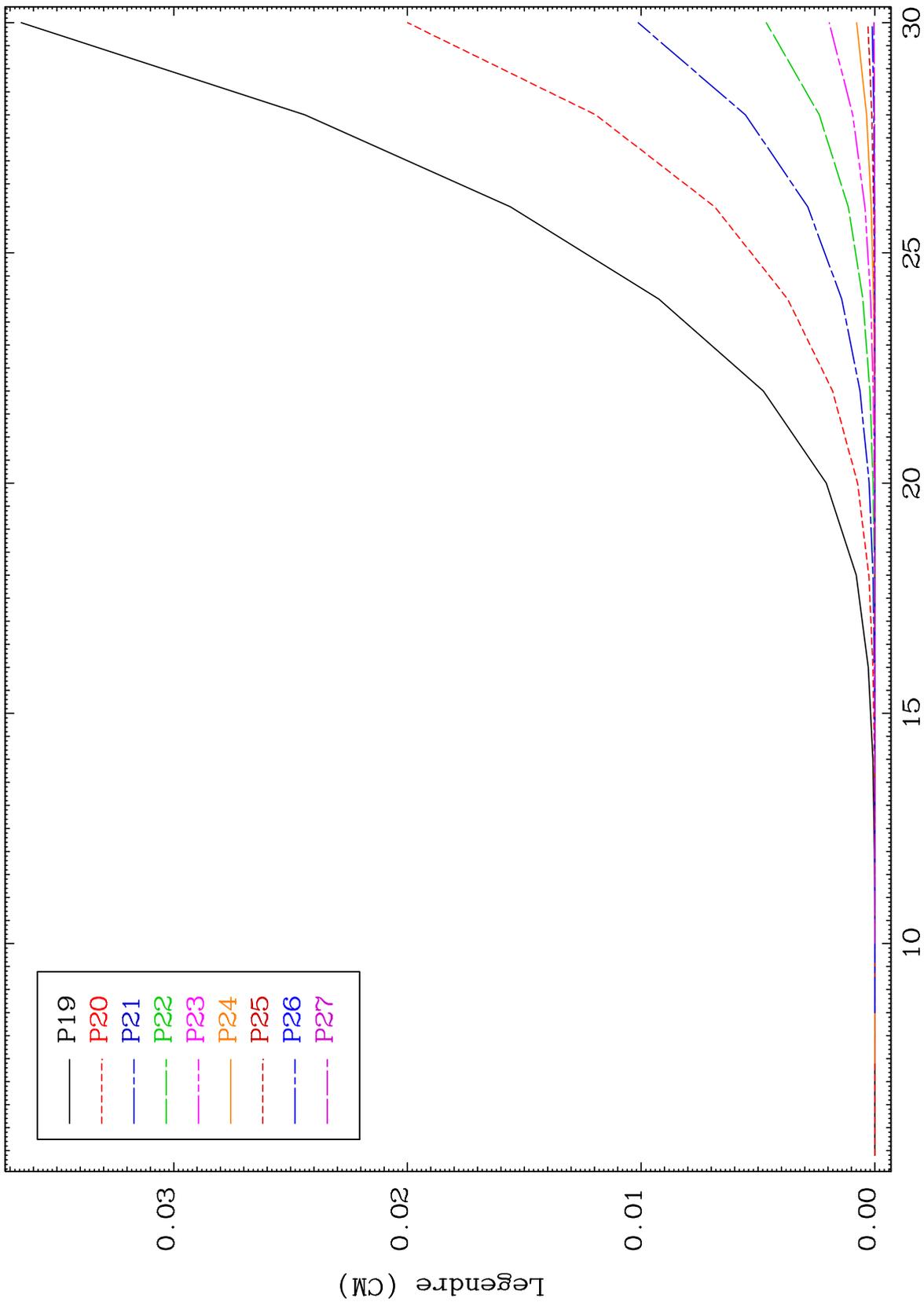
16



MAT 8610

Elastic  
Legendre Coefficients

86-Rn-206



18

Incident Energy (MeV)

86-Rn-206

30

25

20

15

10

Legendre (CM)

0.03

0.02

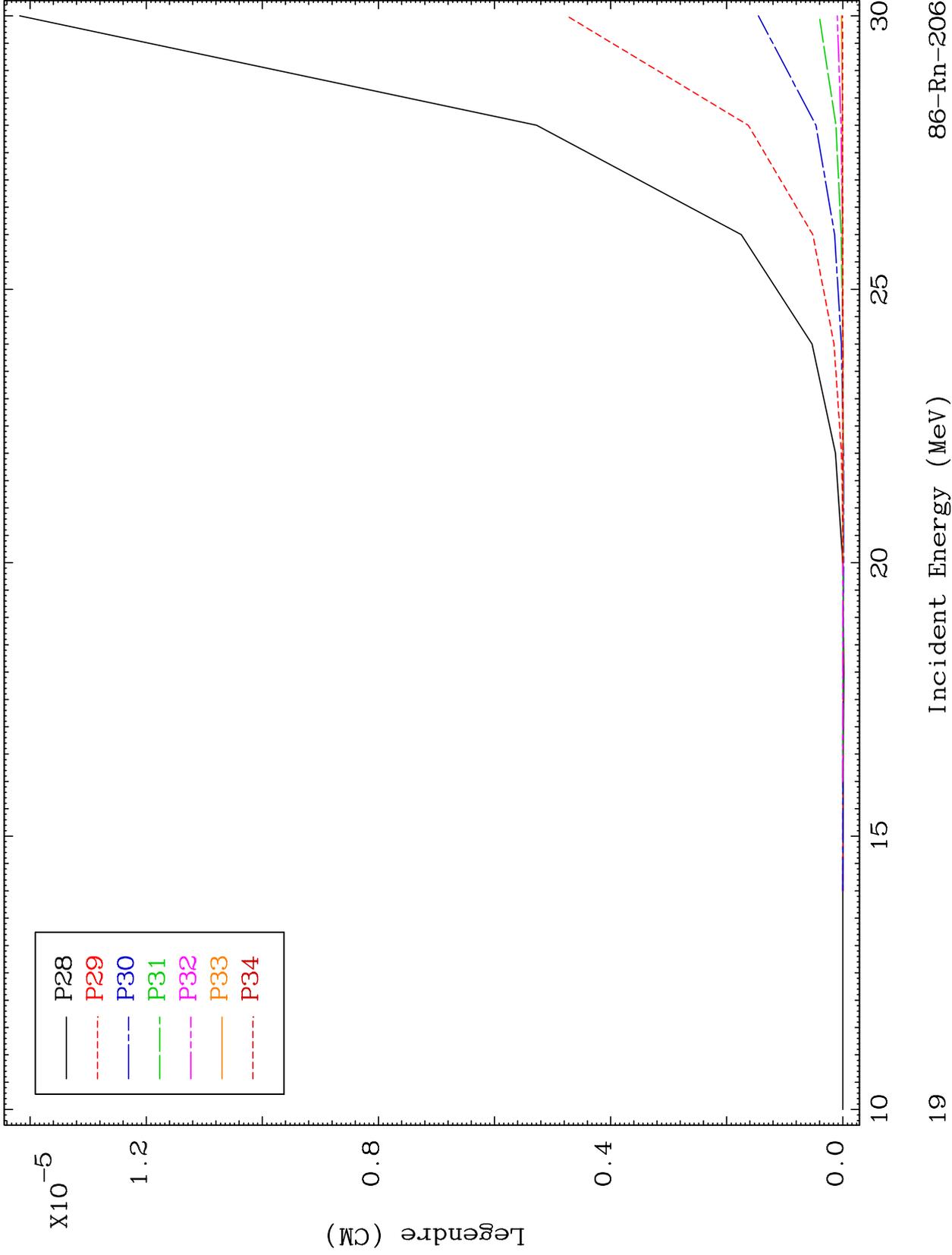
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0.00

MAT 8610

Elastic  
Legendre Coefficients

86-Rn-206



19

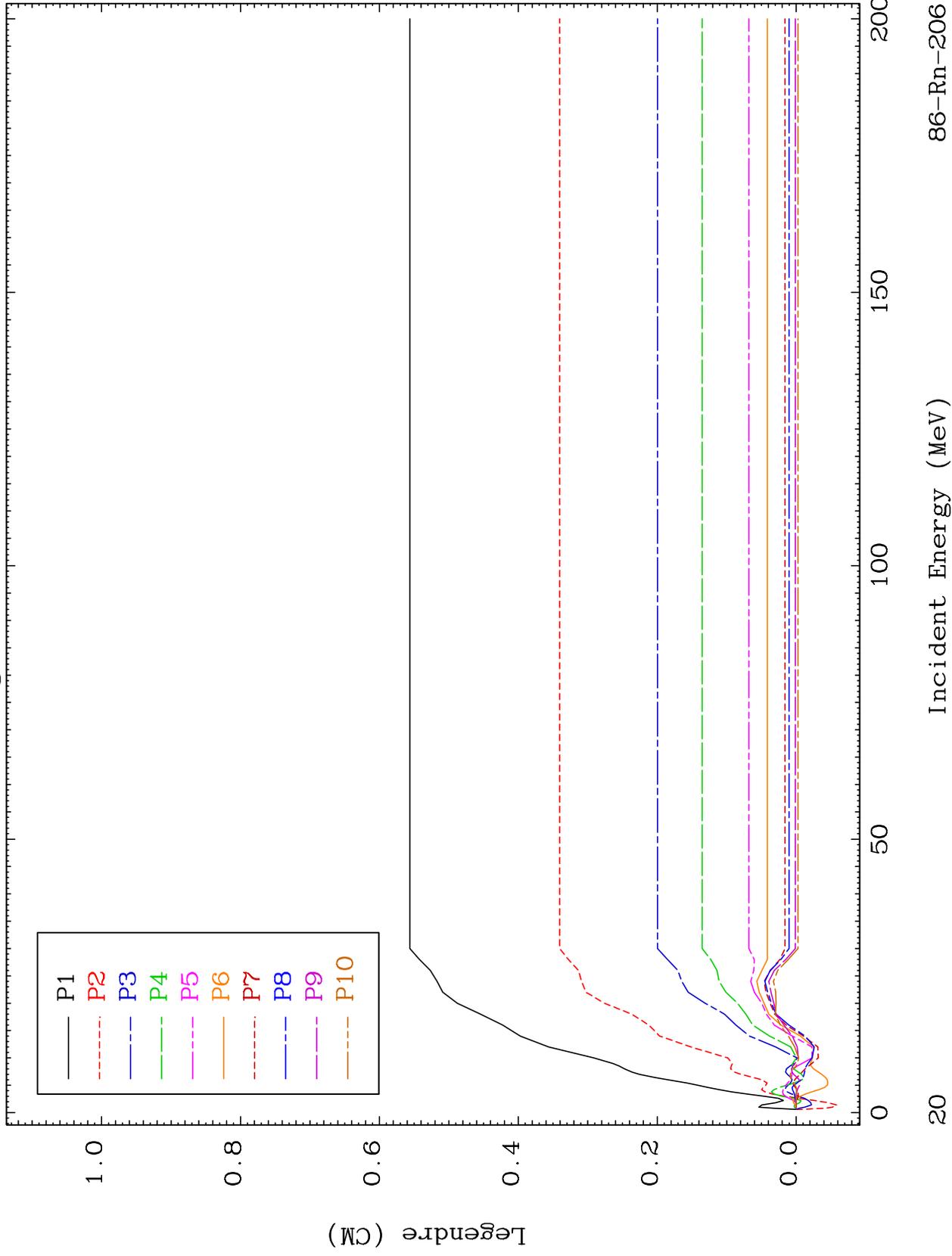
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

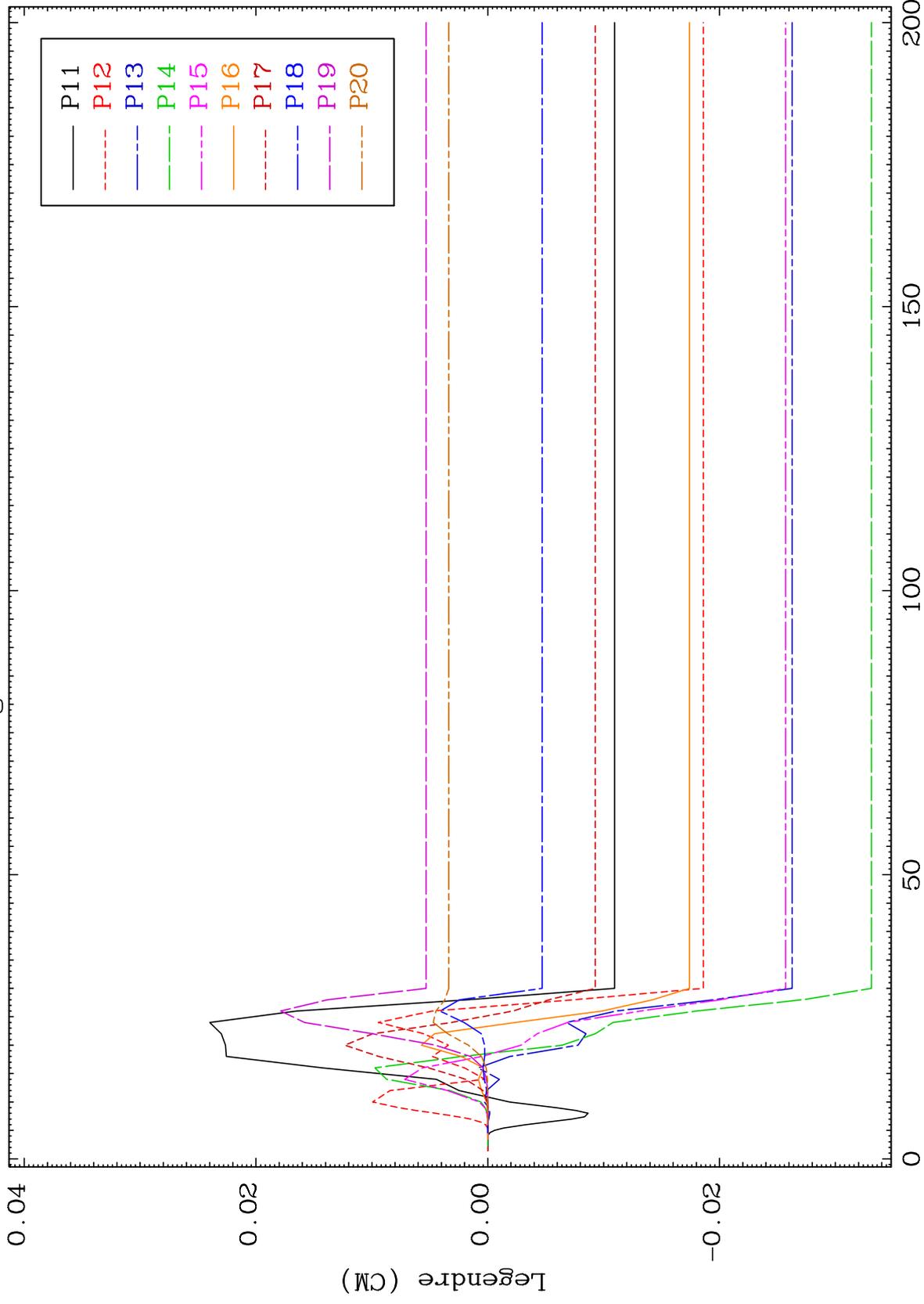
86-Rn-206



MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

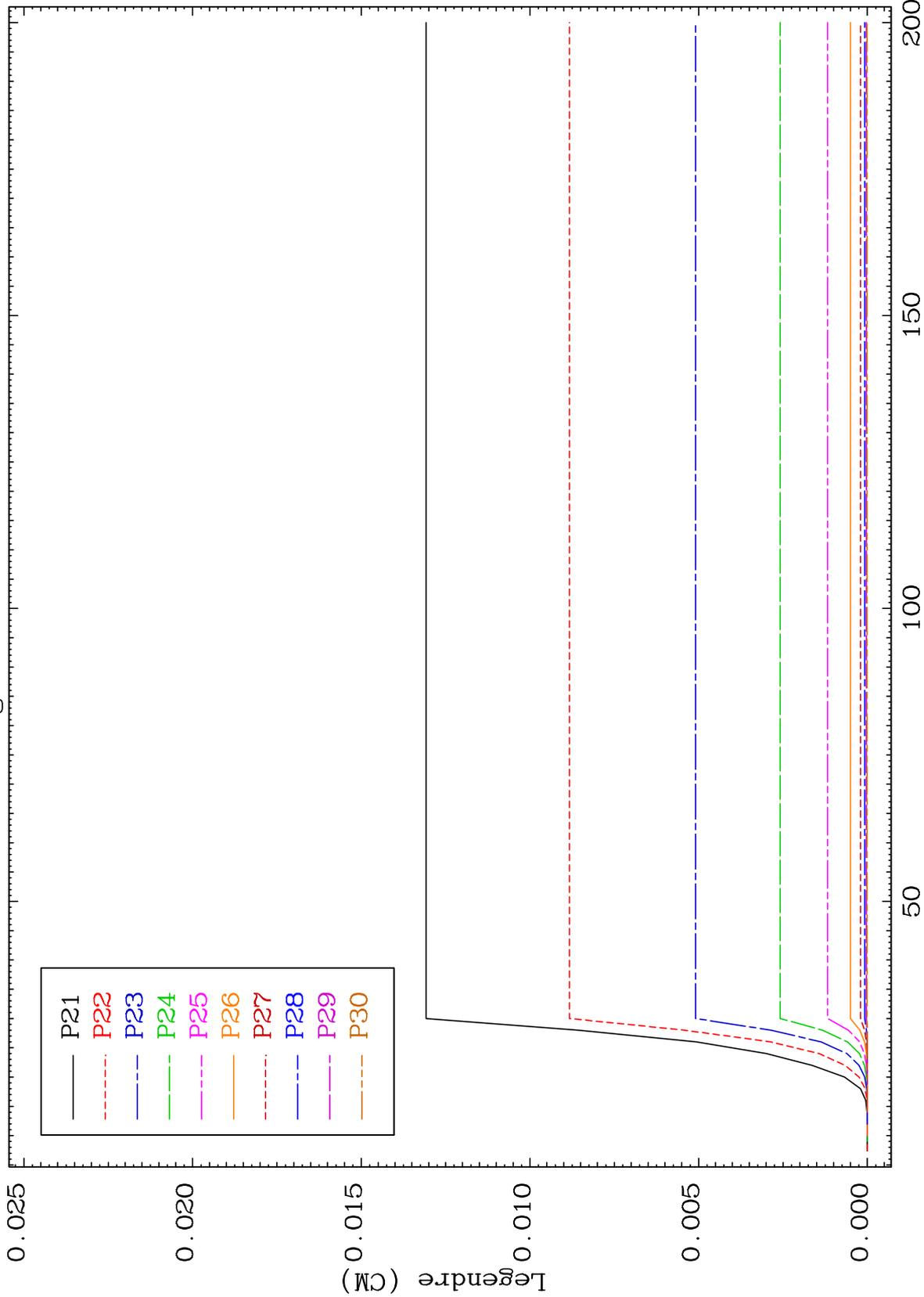
86-Rn-206



21

Incident Energy (MeV)

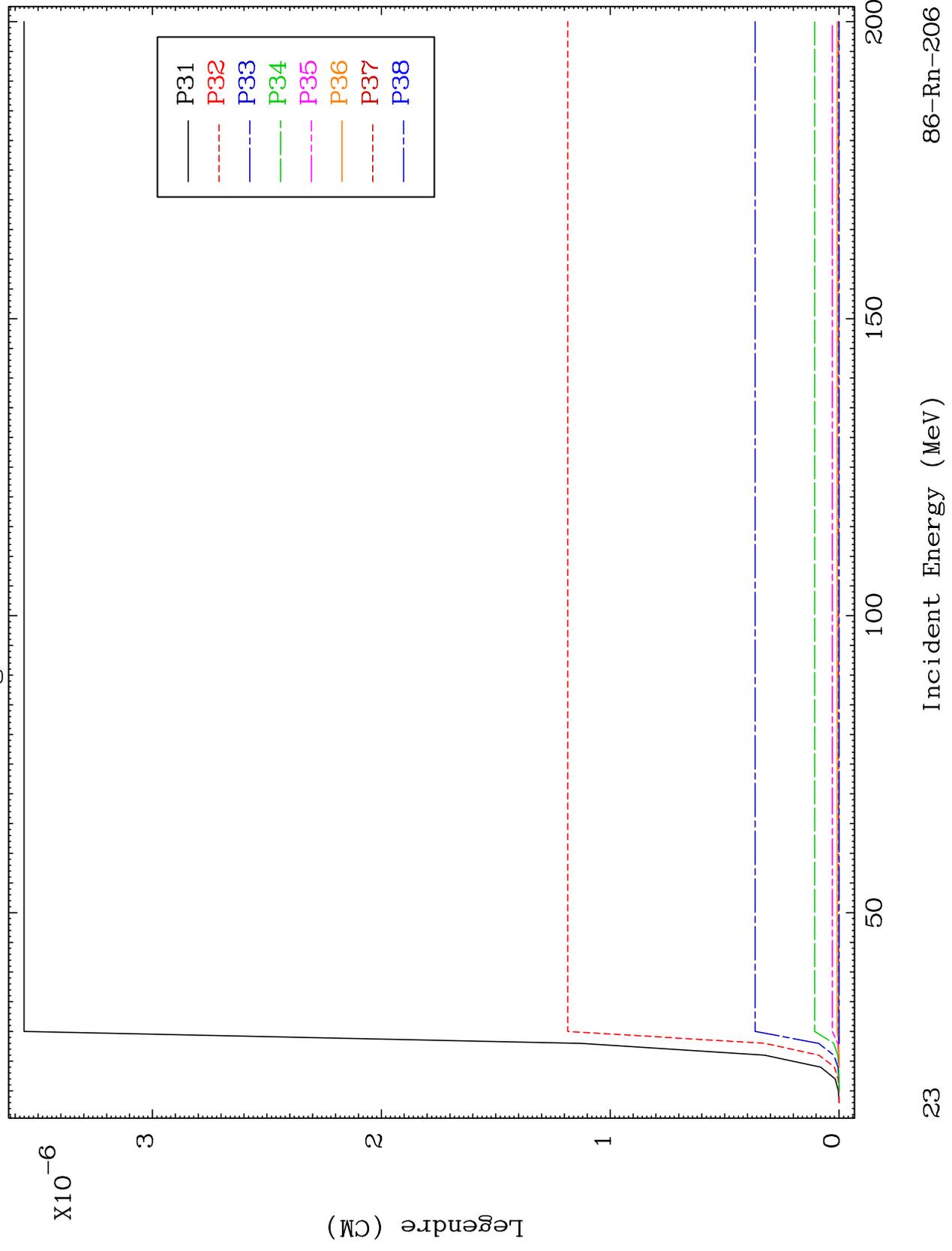
86-Rn-206



MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

86-Rn-206



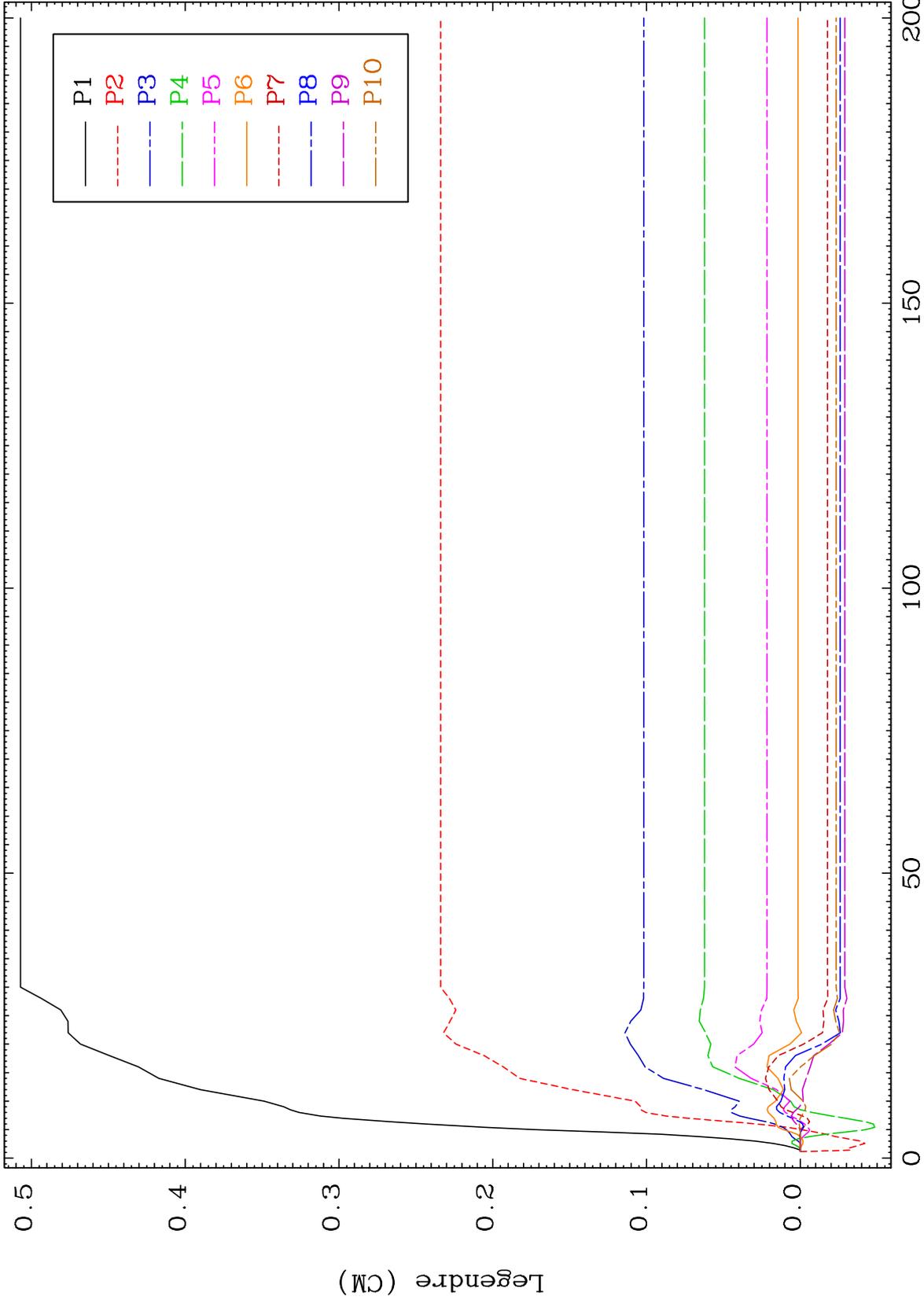
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MAT 8610

MT= 52 (n,n') Level

86-Rn-206

Legendre Coefficients



24

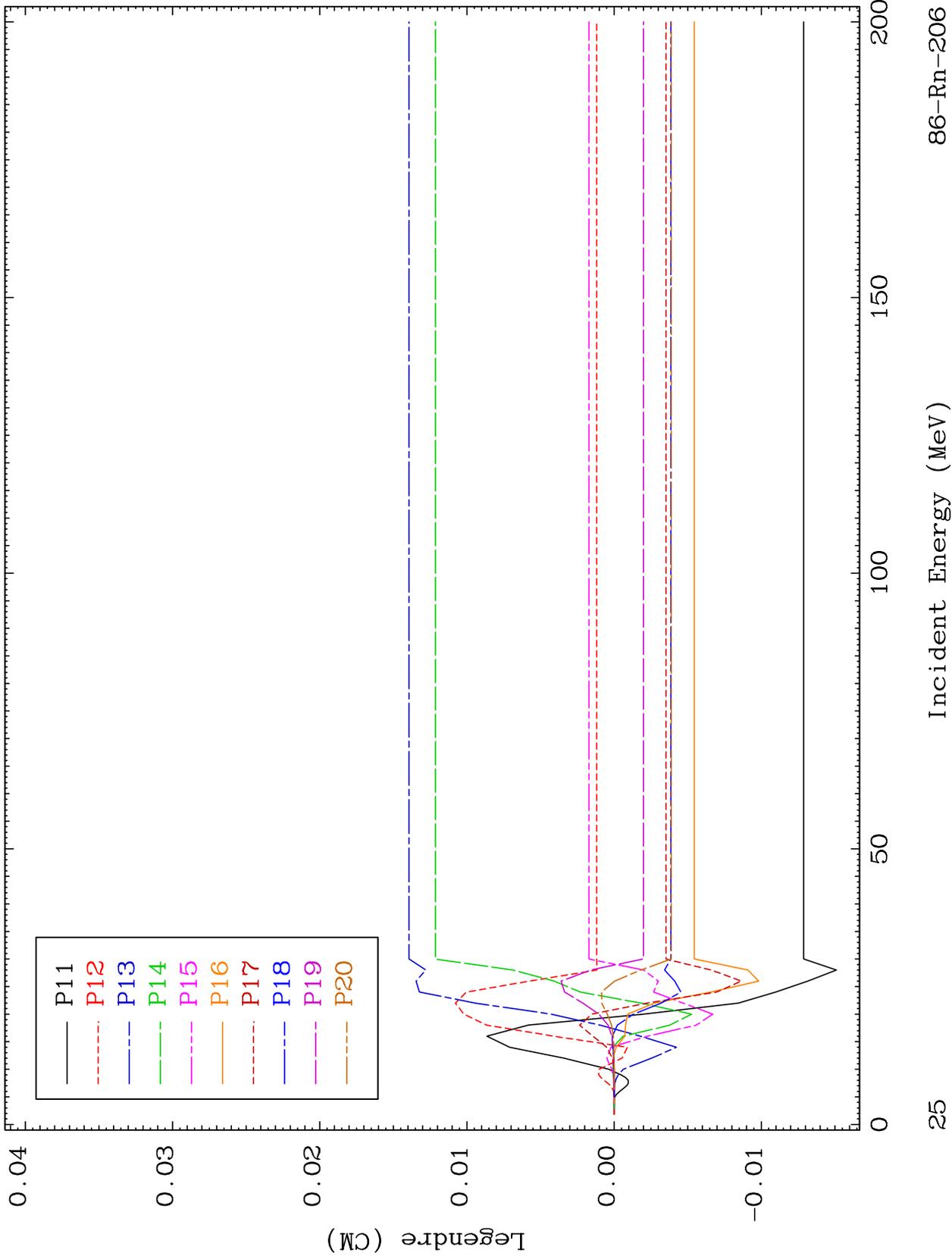
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 52 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

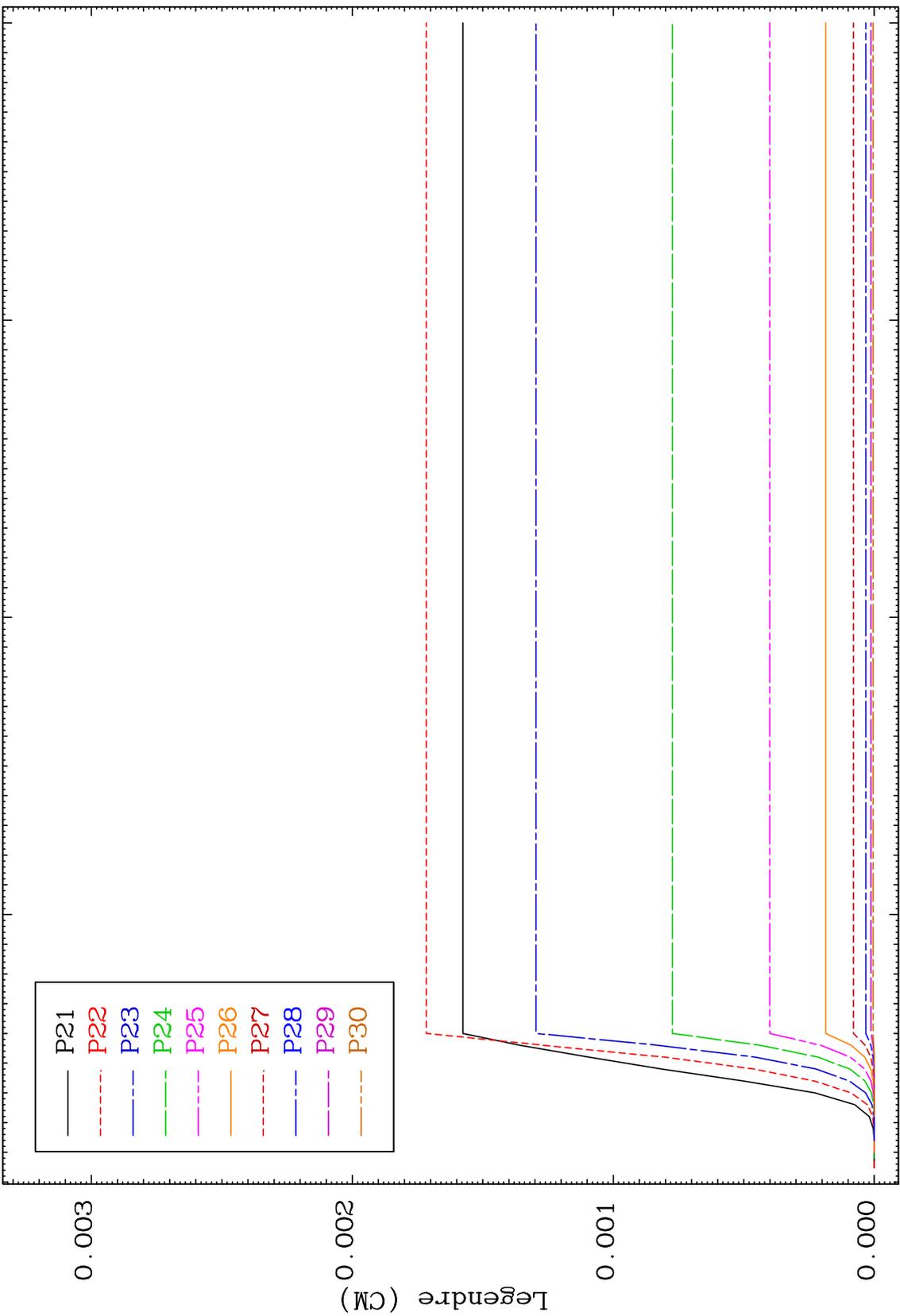
Incident Energy (MeV)

25

MAT 8610

MT= 52 (n,n') Level  
Legendre Coefficients

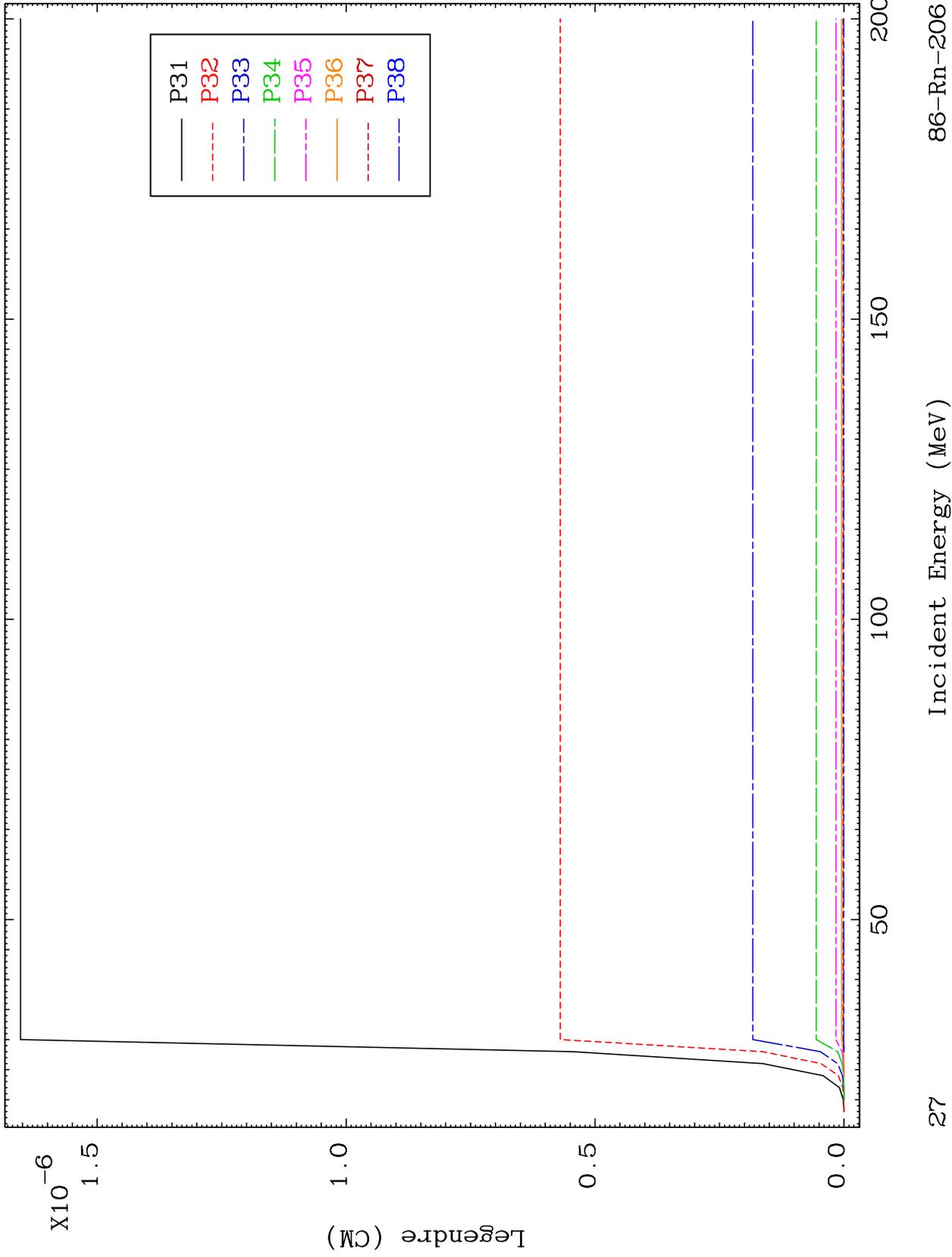
86-Rn-206



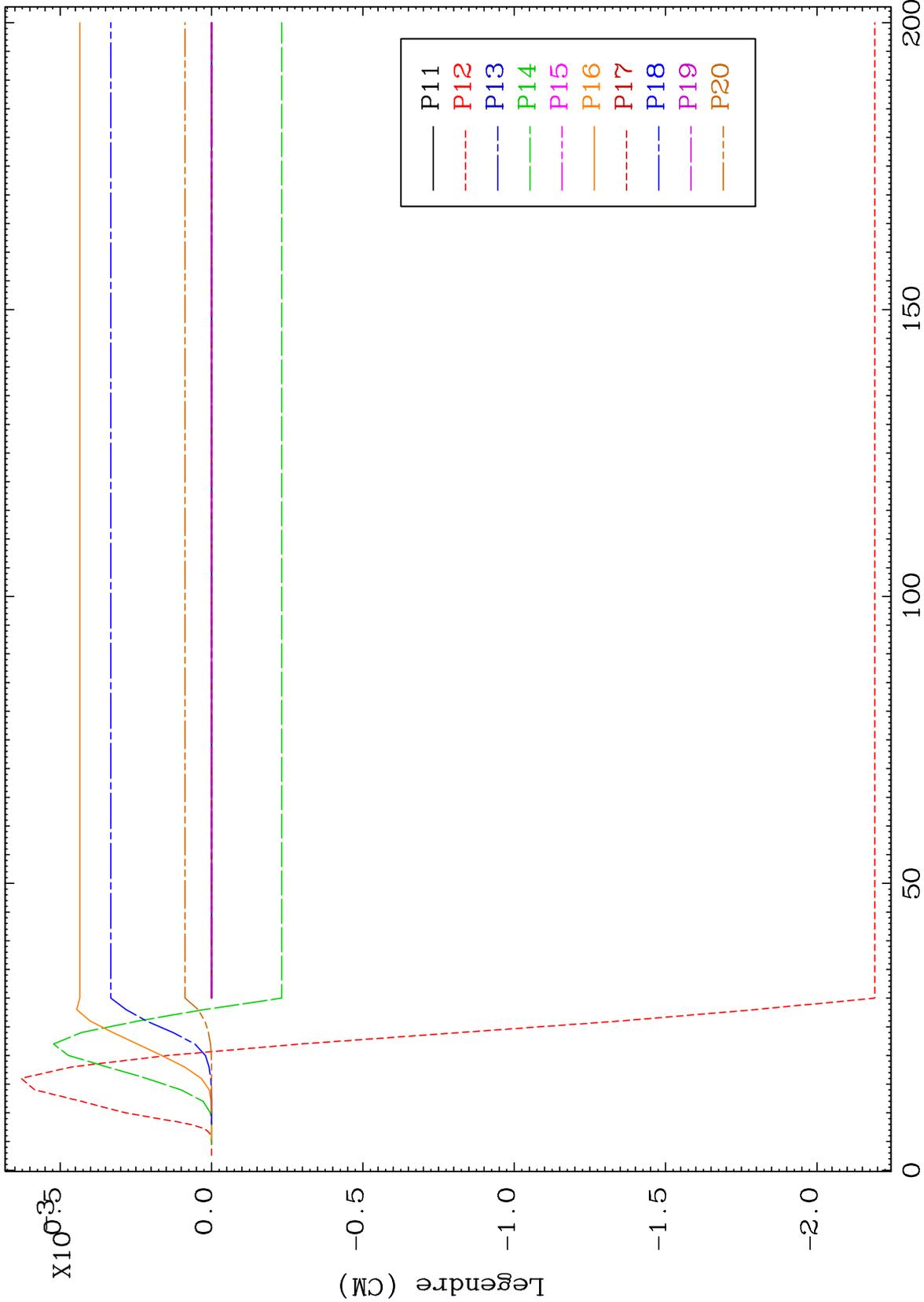
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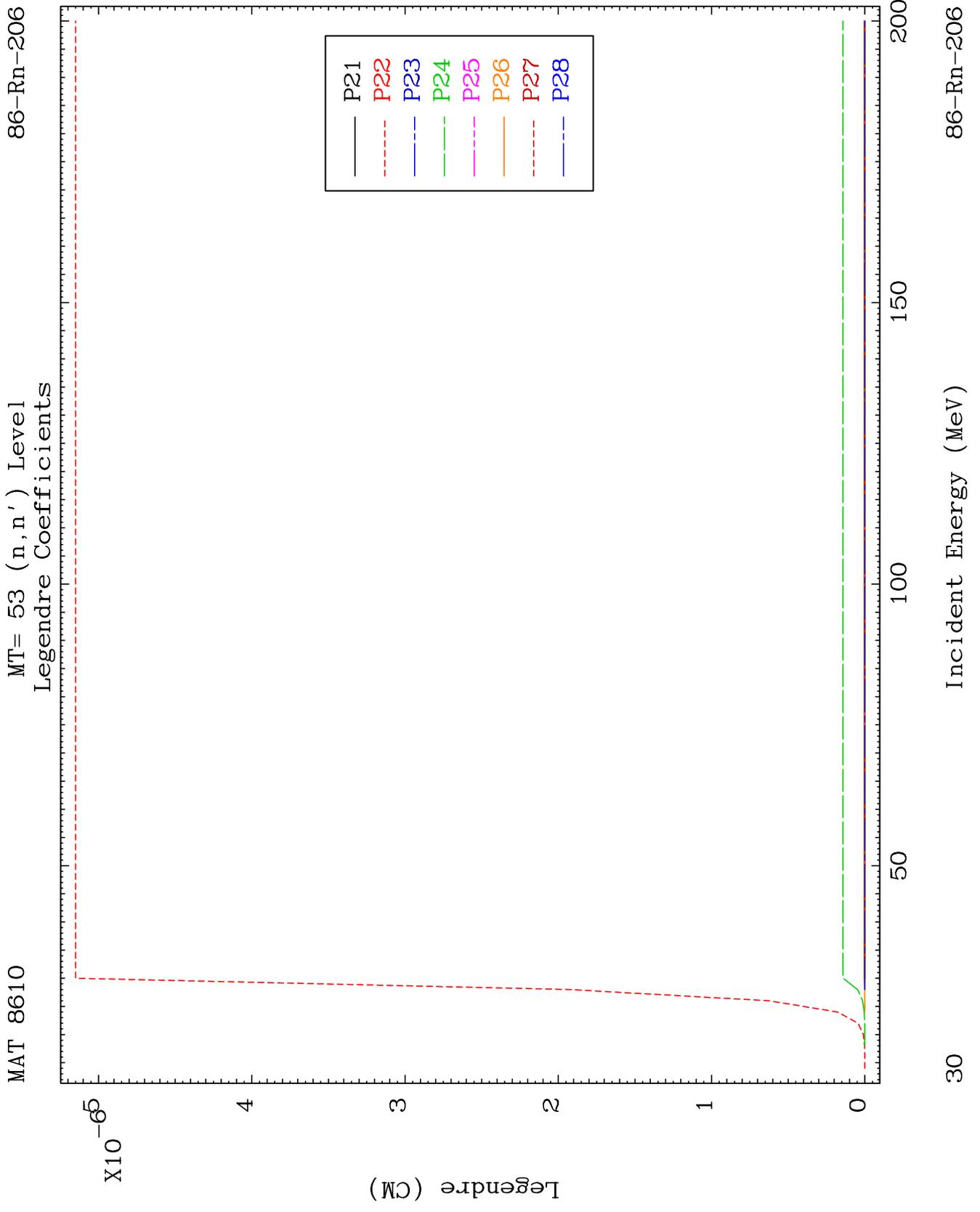
Incident Energy (MeV)

86-Rn-206





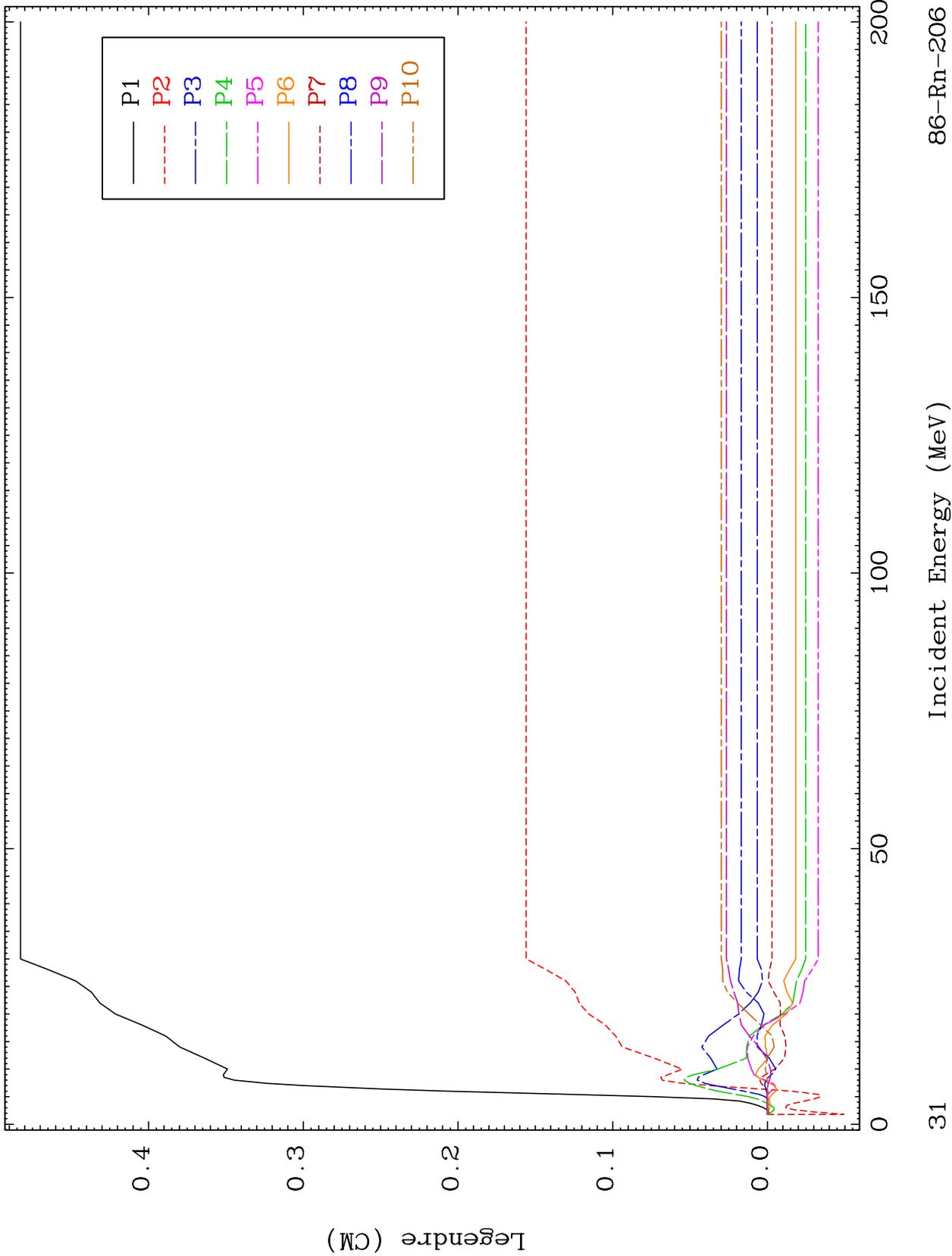




MAT 8610

MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206



31

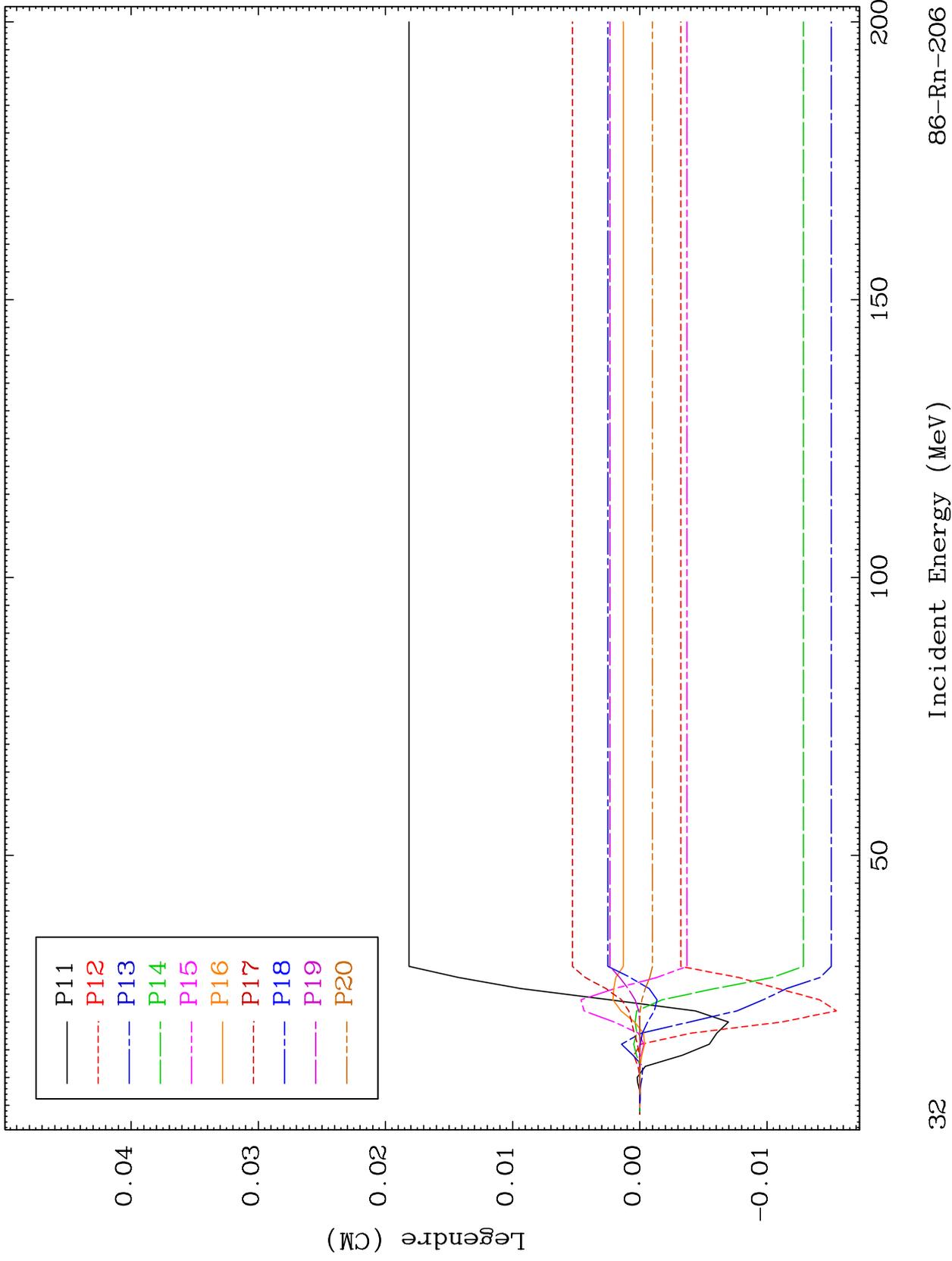
Incident Energy (MeV)

86-Rn-206

MAT 8610

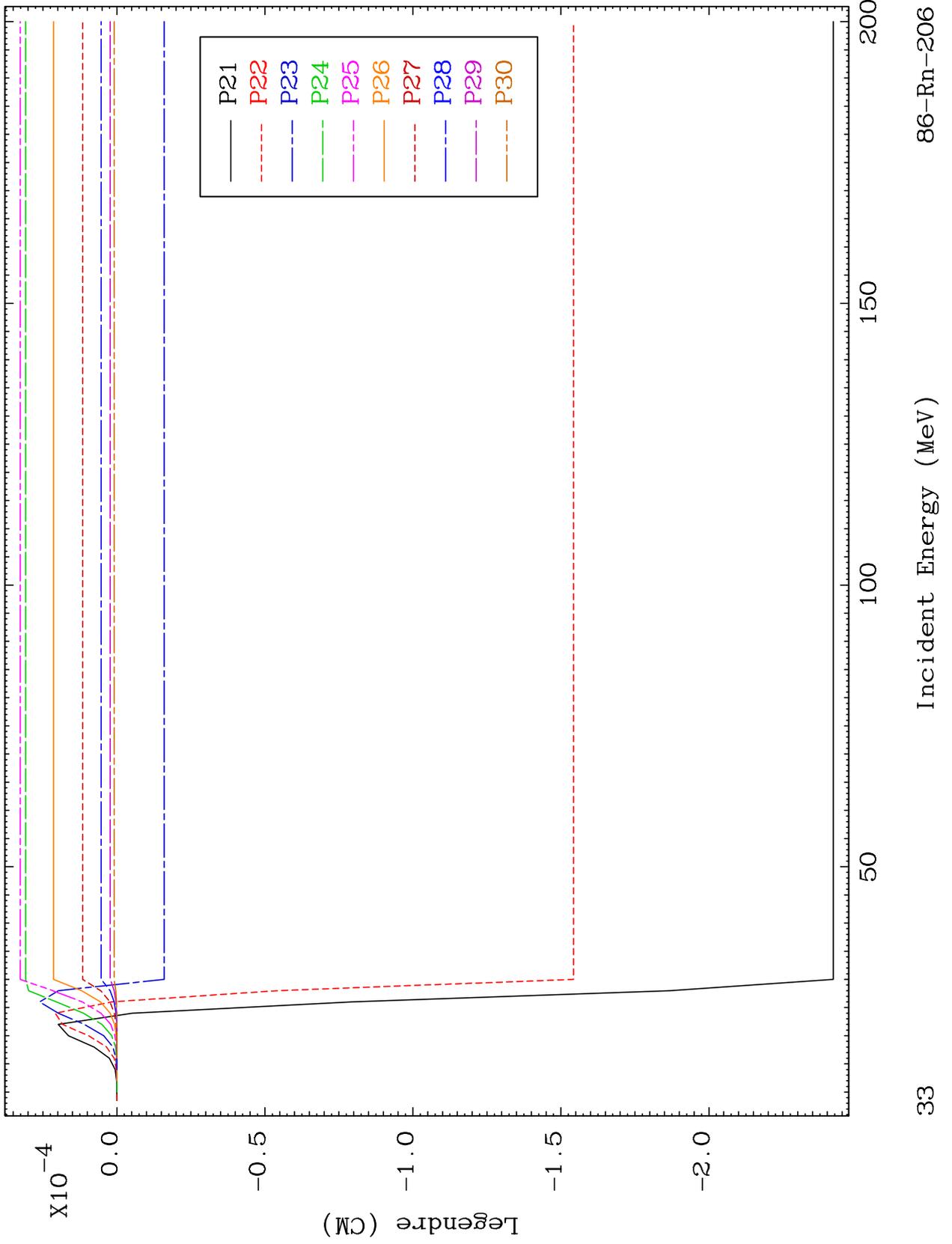
MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206



32

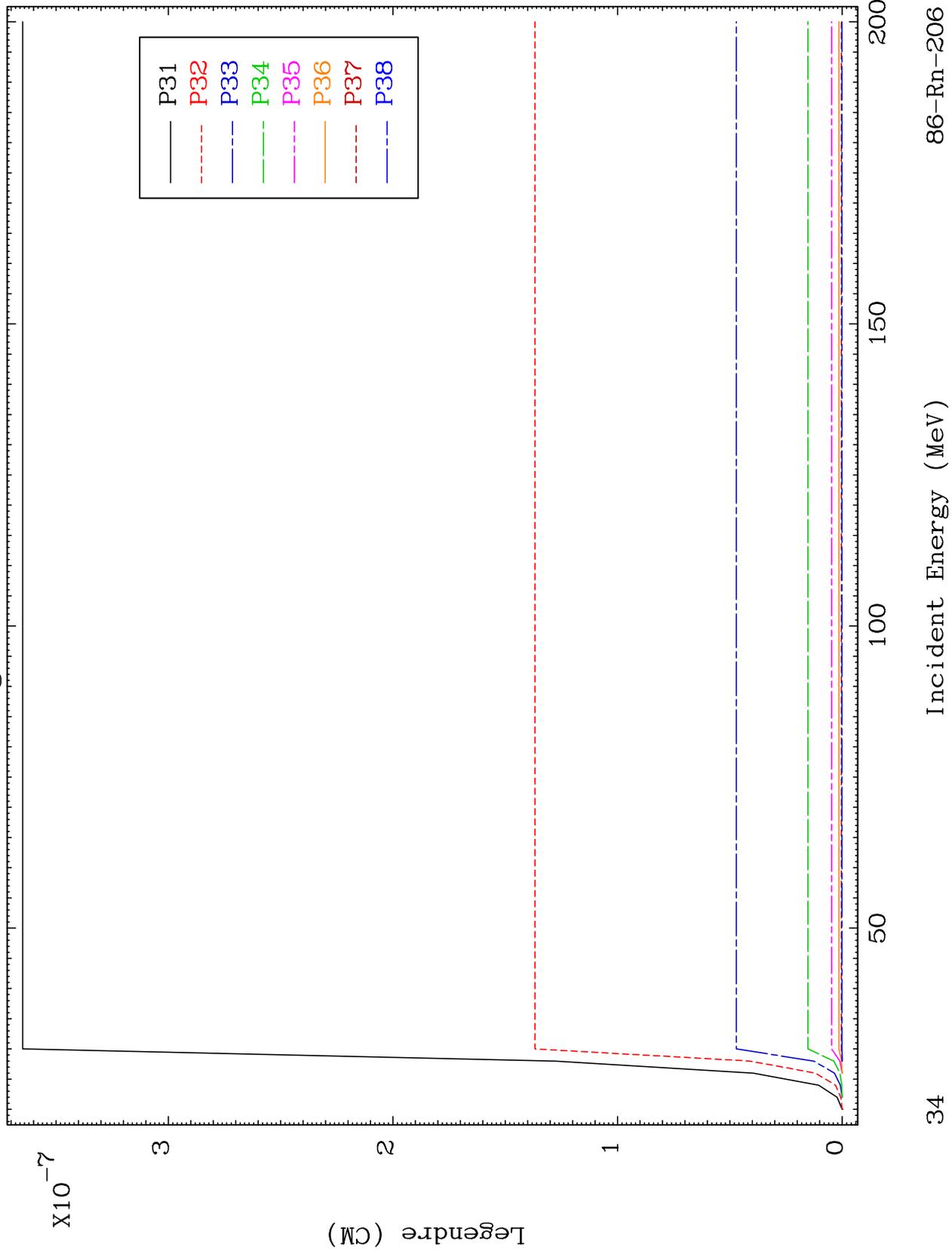
86-Rn-206



MAT 8610

MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206



34

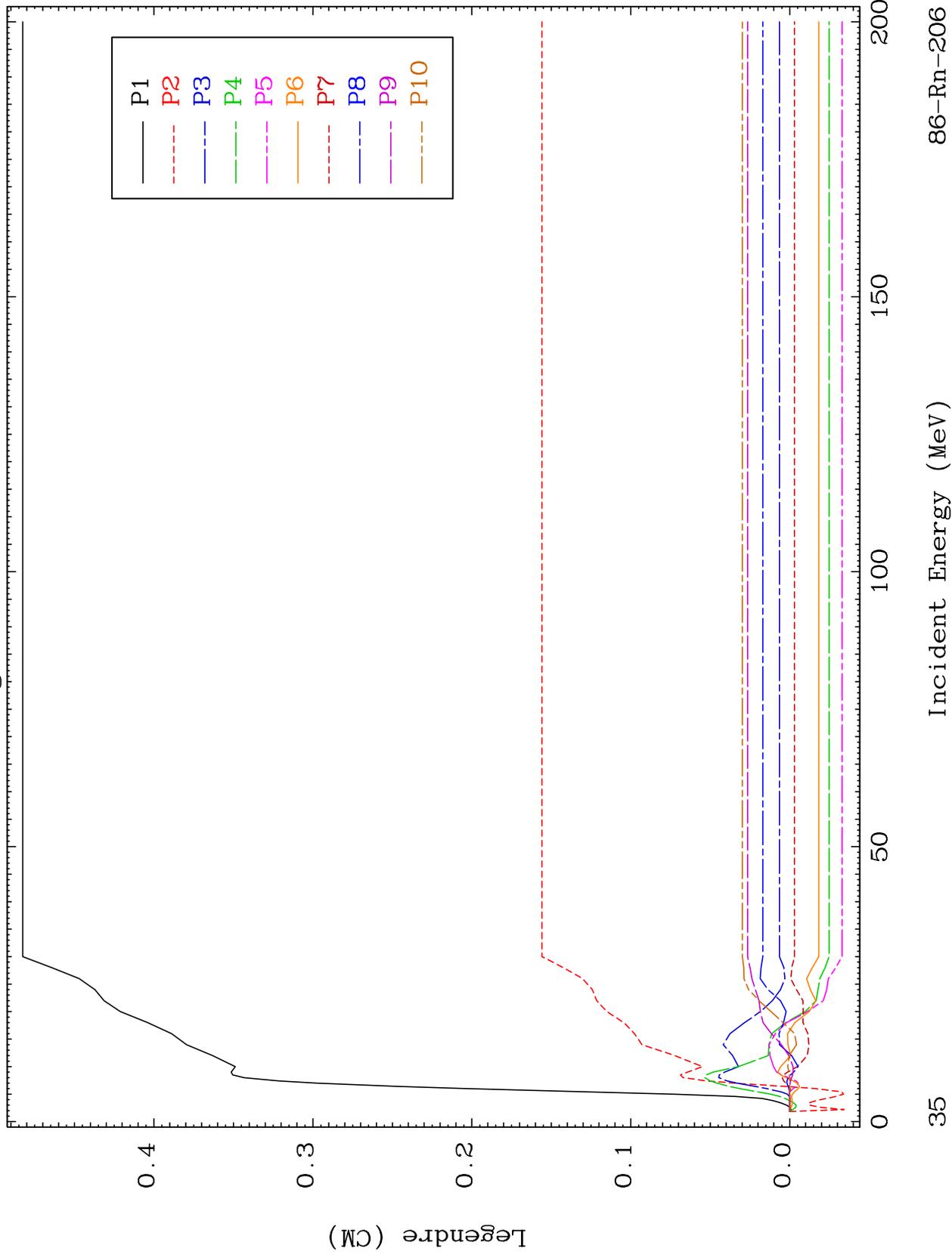
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 55 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

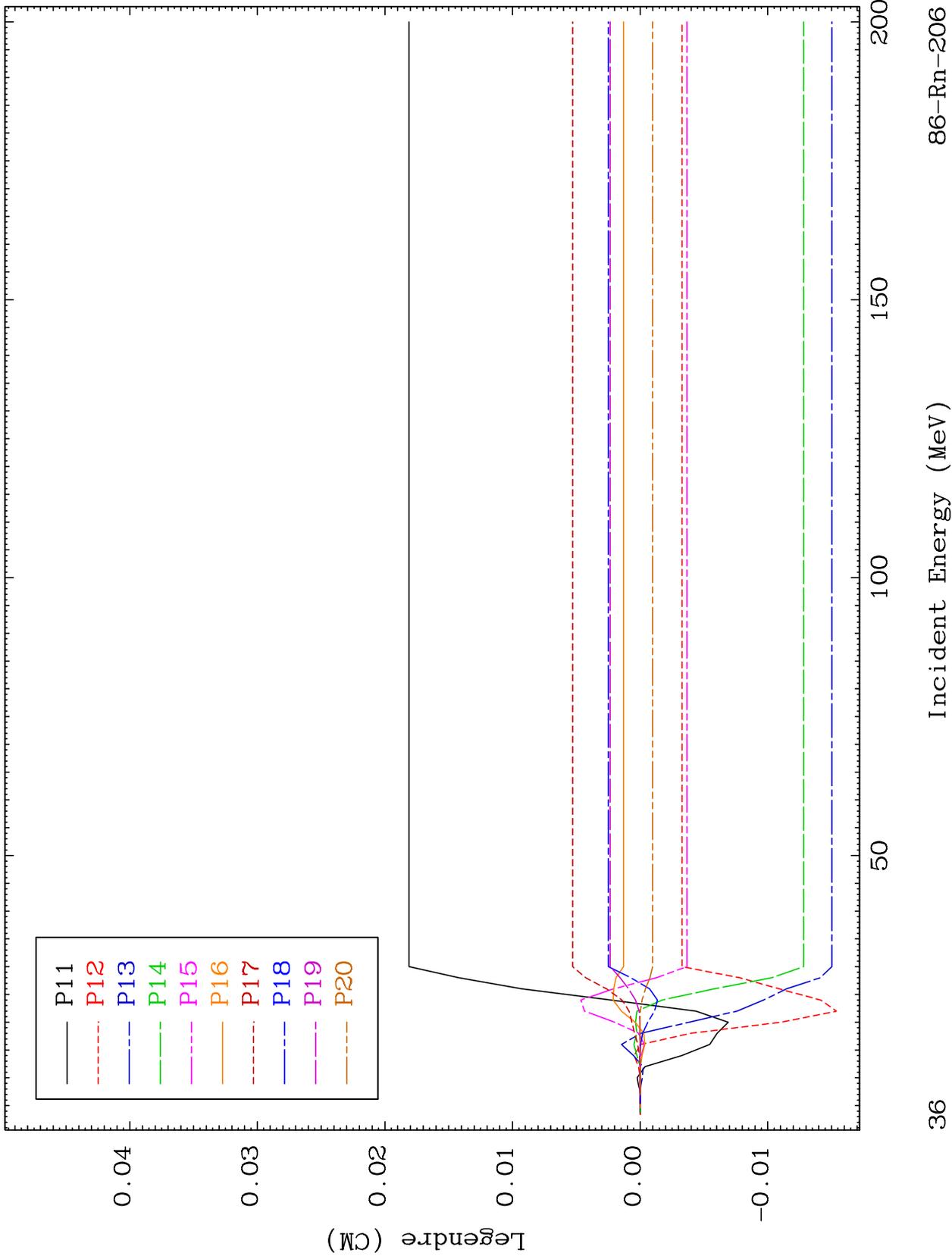
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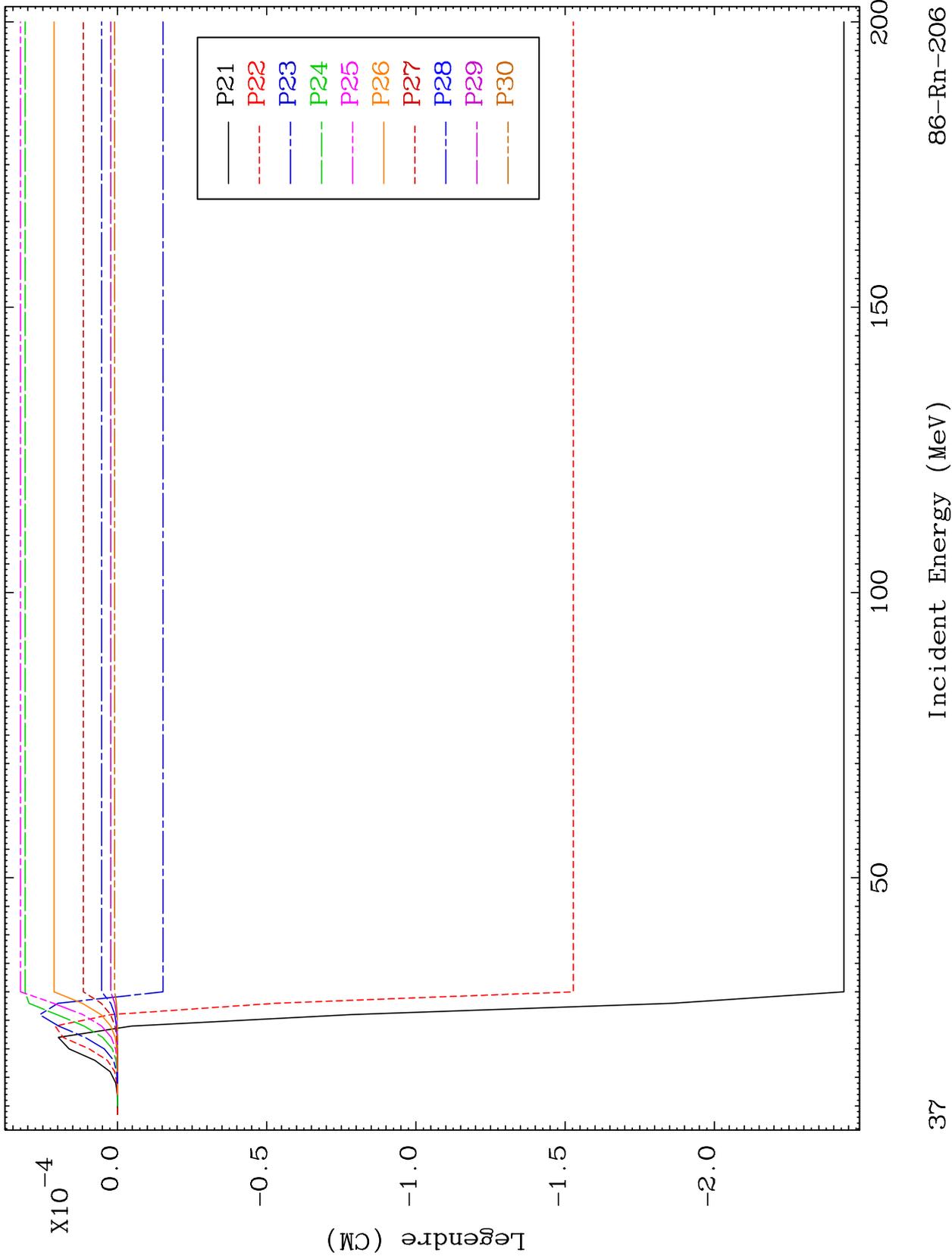
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MAT 8610

MT= 55 (n,n') Level  
Legendre Coefficients

86-Rn-206

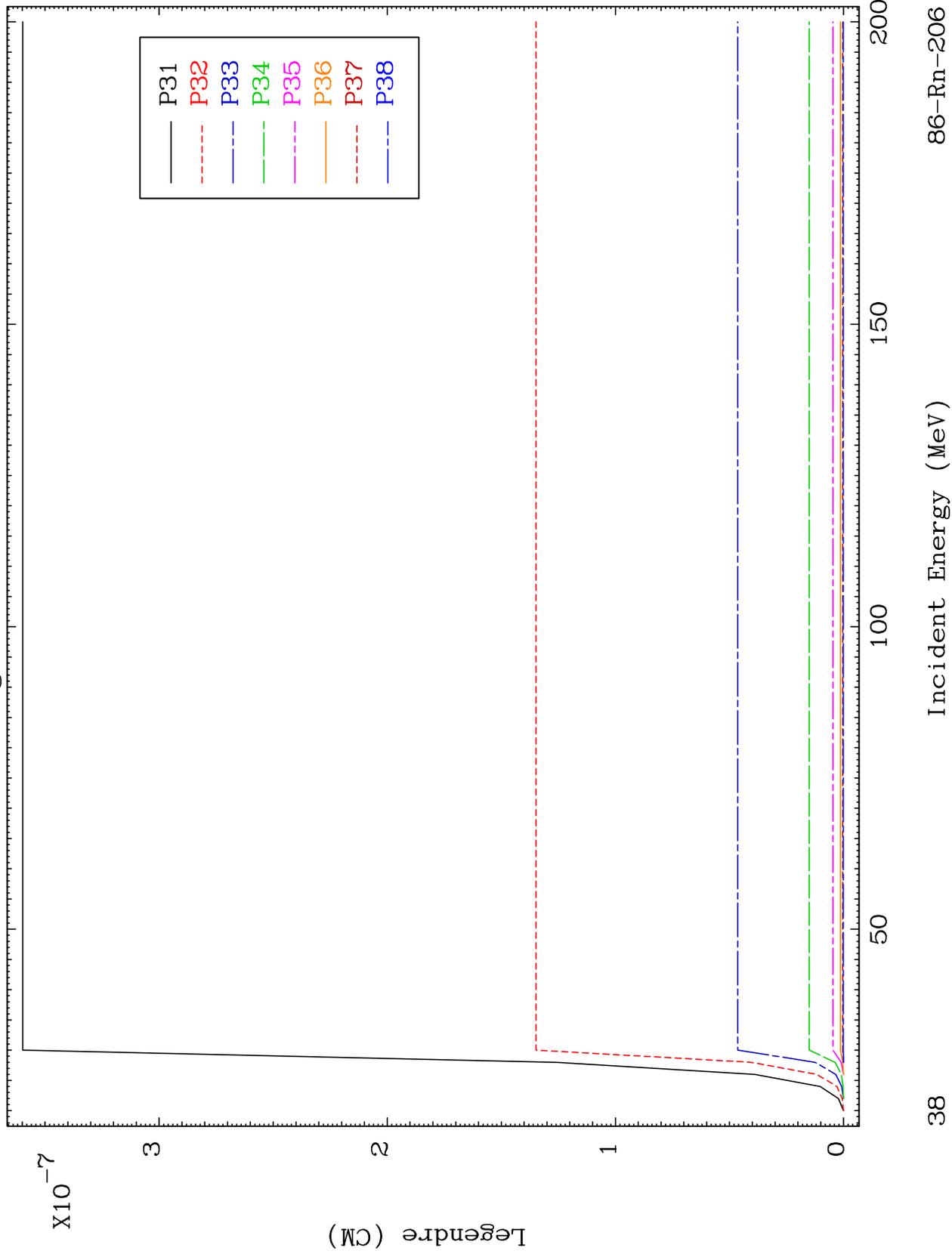




MAT 8610

MT= 55 (n,n') Level  
Legendre Coefficients

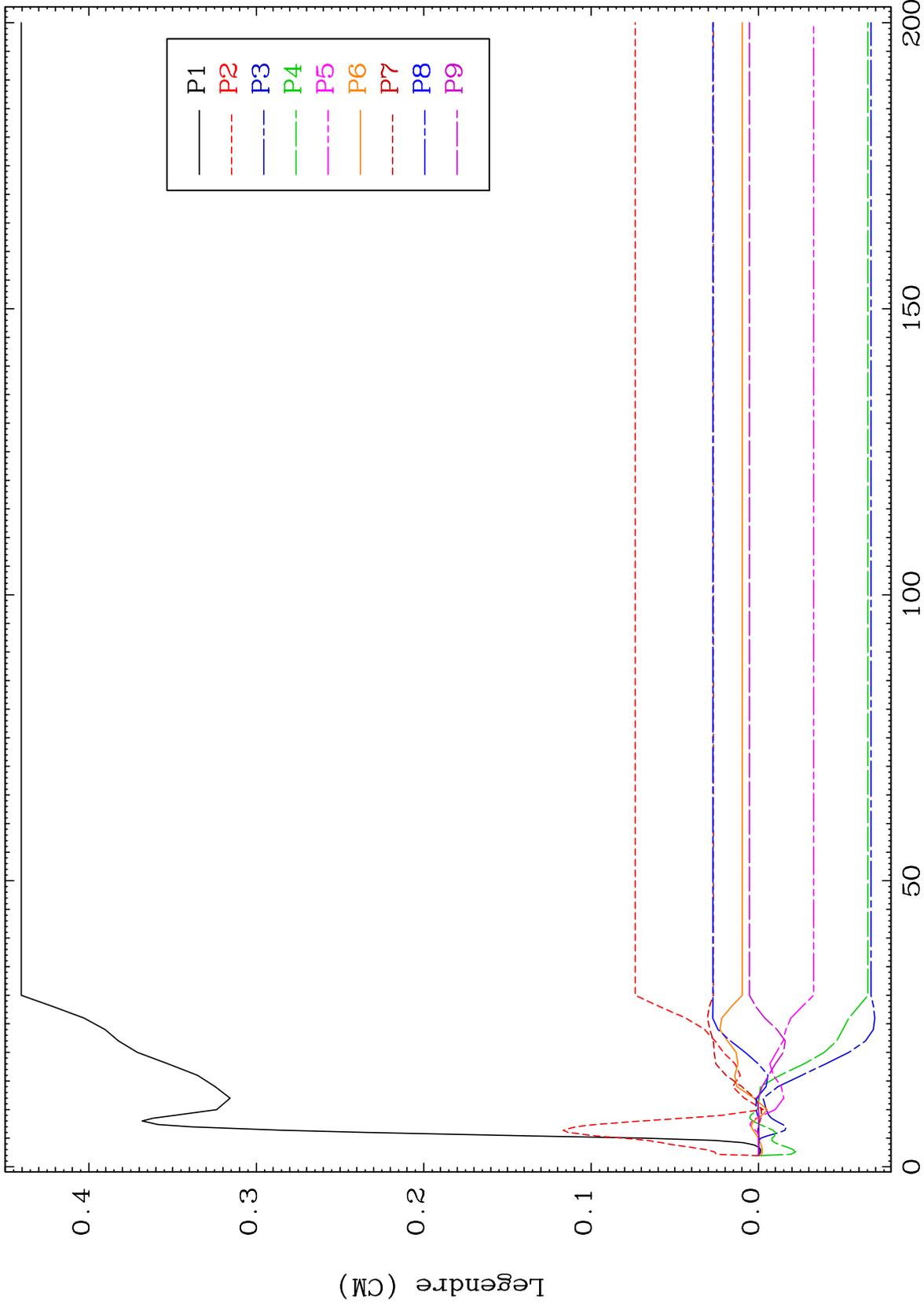
86-Rn-206



MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

86-Rn-206



39

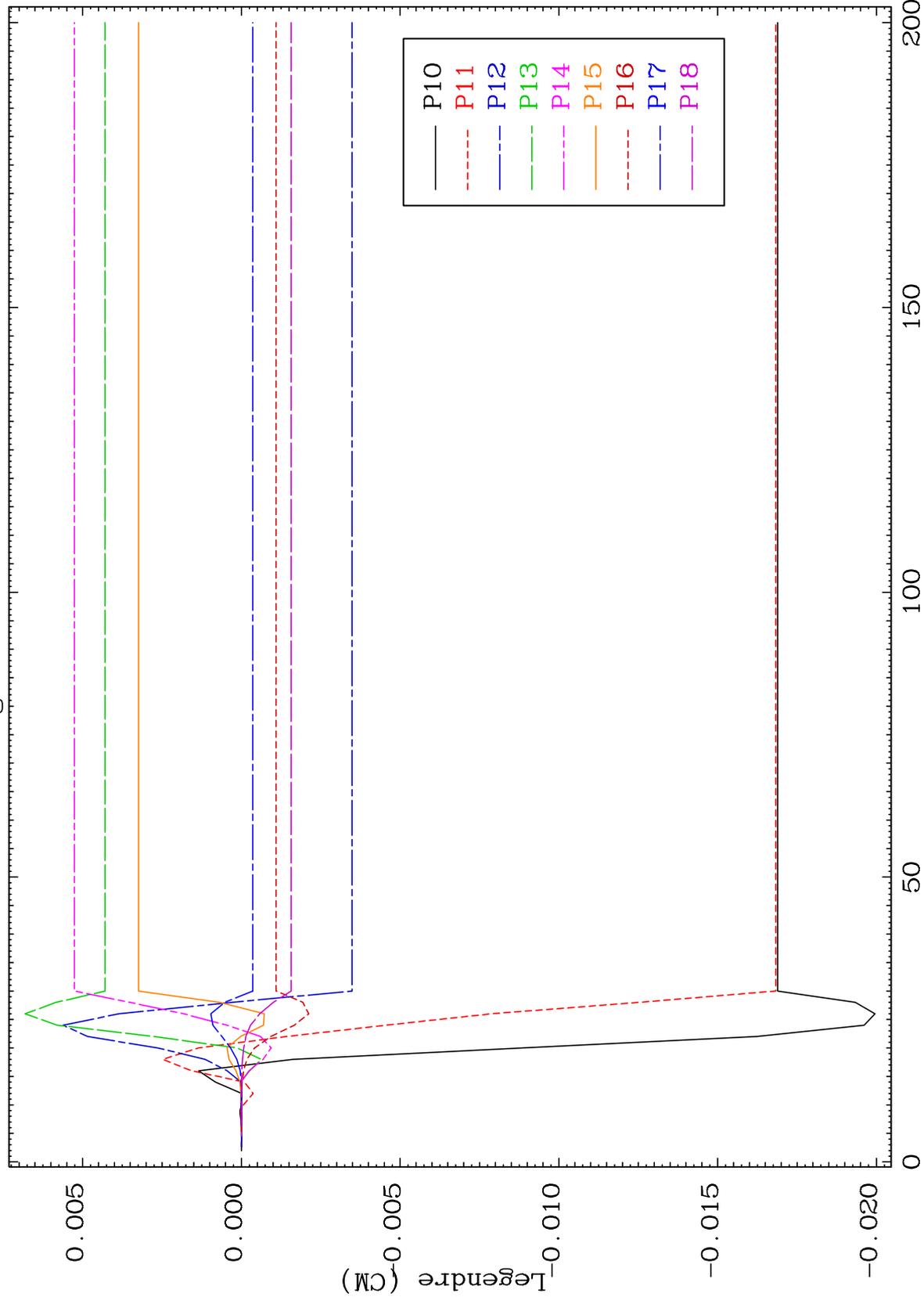
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

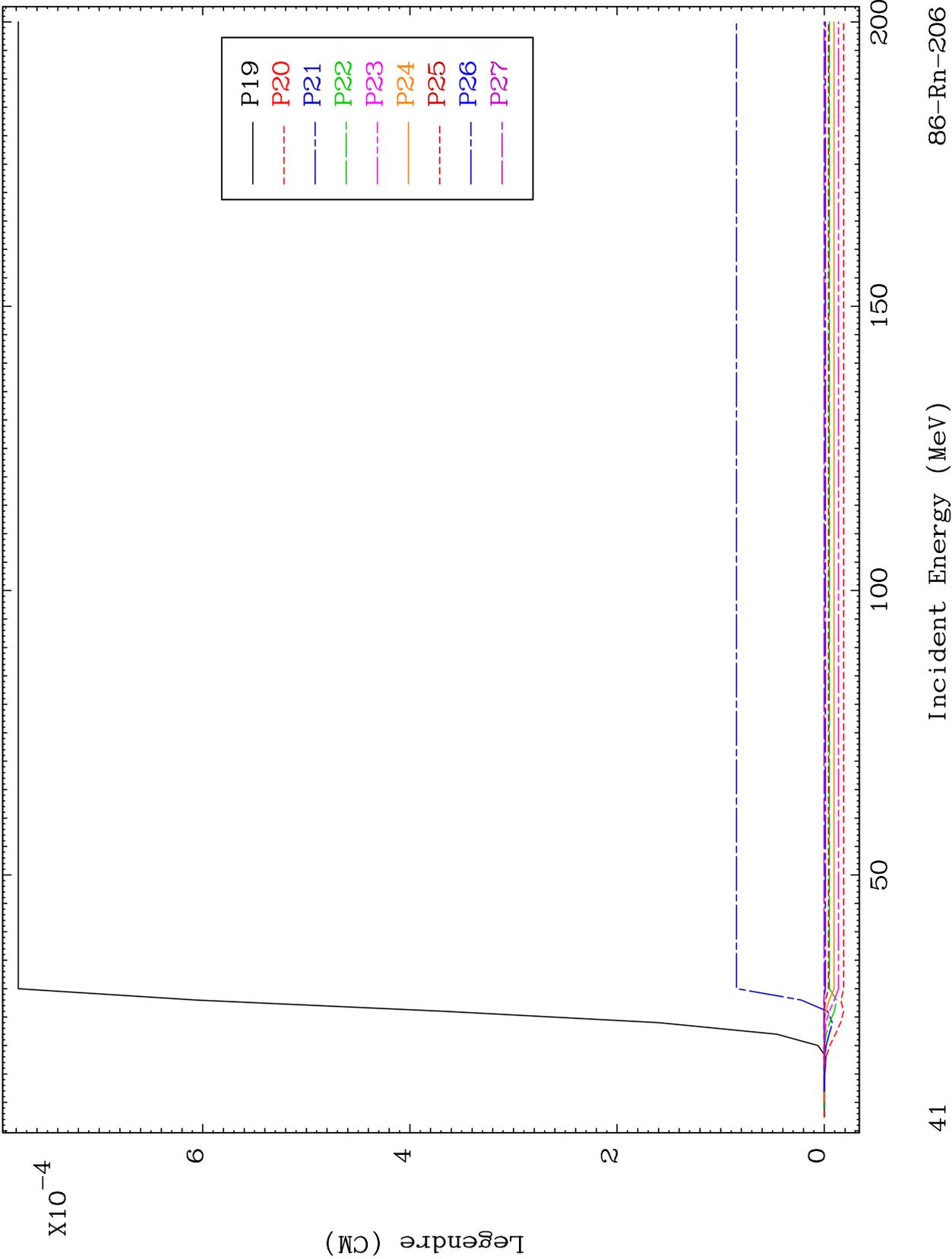
Incident Energy (MeV)

40

MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

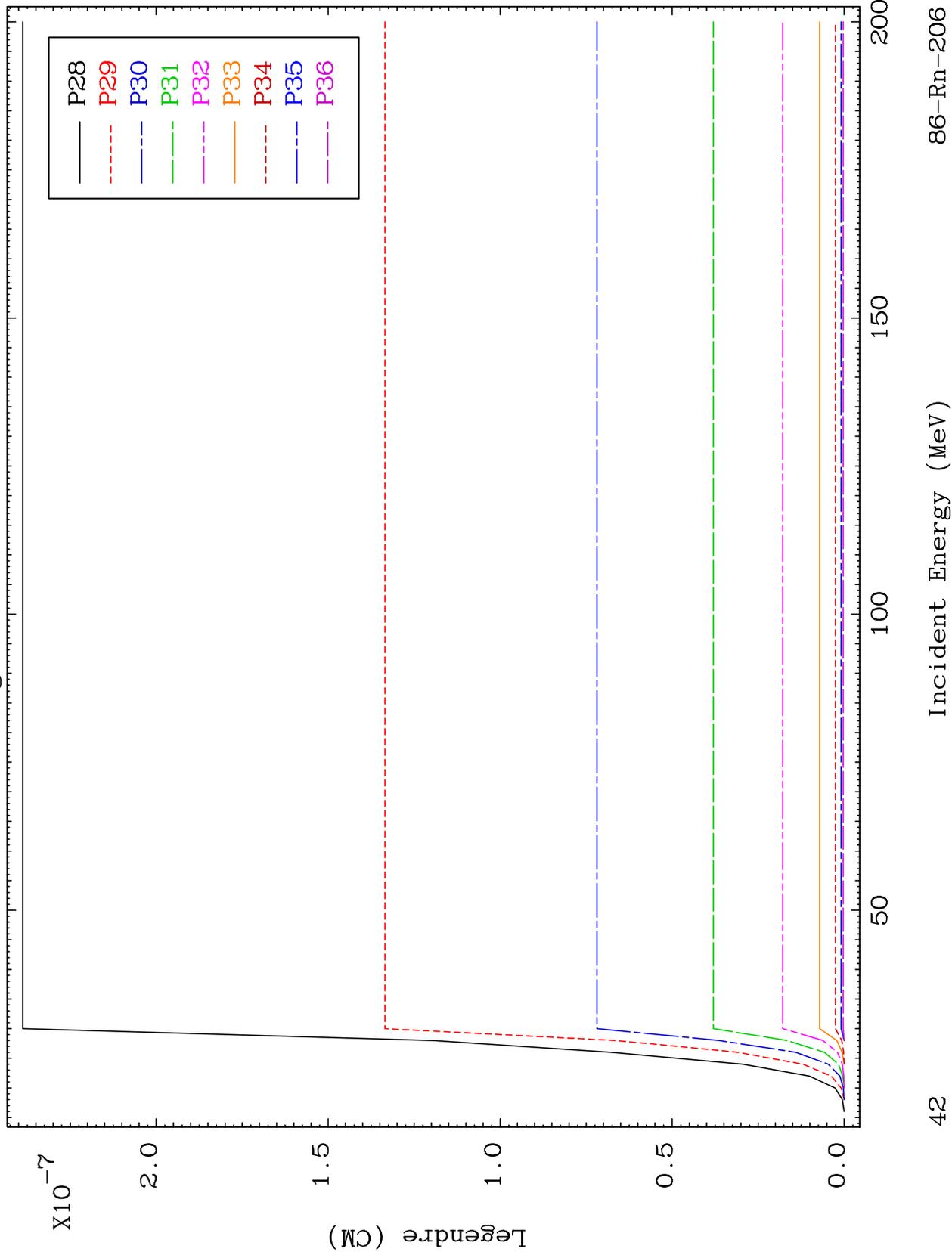
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MAT 8610

MT= 56 (n,n') Level  
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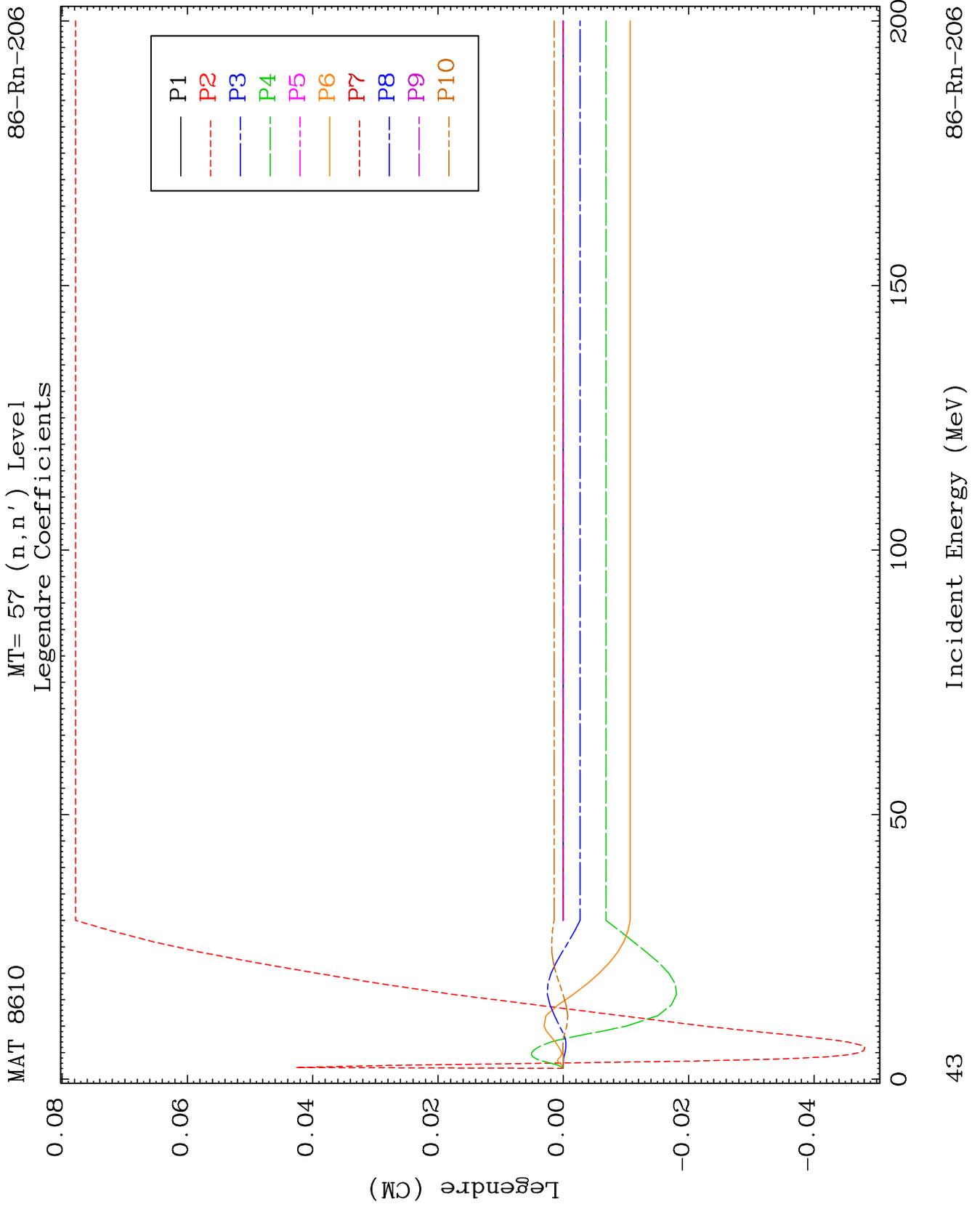
86-Rn-206



42

Incident Energy (MeV)

86-Rn-206



86-Rn-206

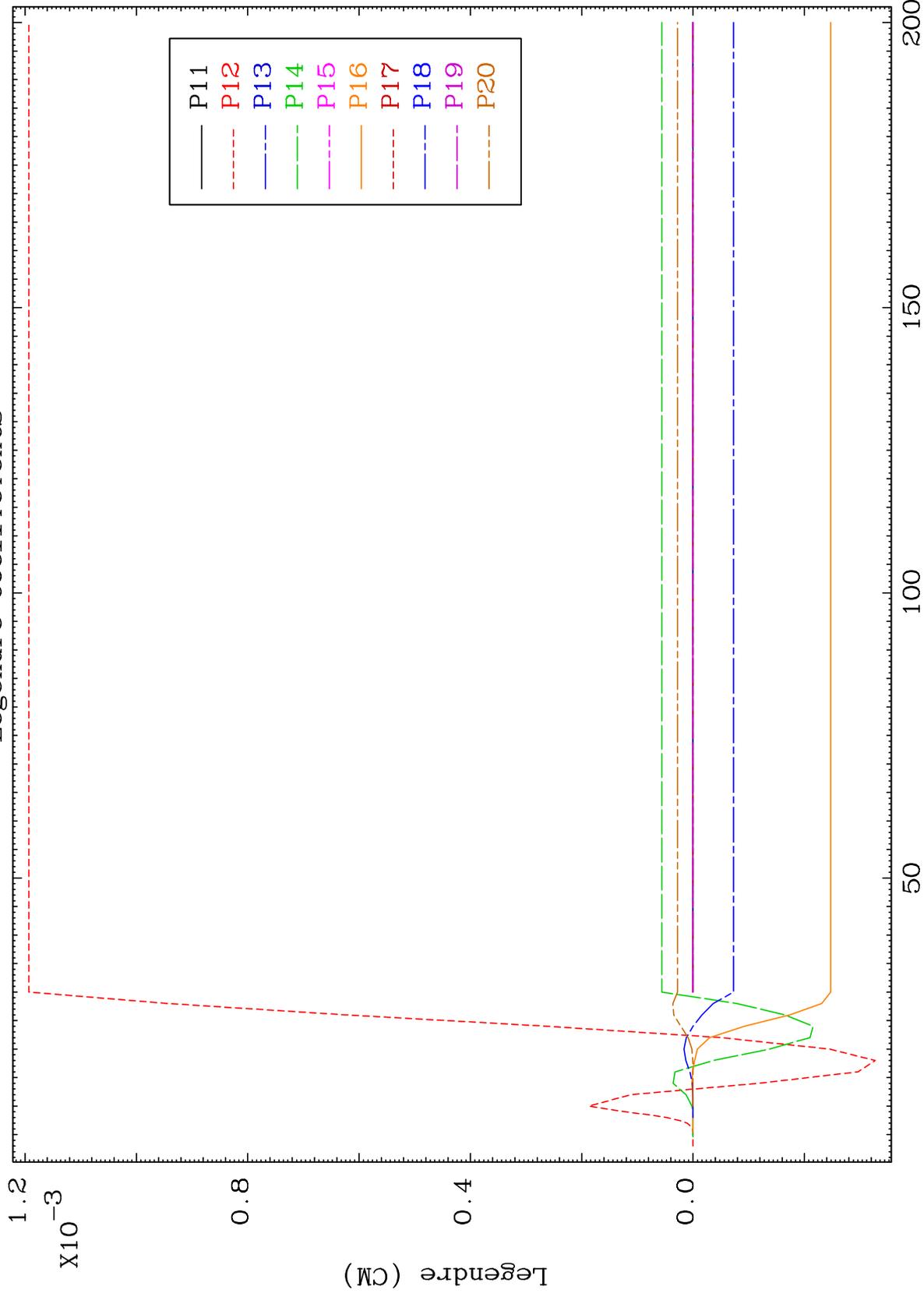
Incident Energy (MeV)

43

MAT 8610

MT= 57 (n,n') Level  
Legendre Coefficients

86-Rn-206



44

Incident Energy (MeV)

86-Rn-206

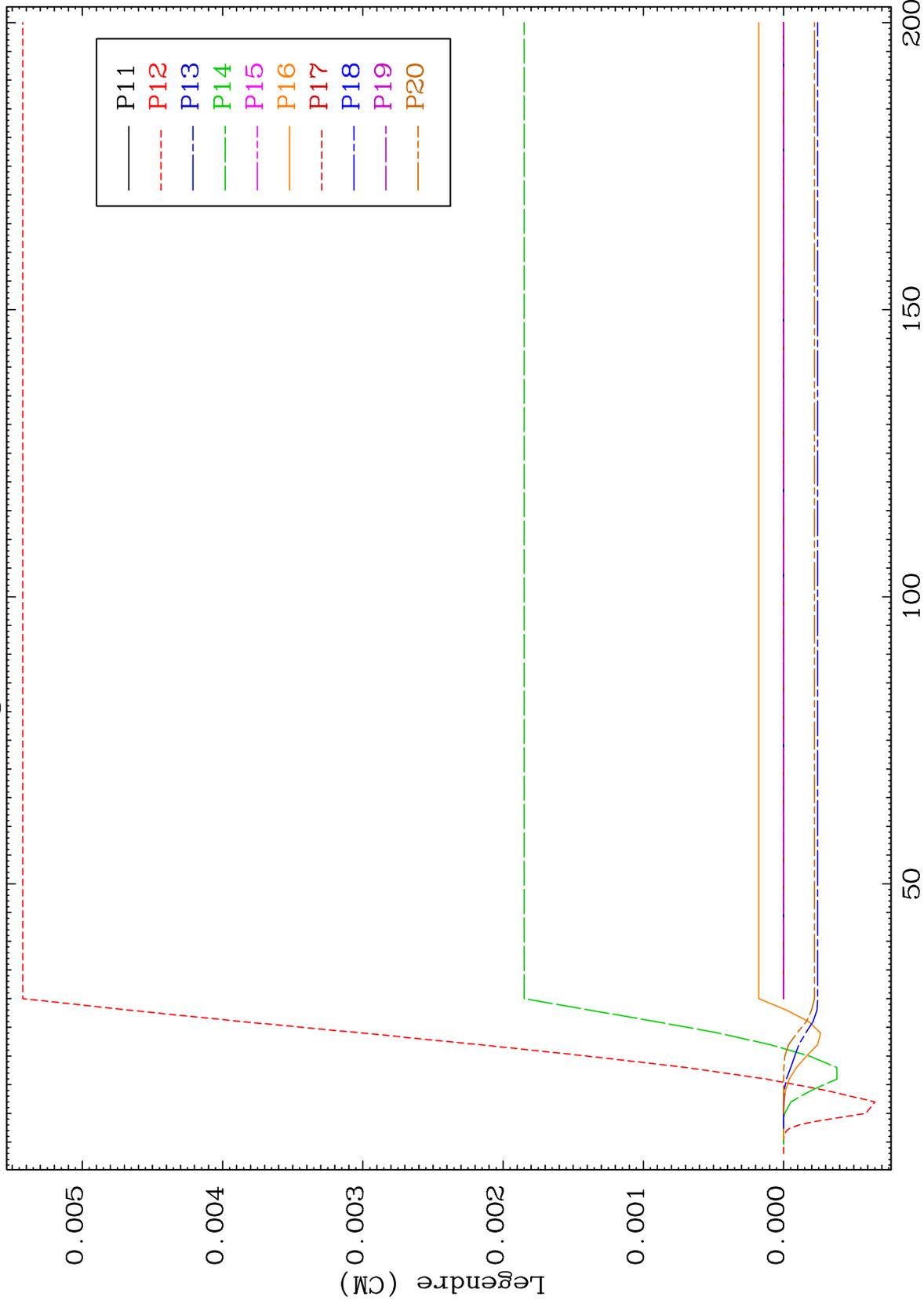




MAT 8610

MT= 58 (n,n') Level  
Legendre Coefficients

86-Rn-206



47

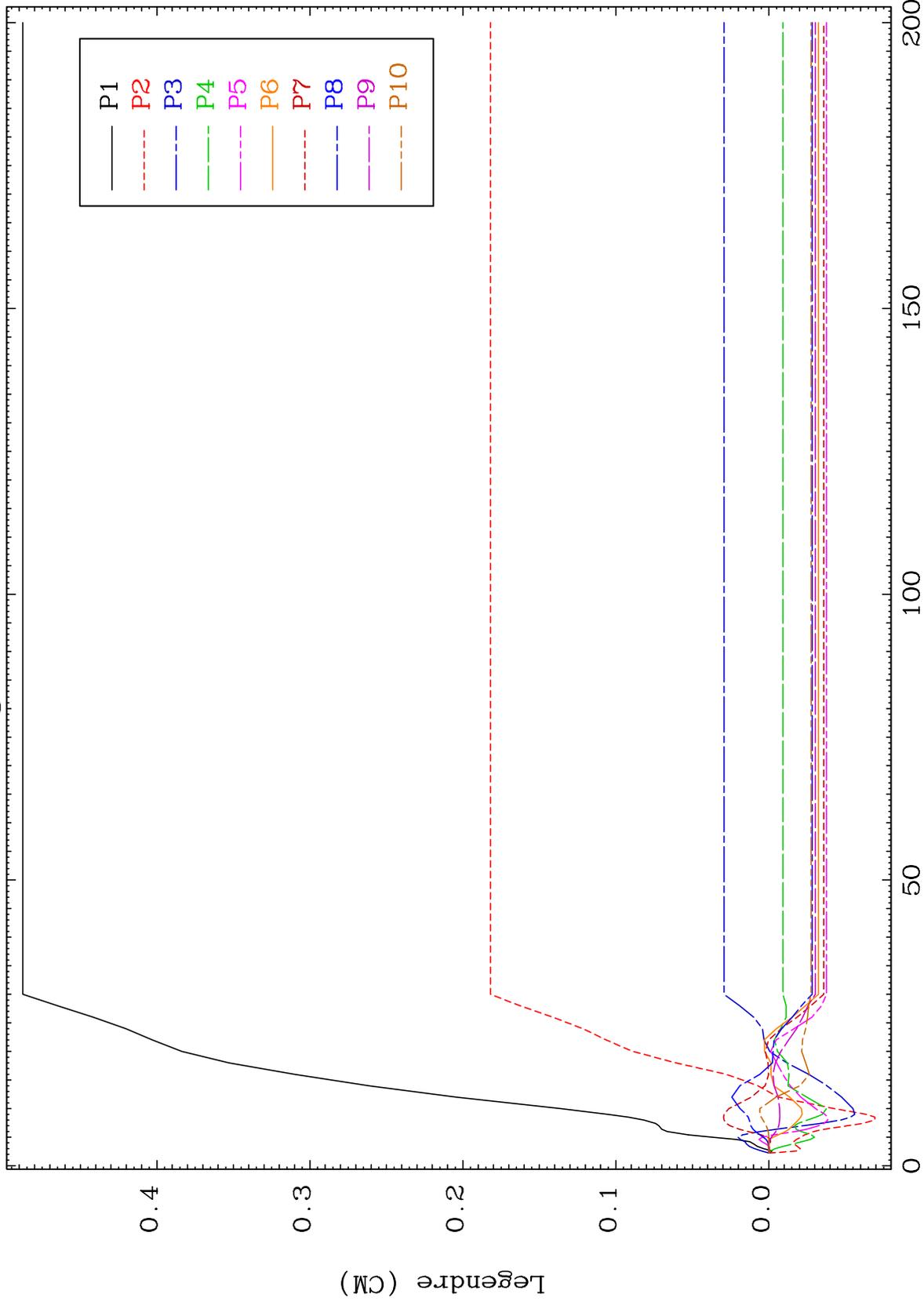
86-Rn-206



MAT 8610

MT= 59 (n,n') Level  
Legendre Coefficients

86-Rn-206



49

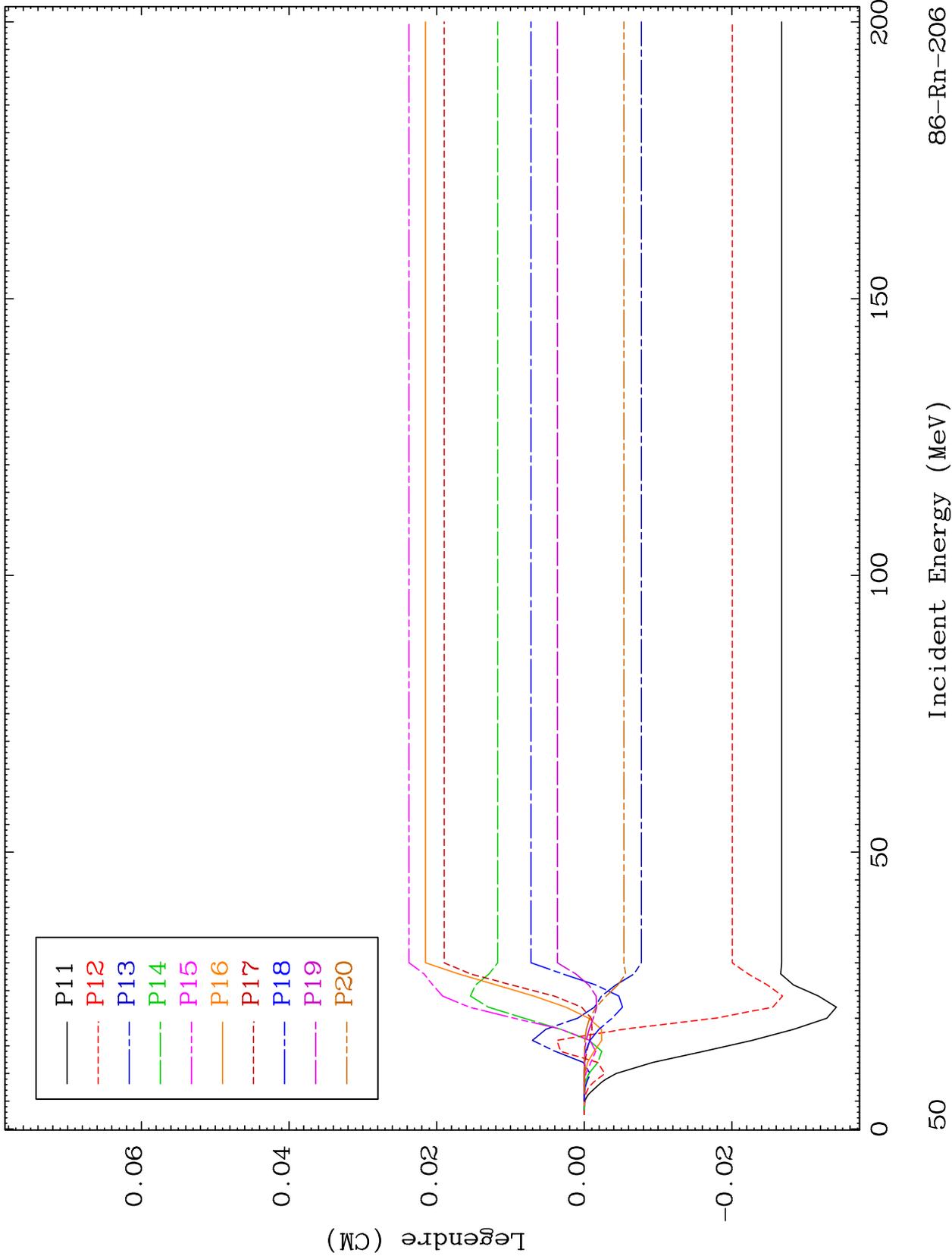
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 59 (n,n') Level Legendre Coefficients

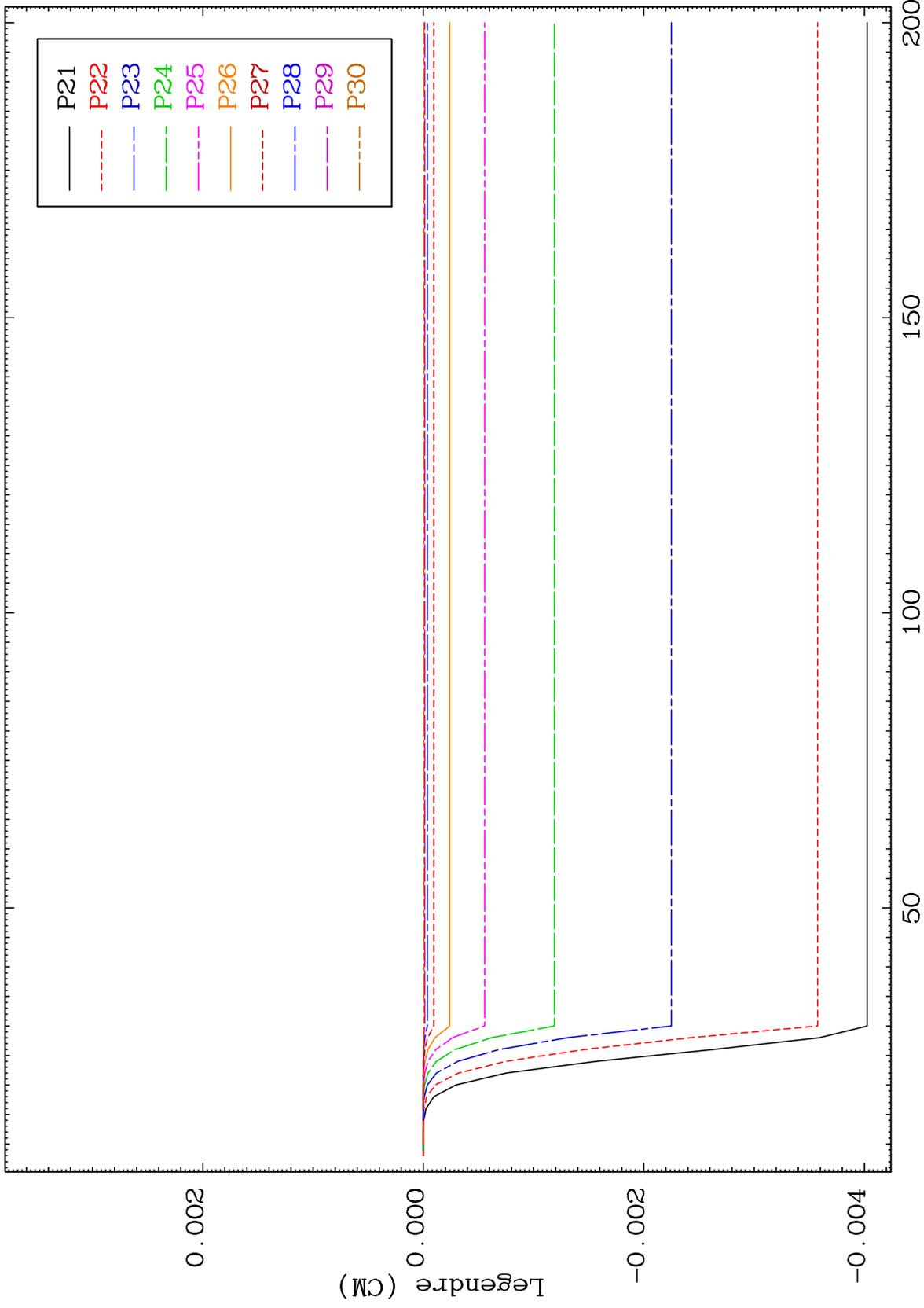
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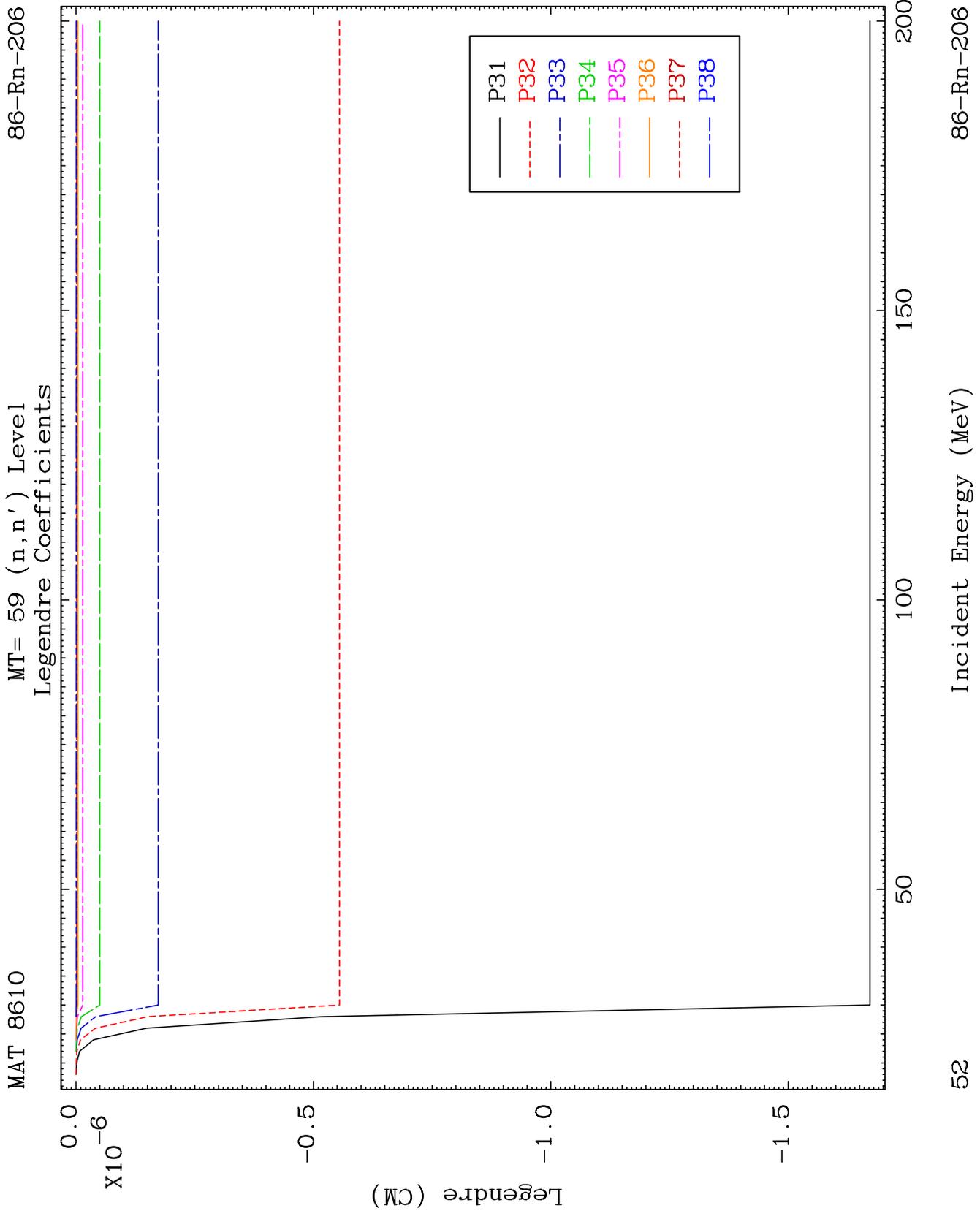


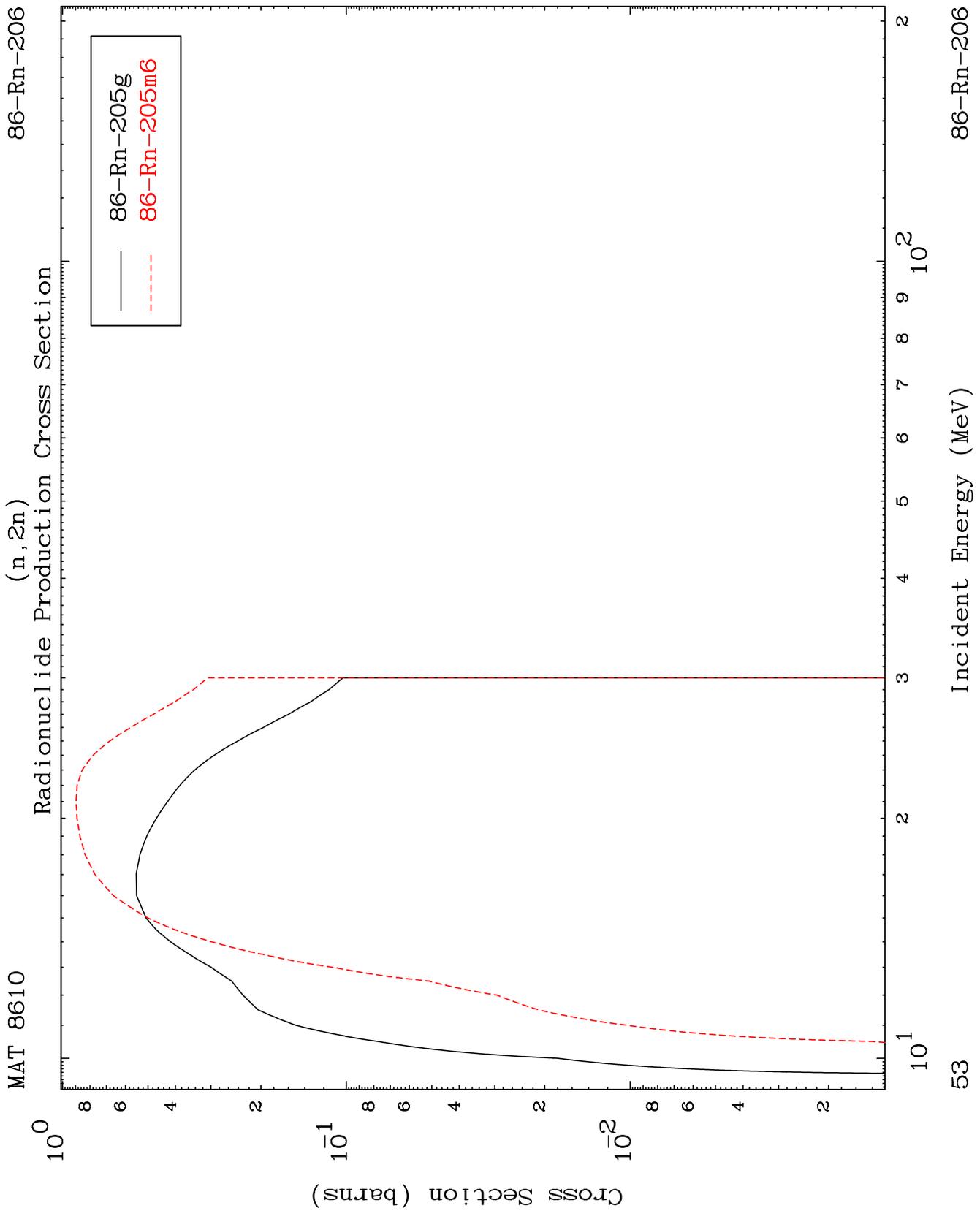
86-Rn-206

Incident Energy (MeV)

50





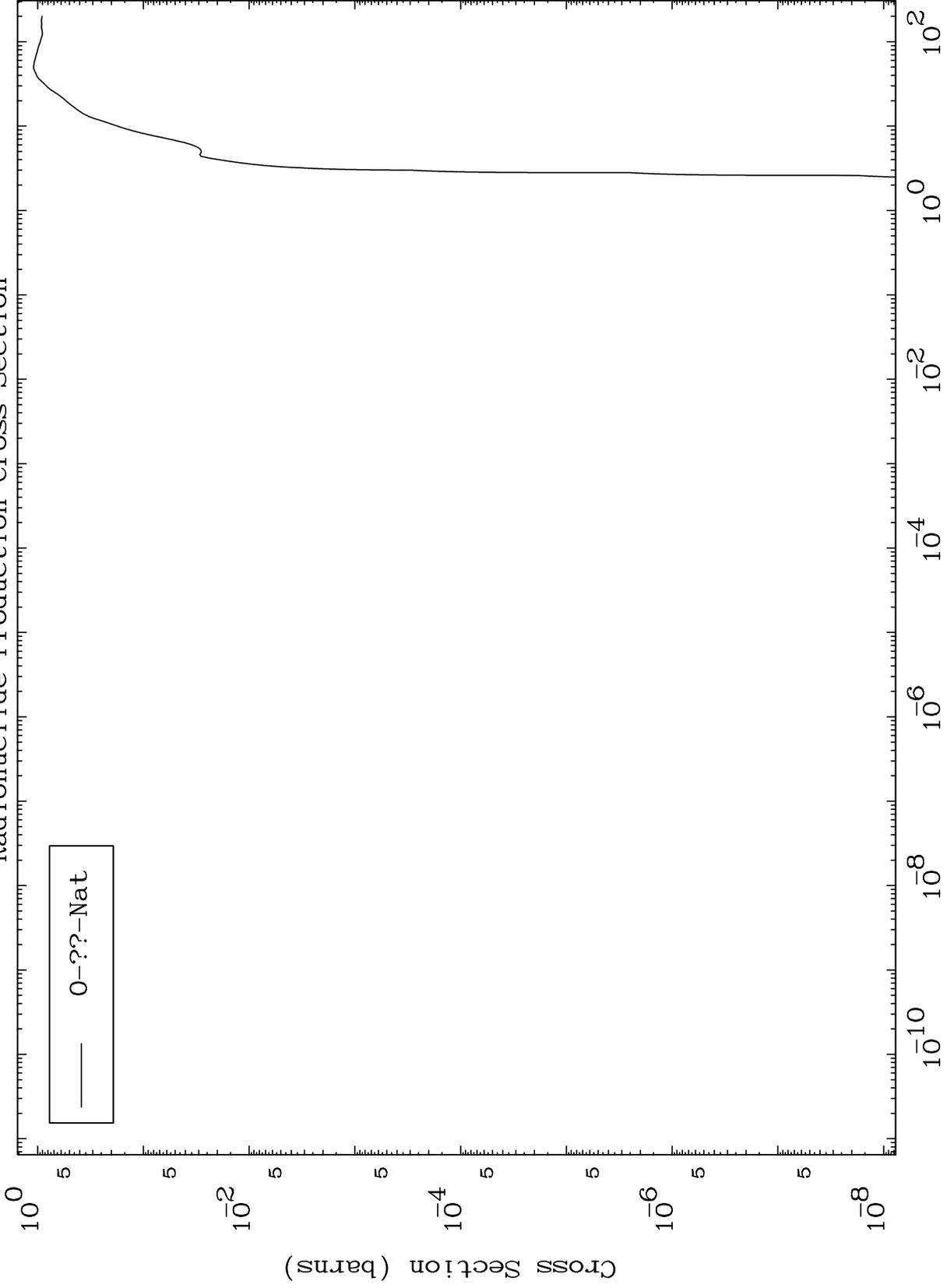


MAT 8610

Fission

86-Rn-206

Radionuclide Production Cross Section

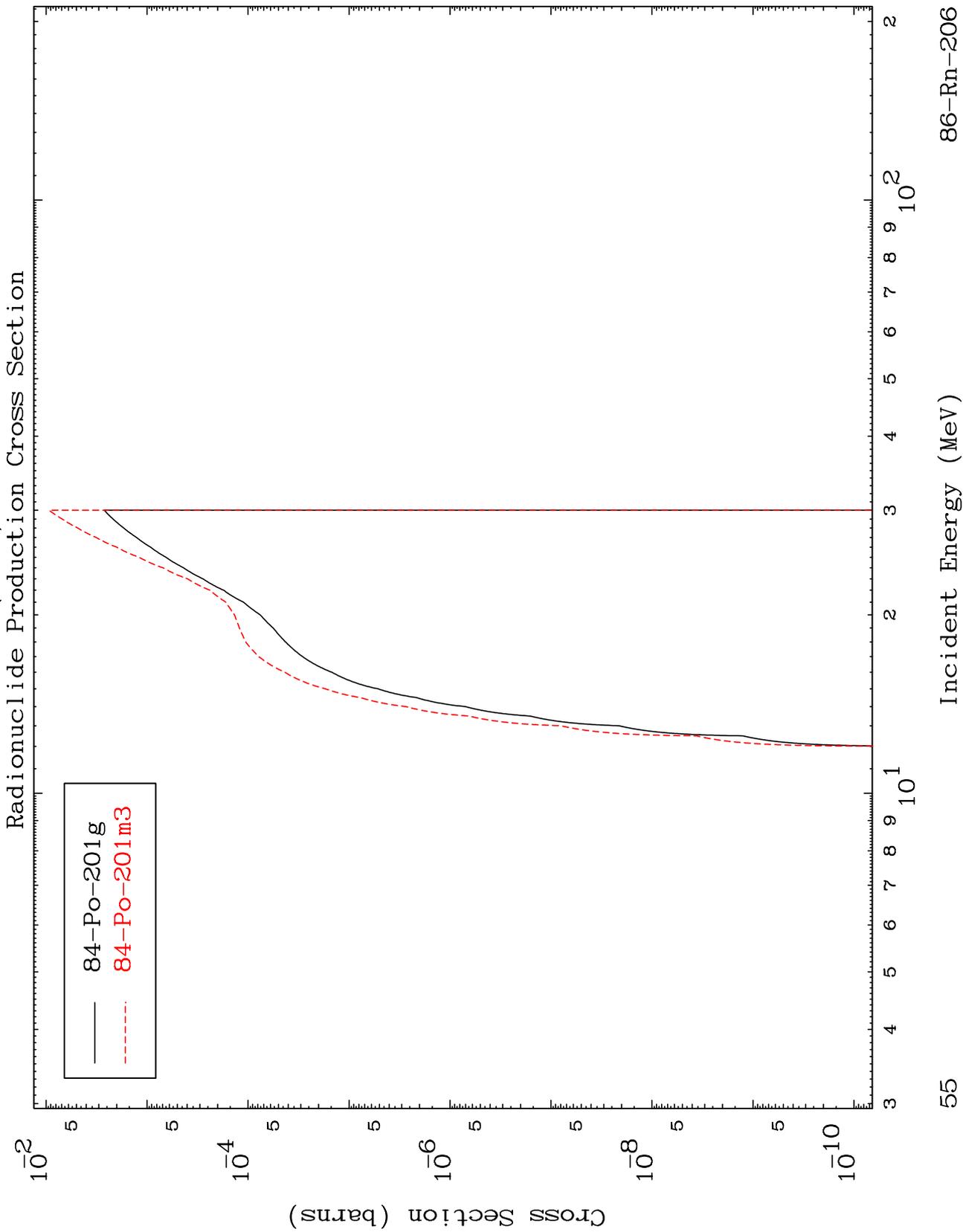


0-??-Nat

MAT 8610

(n,2n)  $\alpha$

86-Rn-206

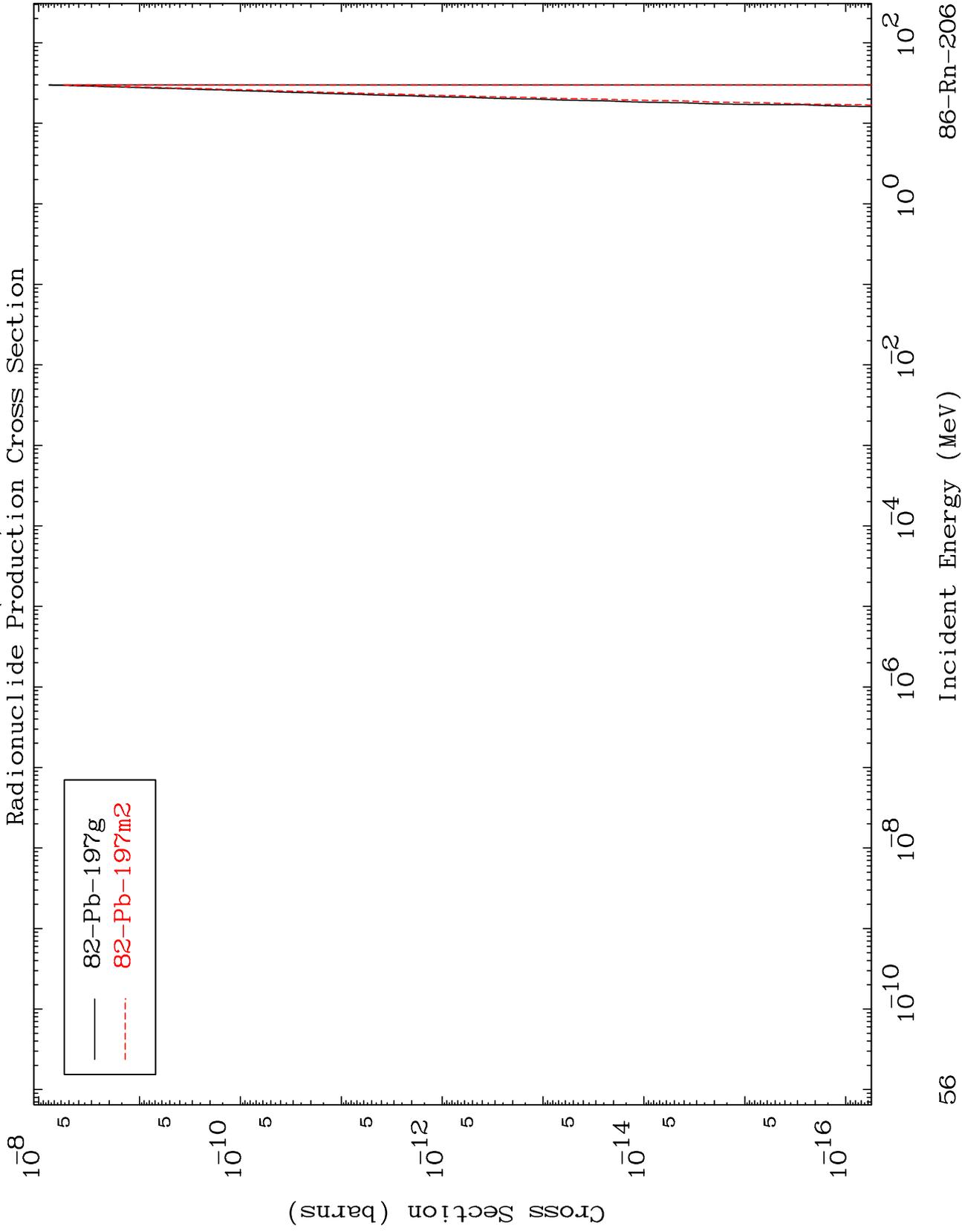


MAT 8610

(n,2n) 2 $\alpha$

86-Rn-206

Radionuclide Production Cross Section



82-Pb-197g  
82-Pb-197m2

56

Incident Energy (MeV)

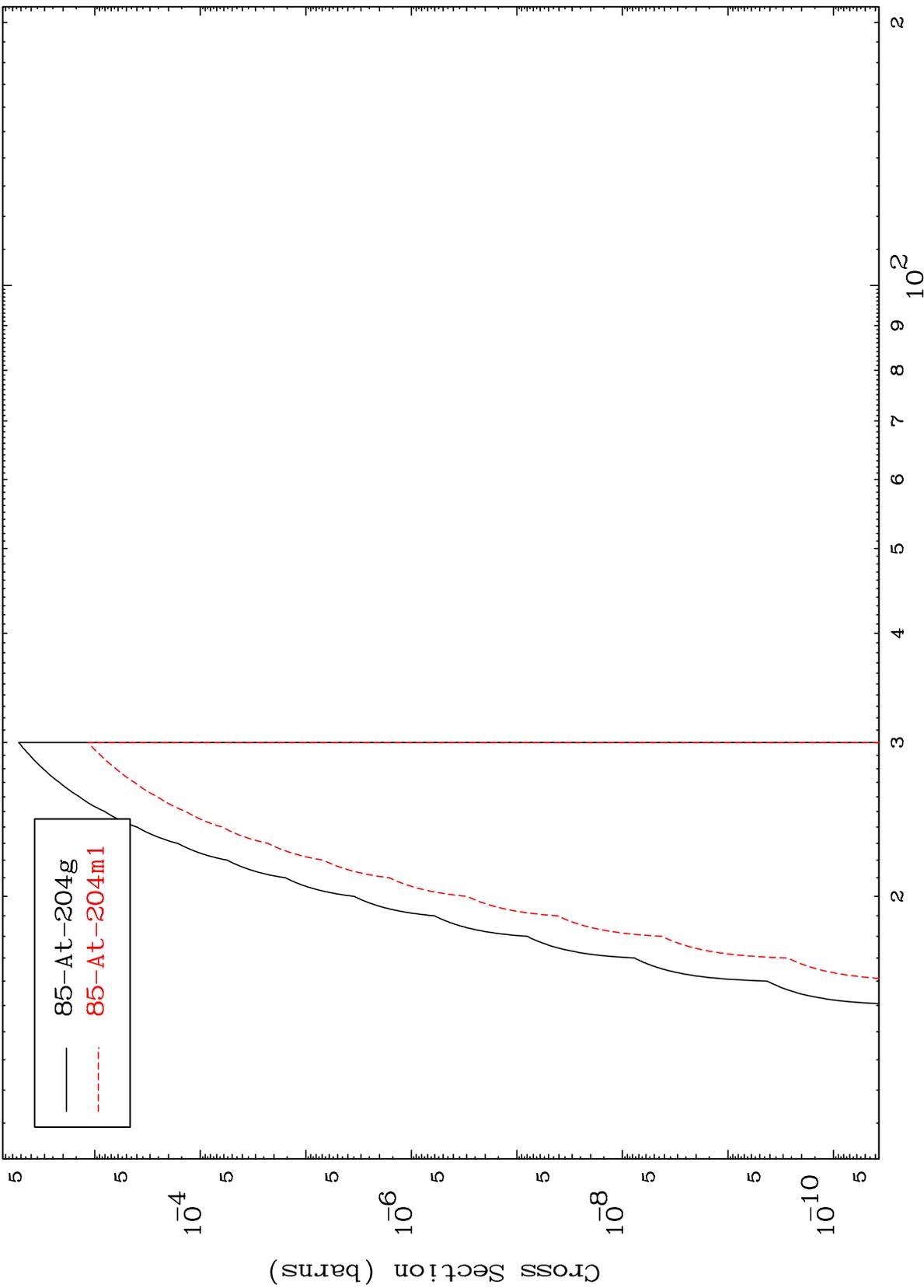
86-Rn-206

MAT 8610

(n,n') d

86-Rn-206

Radionuclide Production Cross Section



57

Incident Energy (MeV)

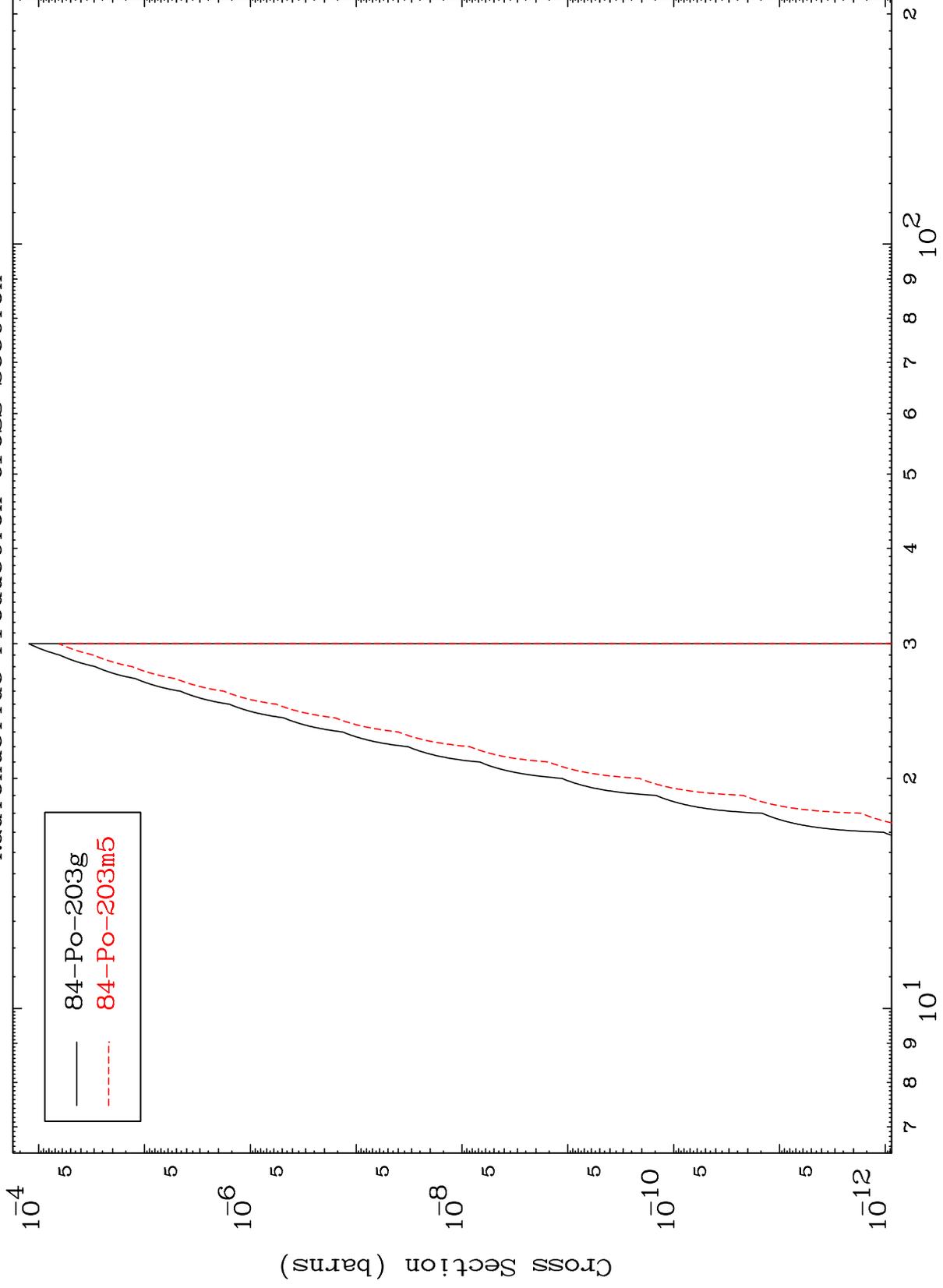
86-Rn-206

MAT 8610

(n,n') He-3

86-Rn-206

Radionuclide Production Cross Section



58

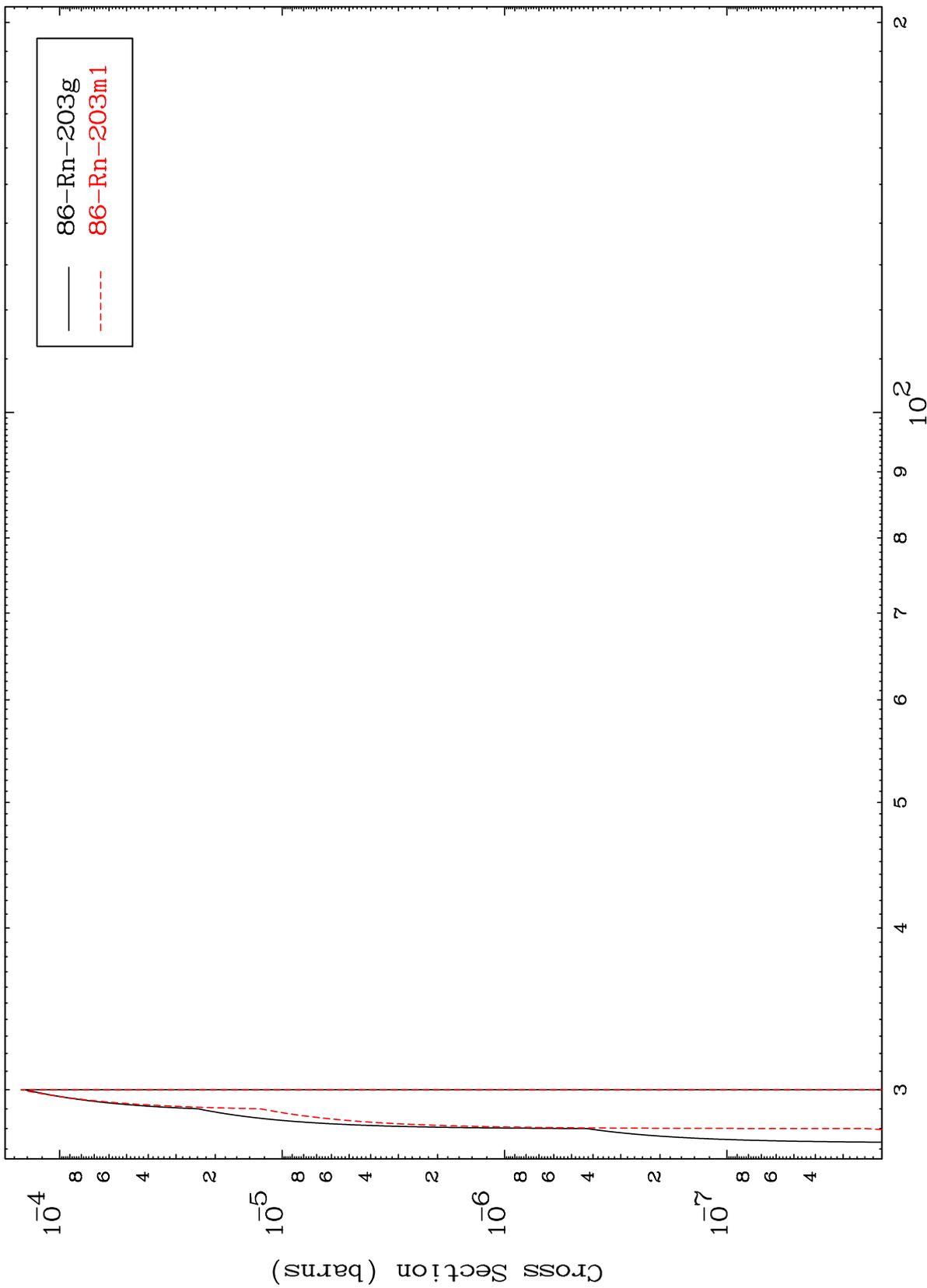
Incident Energy (MeV)

86-Rn-206

MAT 8610

86-Rn-206

(n,4n)  
Radionuclide Production Cross Section



59

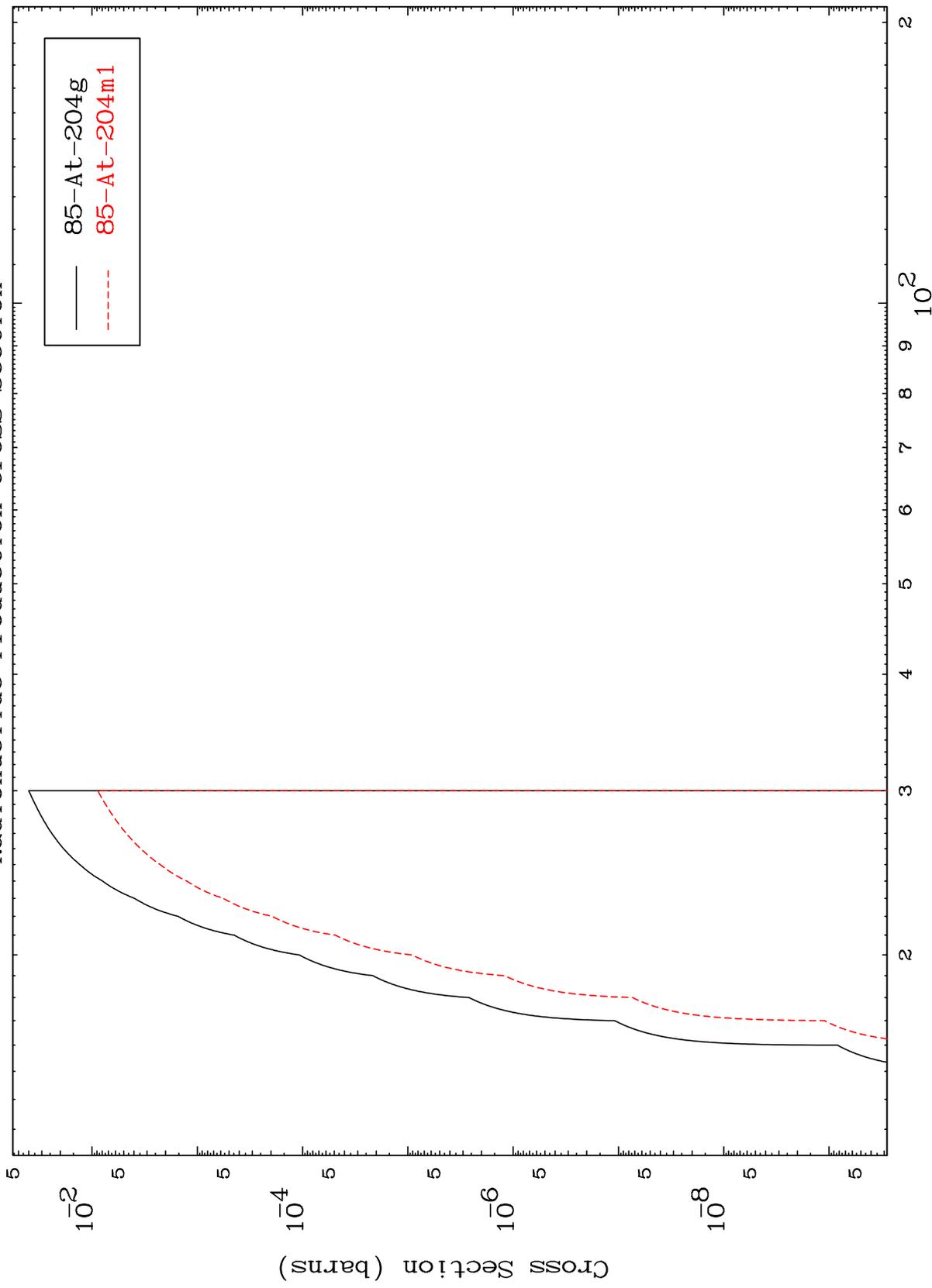
86-Rn-206

Incident Energy (MeV)

MAT 8610

86-Rn-206

(n,2n) p  
Radionuclide Production Cross Section



60

Incident Energy (MeV)

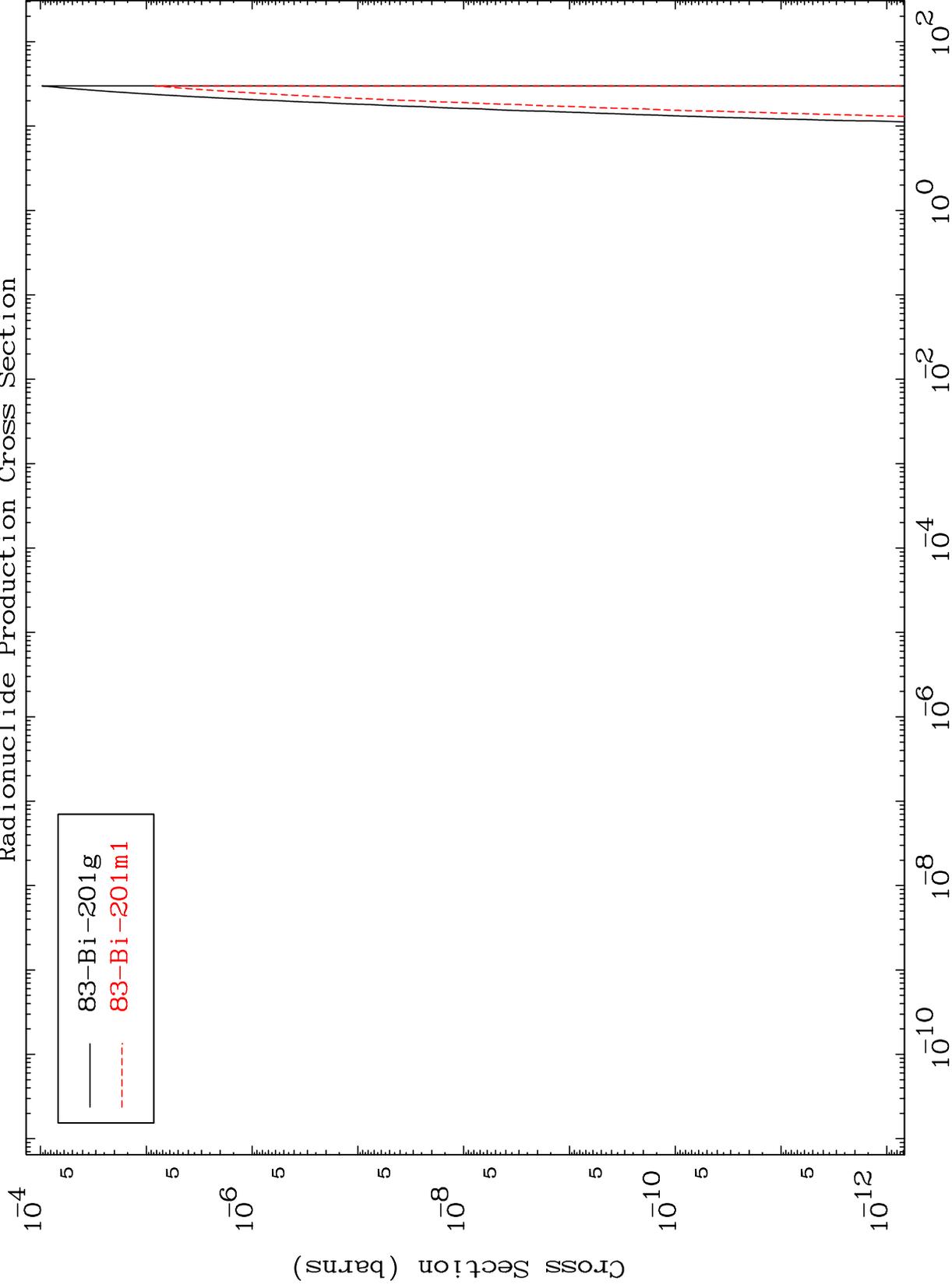
86-Rn-206

MAT 8610

(n,n') p  $\alpha$

86-Rn-206

Radionuclide Production Cross Section



83-Bi-201g  
83-Bi-201m1

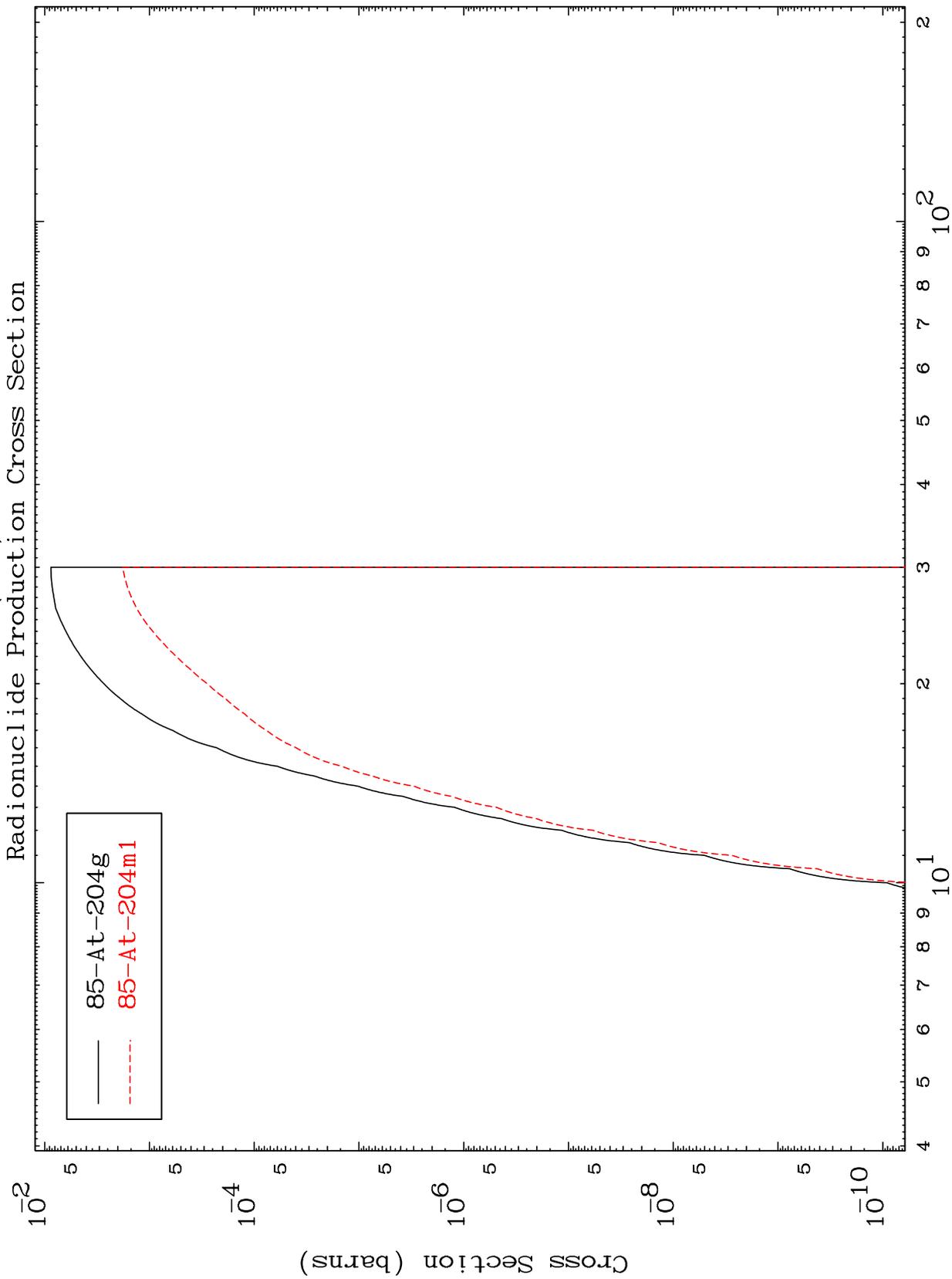
Incident Energy (MeV)

86-Rn-206

61

MAT 8610

86-Rn-206



86-Rn-206

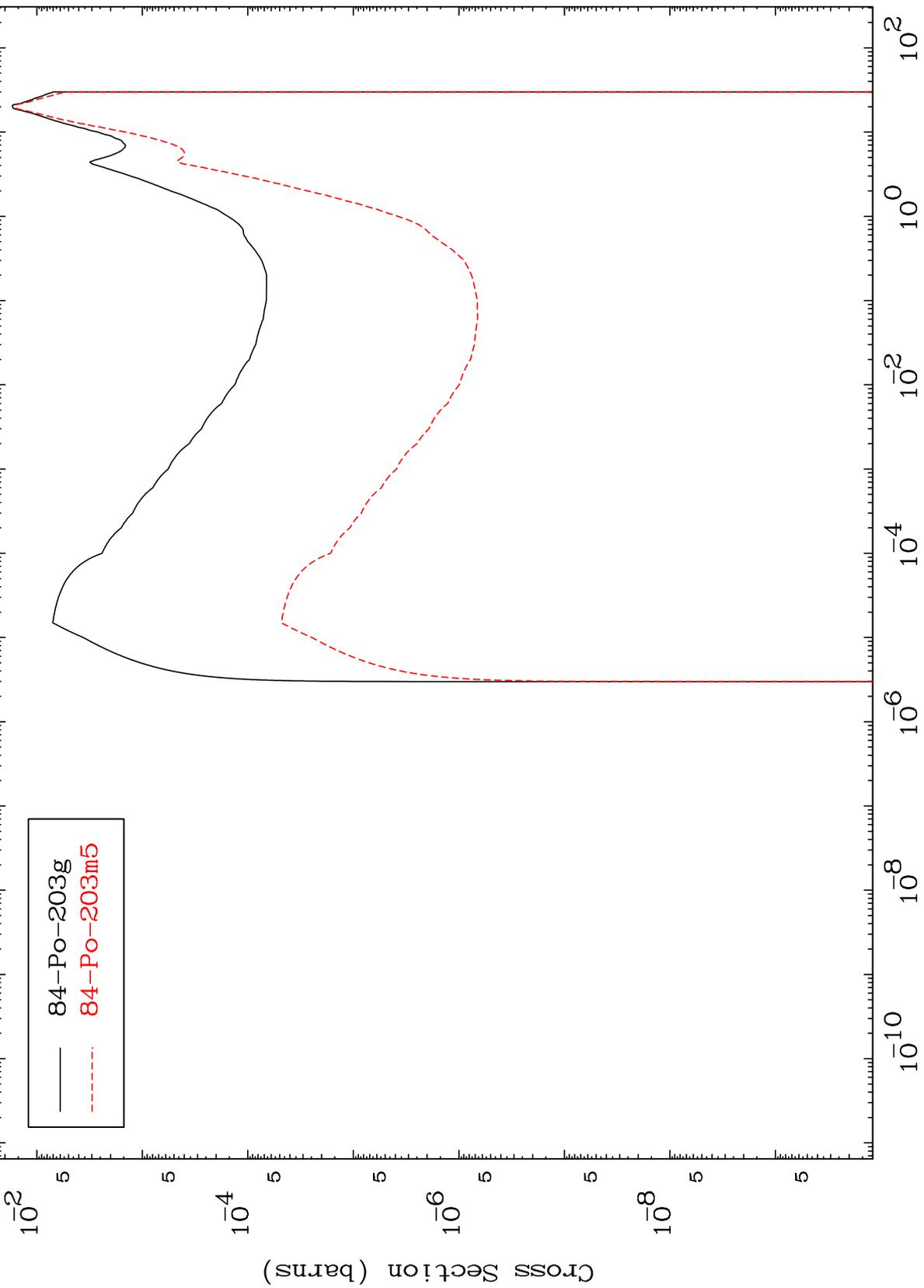
Incident Energy (MeV)

62

MAT 8610

86-Rn-206

(n,  $\alpha$ )  
Radionuclide Production Cross Section



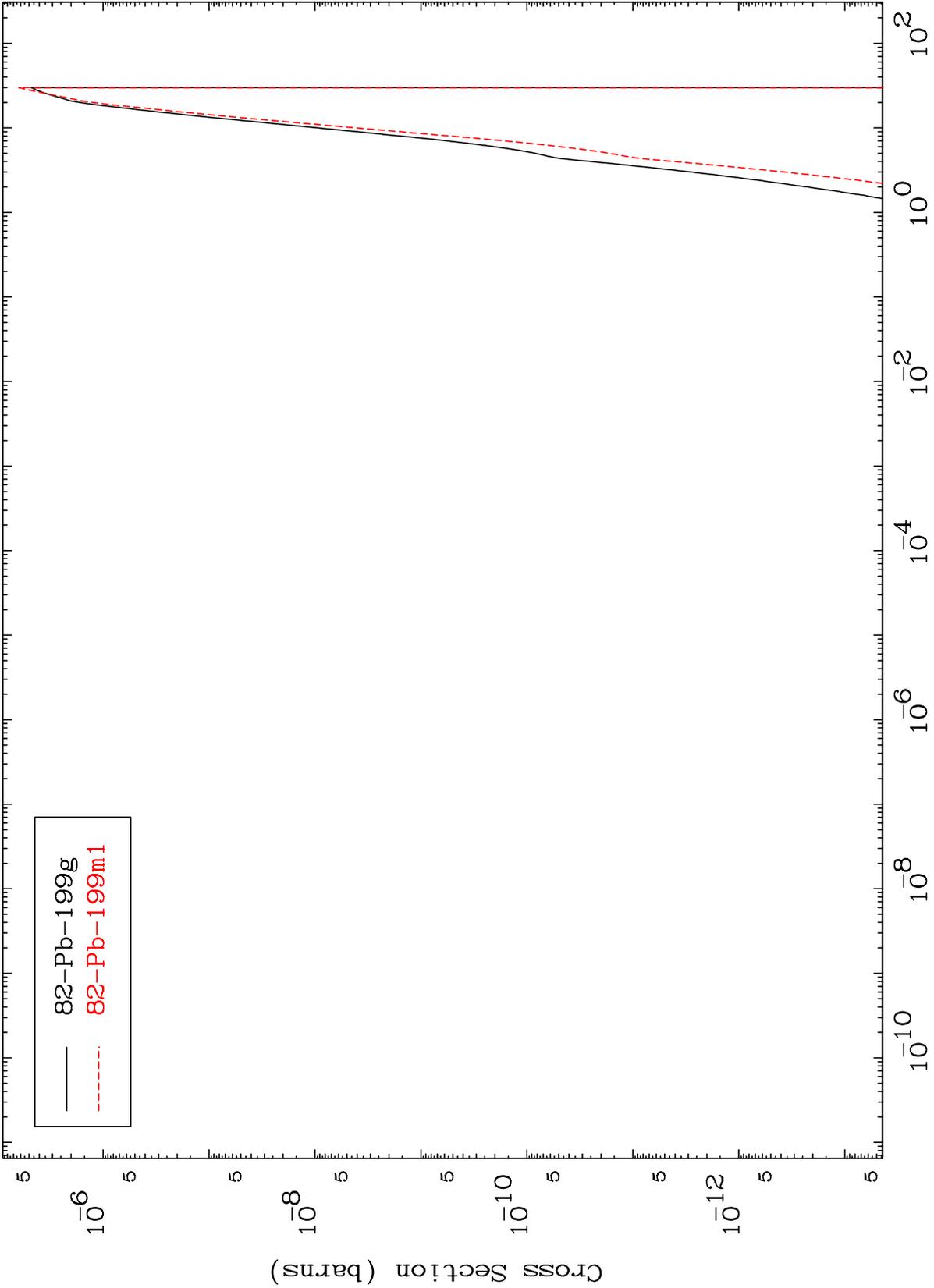
— 84-Po-203g  
- - - 84-Po-203m5

MAT 8610

(n,2α)

86-Rn-206

Radionuclide Production Cross Section



82-Pb-199g  
82-Pb-199m1

64

Incident Energy (MeV)

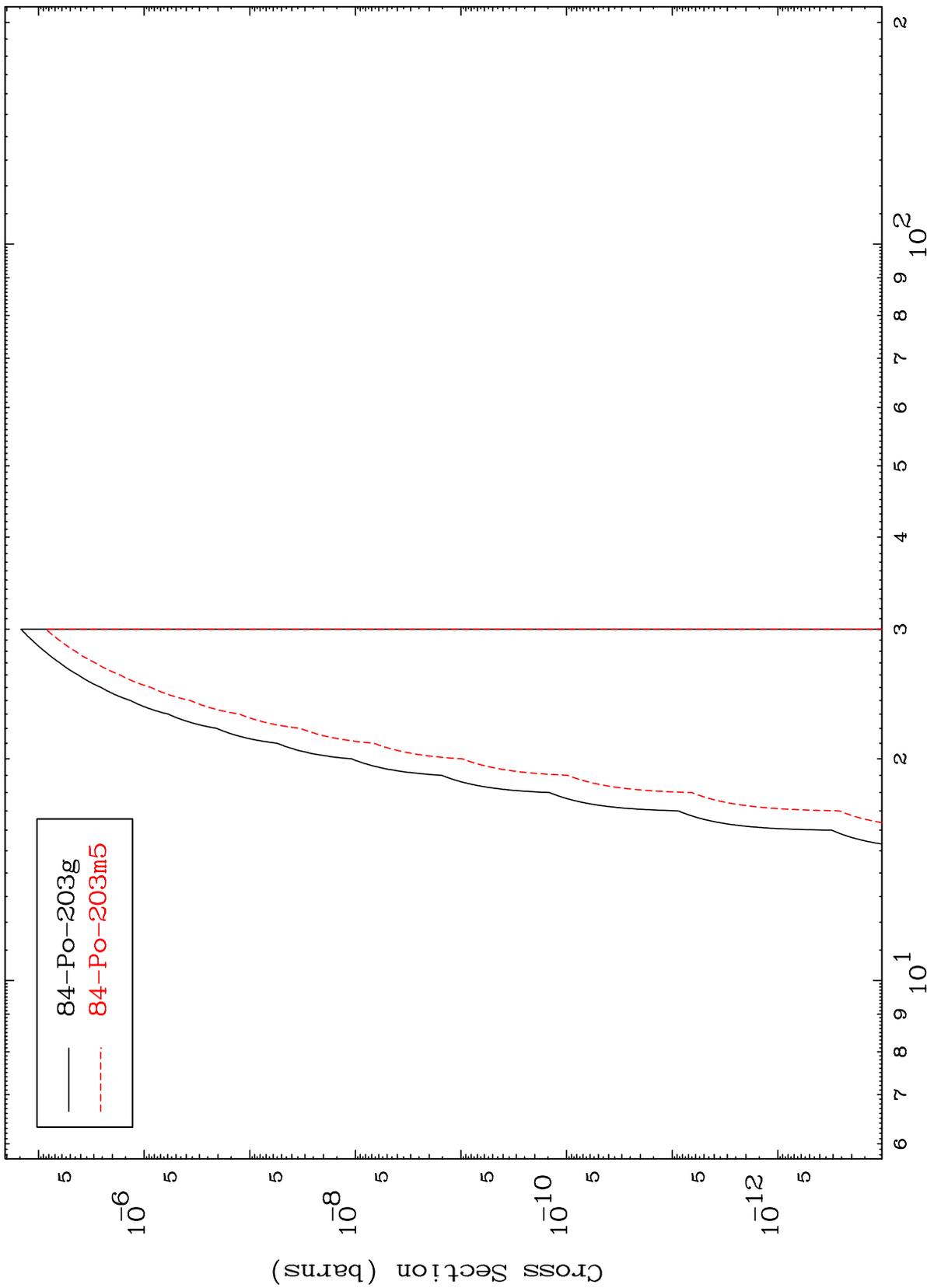
86-Rn-206

MAT 8610

(n,p) t

86-Rn-206

Radionuclide Production Cross Section



65

Incident Energy (MeV)

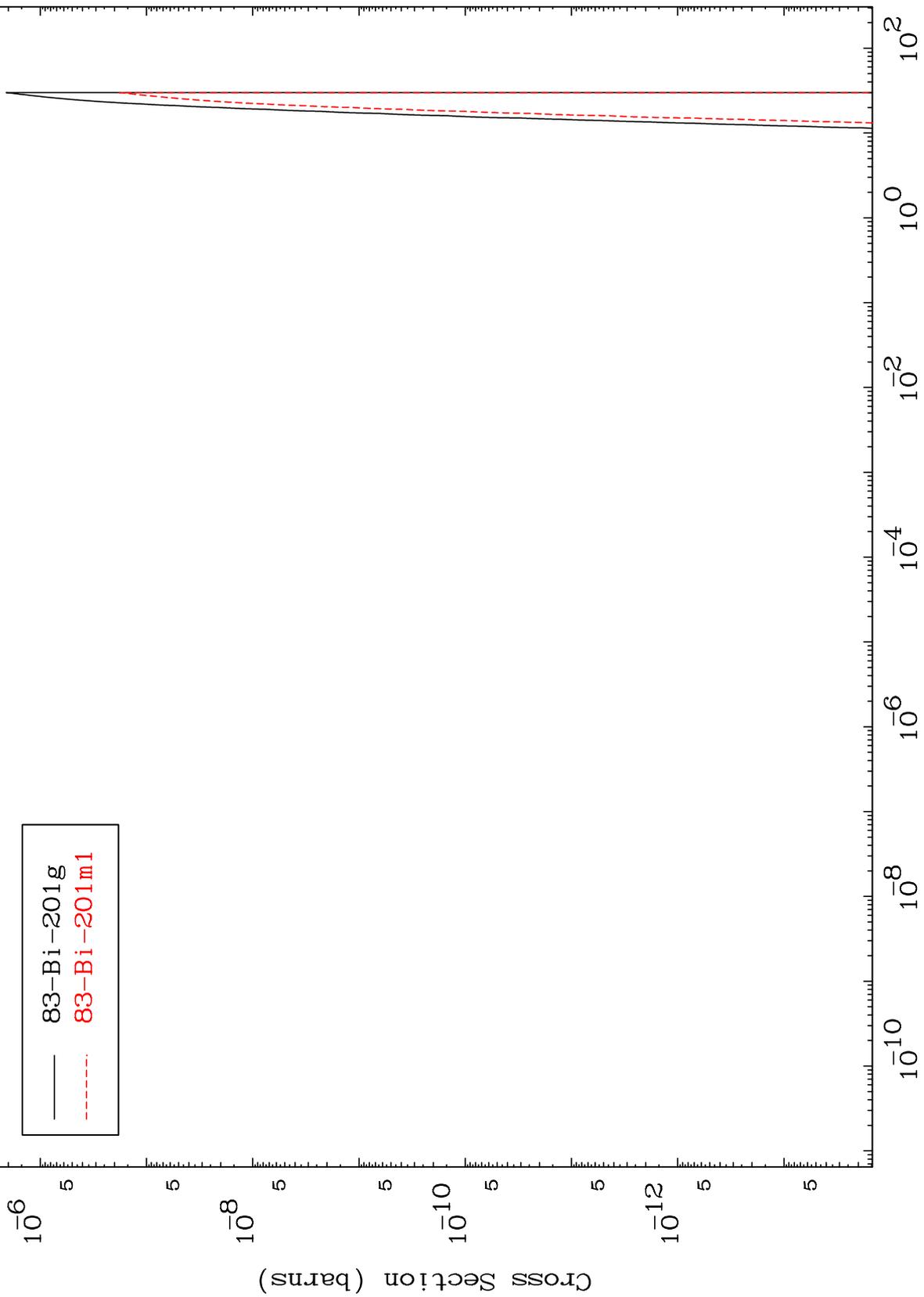
86-Rn-206

MAT 8610

(n,d)  $\alpha$

86-Rn-206

Radionuclide Production Cross Section



83-Bi-201g  
83-Bi-201m1