

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

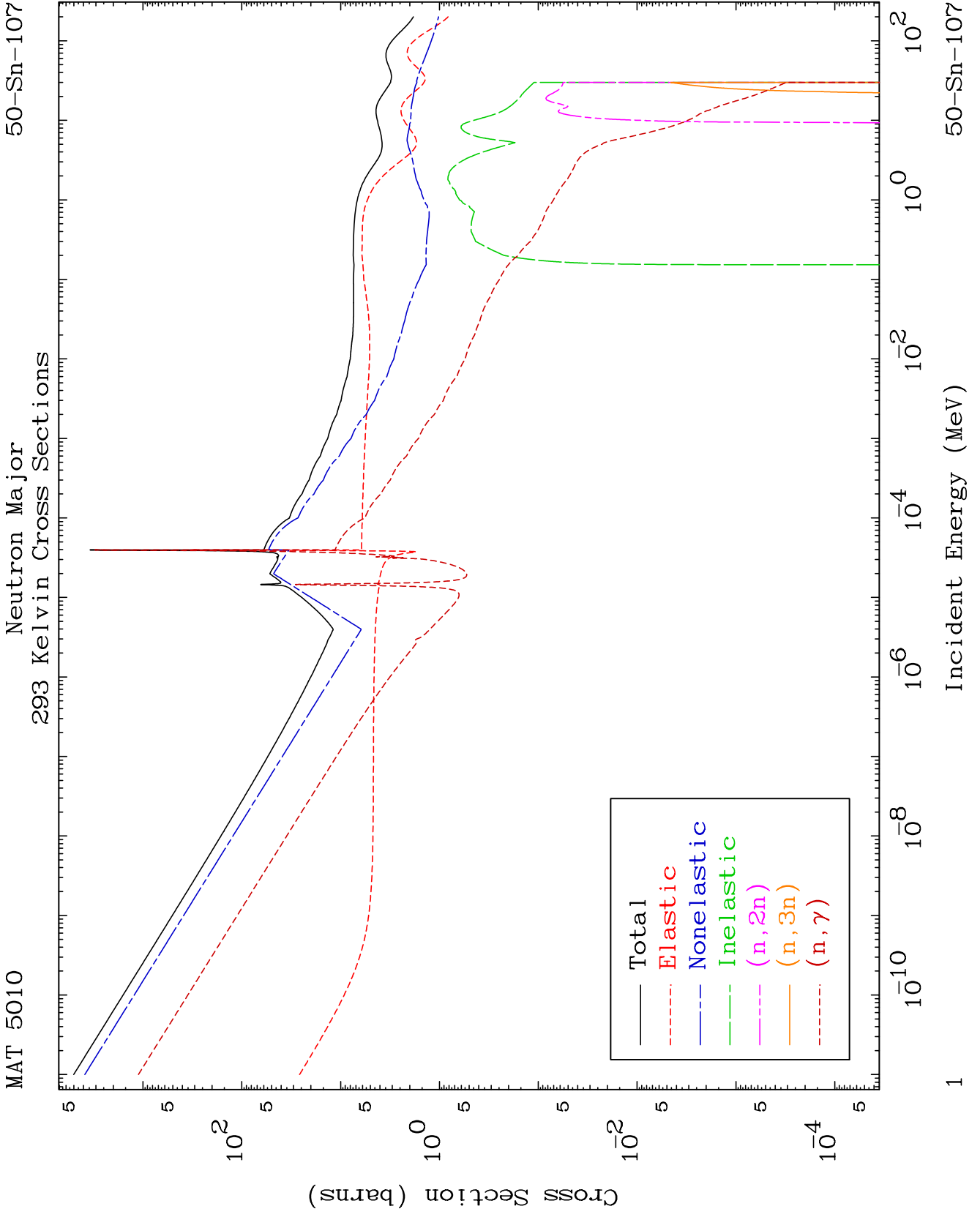
Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

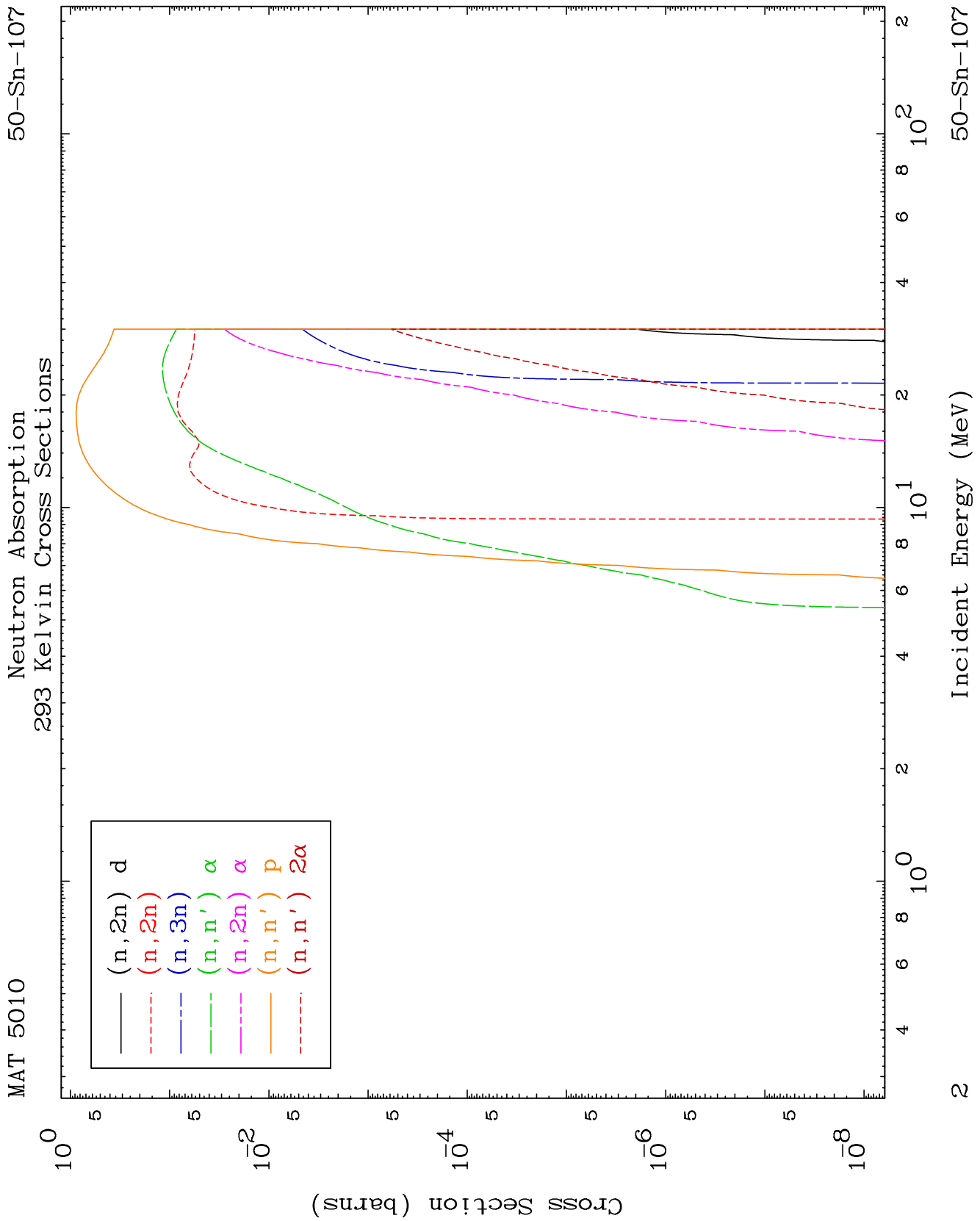
Tele: 925-443-1911

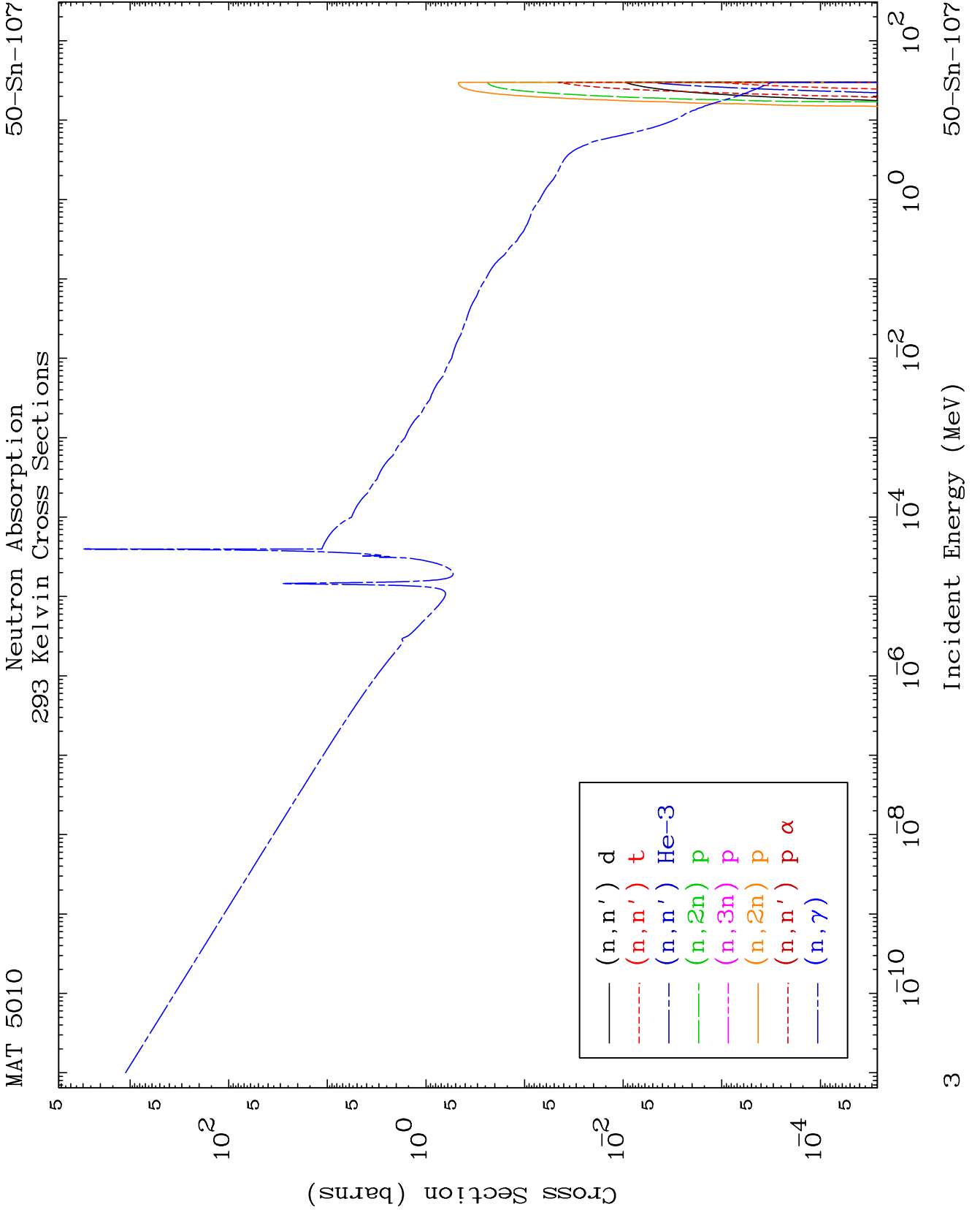
E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start



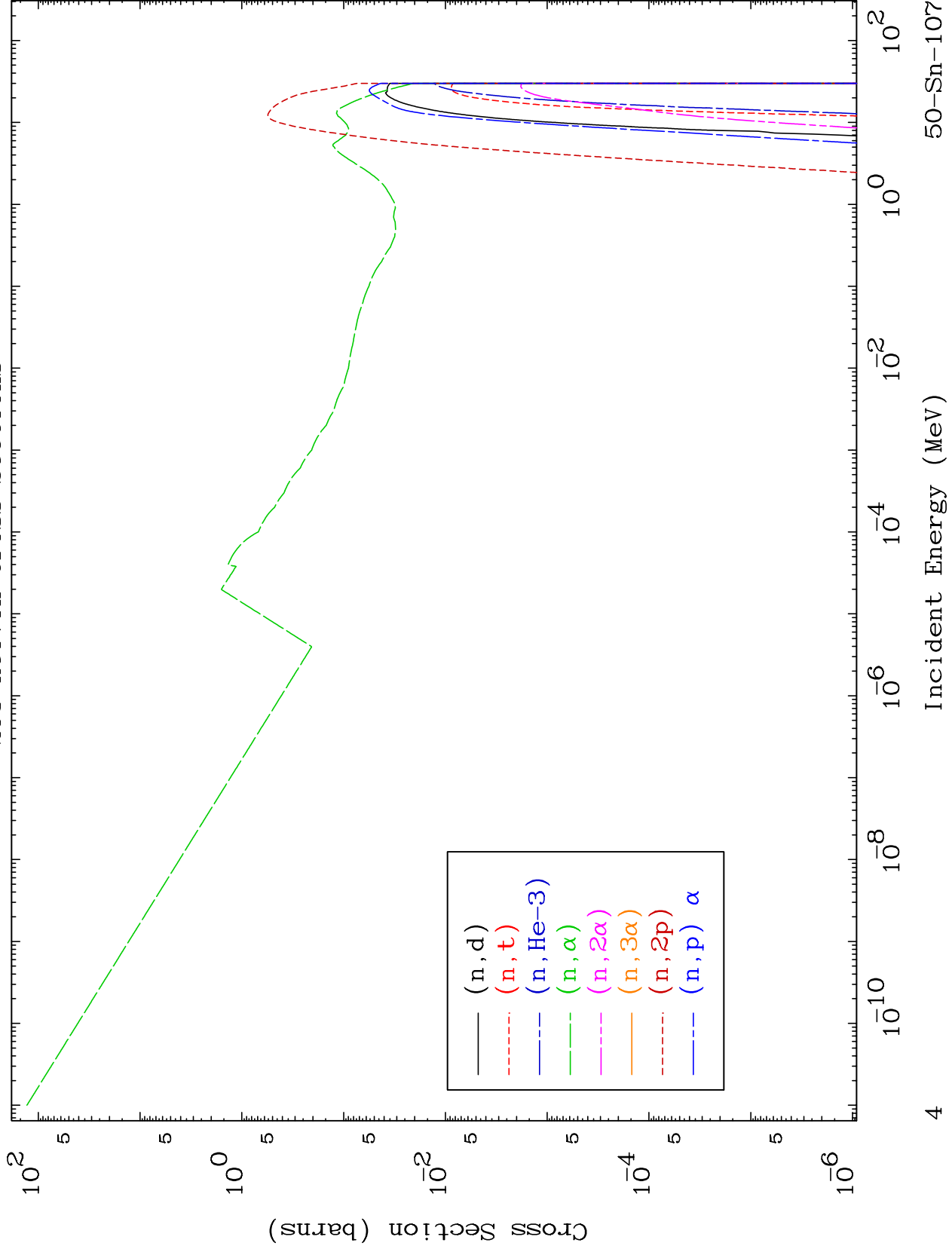




MAT 5010

Neutron Absorption  
293 Kelvin Cross Sections

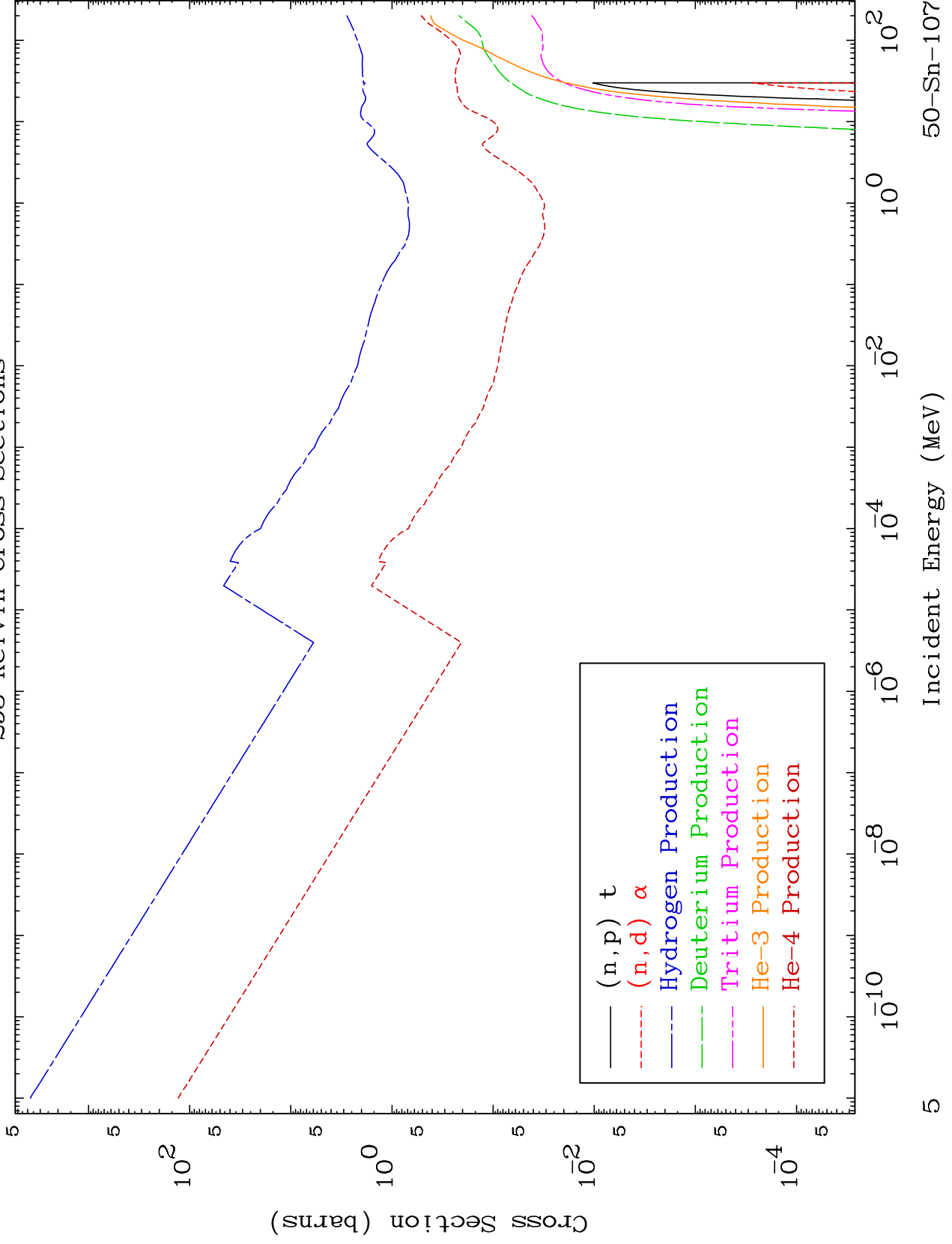
50-Sn-107

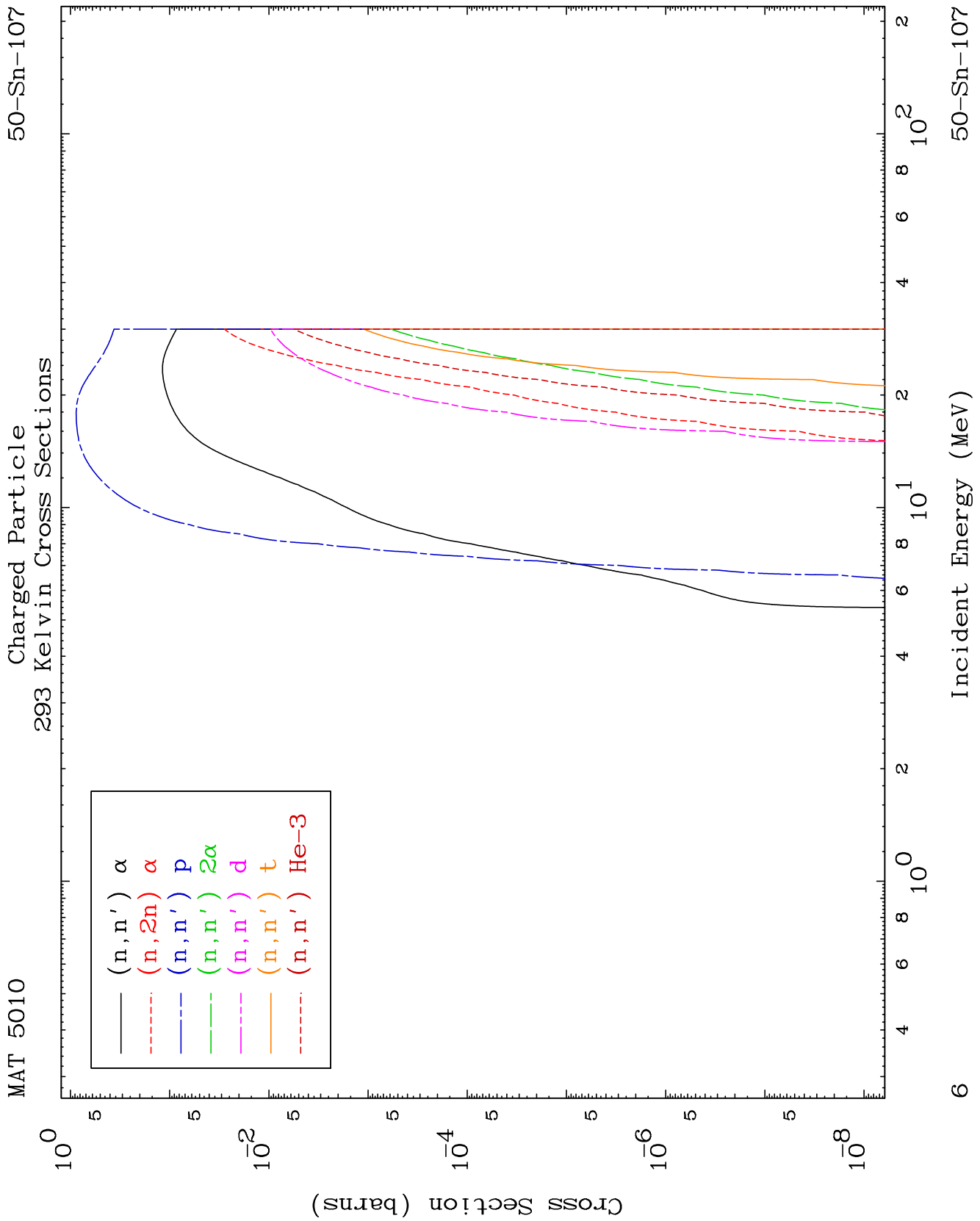


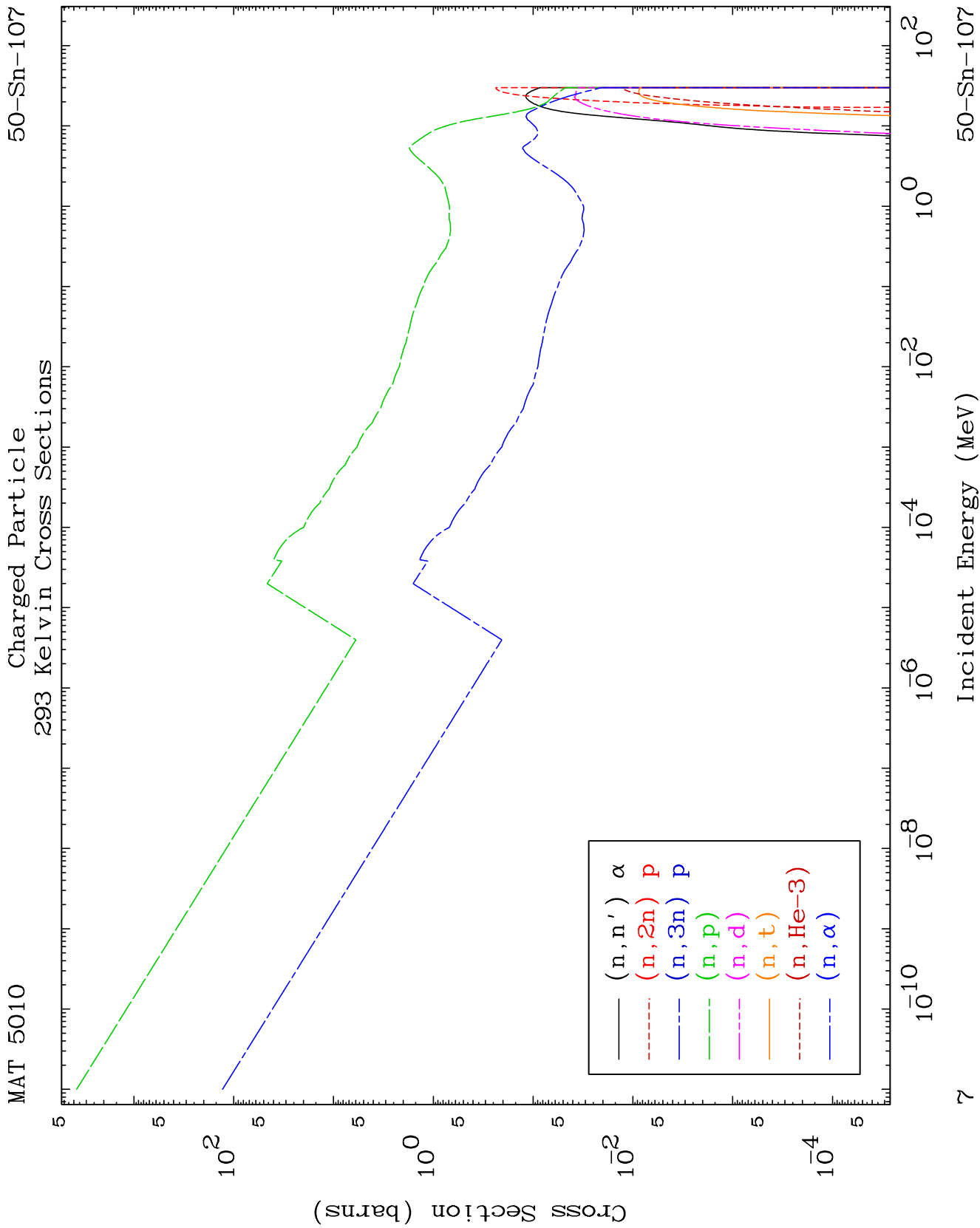
MAT 5010

Neutron Absorption  
293 Kelvin Cross Sections

50-Sn-107



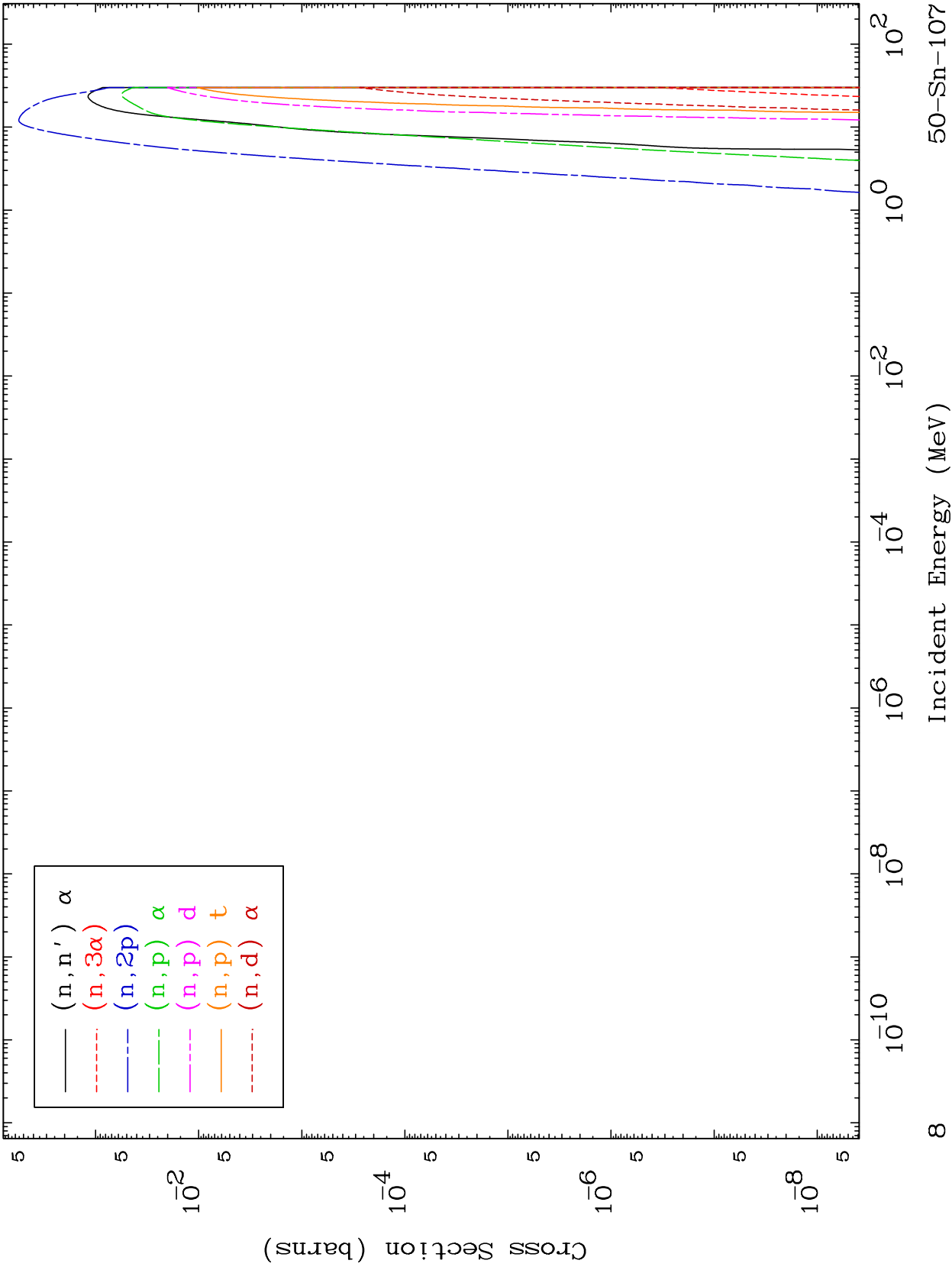


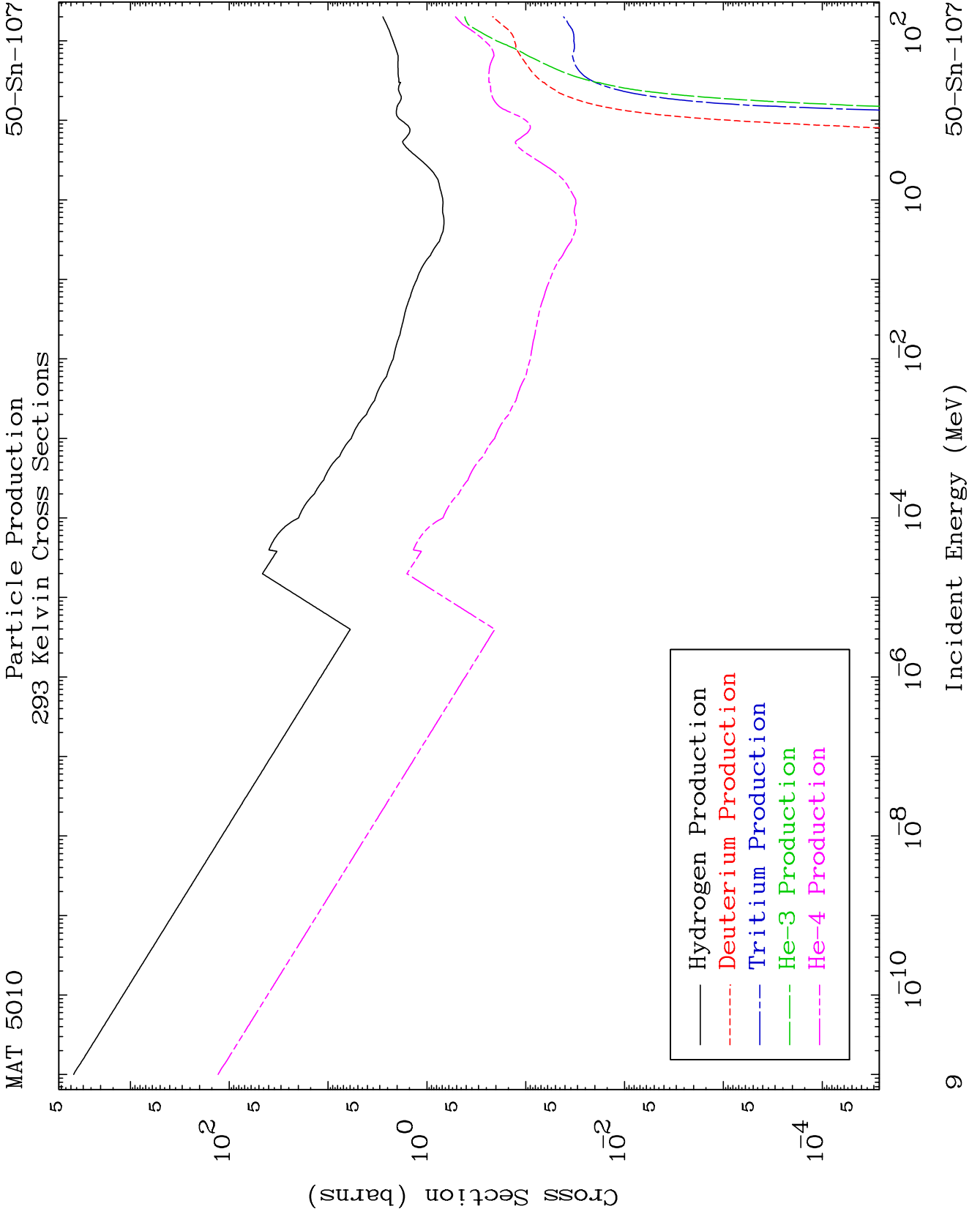


MAT 5010

Charged Particle  
293 Kelvin Cross Sections

50-Sn-107



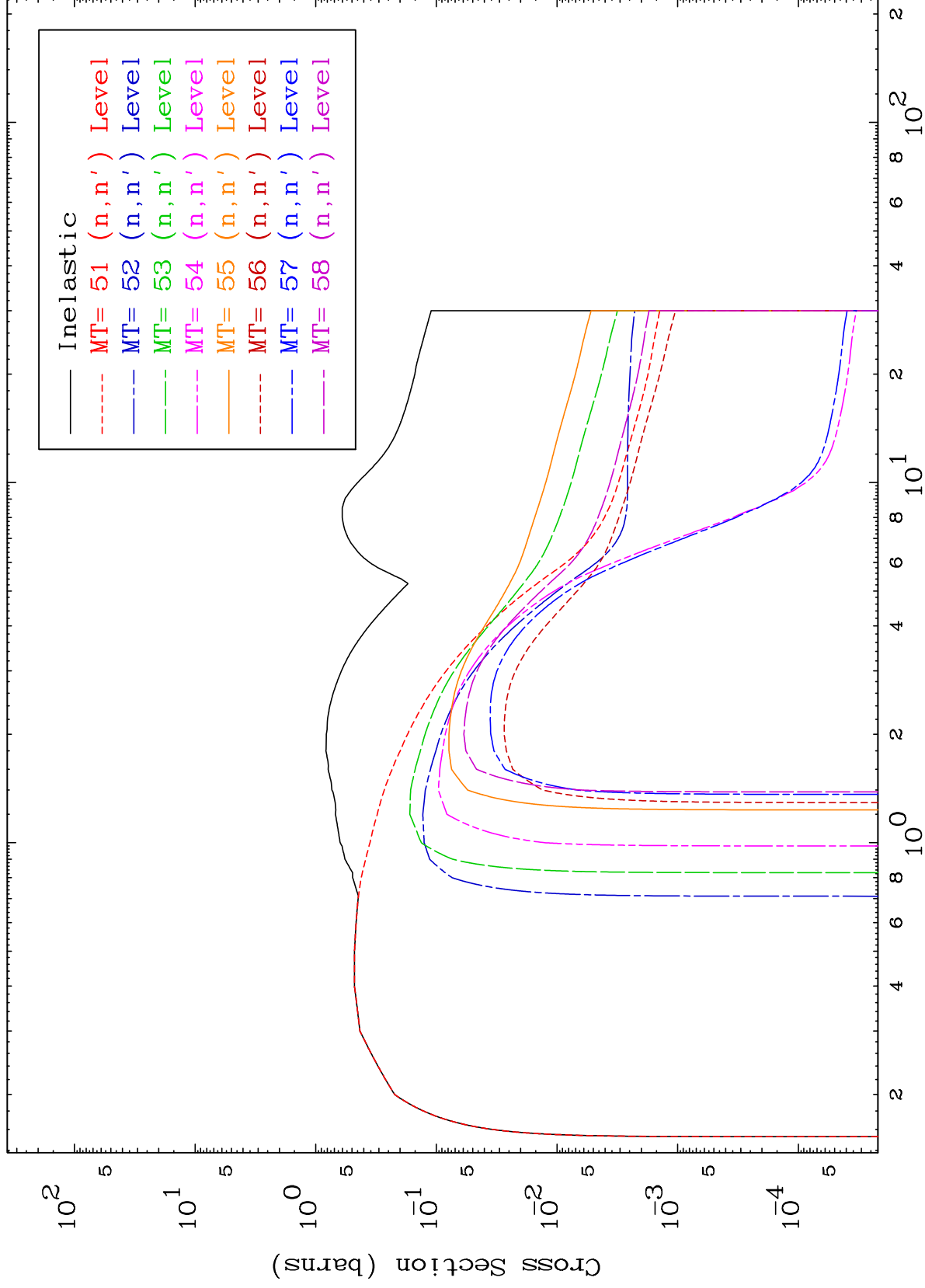


MAT 5010

(n,n') Levels

293 Kelvin Cross Sections

50-Sn-107



10

Incident Energy (MeV)

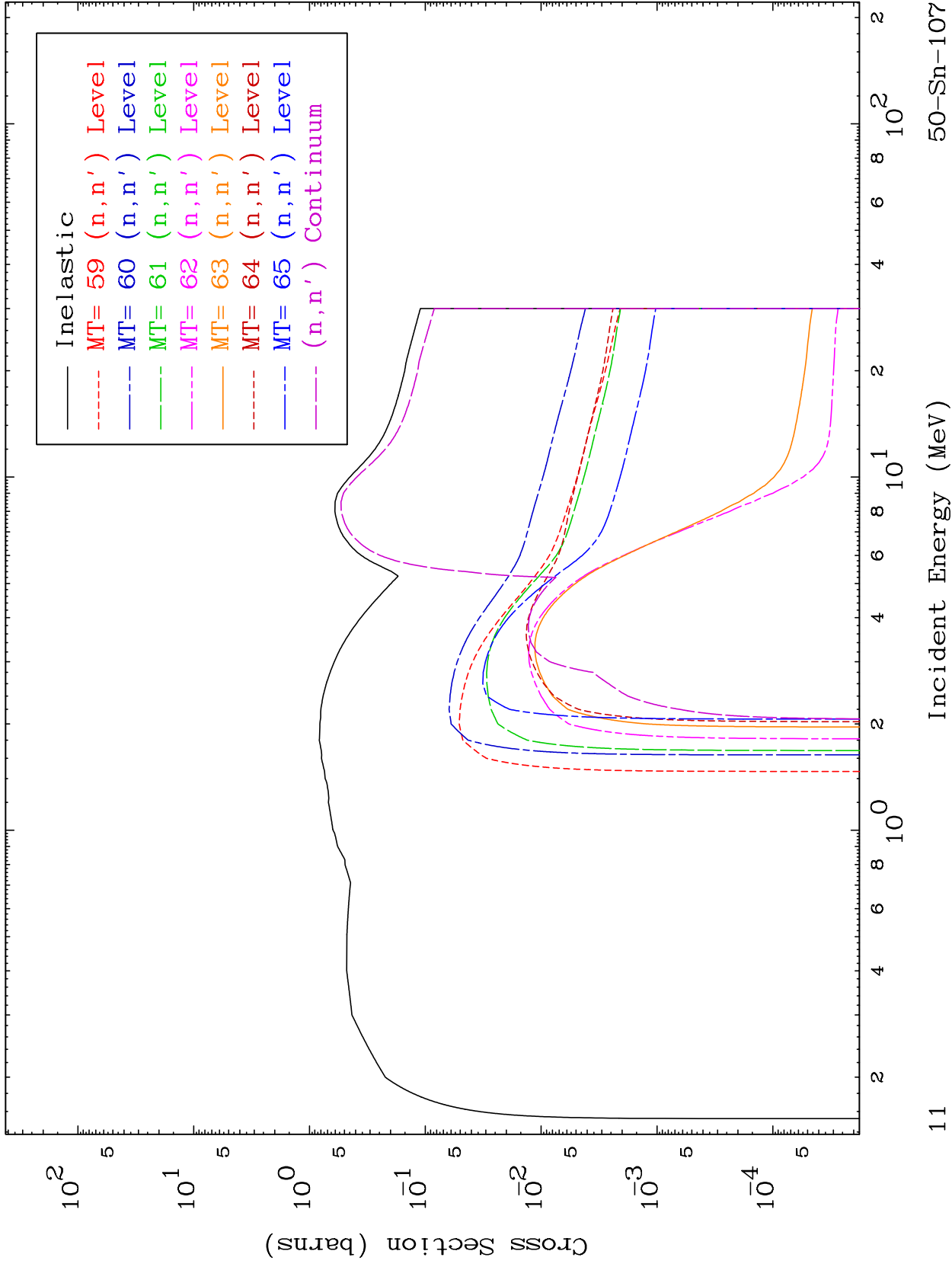
50-Sn-107

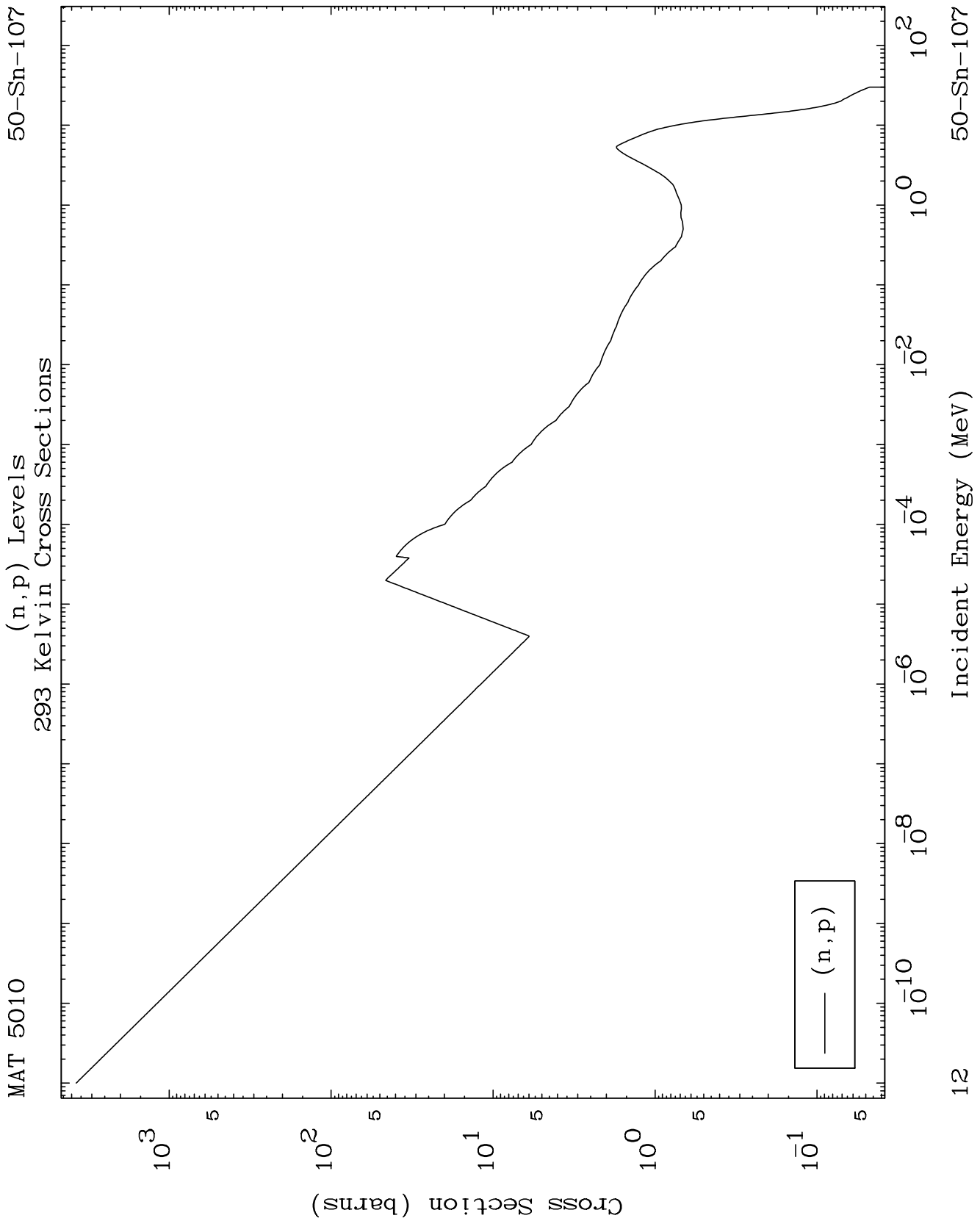
MAT 5010

(n,n') Levels

293 Kelvin Cross Sections

50-Sn-107

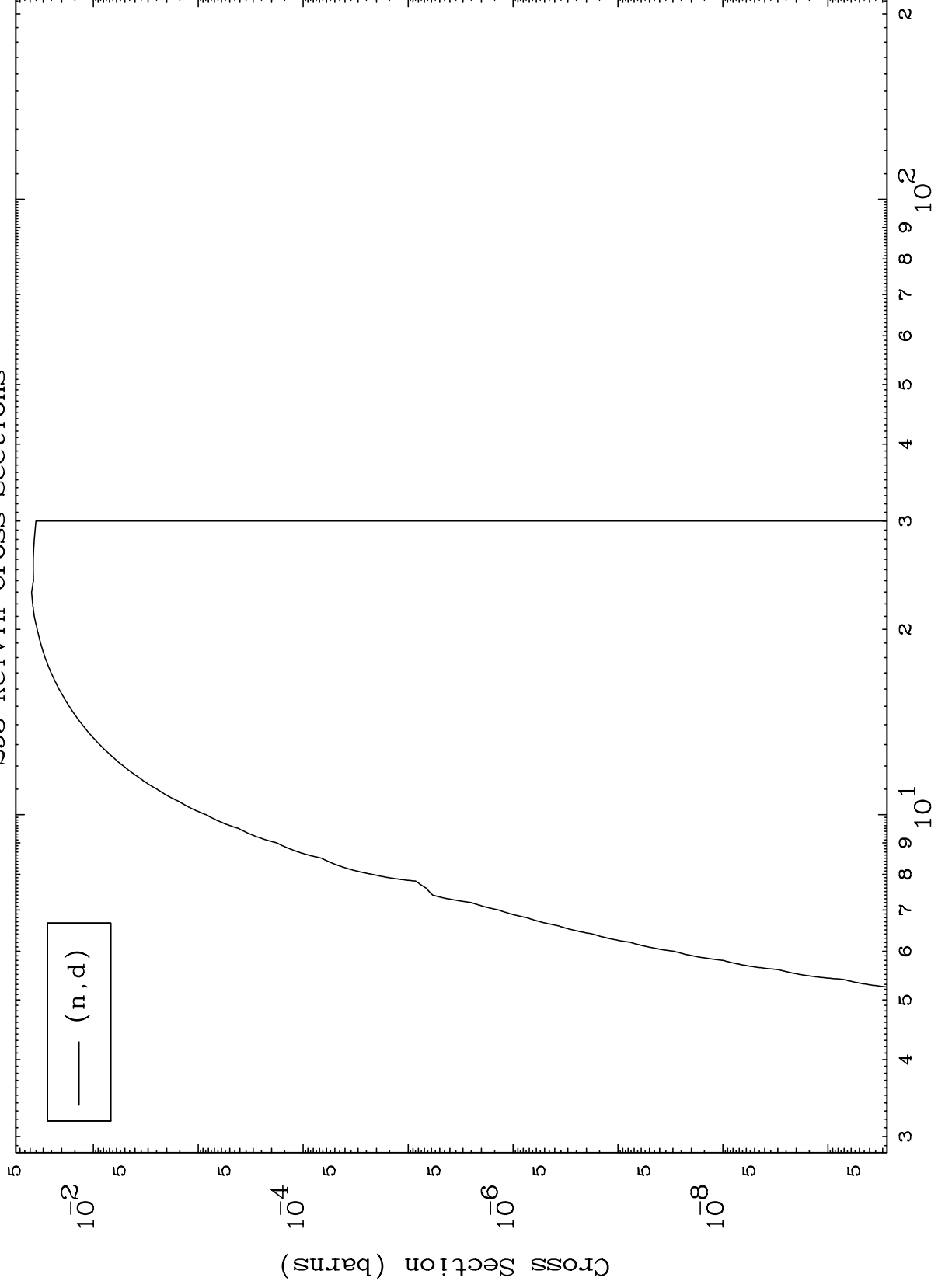




MAT 5010

(n,d) Levels  
293 Kelvin Cross Sections

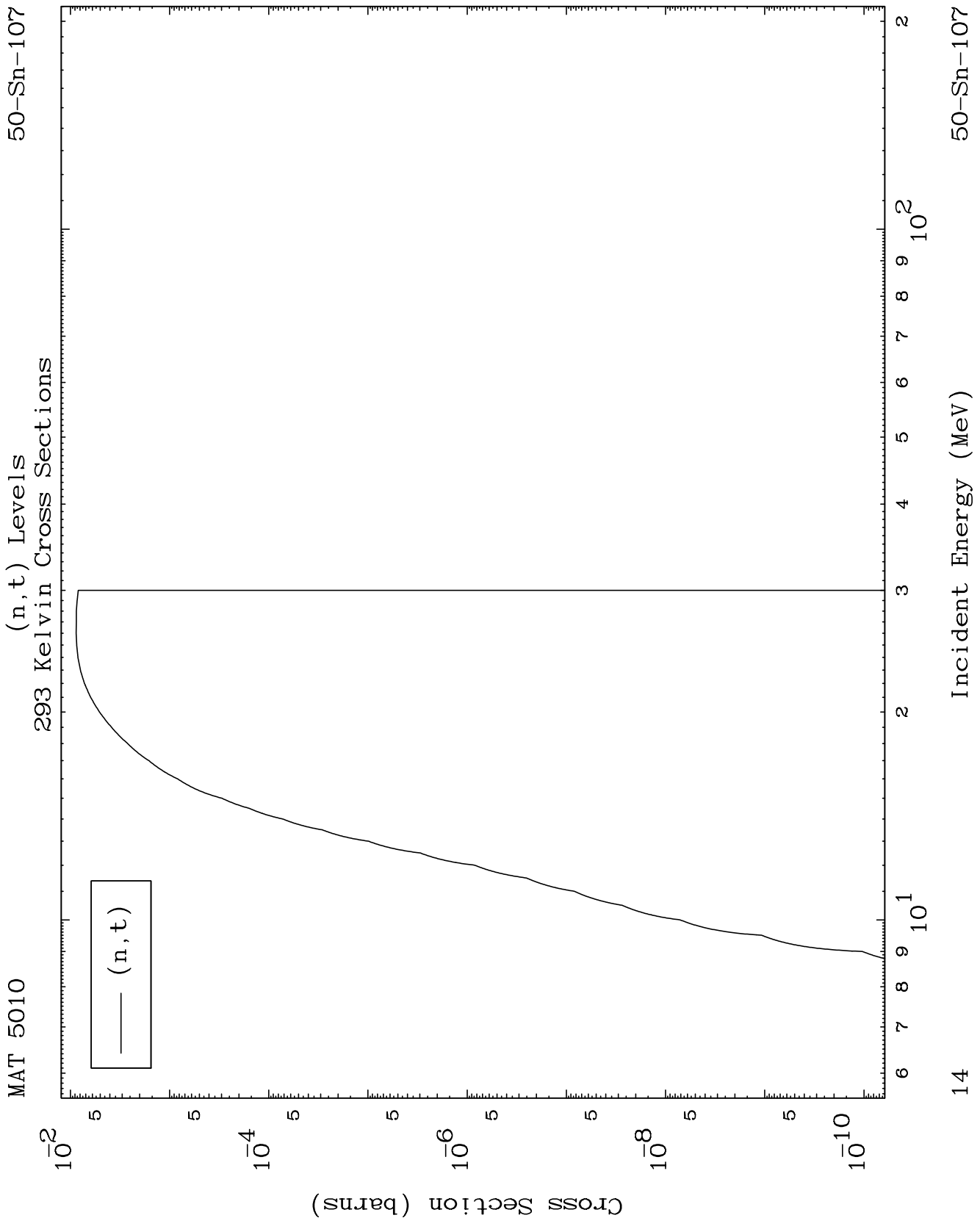
50-Sn-107



Incident Energy (MeV)

50-Sn-107

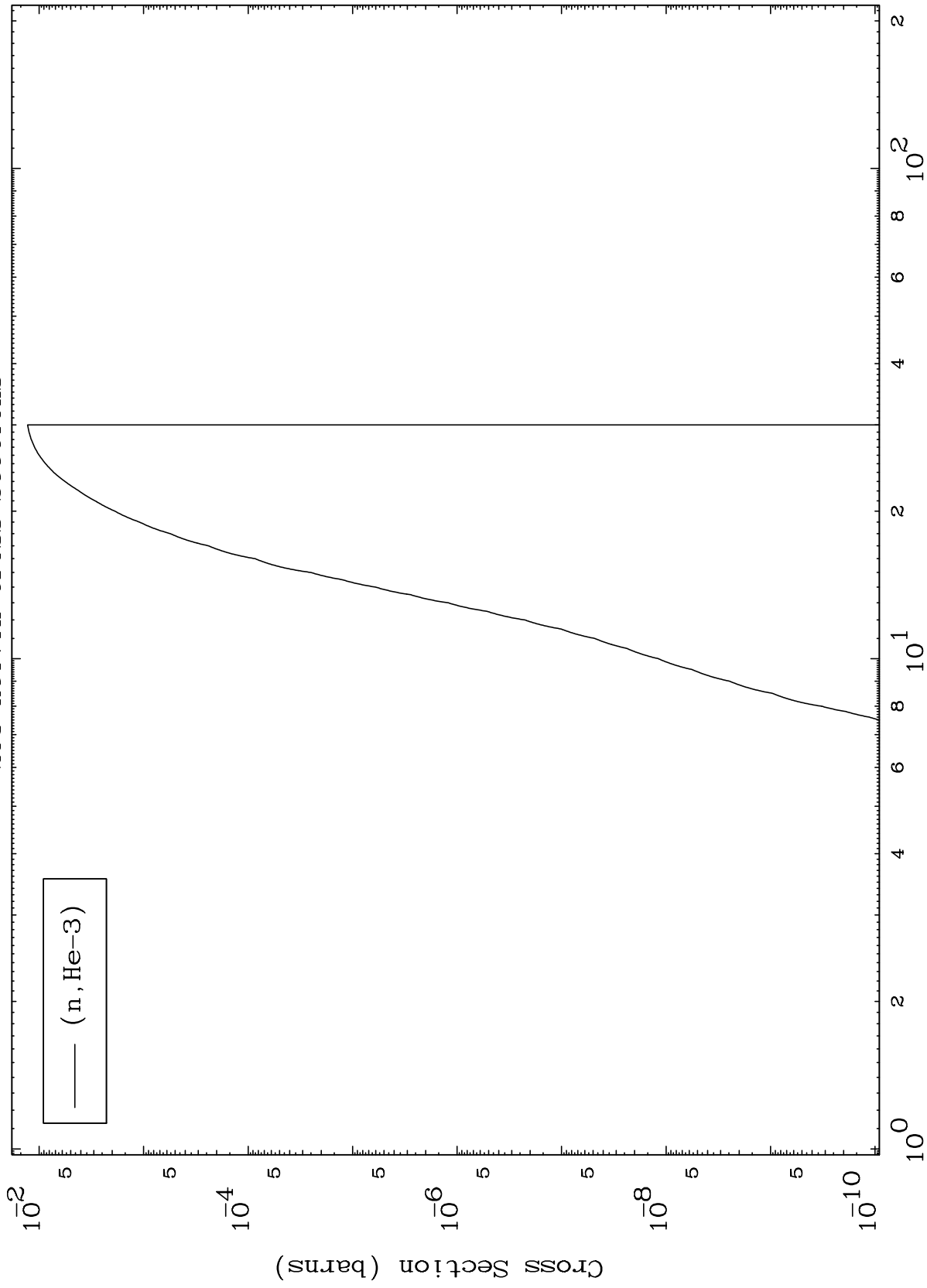
13



MAT 5010

(n,He3) Levels  
293 Kelvin Cross Sections

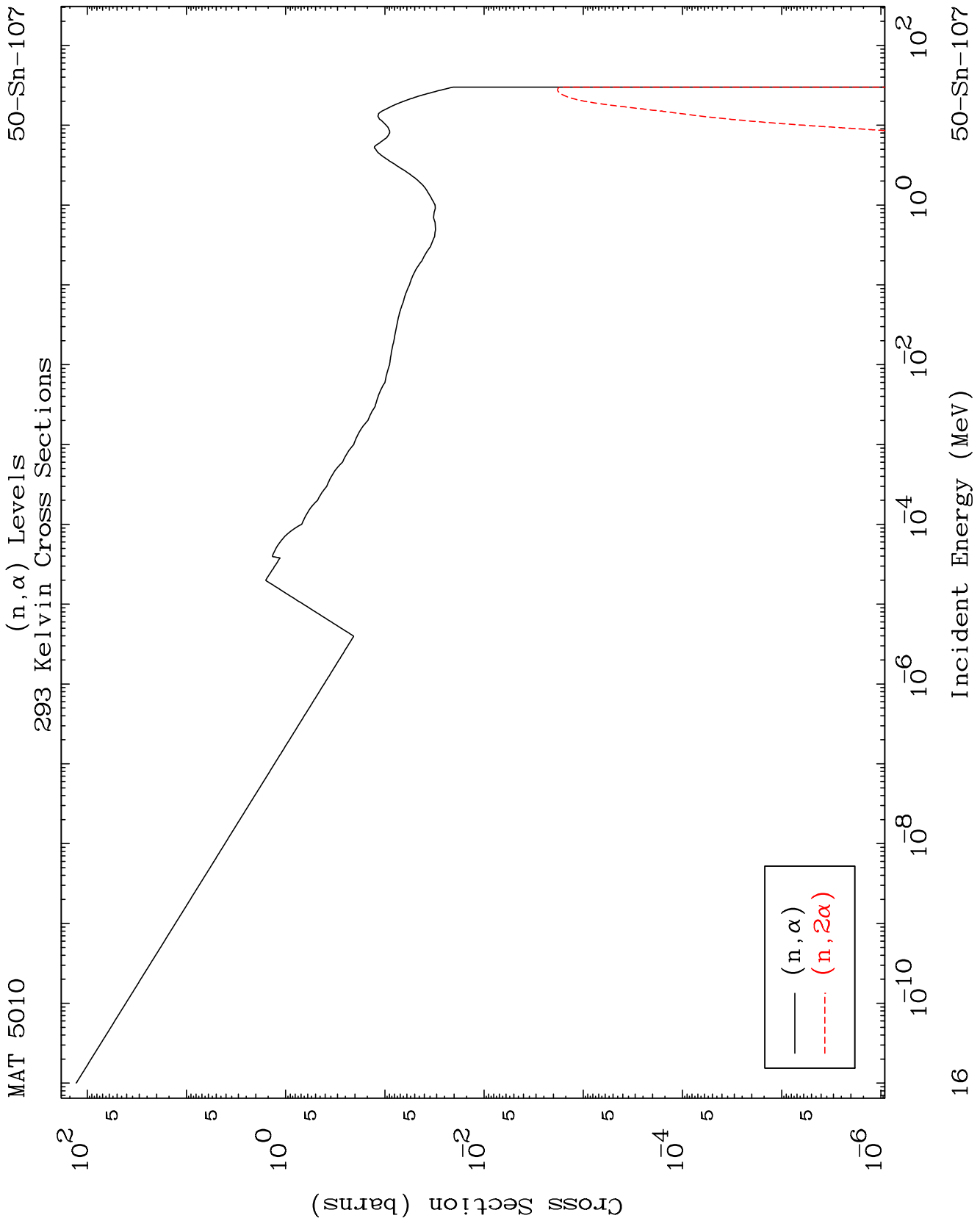
50-Sn-107



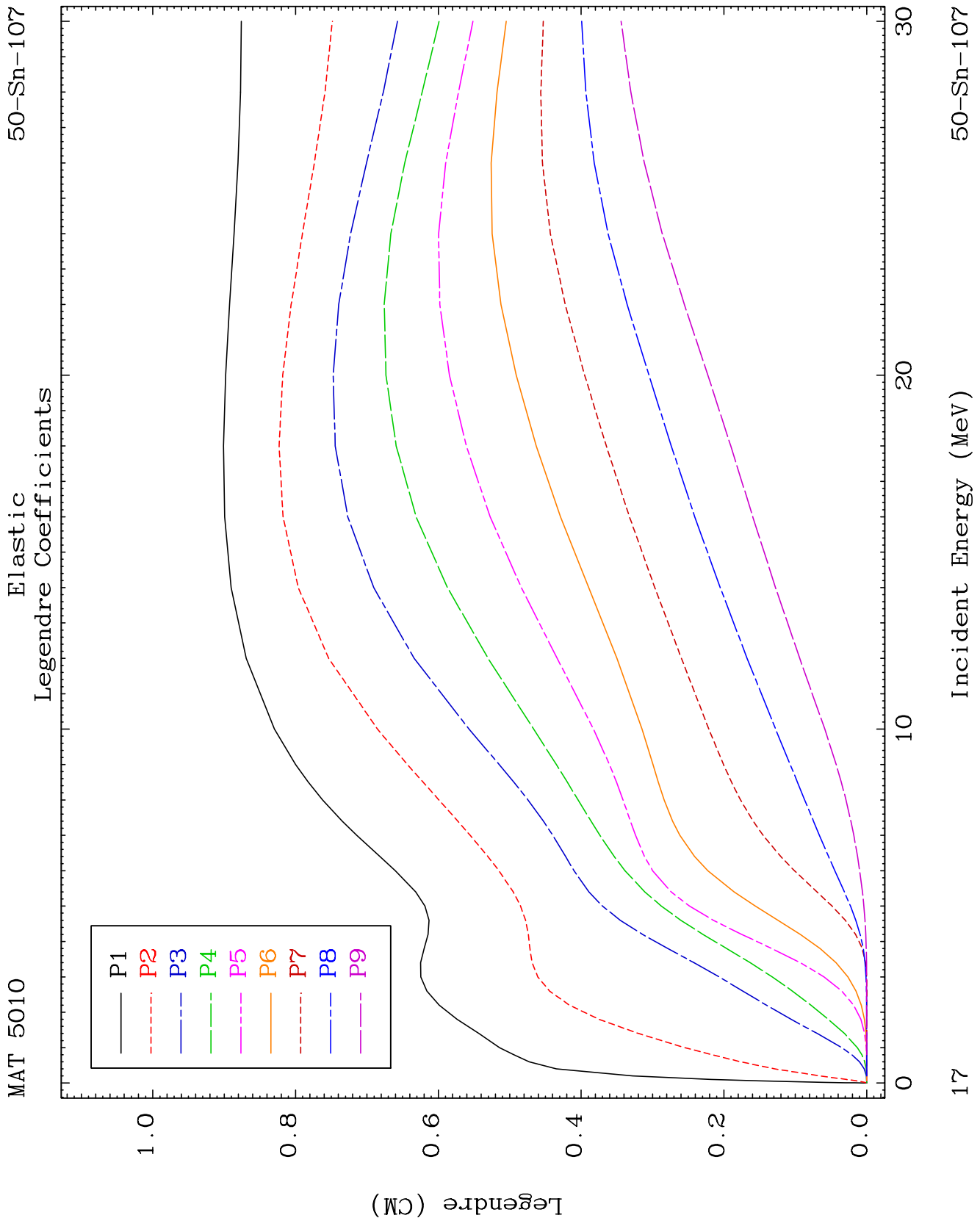
Incident Energy (MeV)

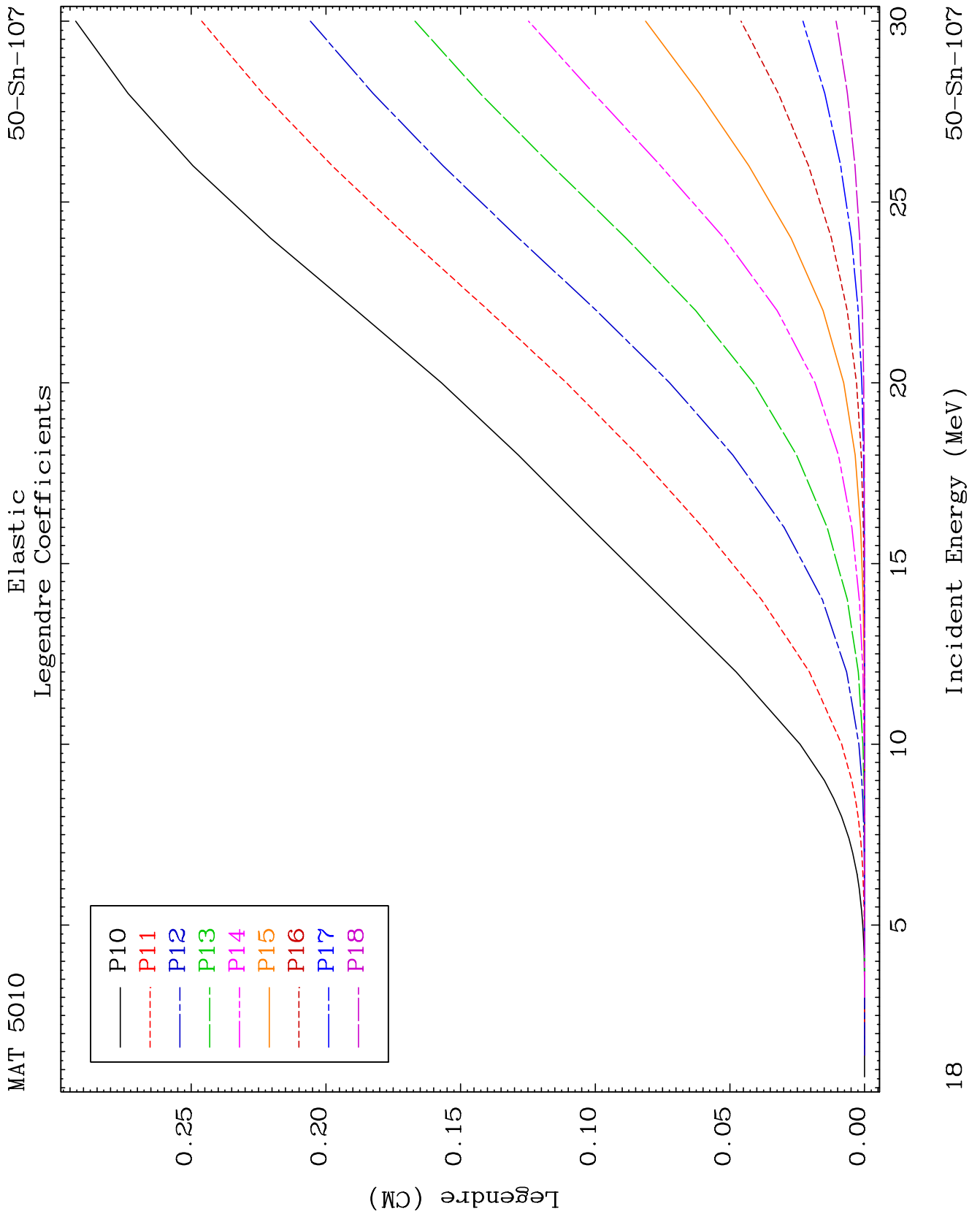
50-Sn-107

15



MAT 5010

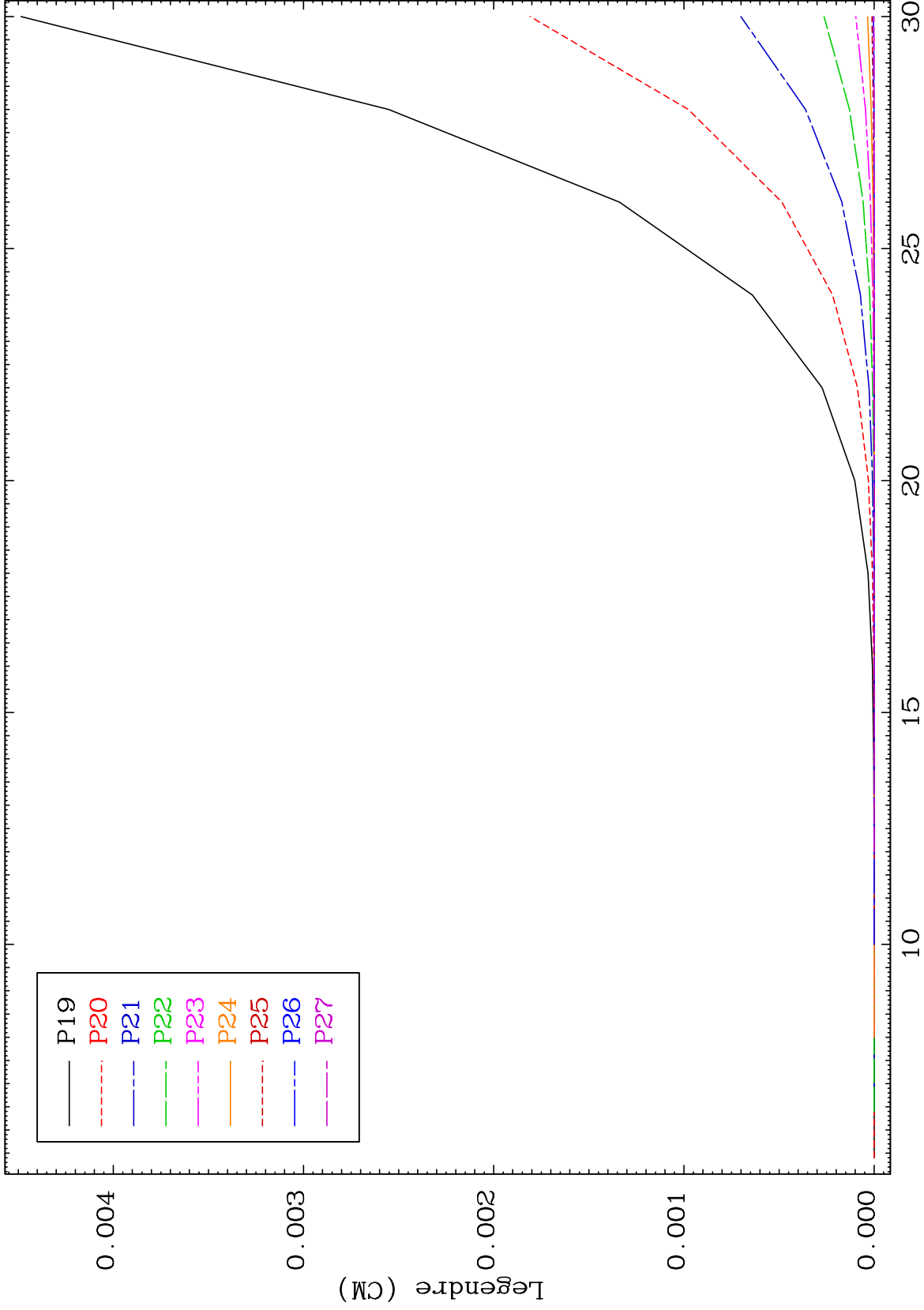




MAT 5010

Elastic  
Legendre Coefficients

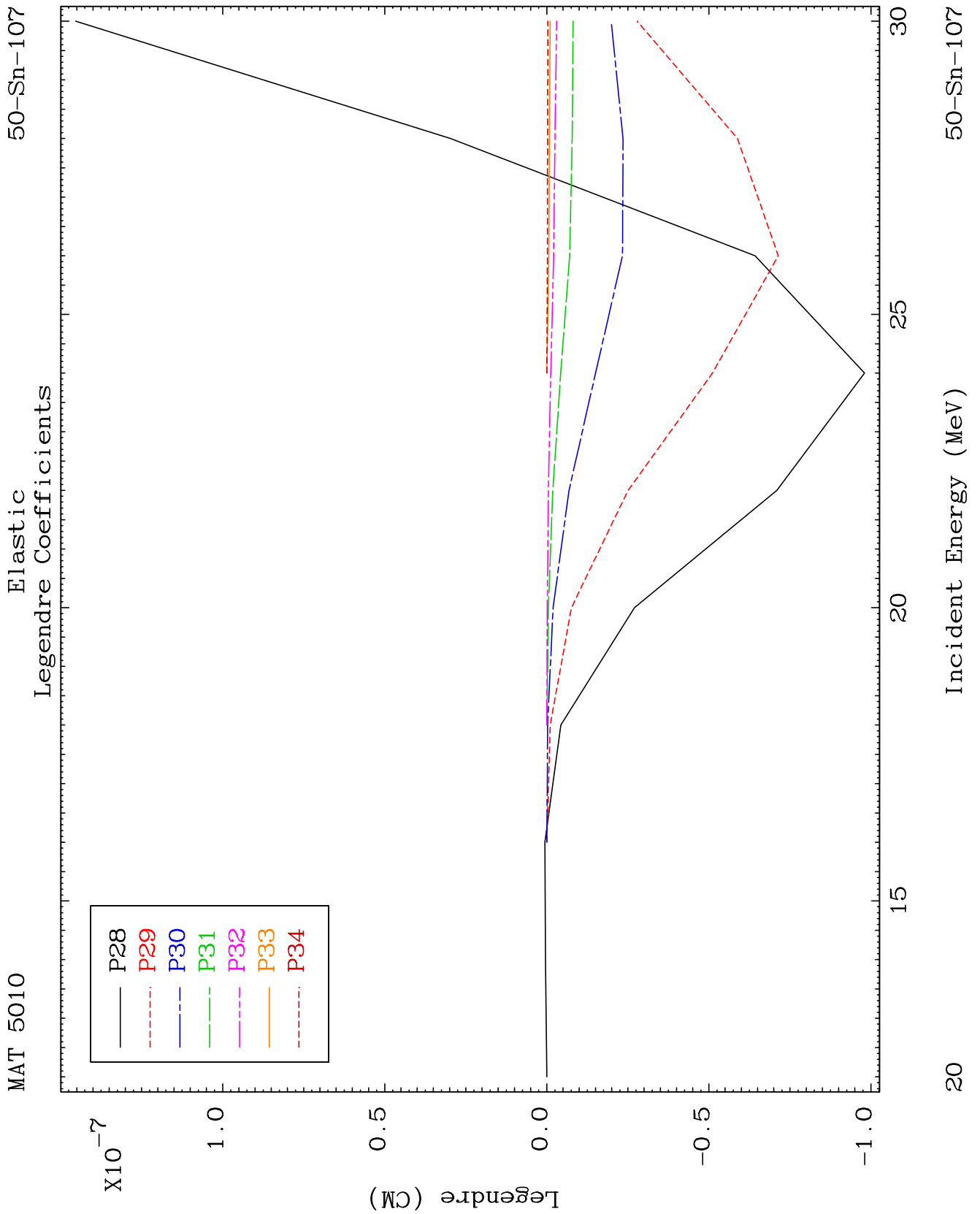
50-Sn-107



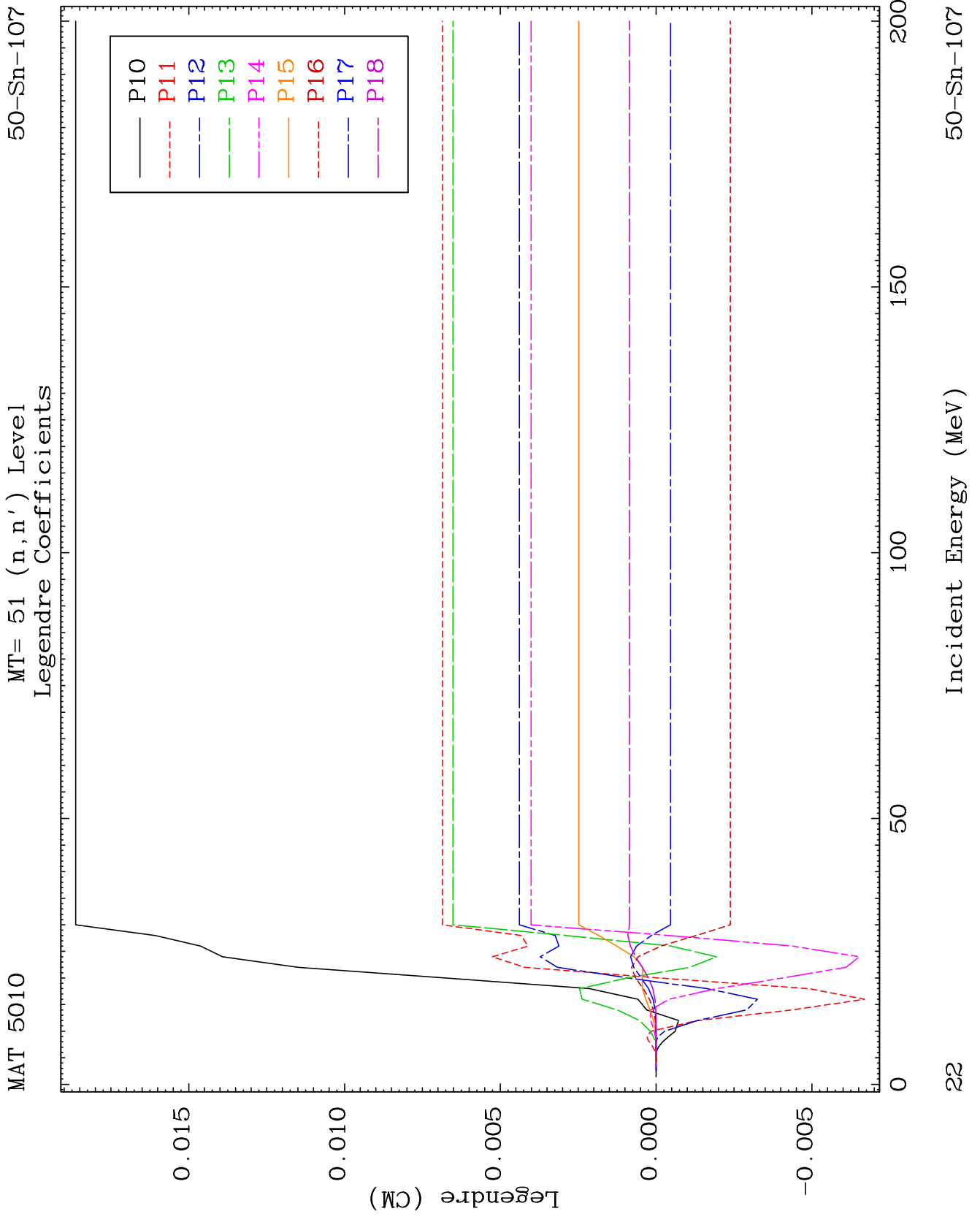
19

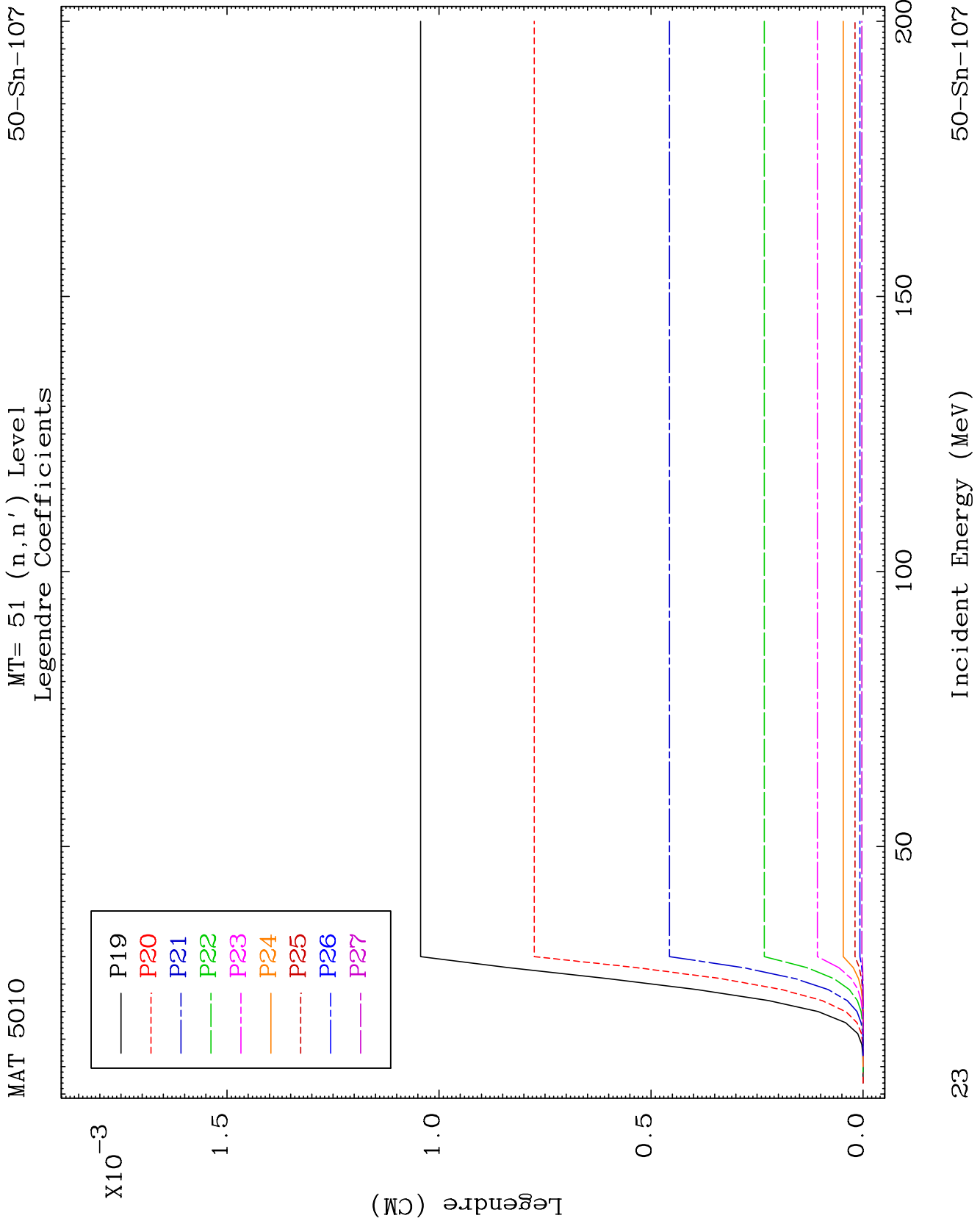
Incident Energy (MeV)

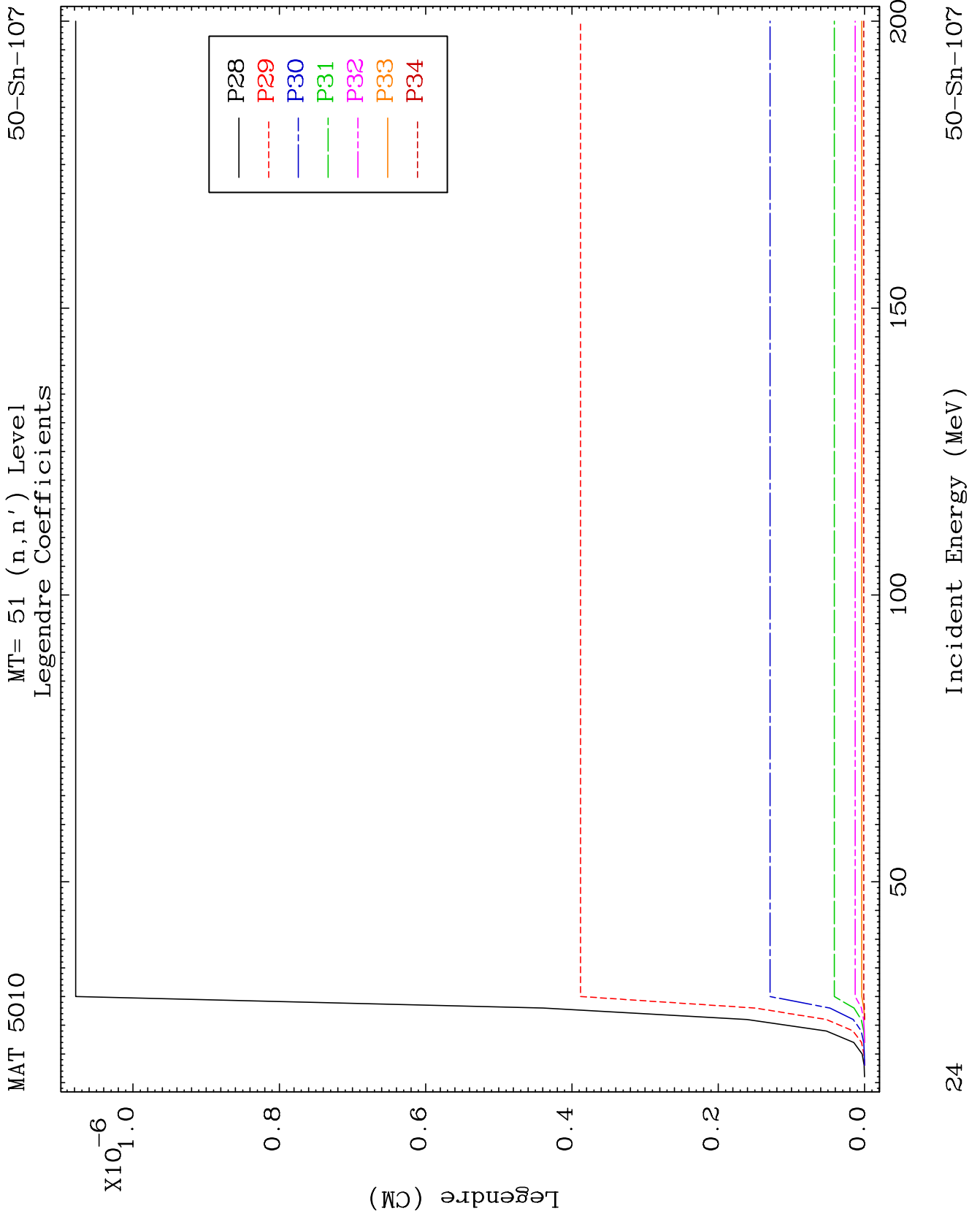
50-Sn-107



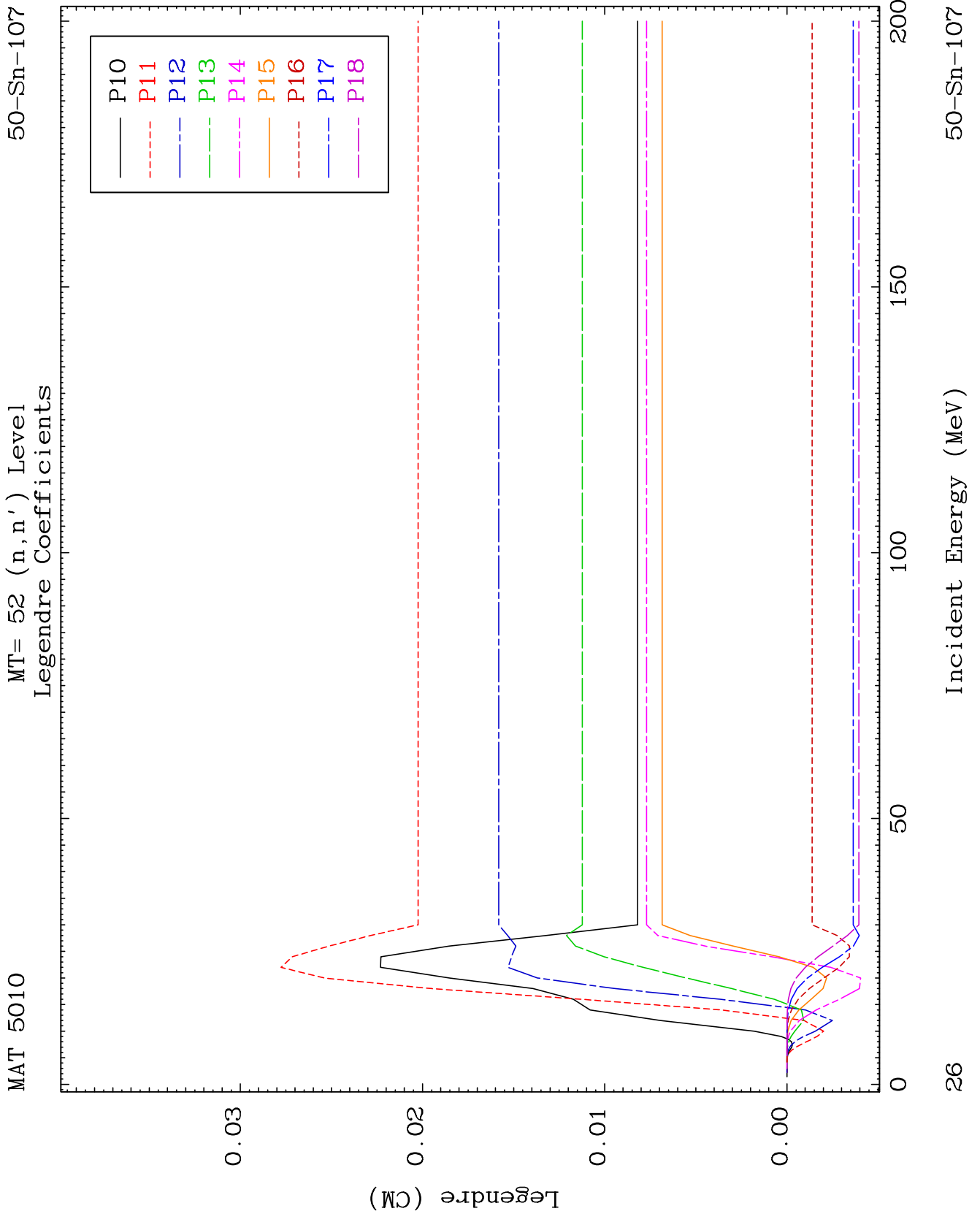


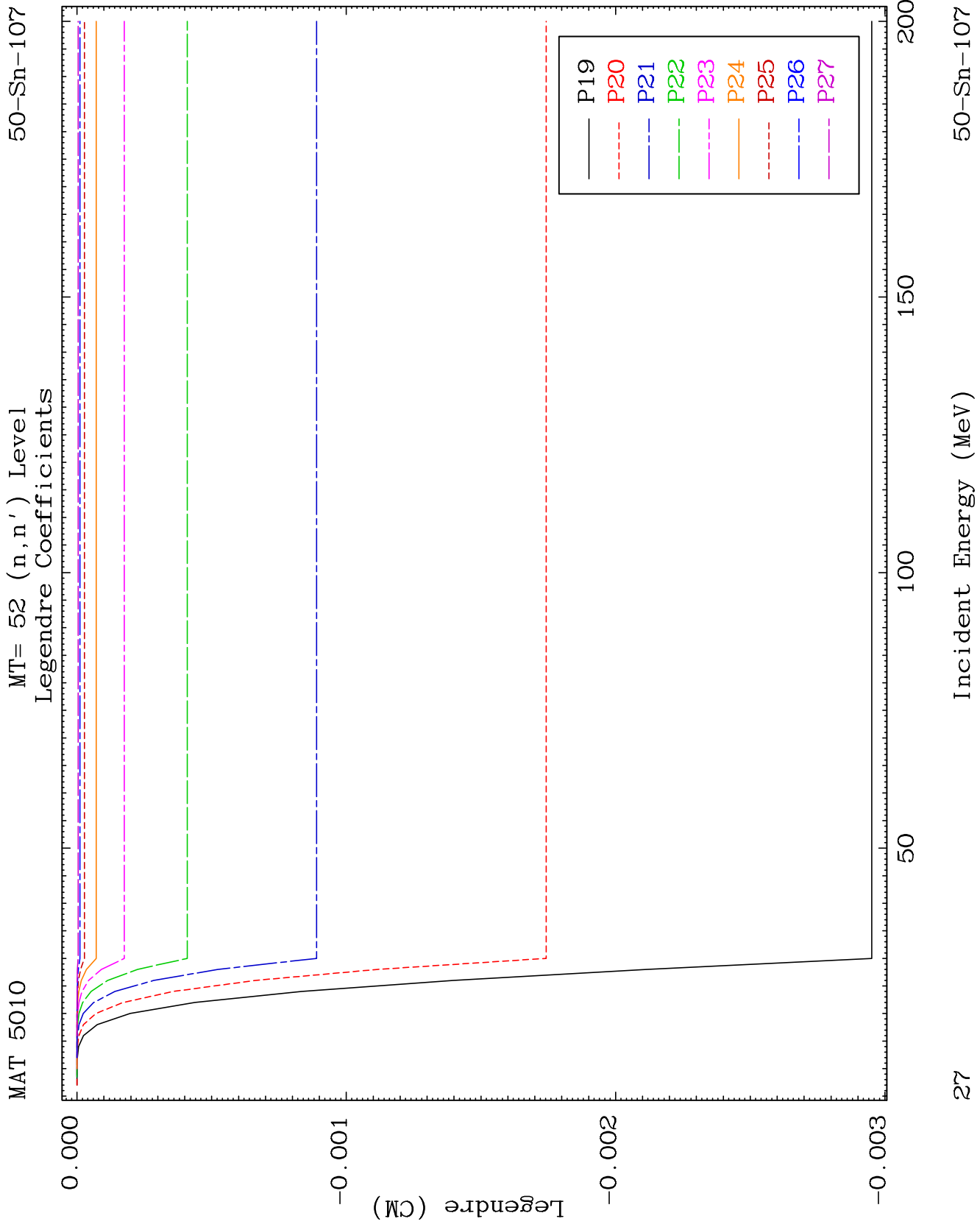


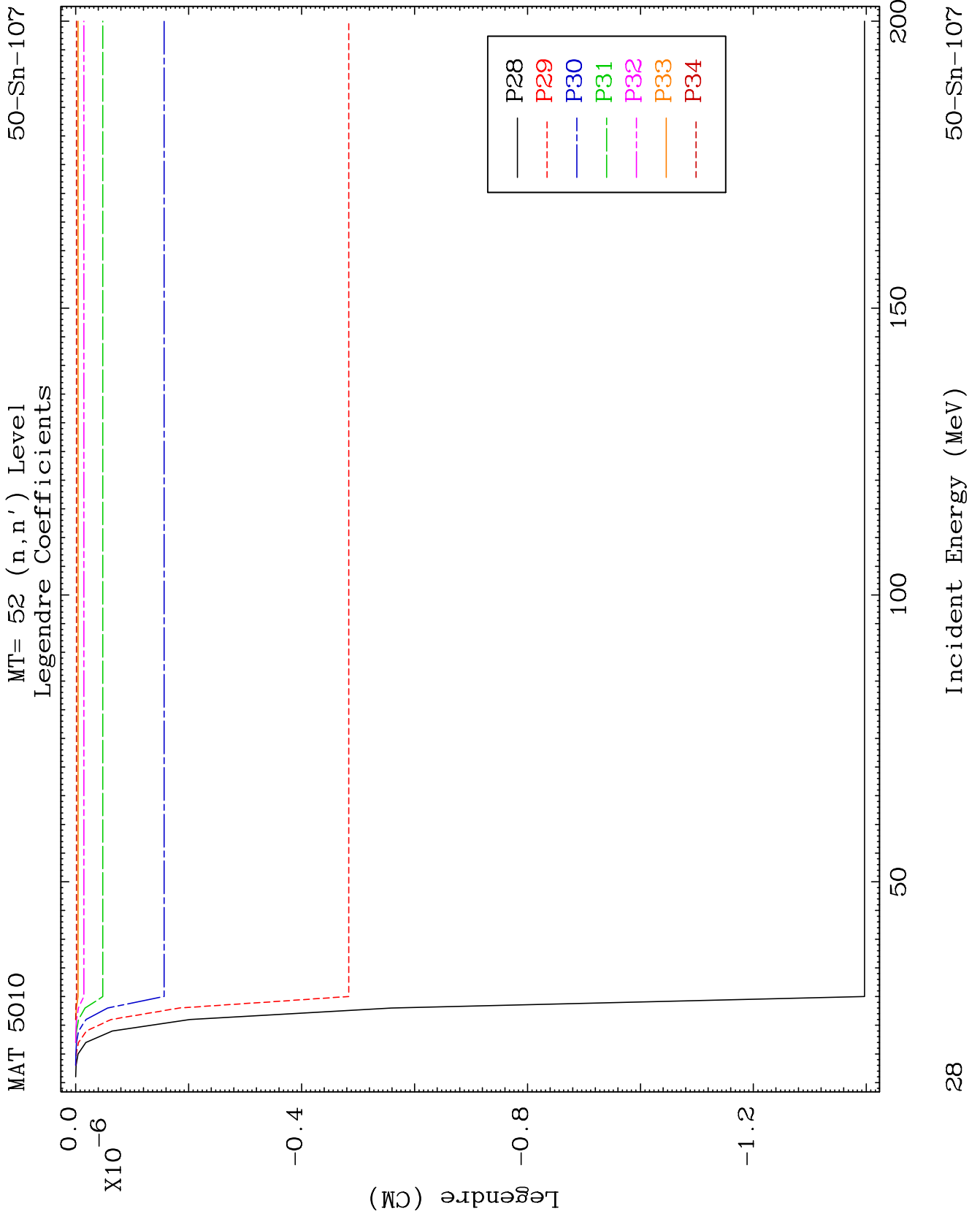


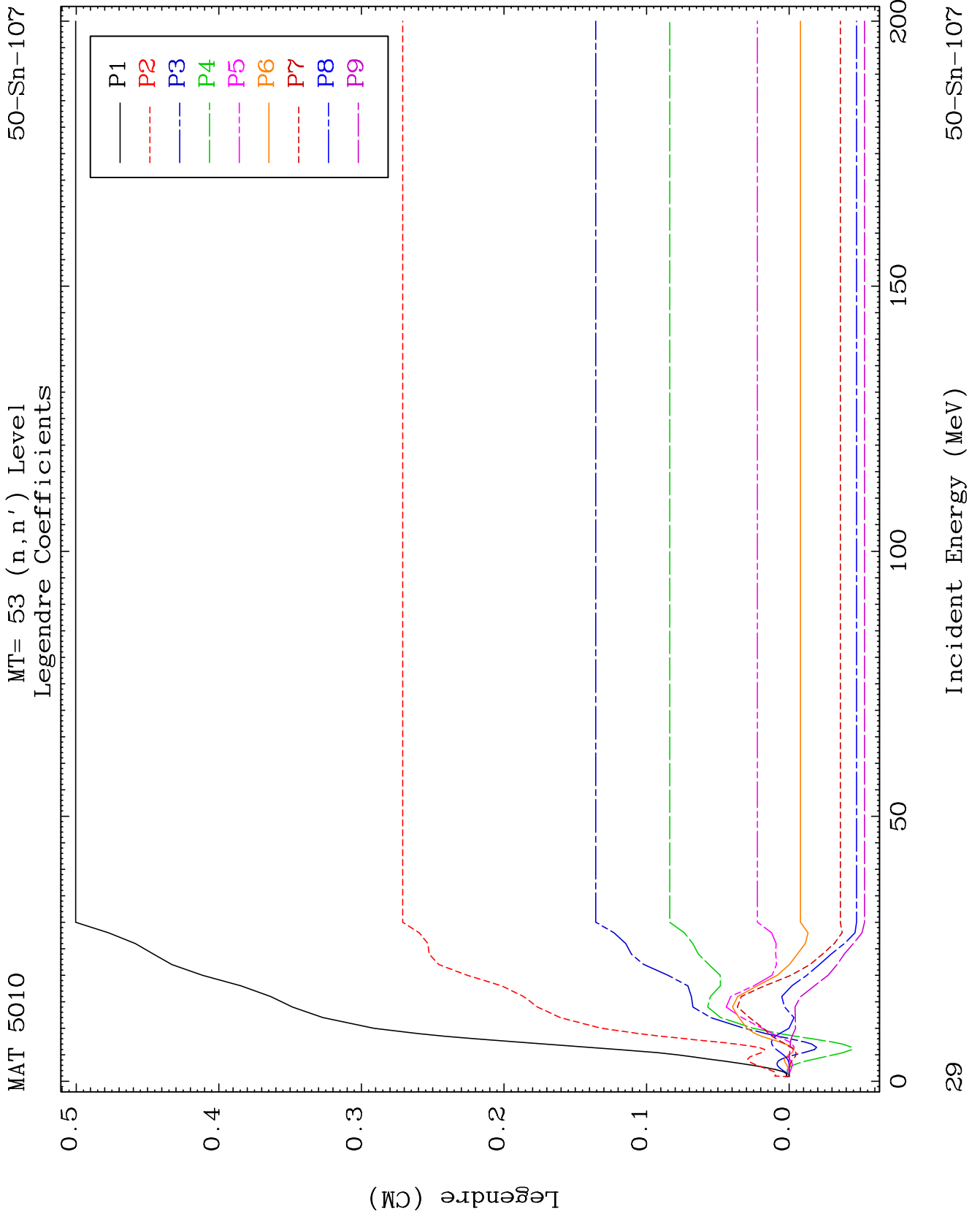


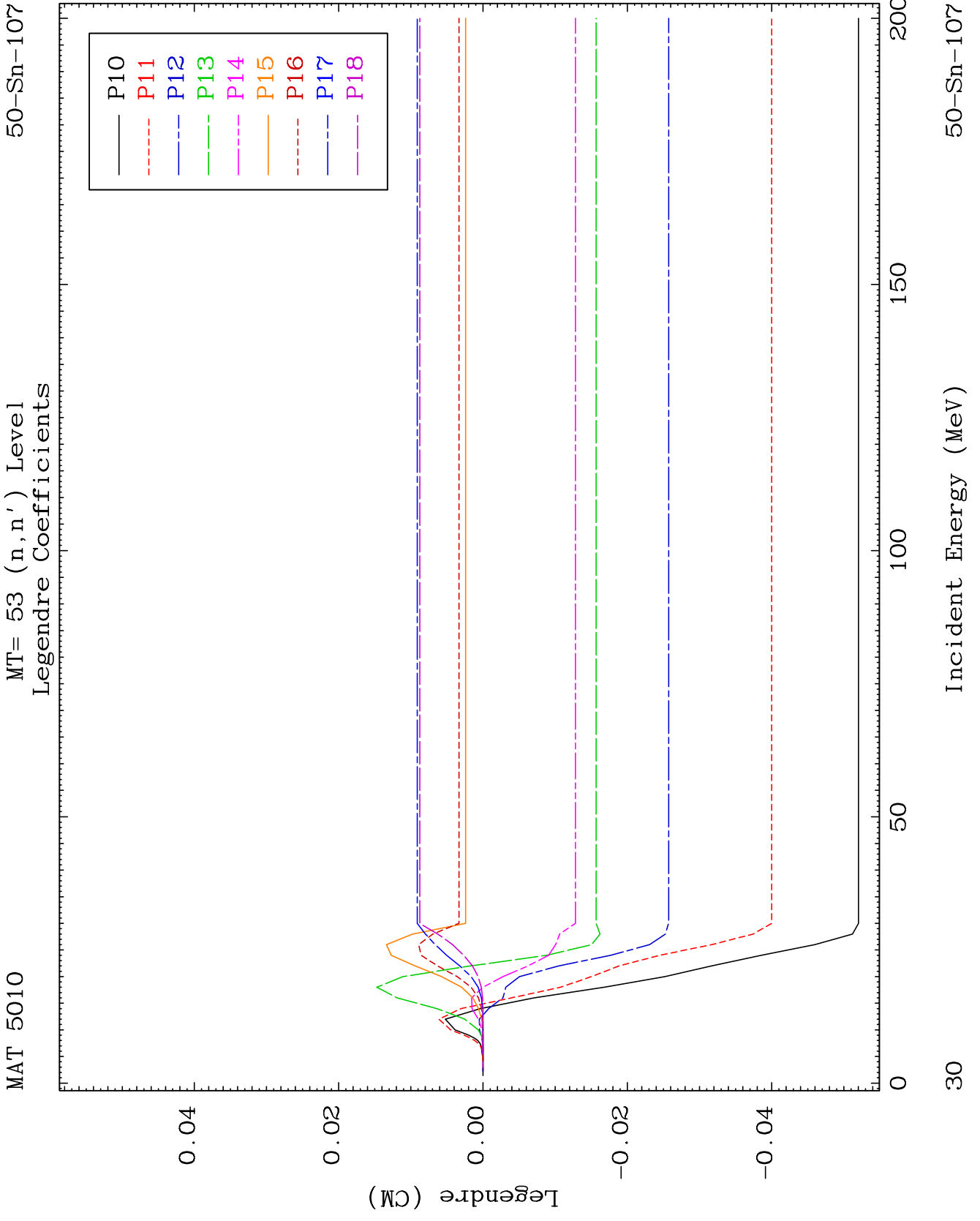


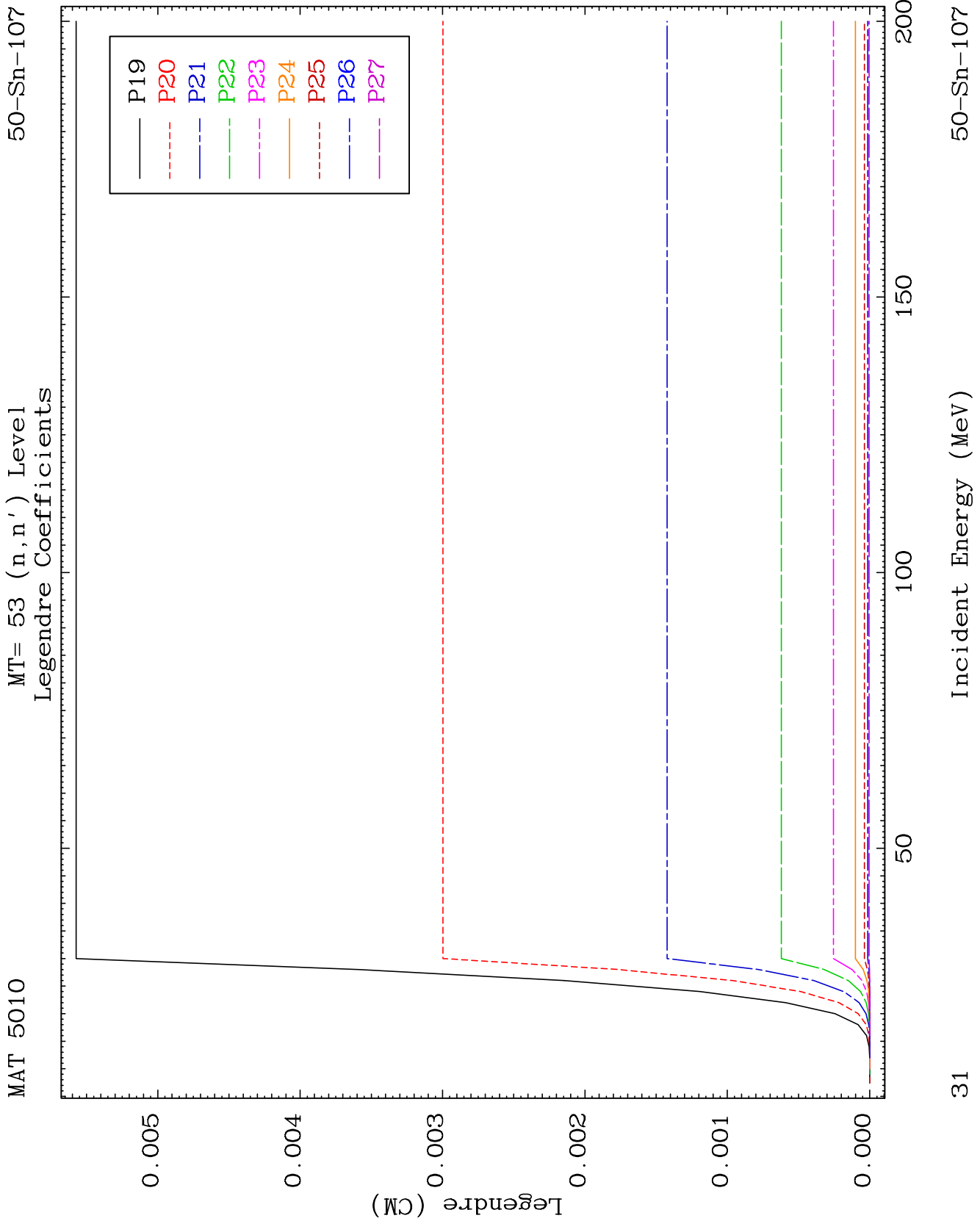


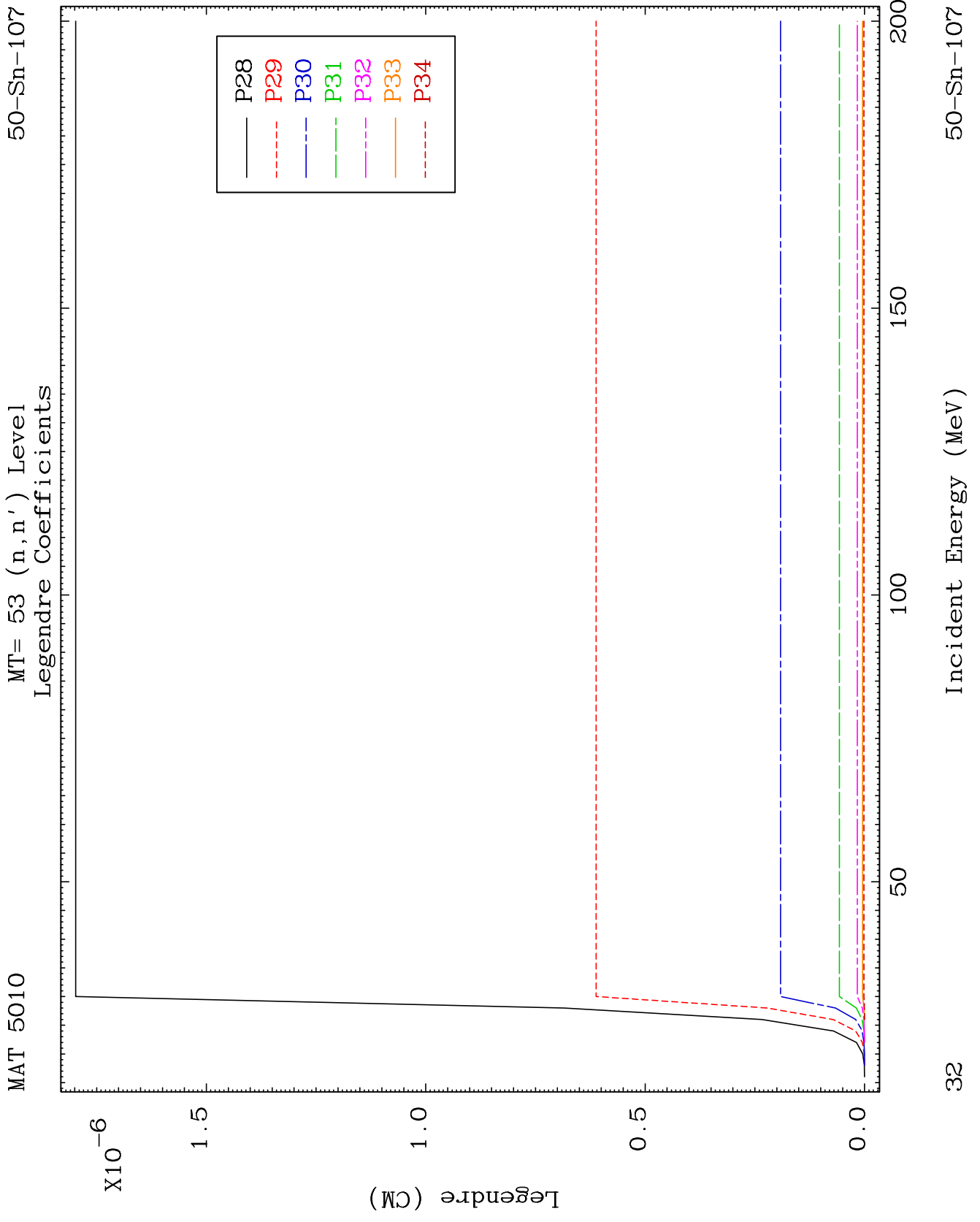




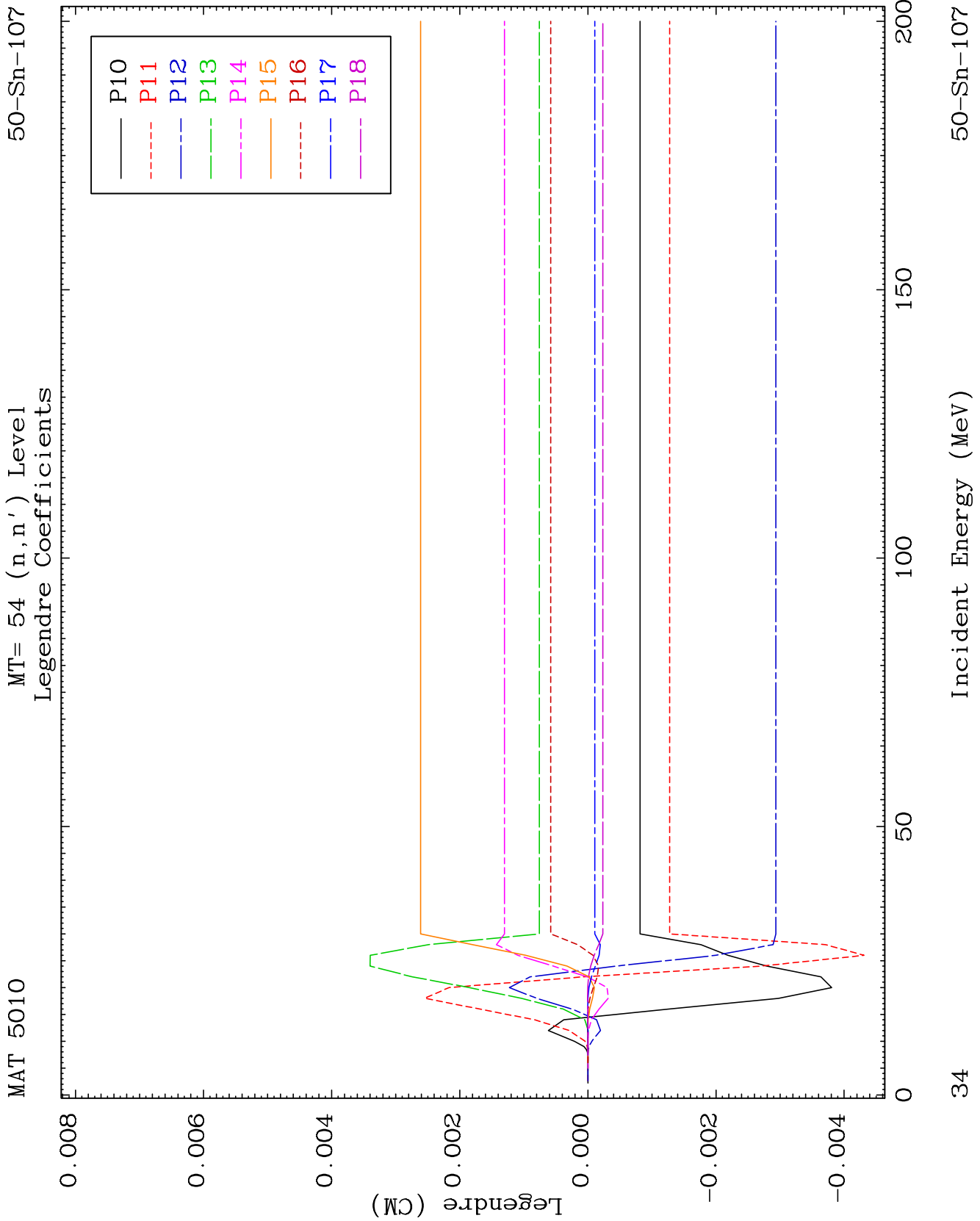


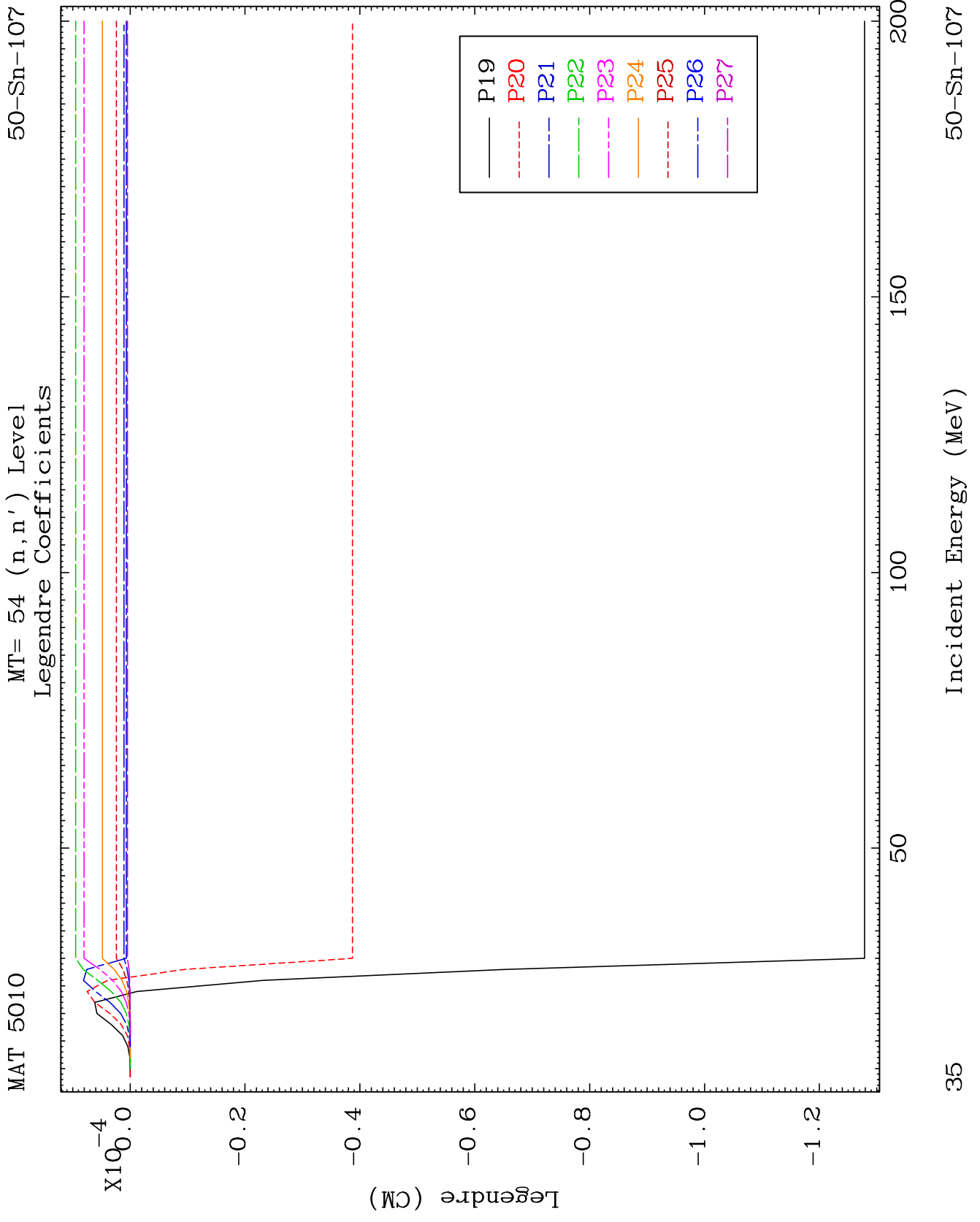


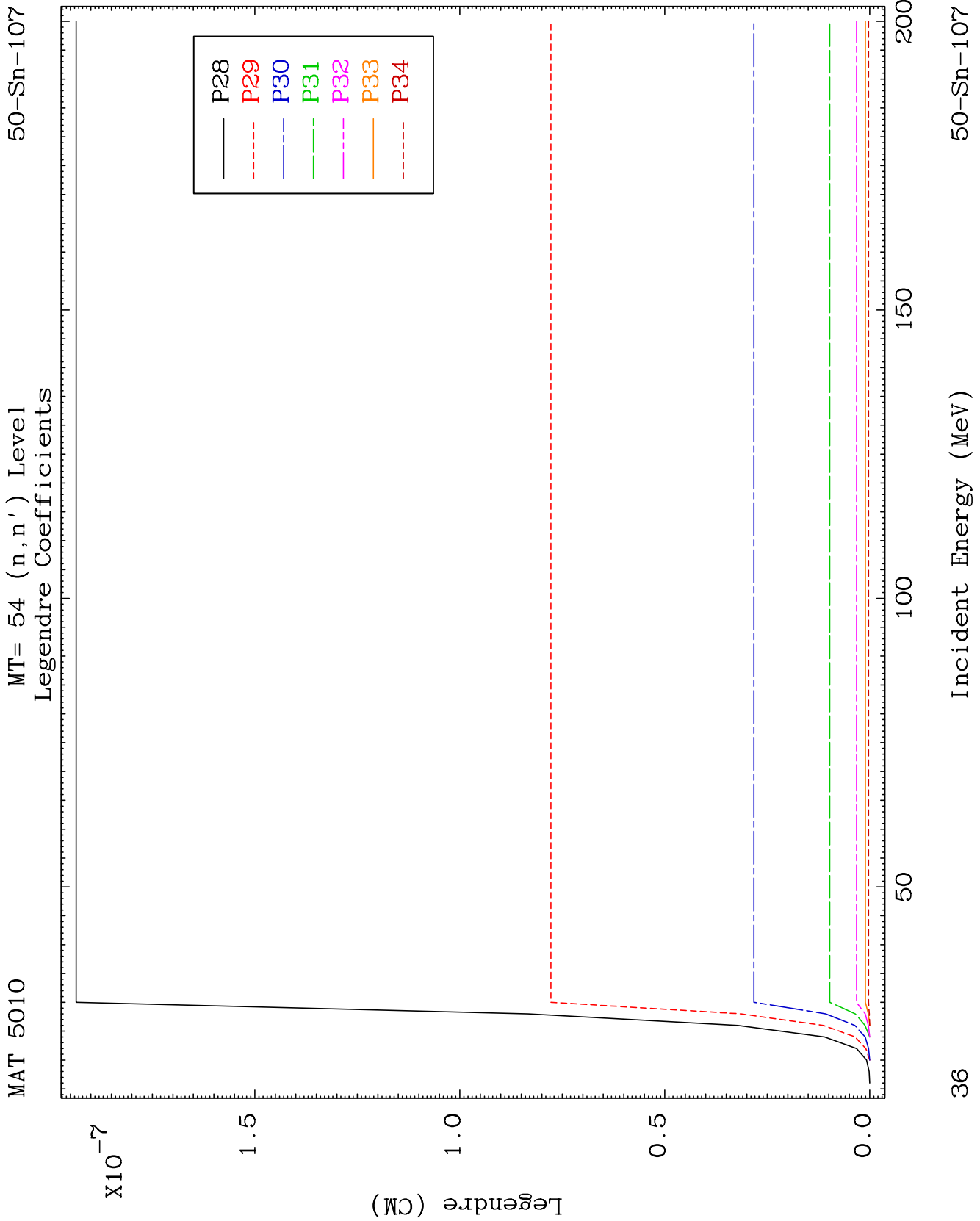


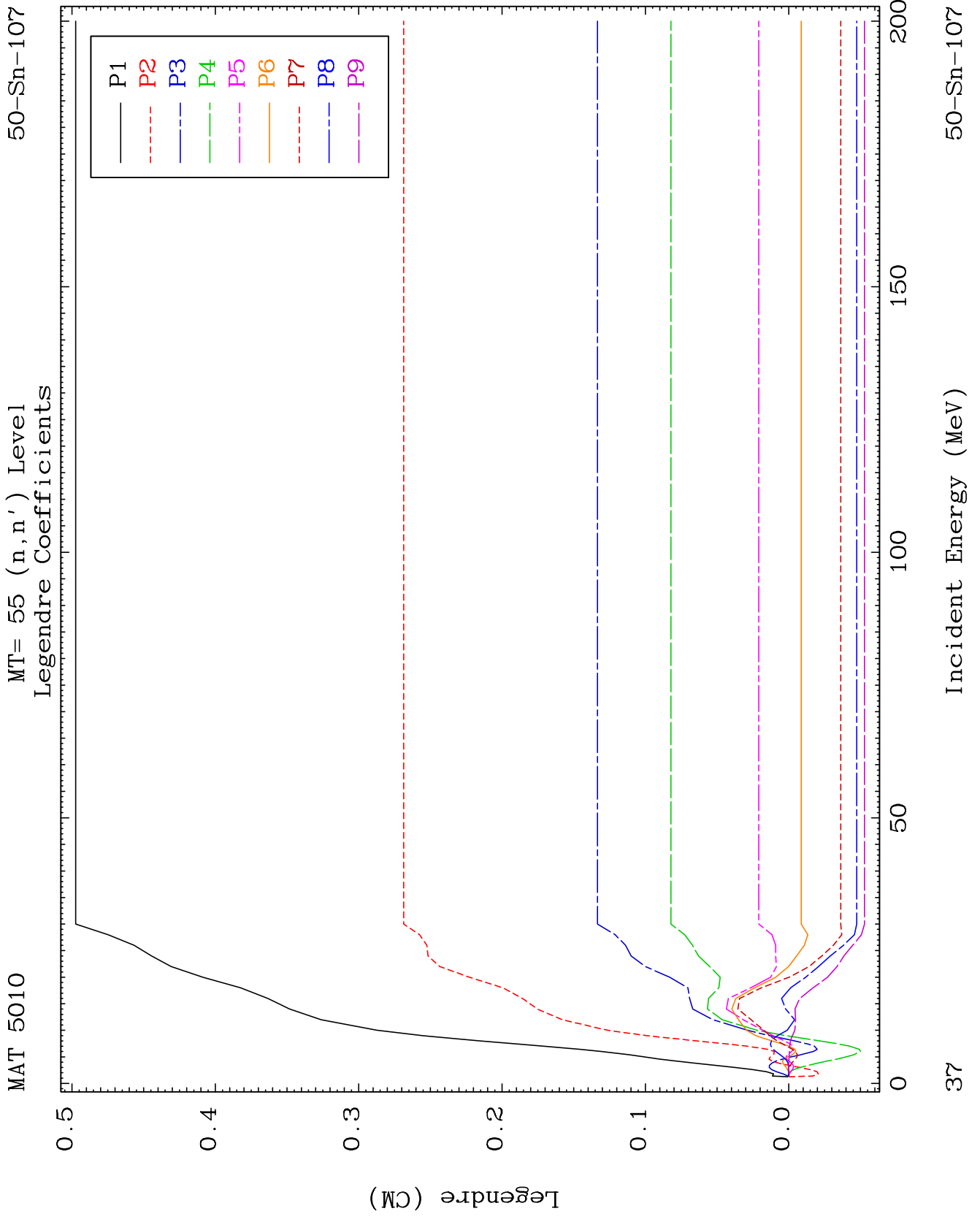


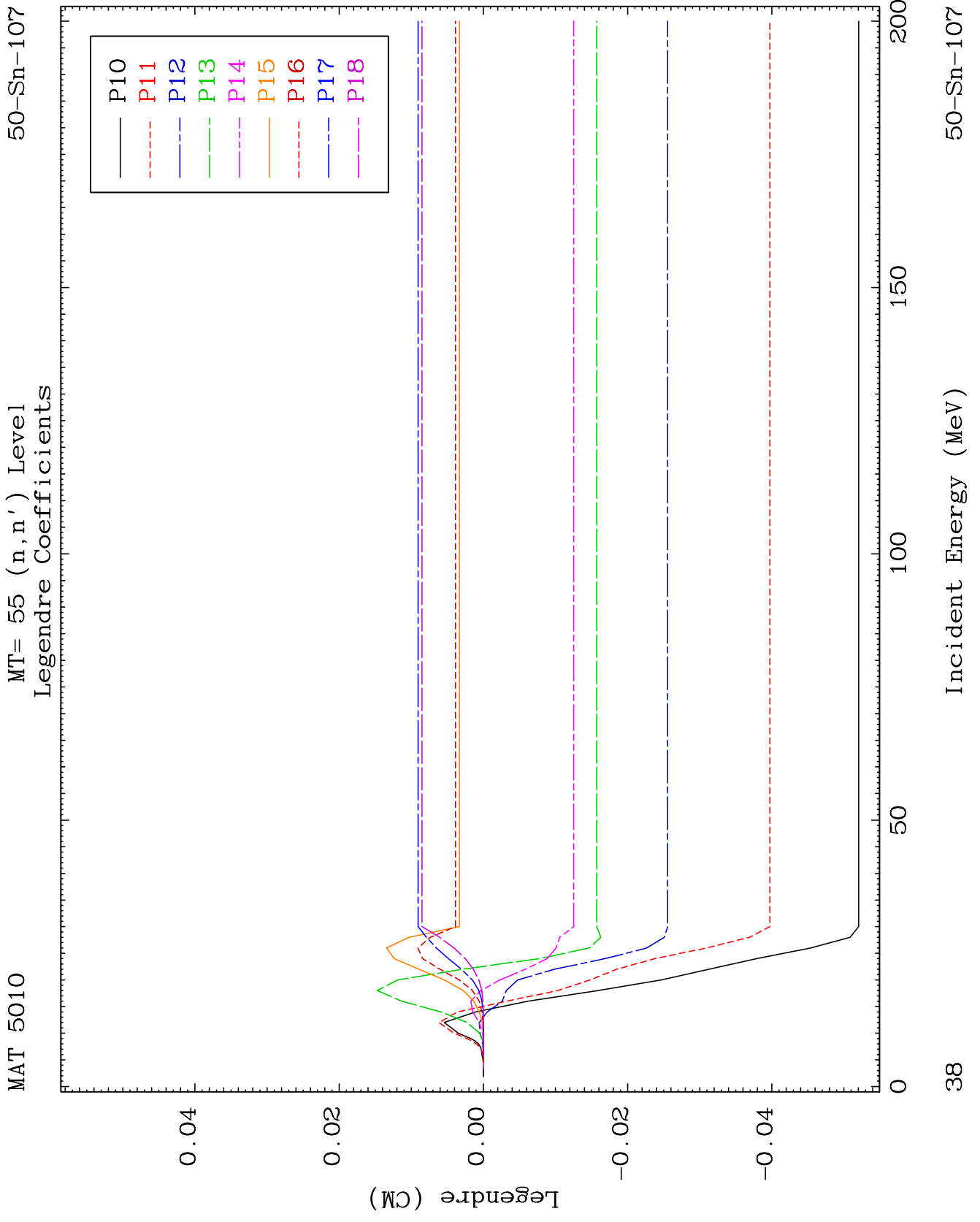


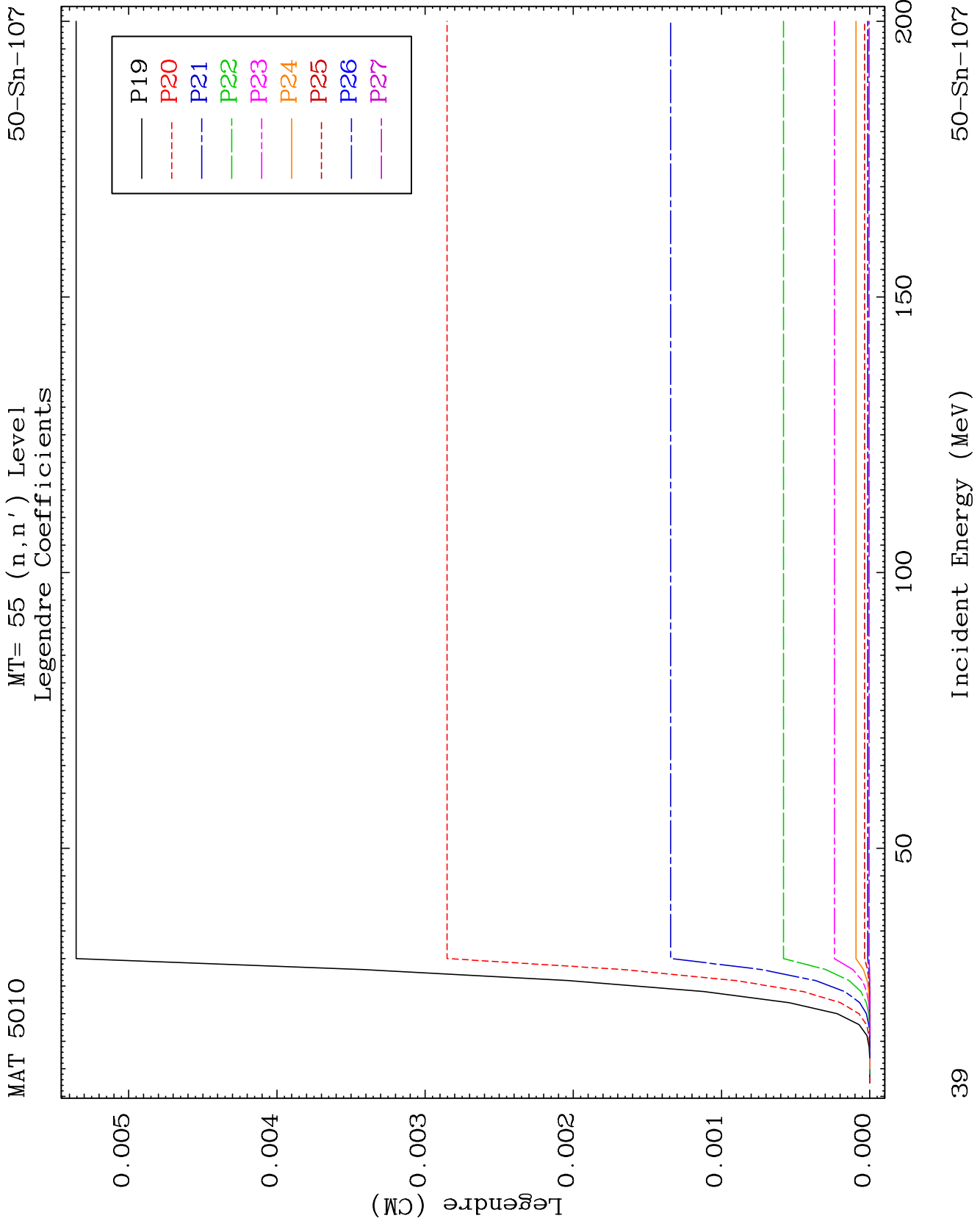




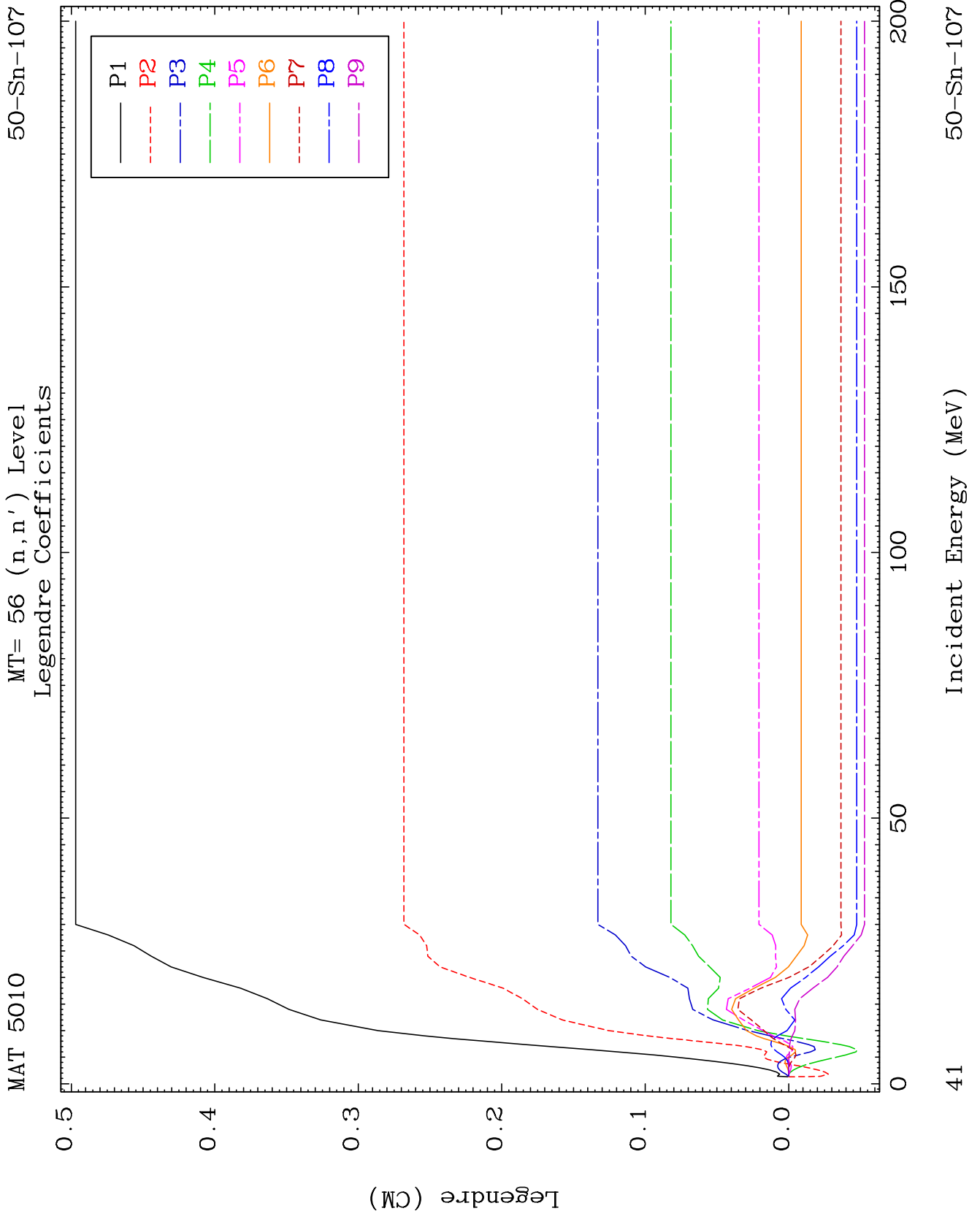


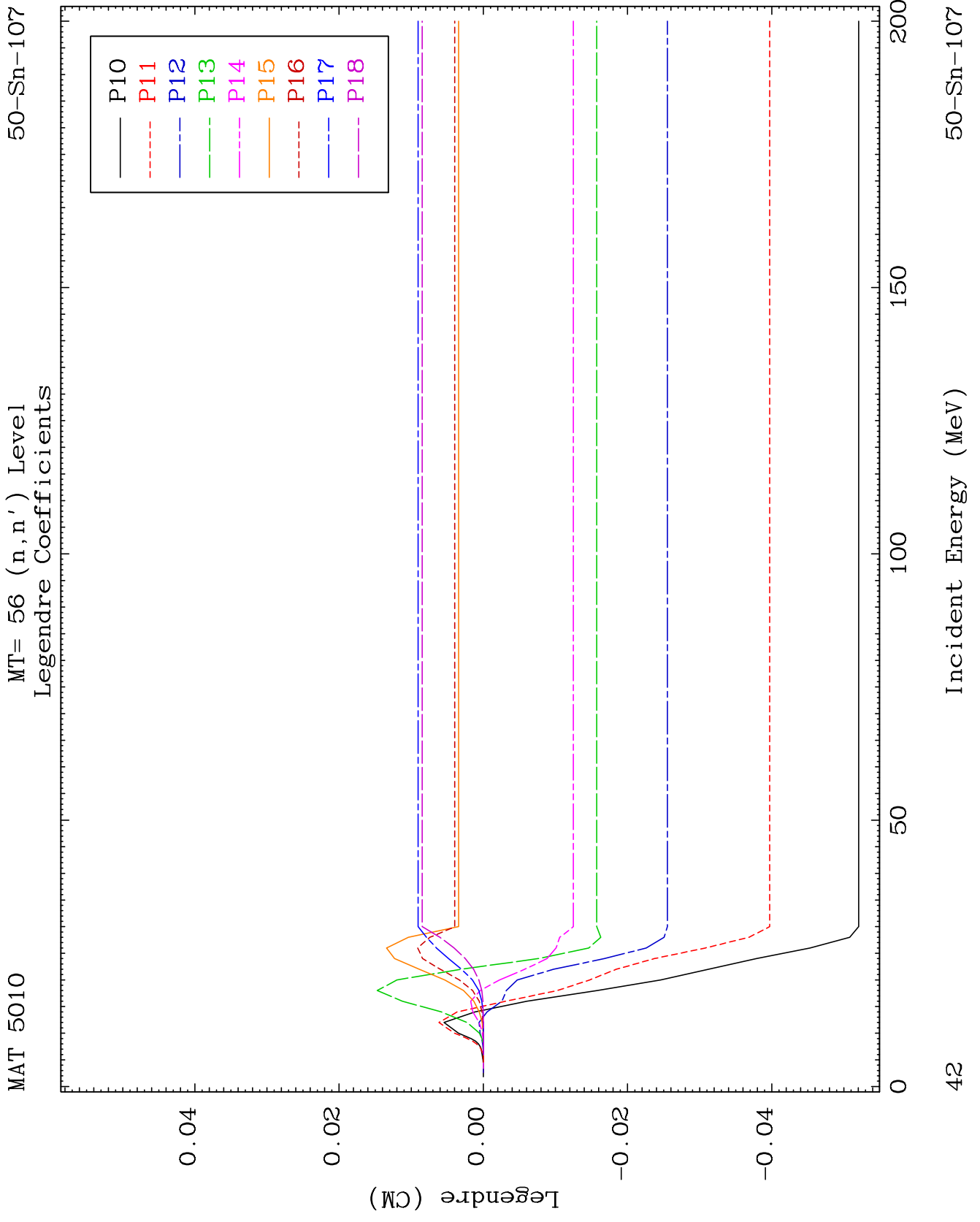


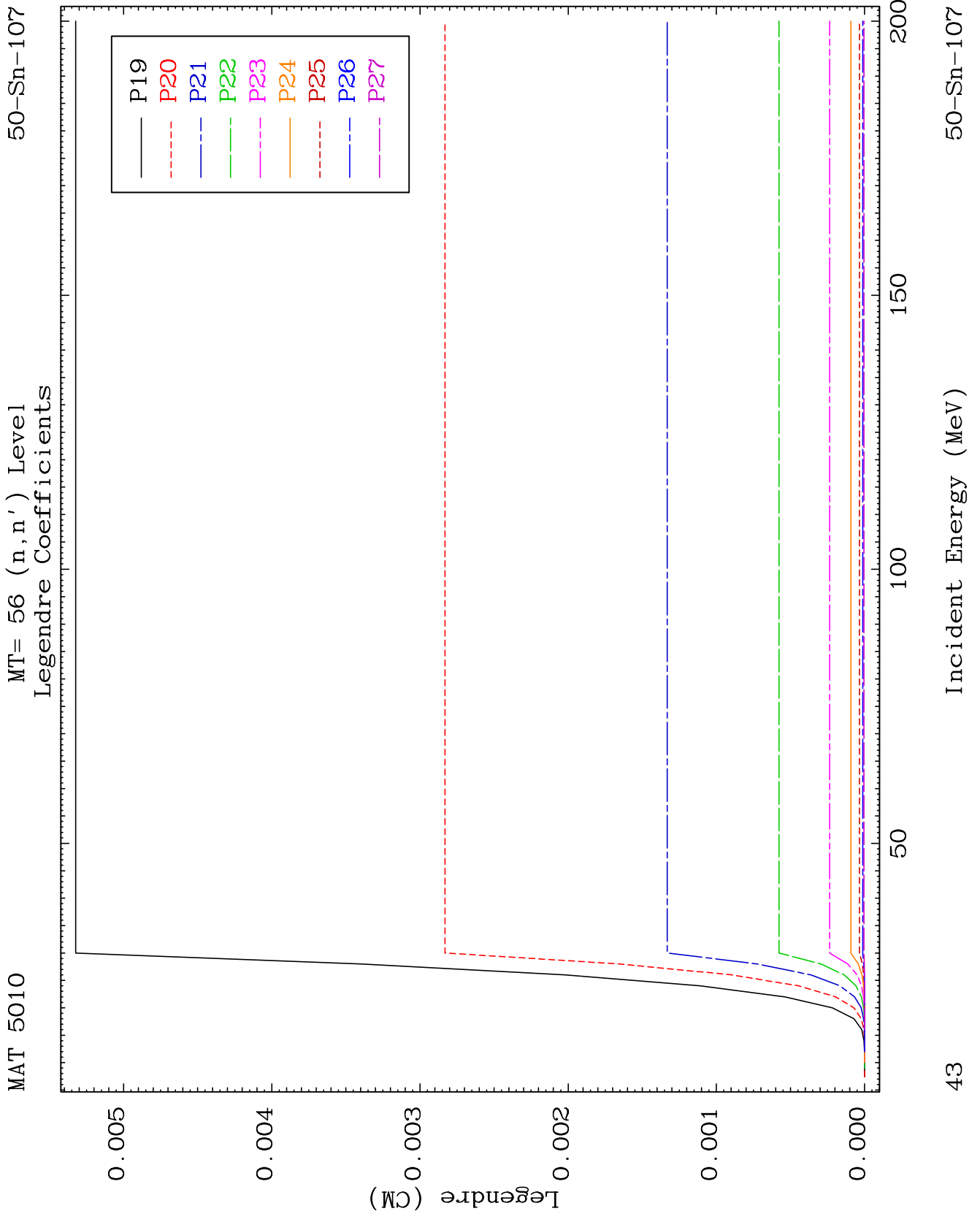


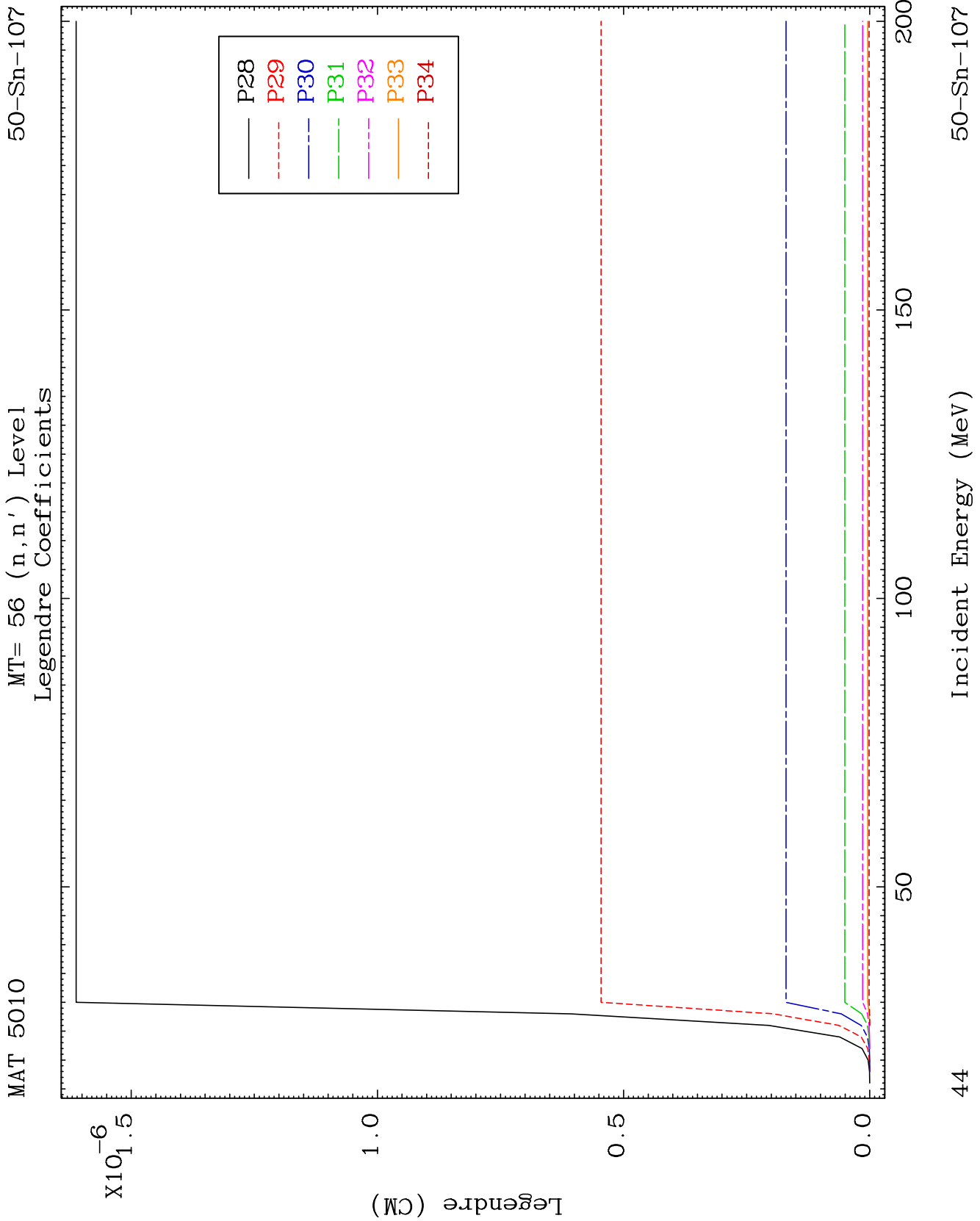




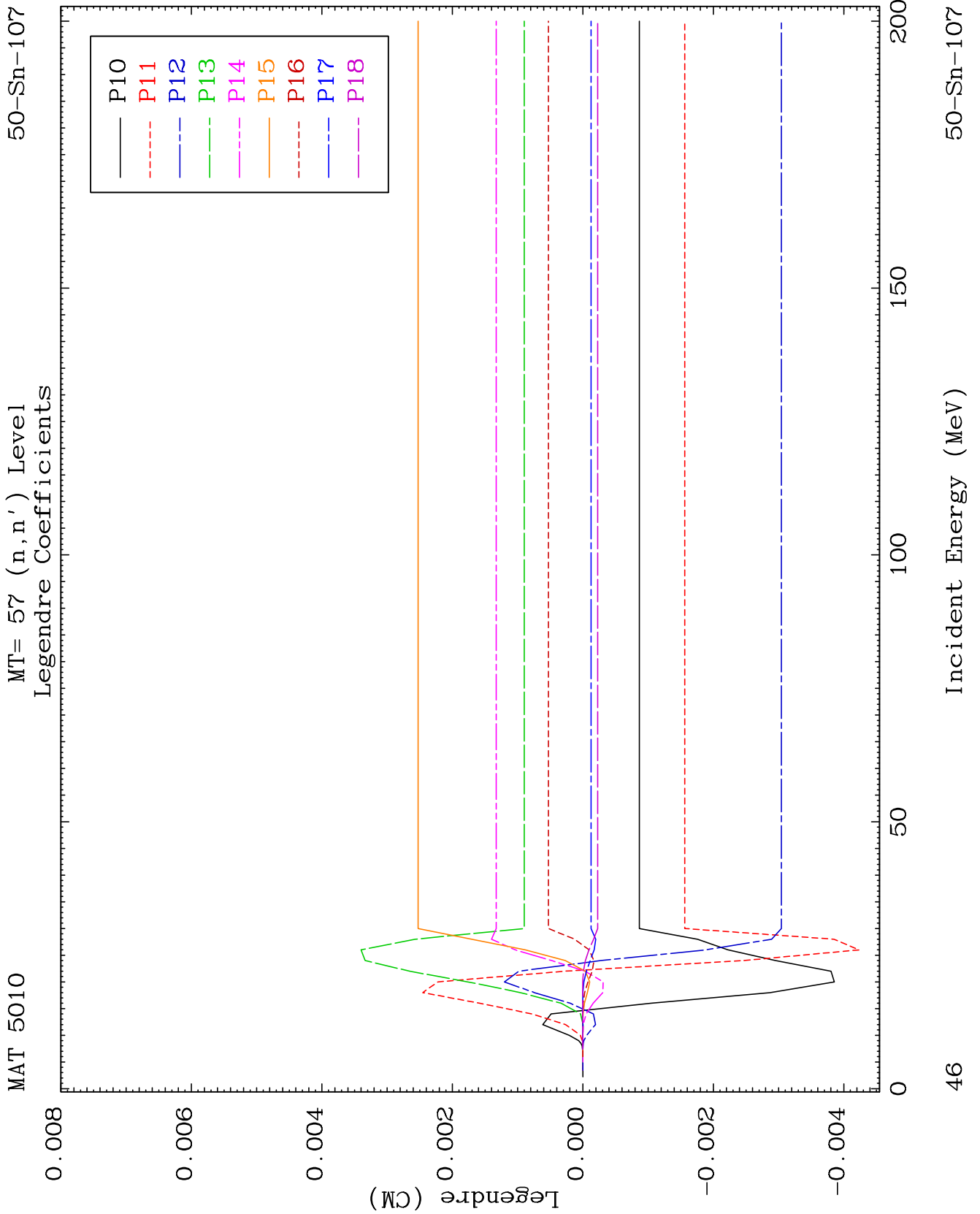


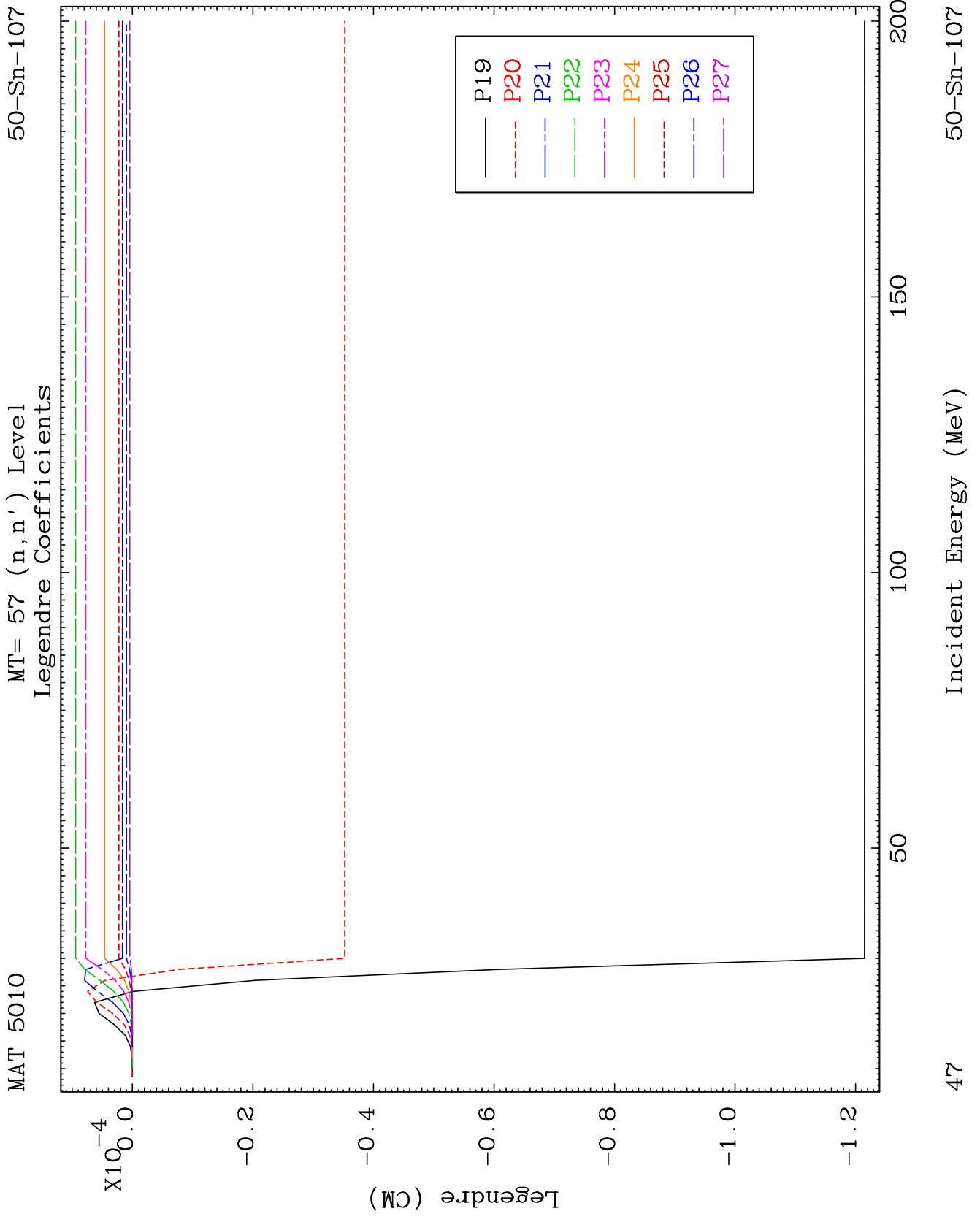


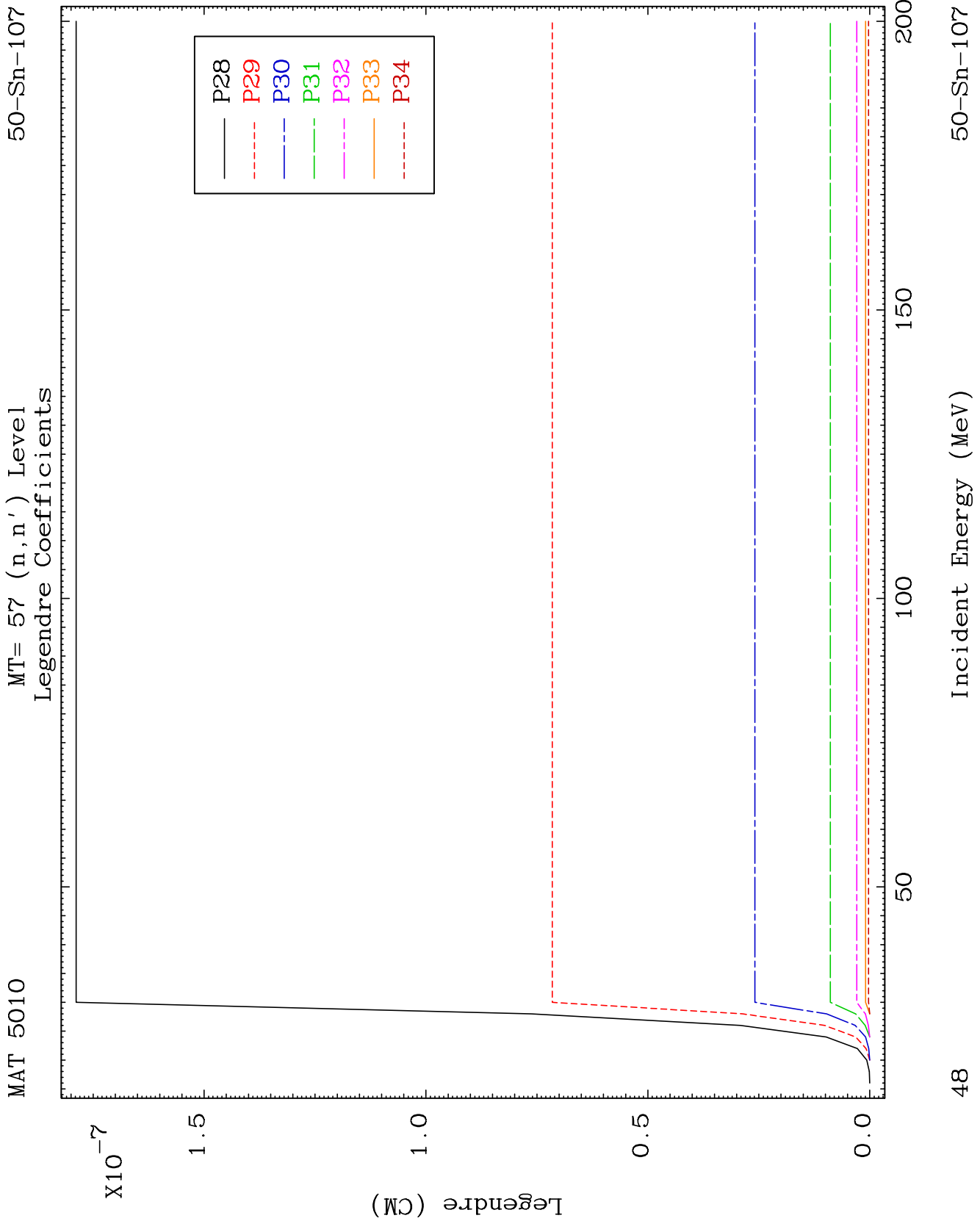






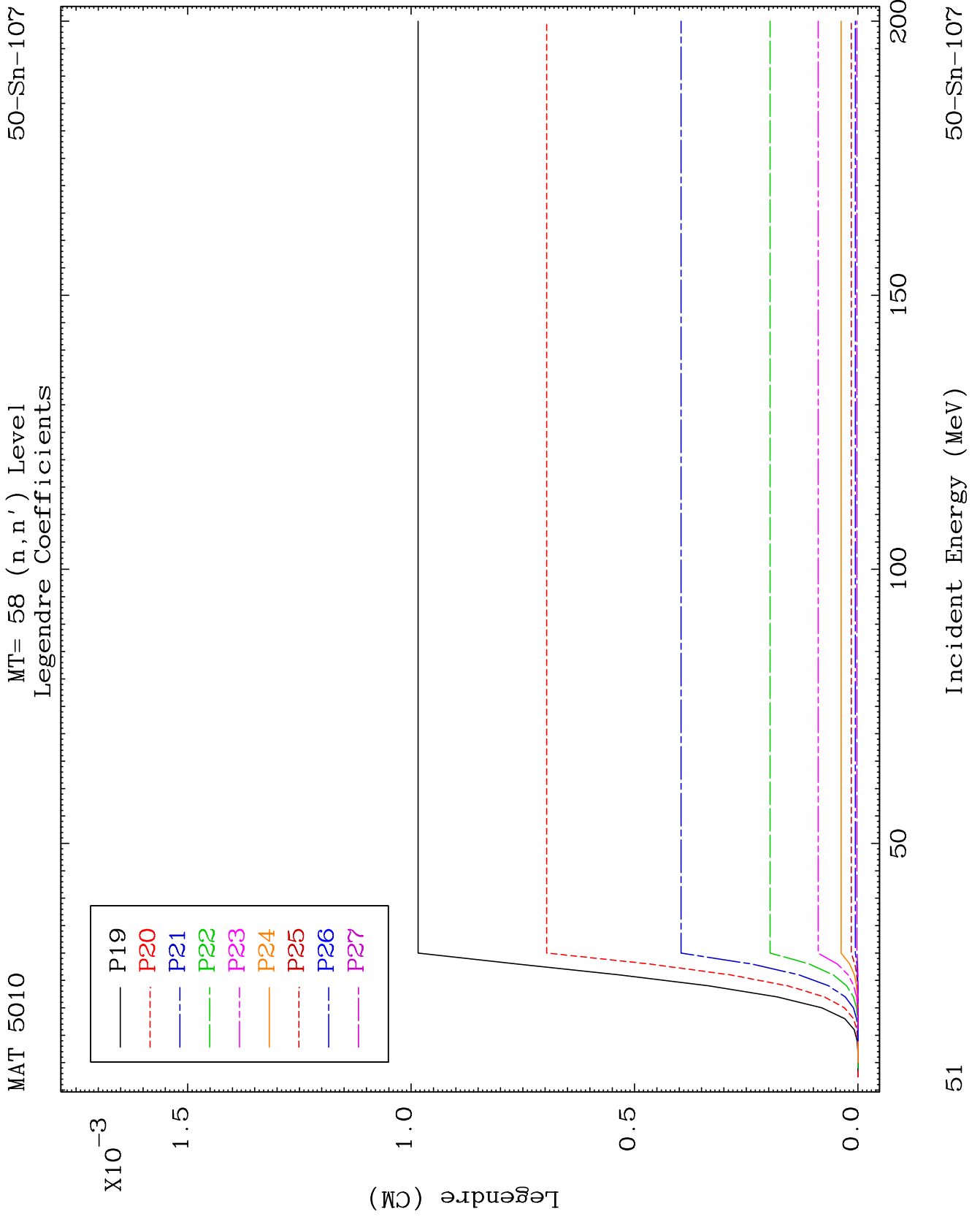


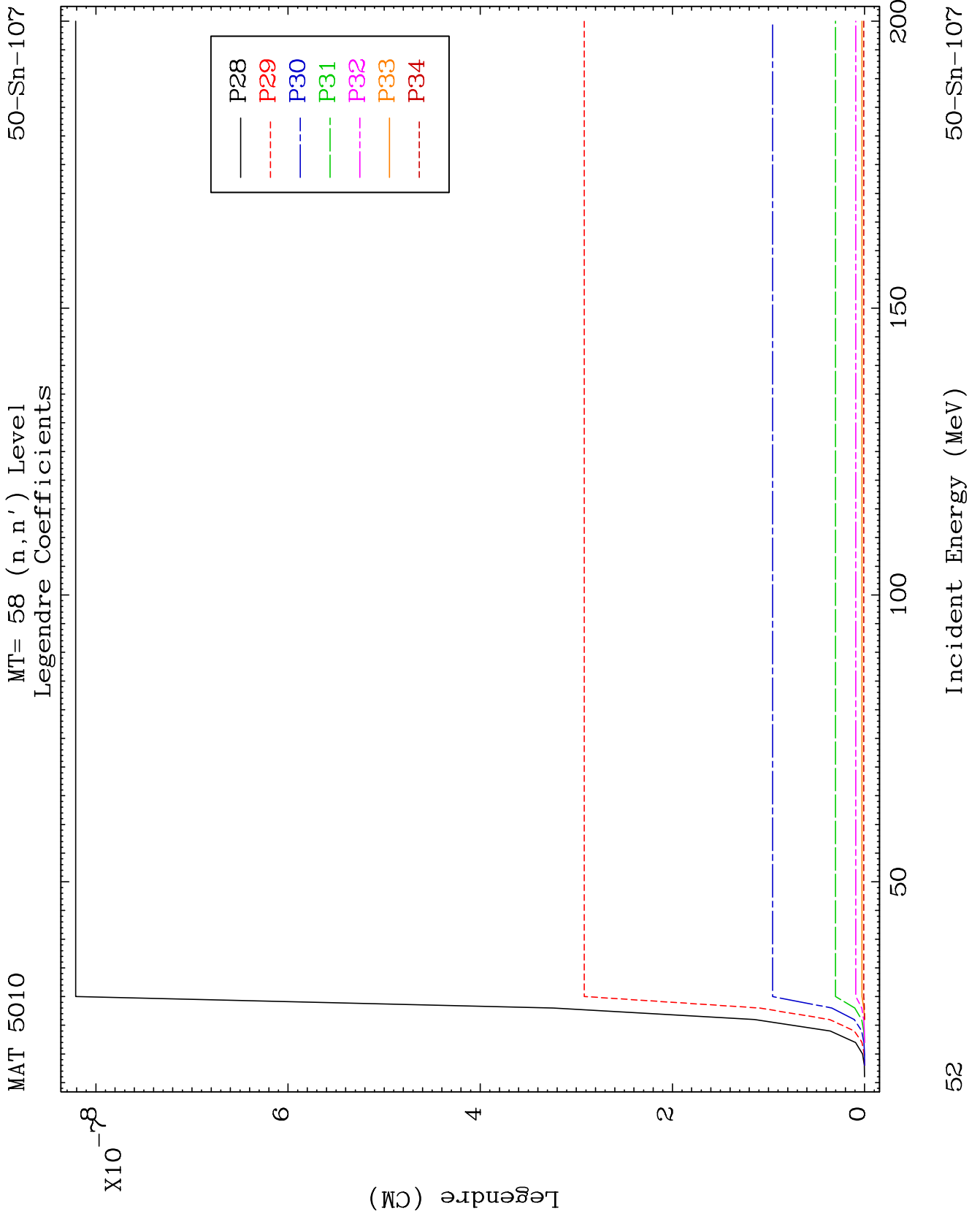


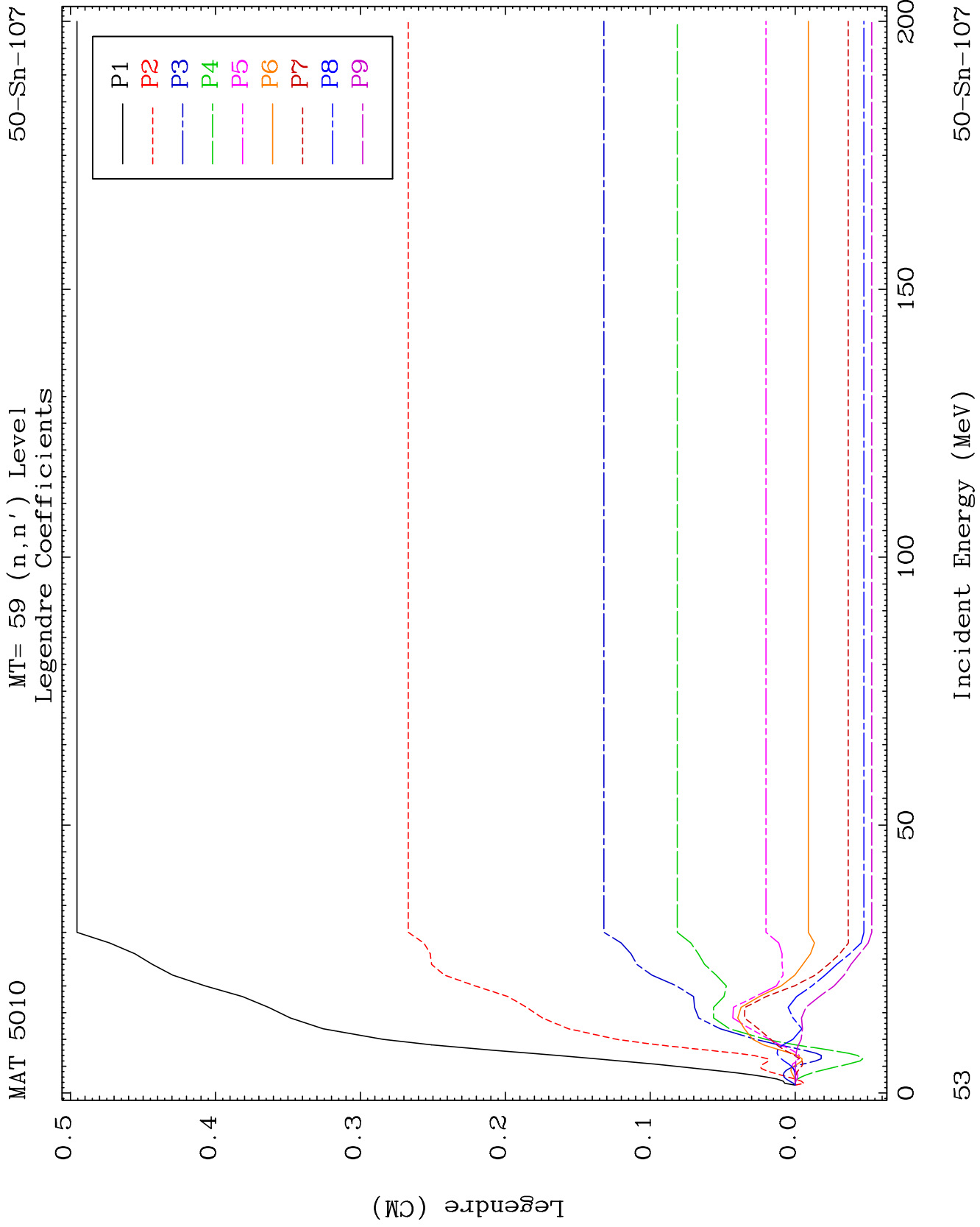


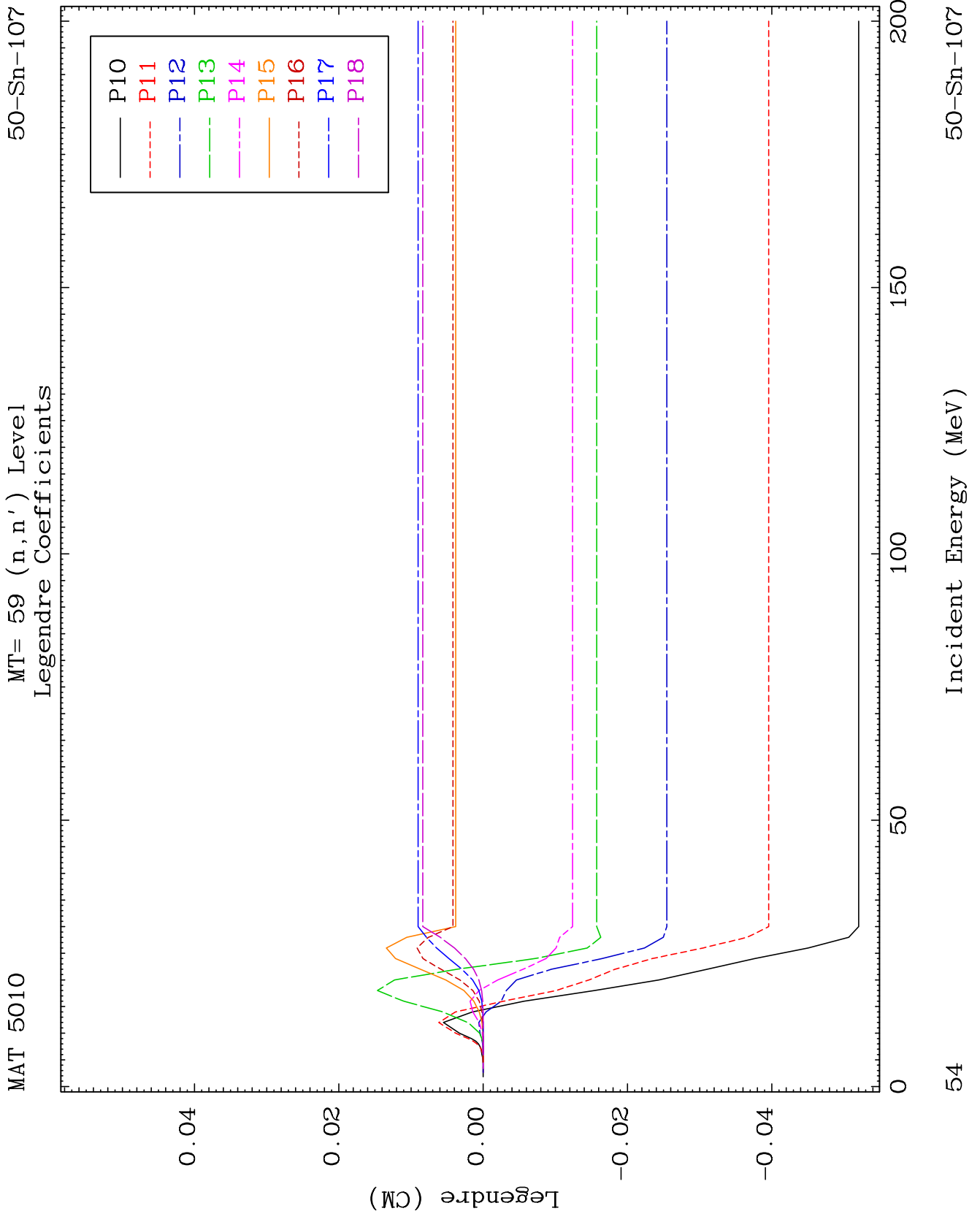




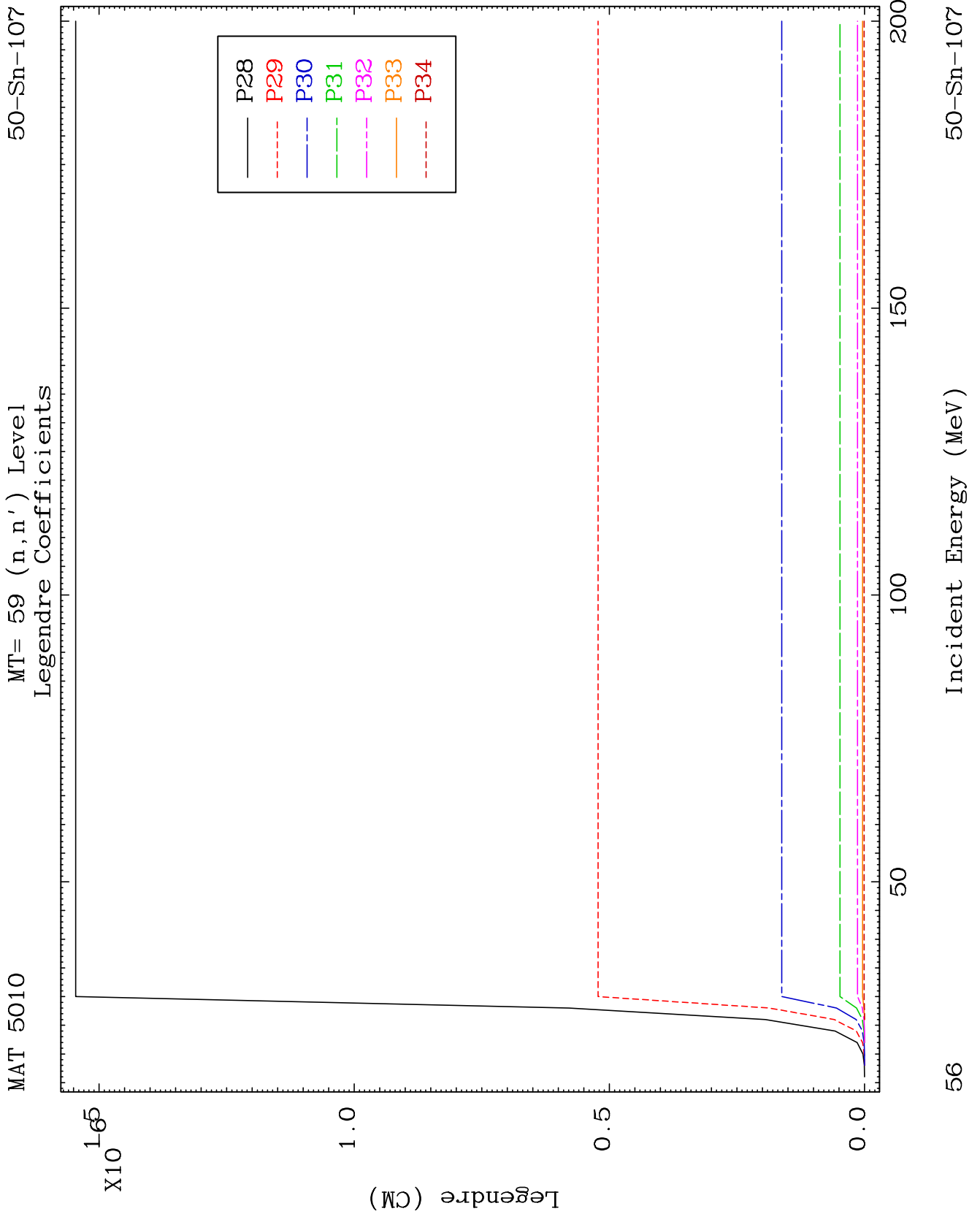


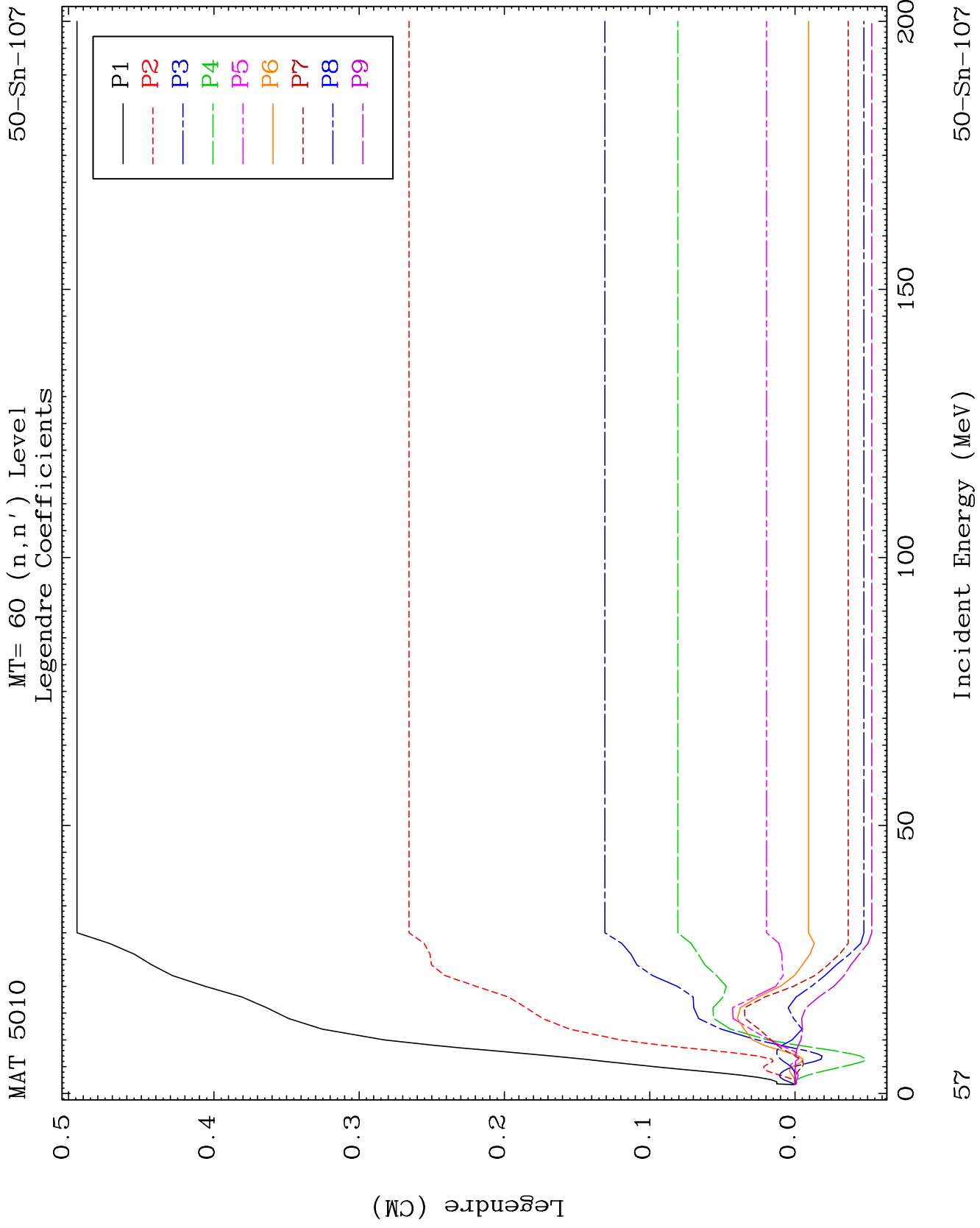


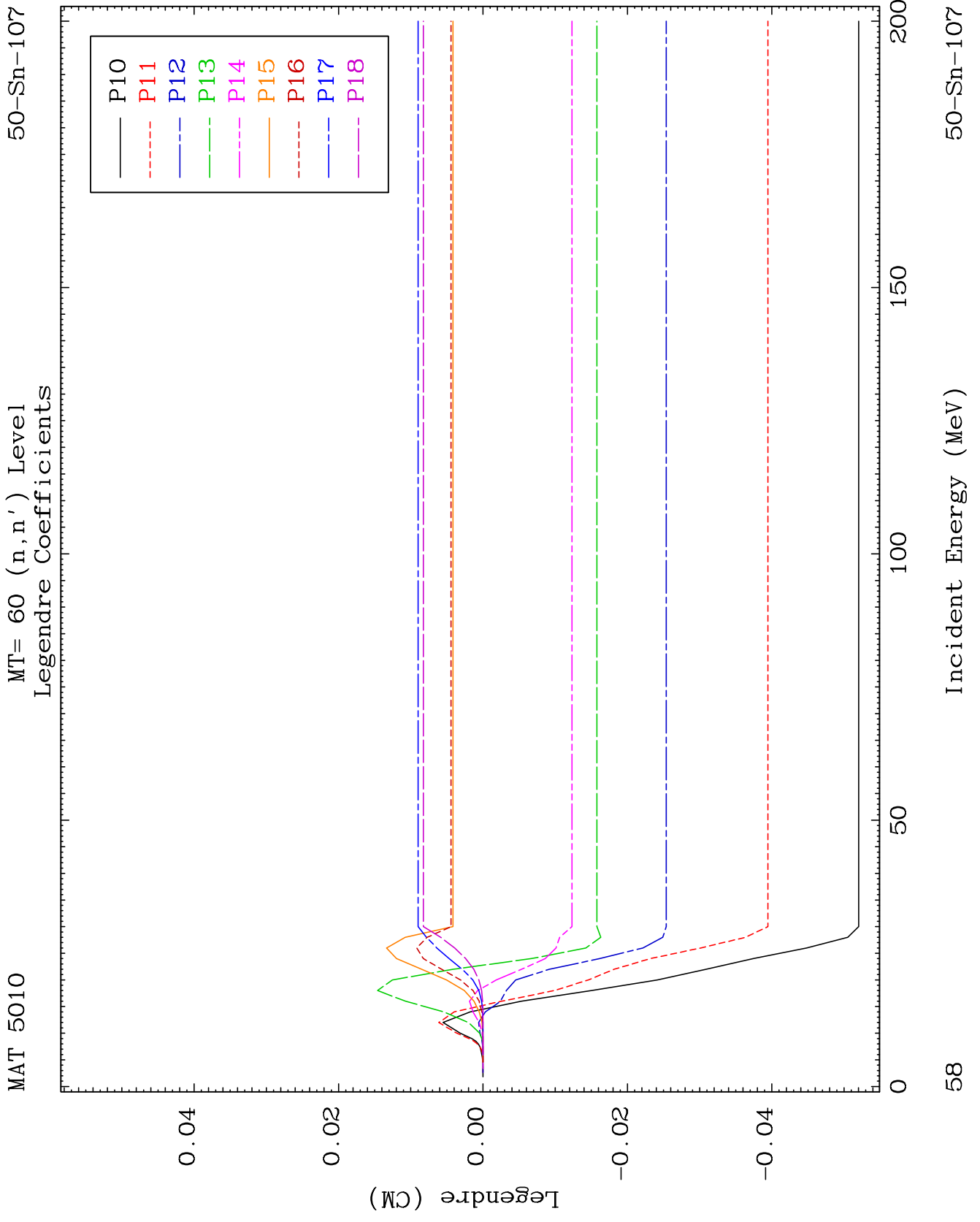


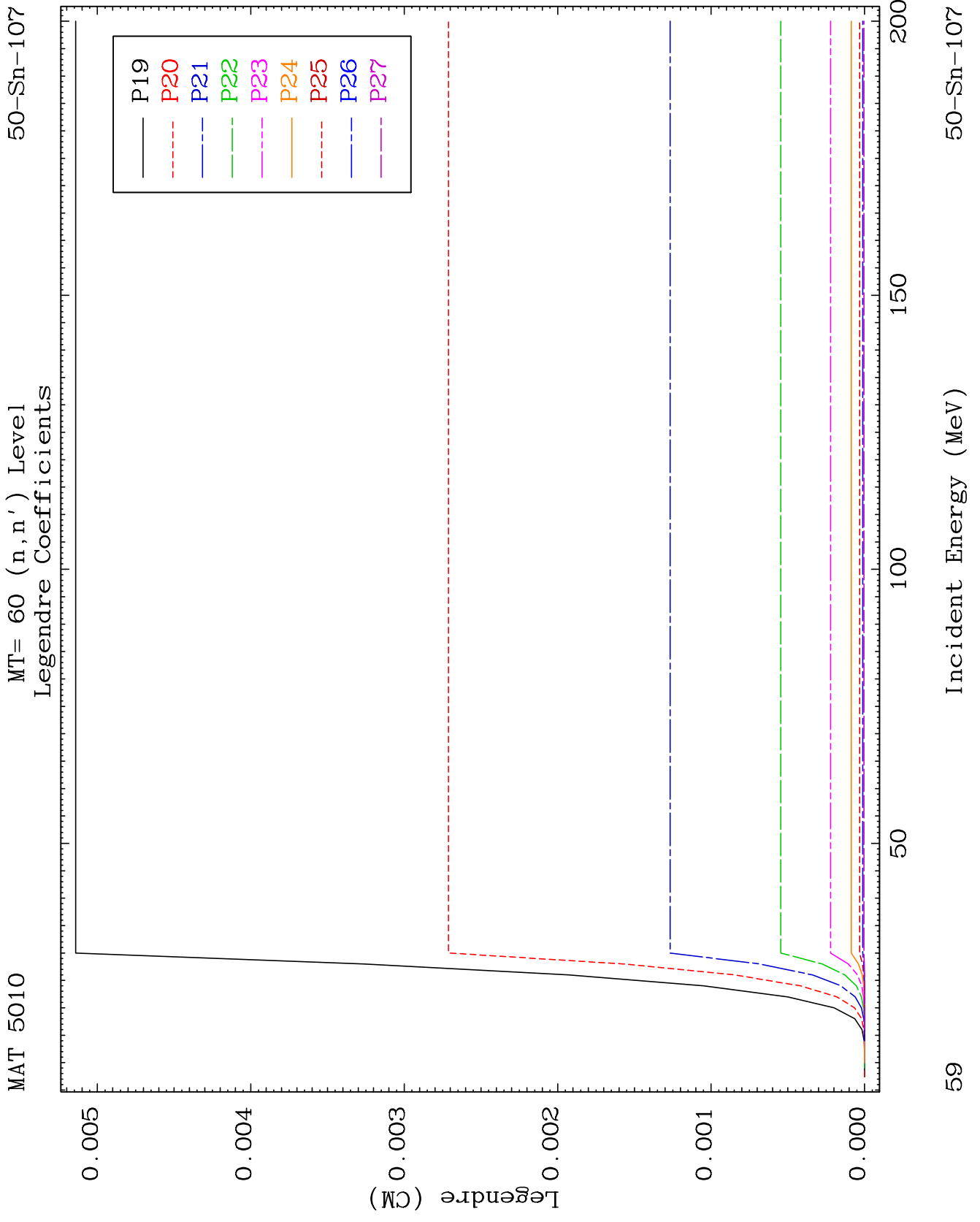




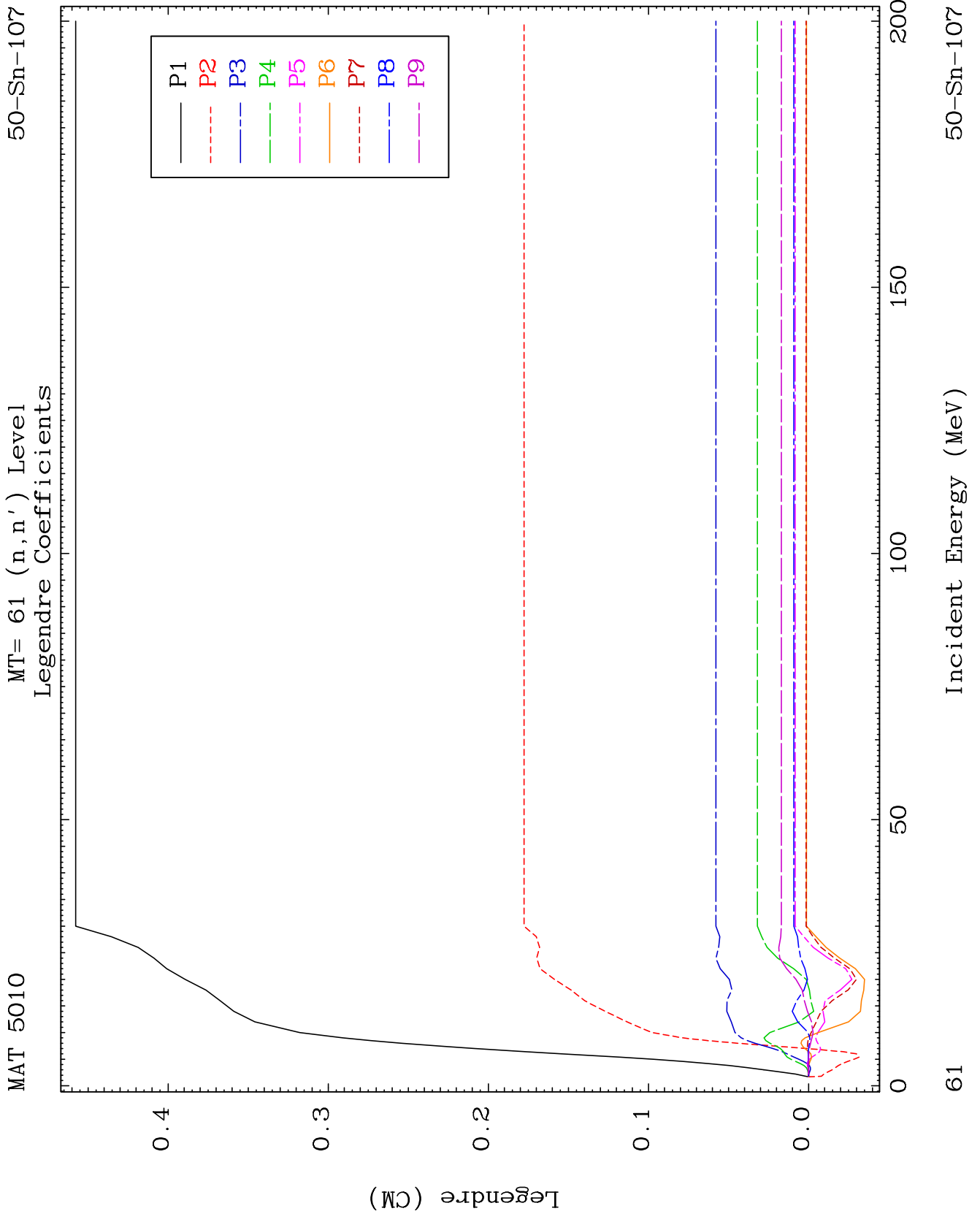


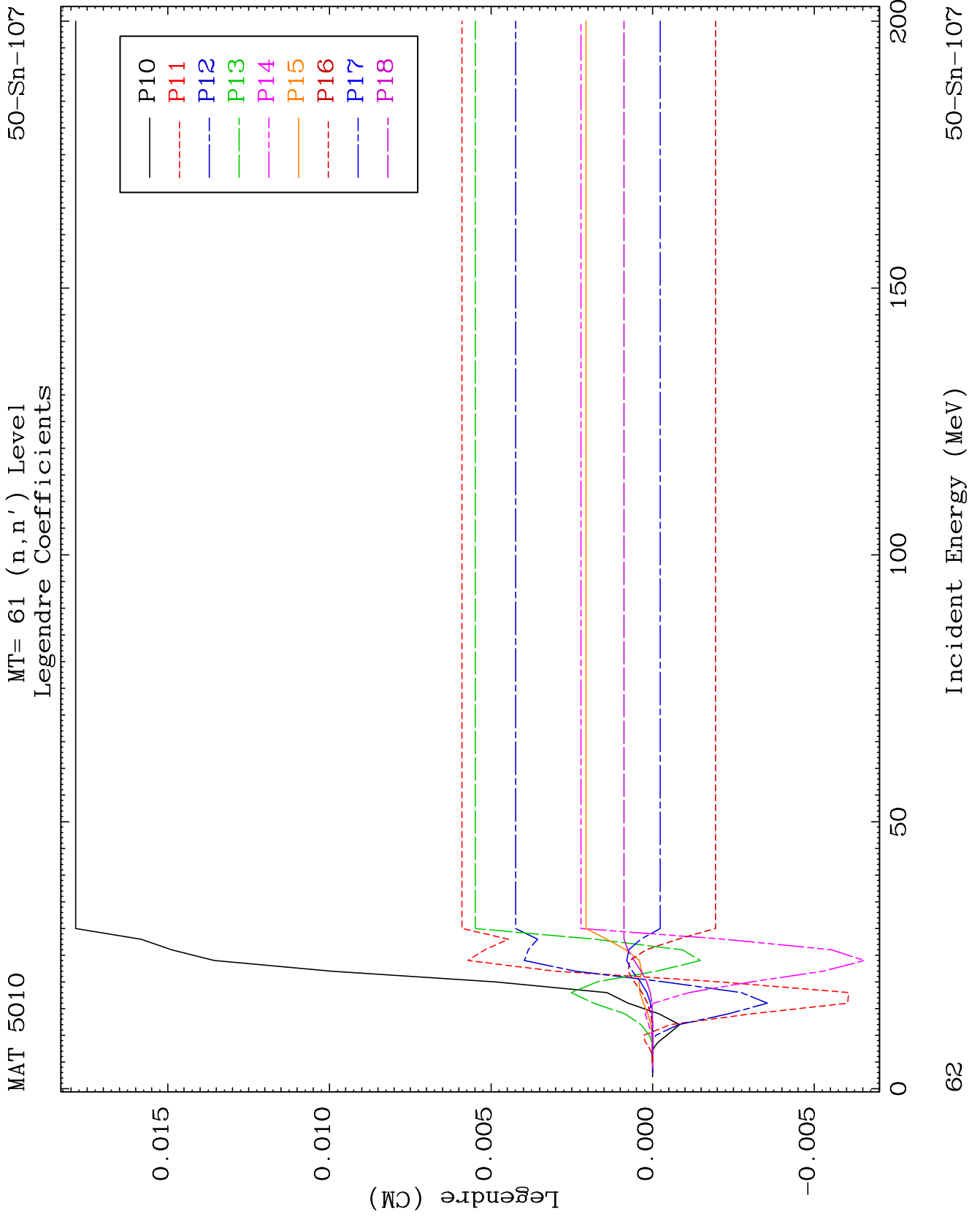


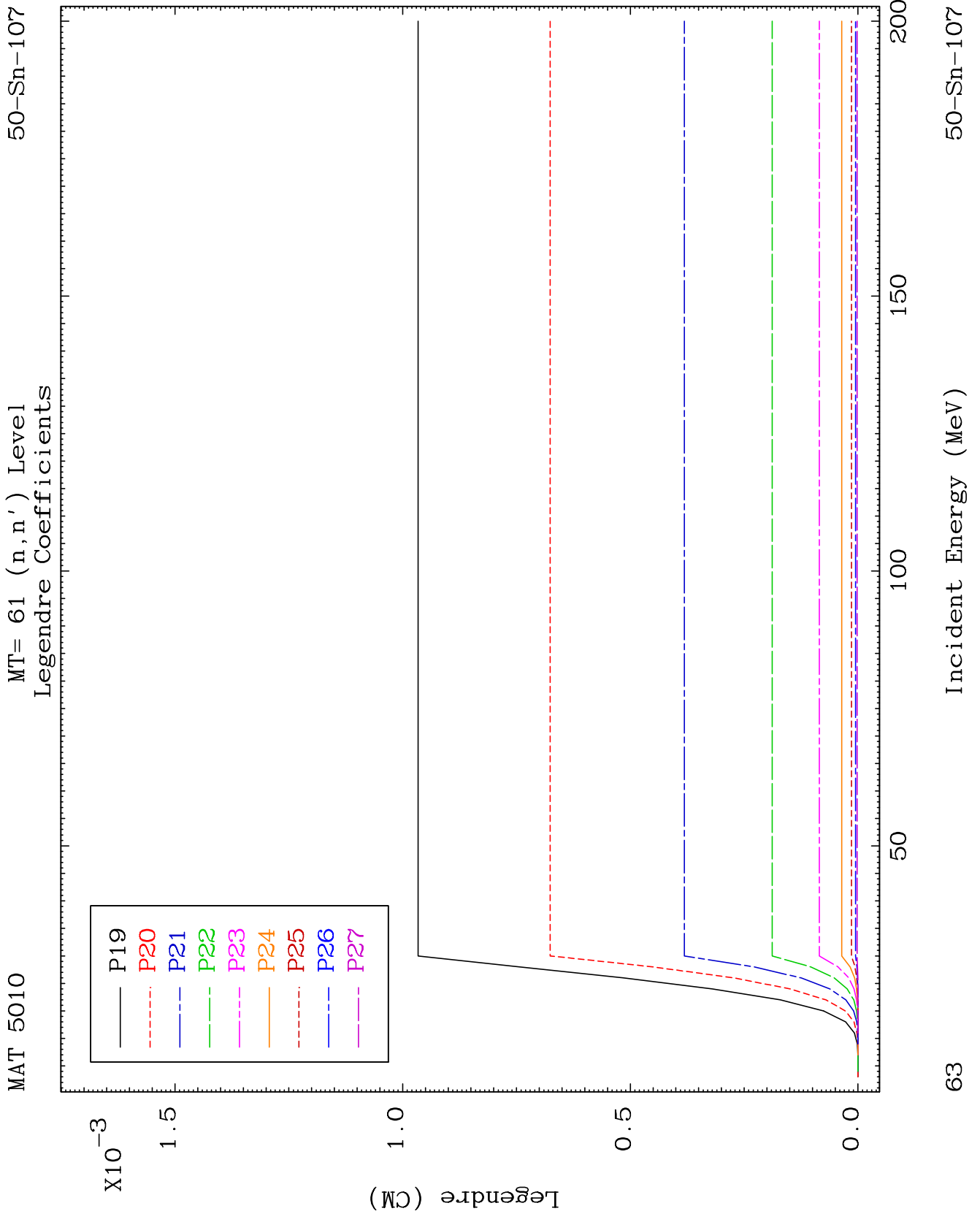


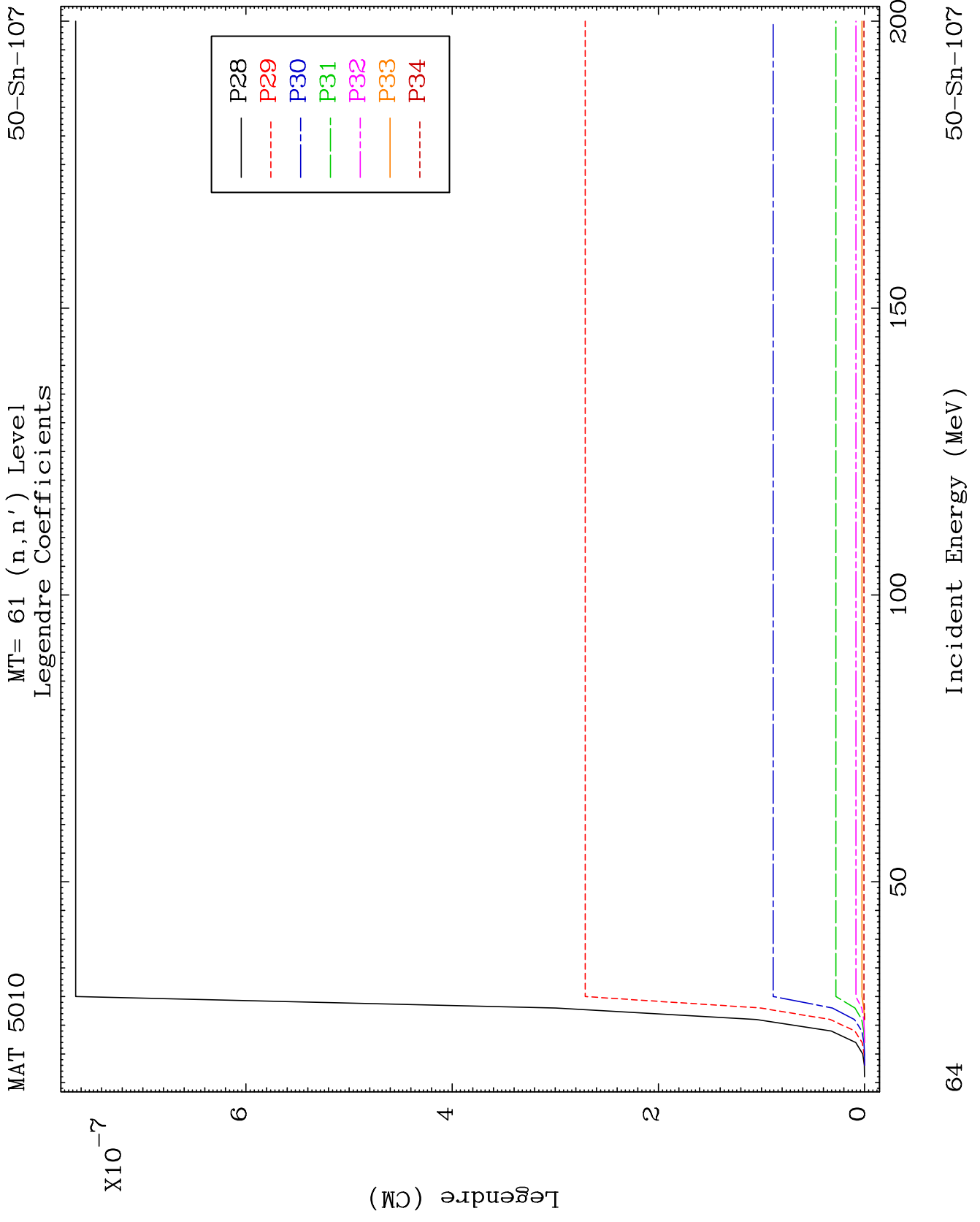










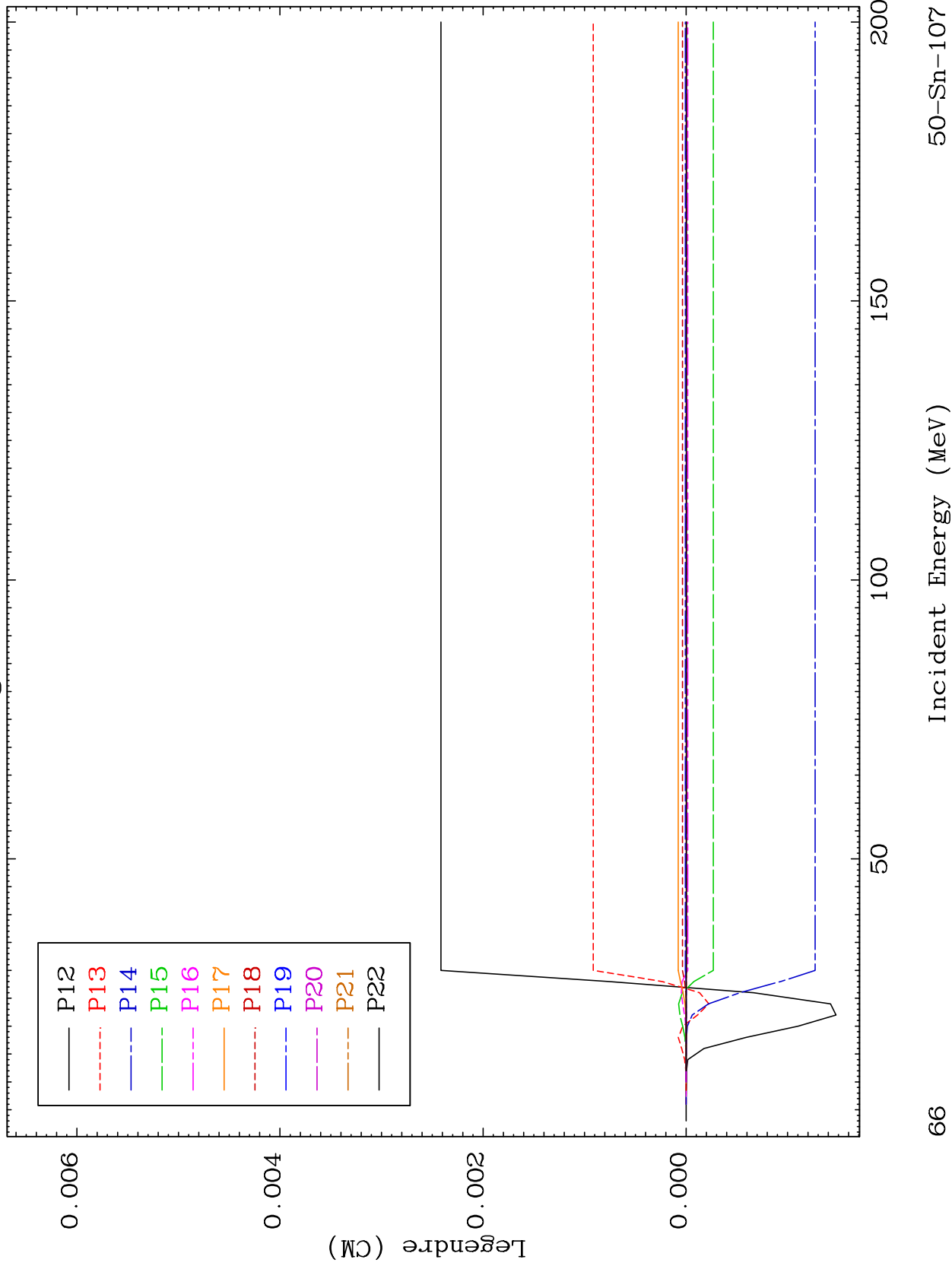


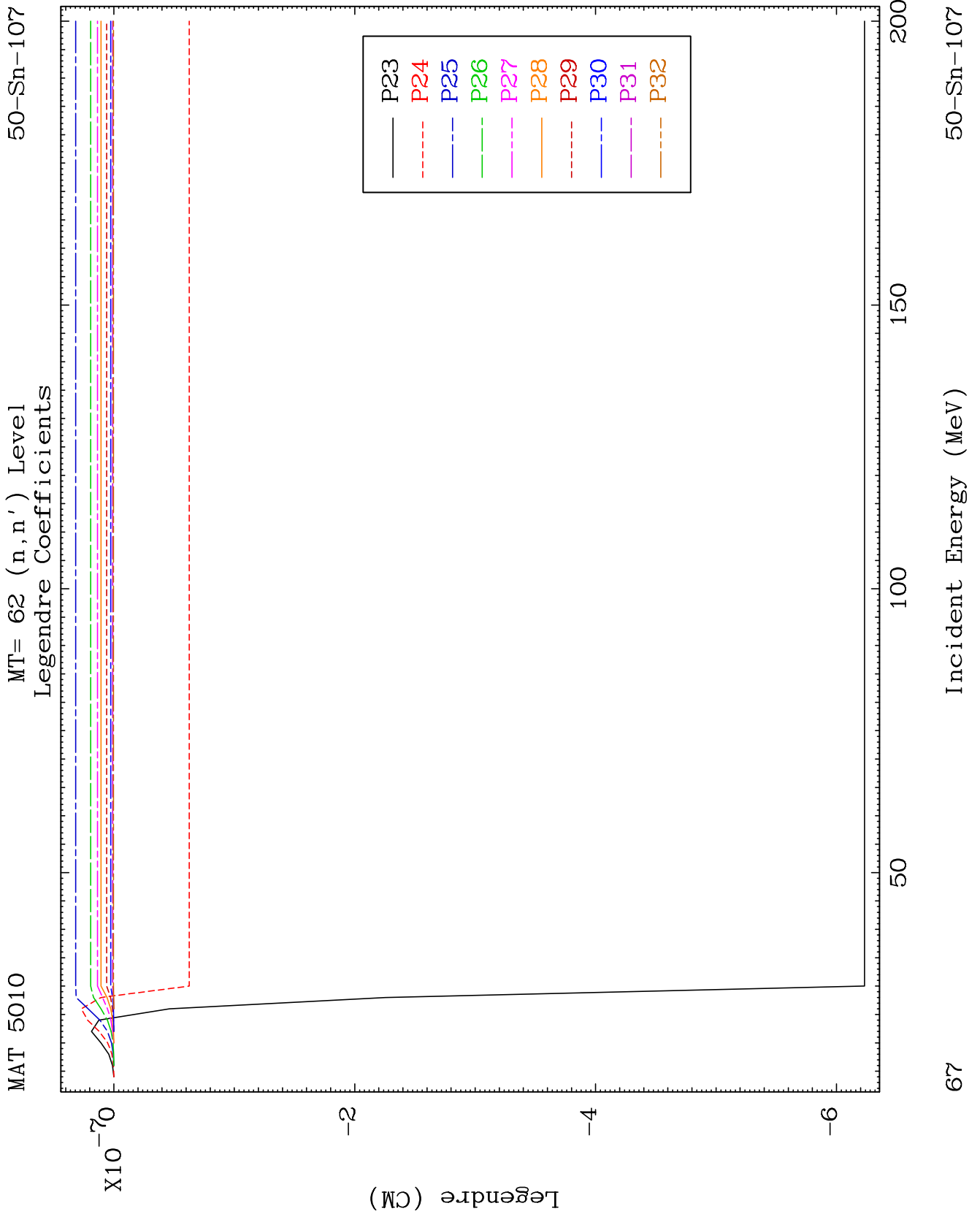


MAT 5010

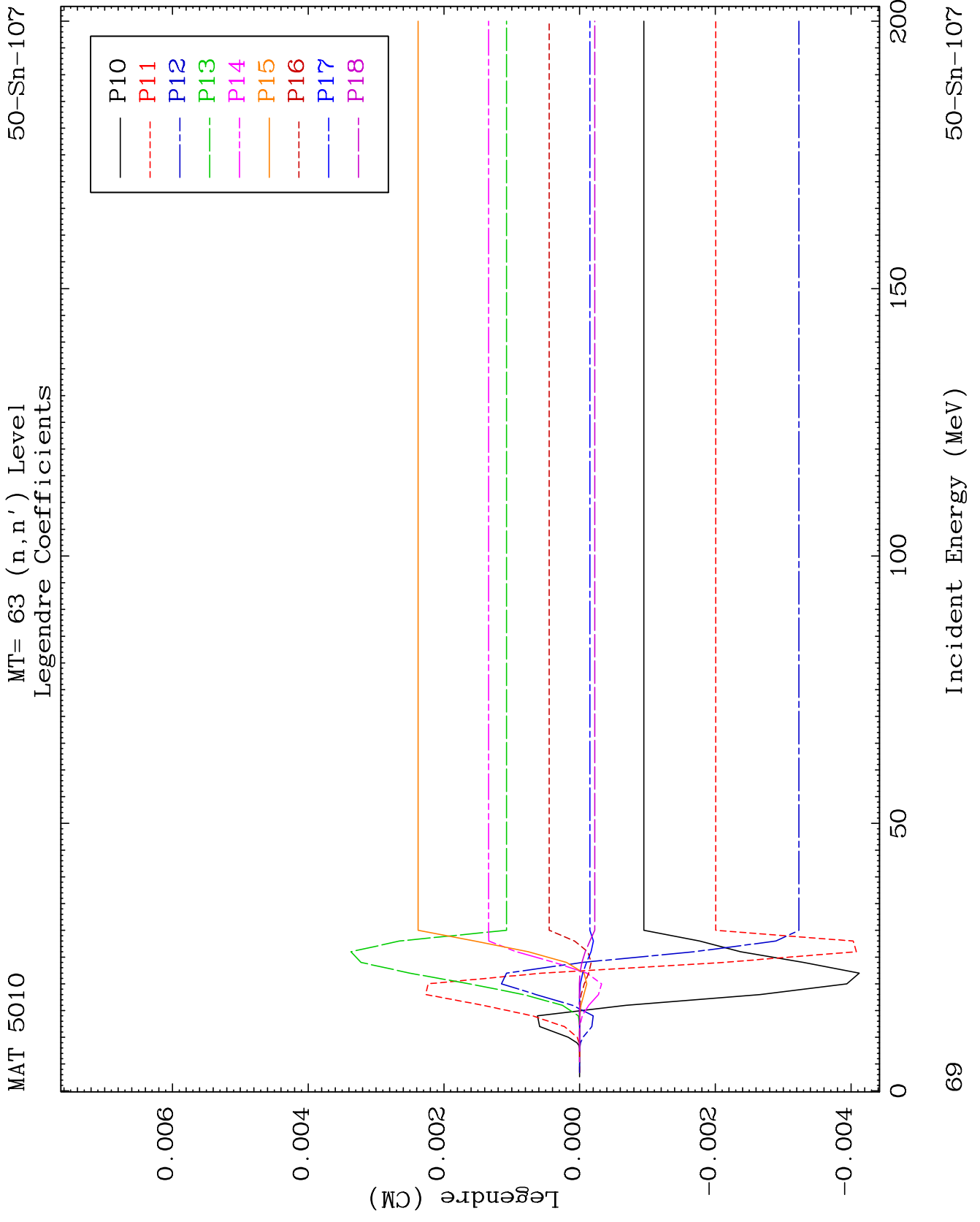
MT= 62 (n,n') Level  
Legendre Coefficients

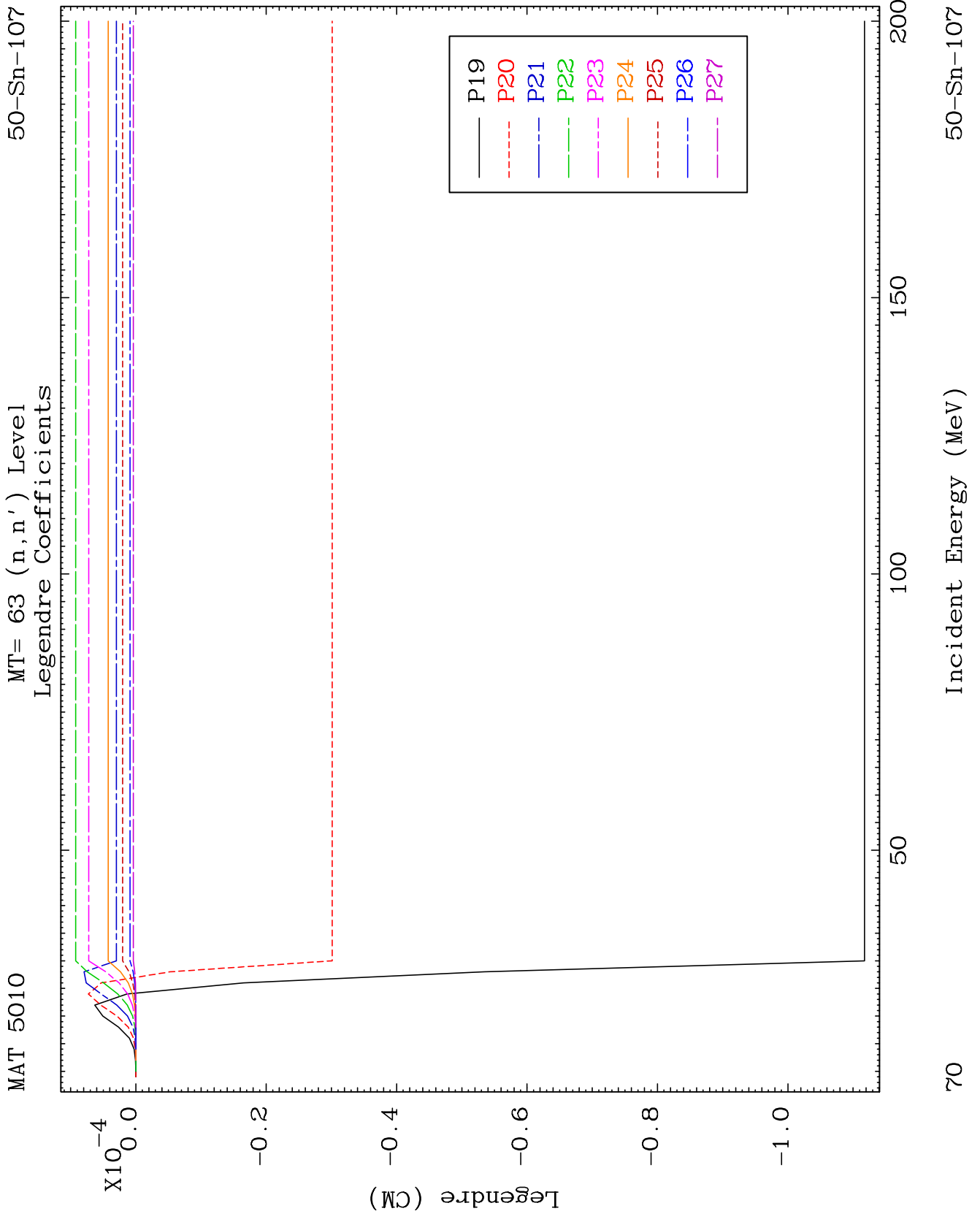
50-Sn-107

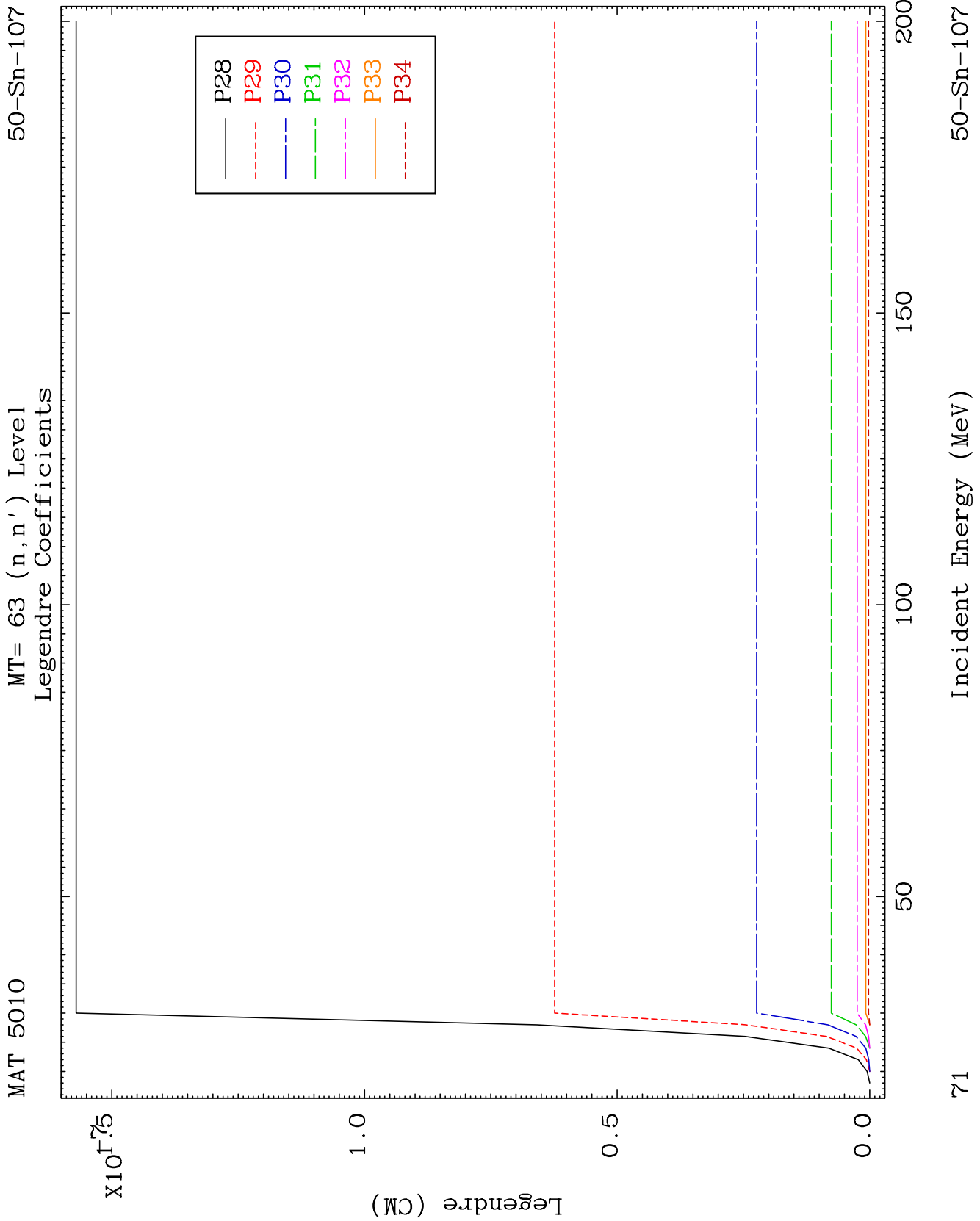


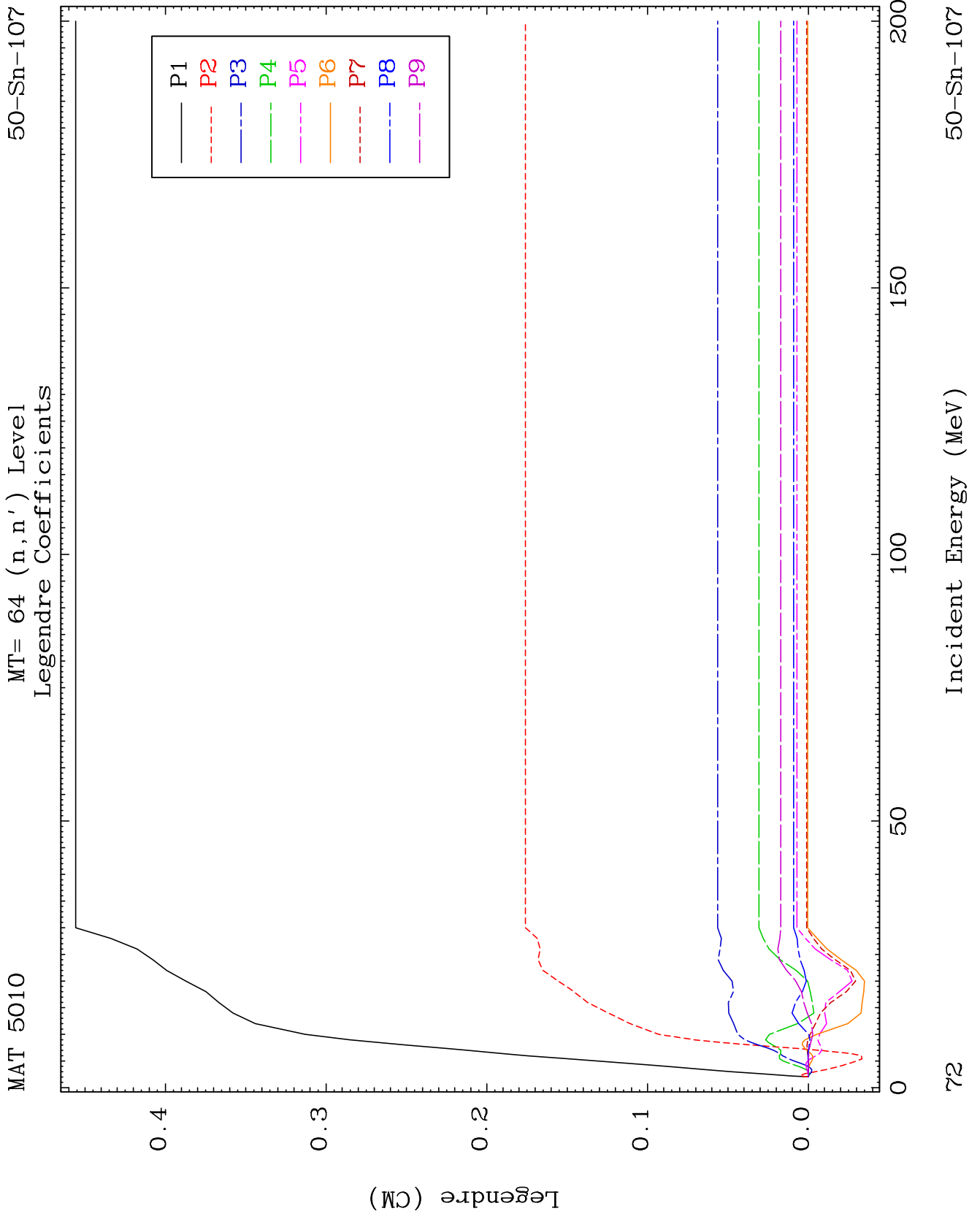




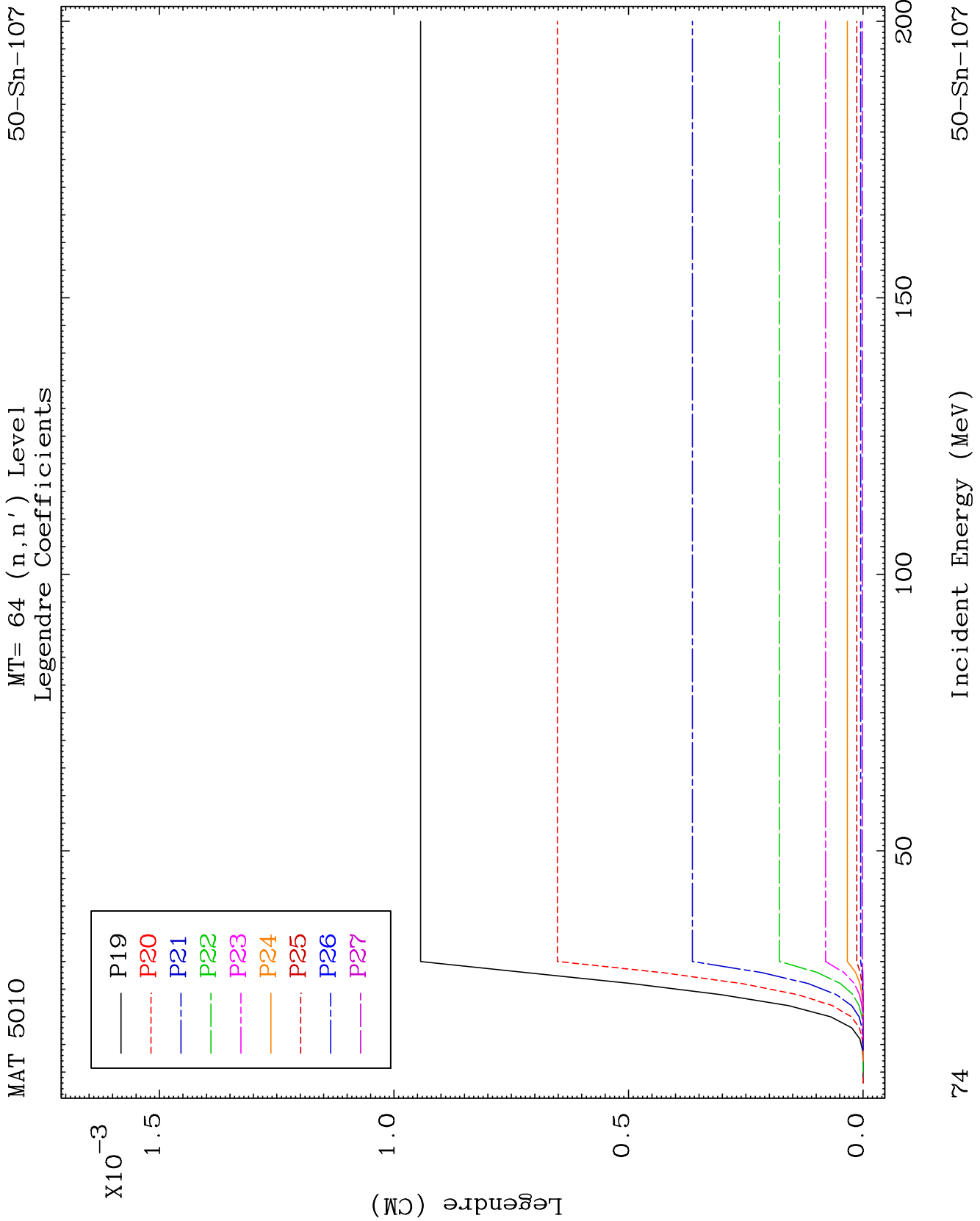


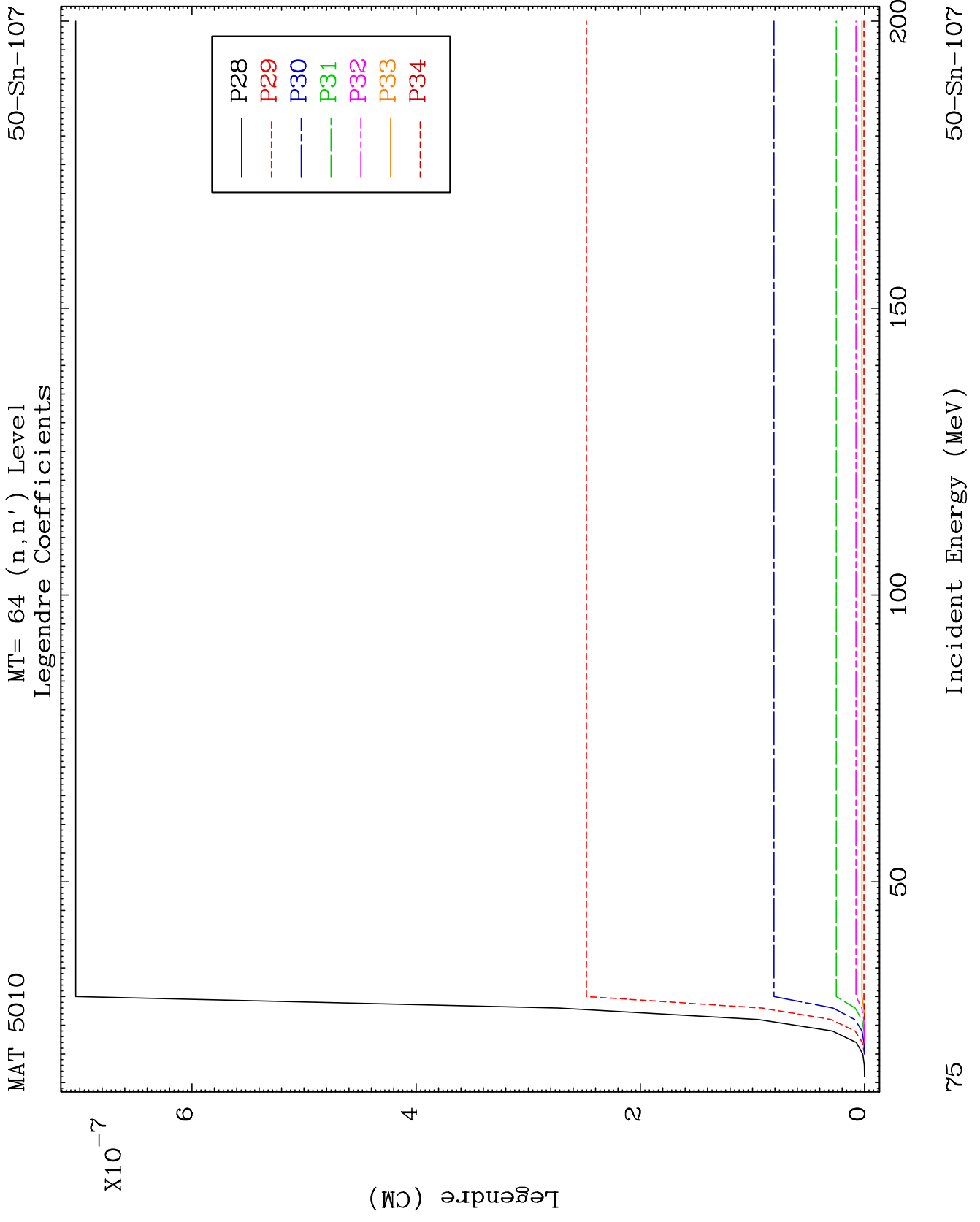






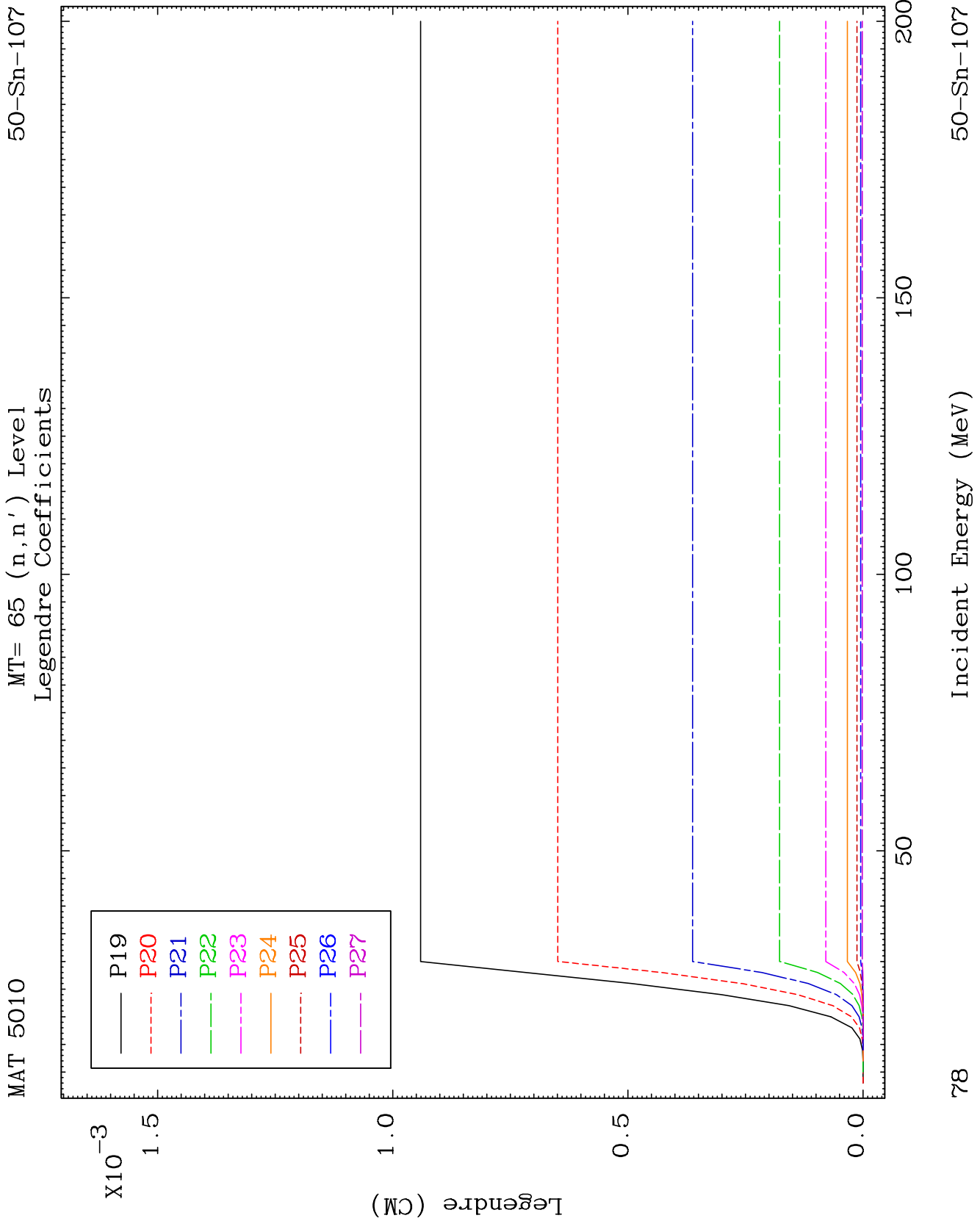


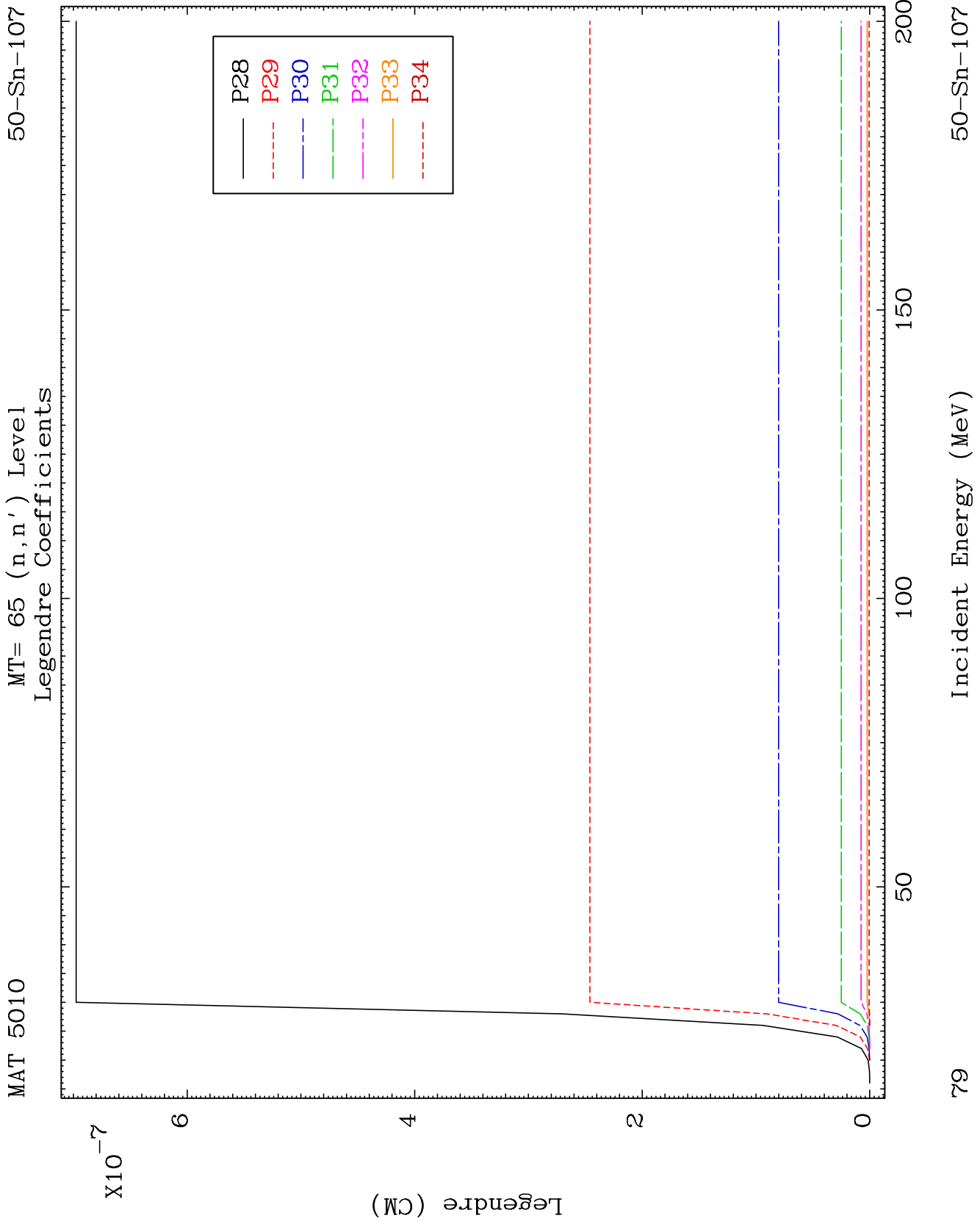


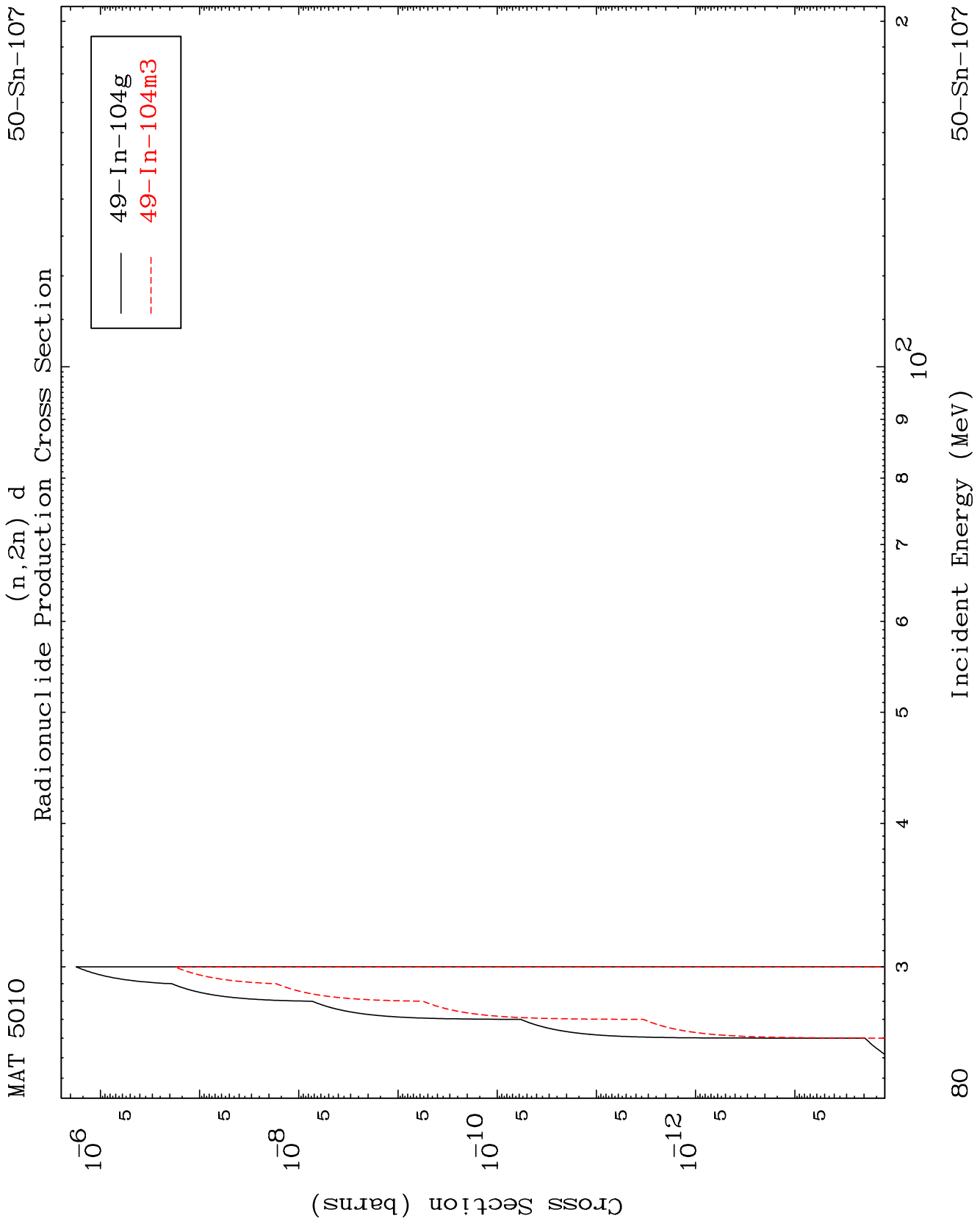










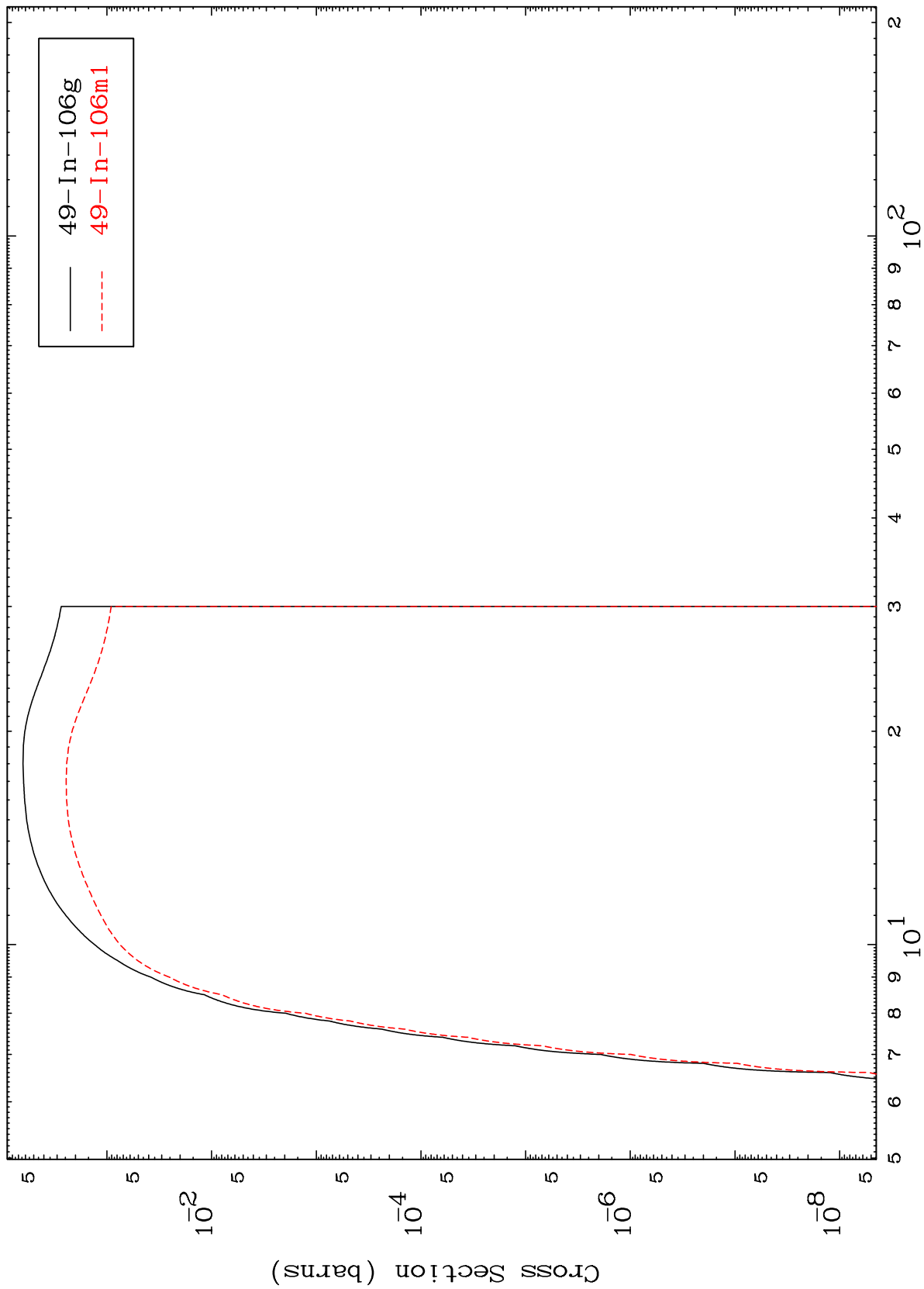


MAT 5010

(n,n') p

50-Sn-107

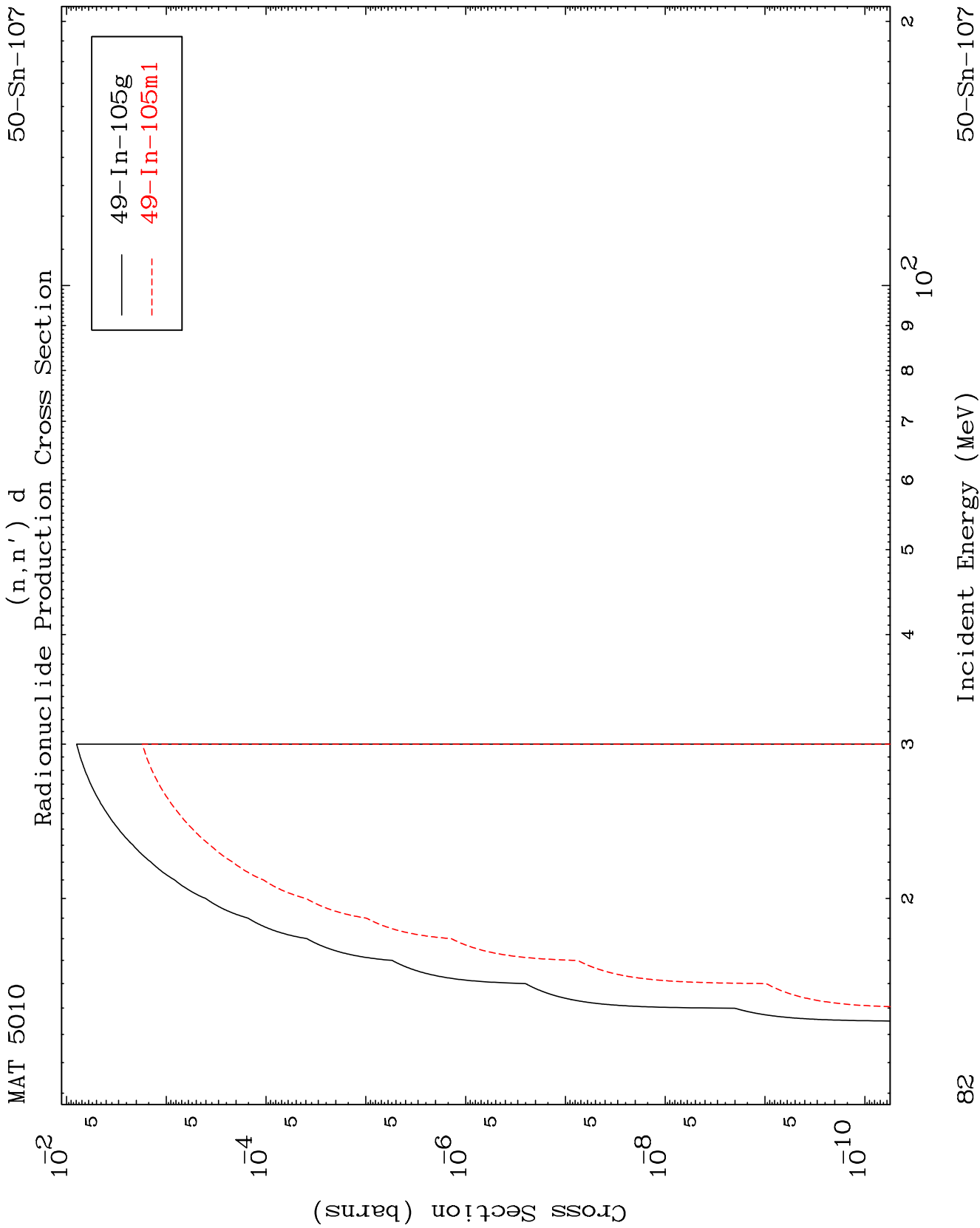
Radionuclide Production Cross Section



81

Incident Energy (MeV)

50-Sn-107

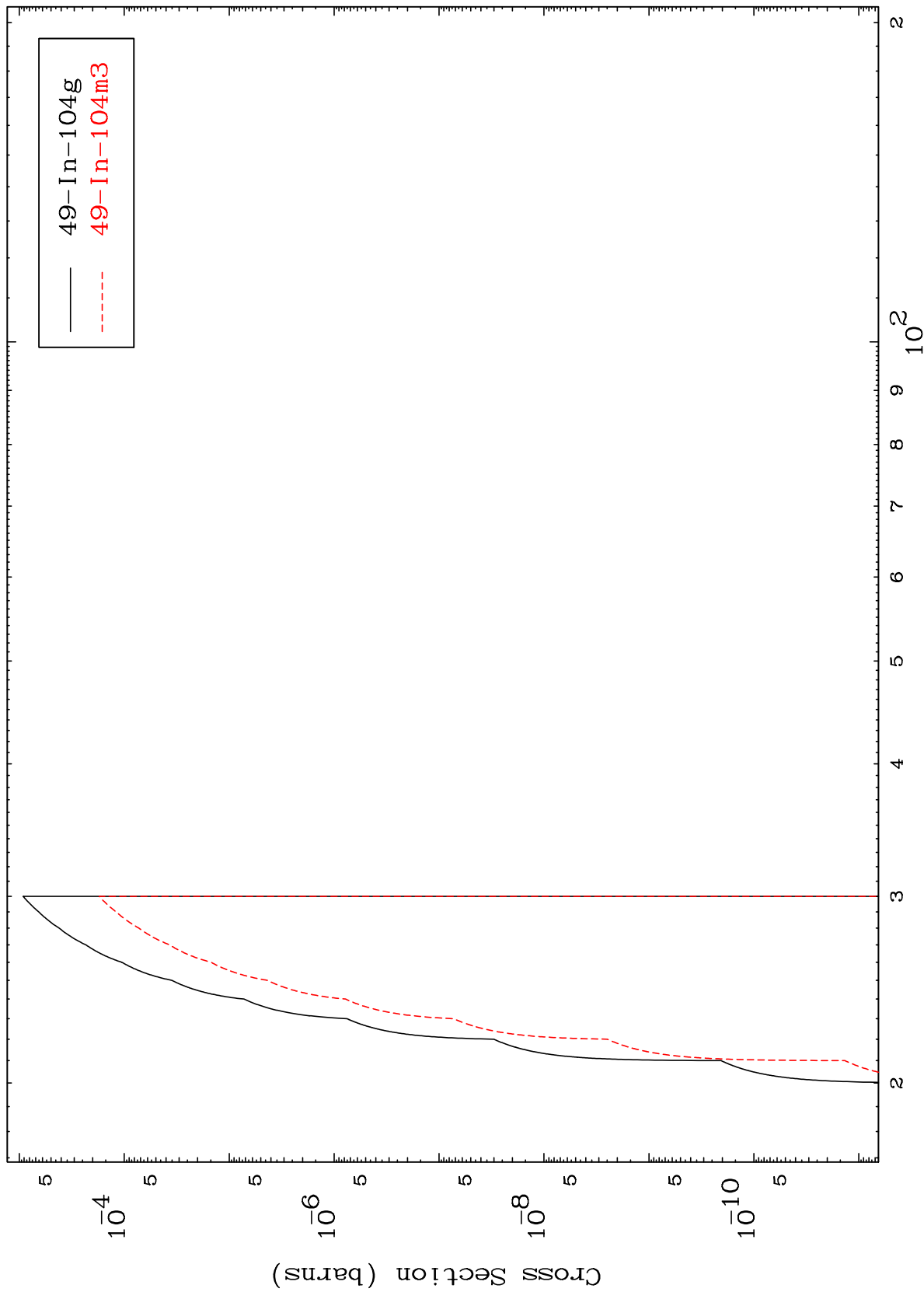


MAT 5010

(n,n') t

50-Sn-107

Radionuclide Production Cross Section



83

Incident Energy (MeV)

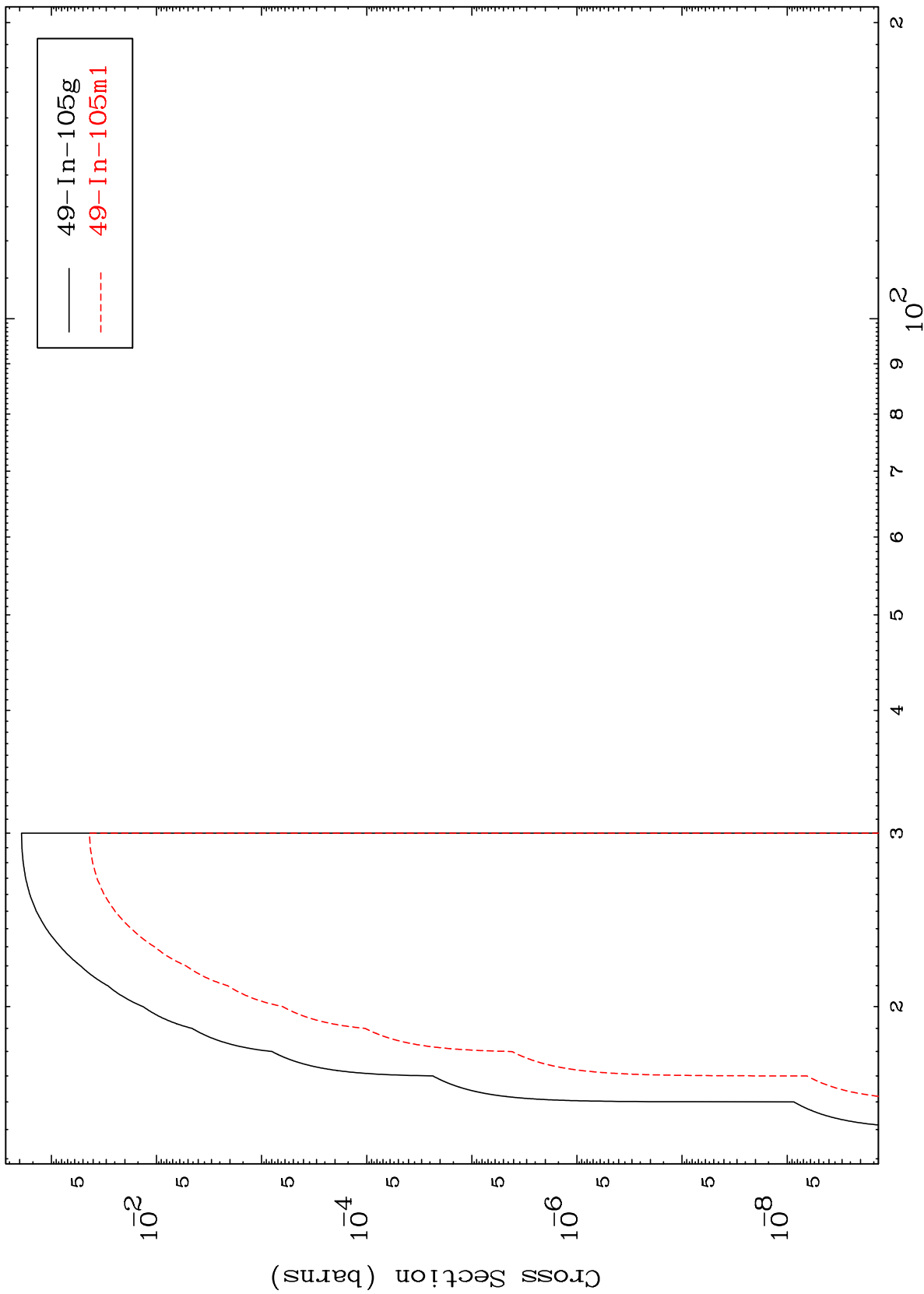
50-Sn-107

MAT 5010

(n,2n) p

50-Sn-107

Radionuclide Production Cross Section



84

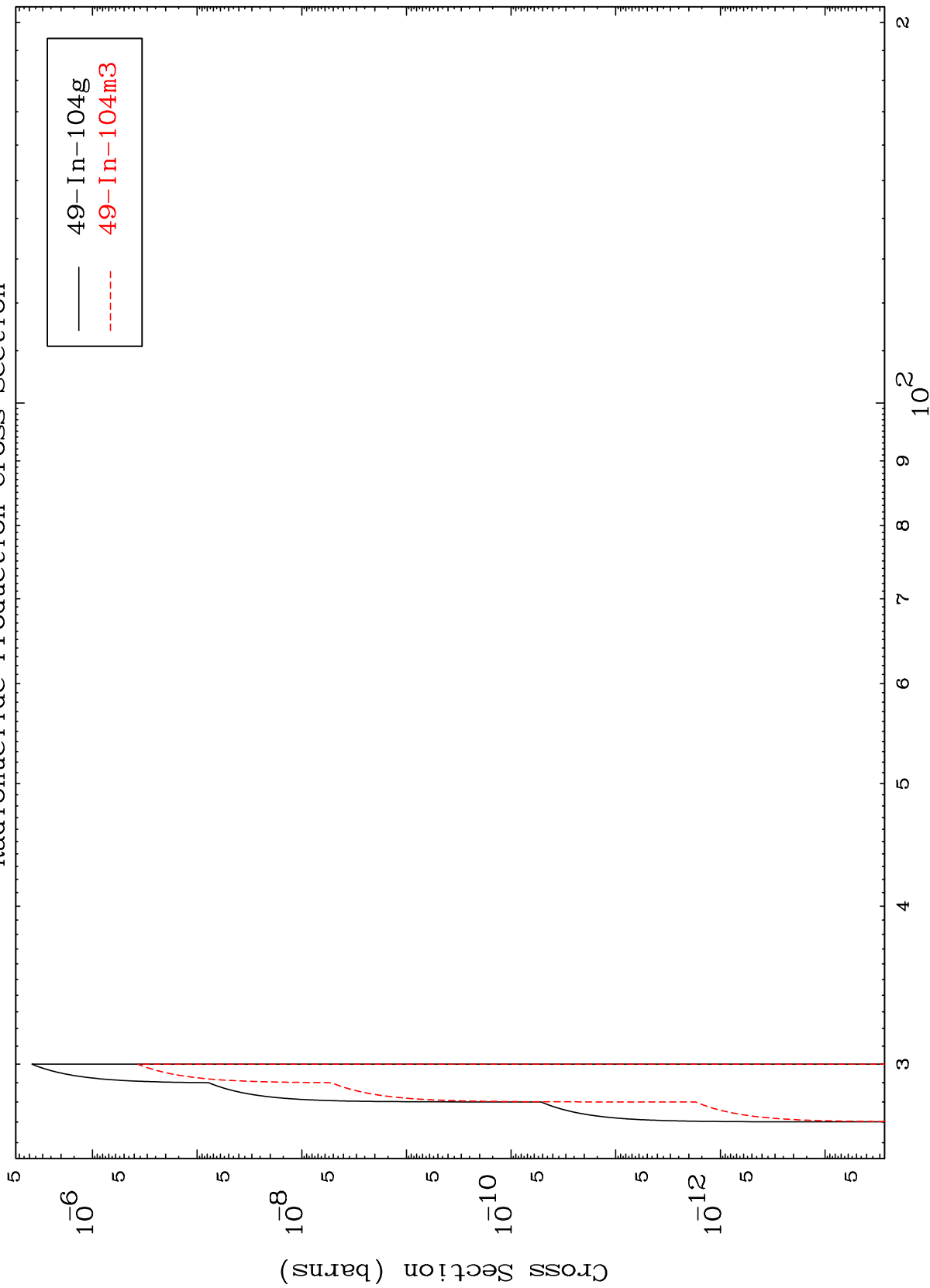
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

(n,3n) p  
Radionuclide Production Cross Section



85

50-Sn-107

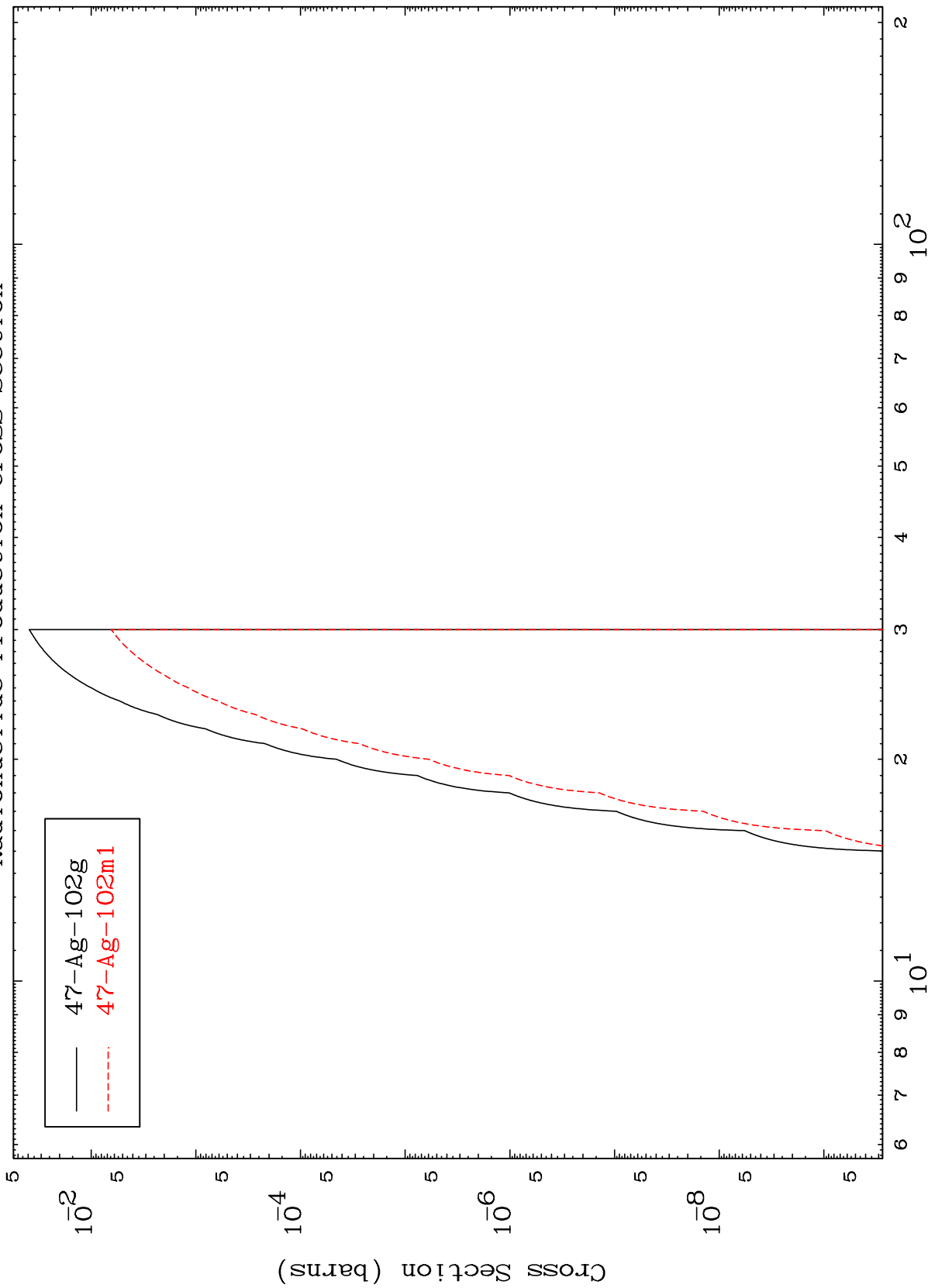
Incident Energy (MeV)

MAT 5010

(n,n') p  $\alpha$

50-Sn-107

Radionuclide Production Cross Section



— 47-Ag-102g  
- - - 47-Ag-102m1

86

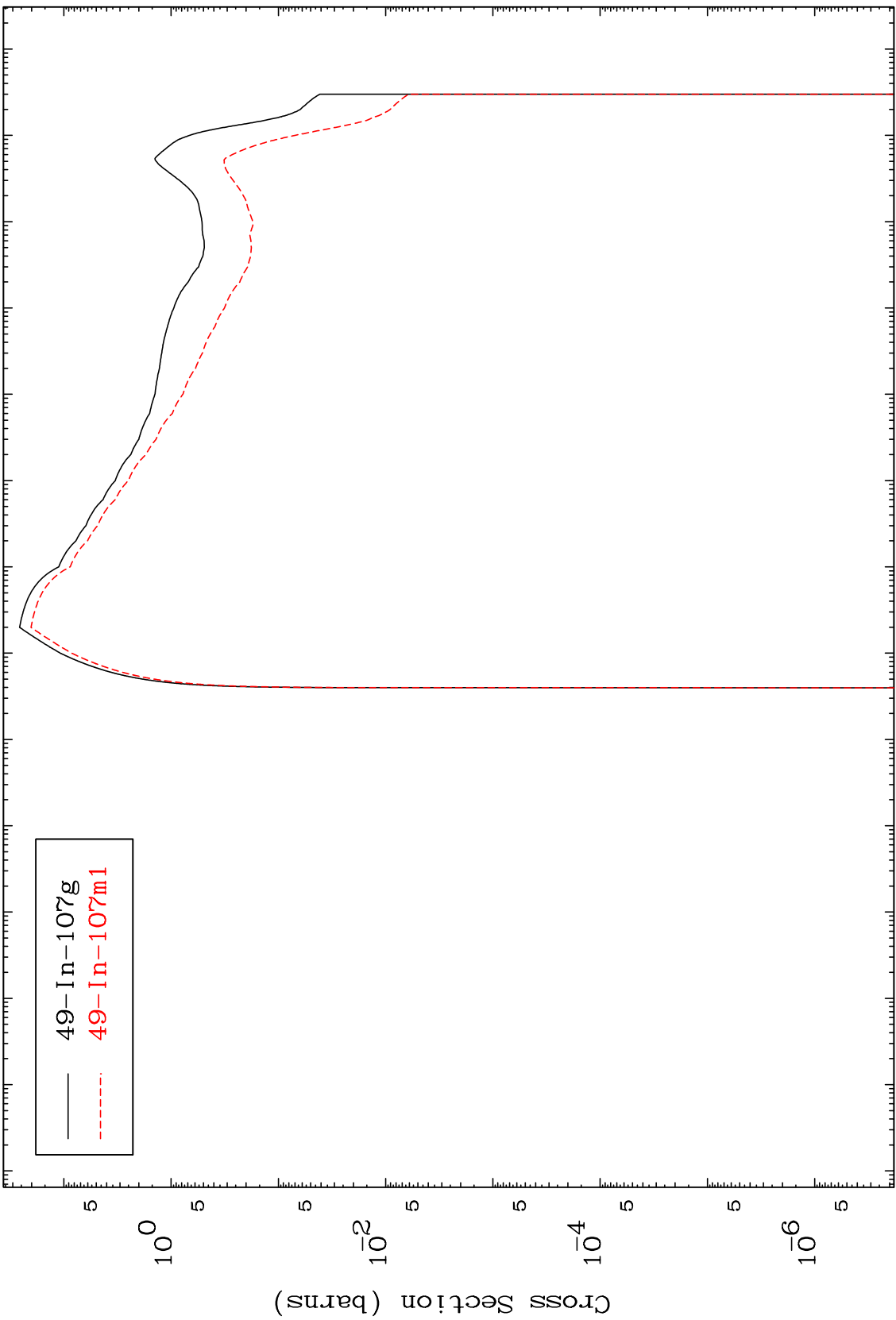
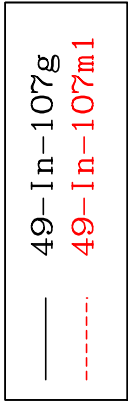
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

(n,p)  
Radionuclide Production Cross Section



50-Sn-107

$10^0$

$10^{-2}$

$10^{-4}$

$10^{-6}$

$10^{-8}$

$10^{-10}$

$10^0$

$10^2$

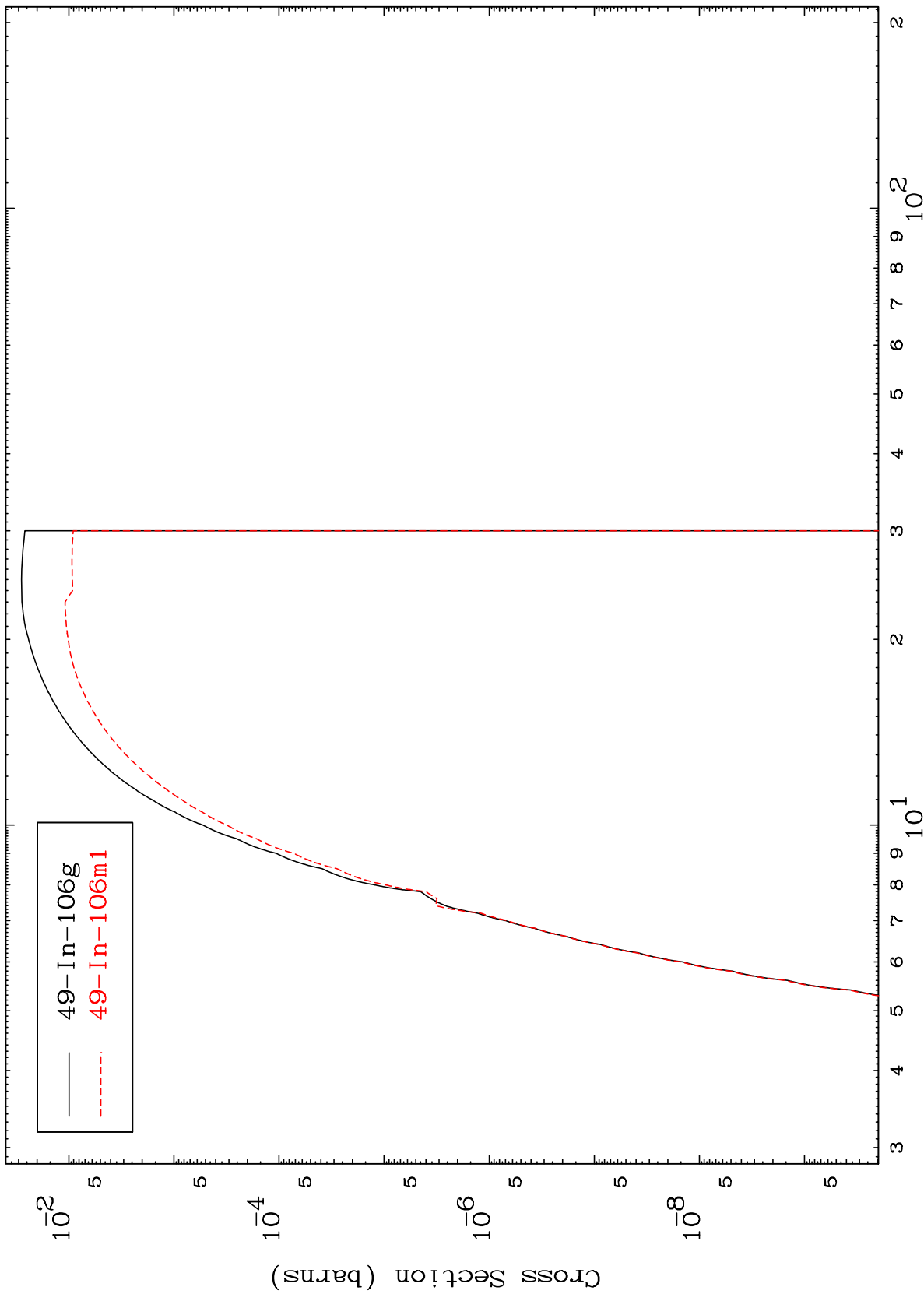
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

(n,d)  
Radionuclide Production Cross Section



88

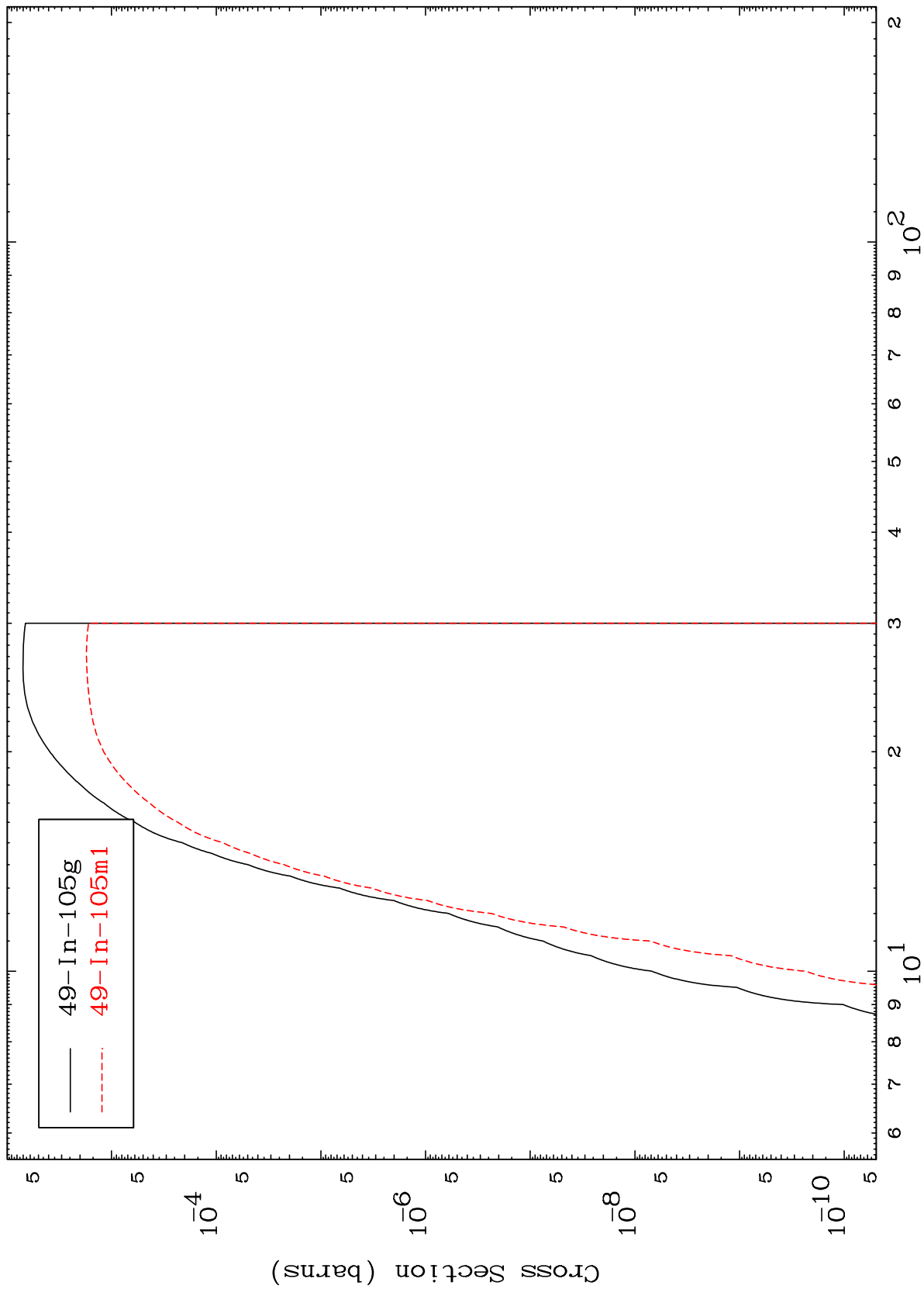
50-Sn-107

Incident Energy (MeV)

MAT 5010

50-Sn-107

(n, t)  
Radionuclide Production Cross Section



— 49-In-105g  
- - - 49-In-105m1

89

Incident Energy (MeV)

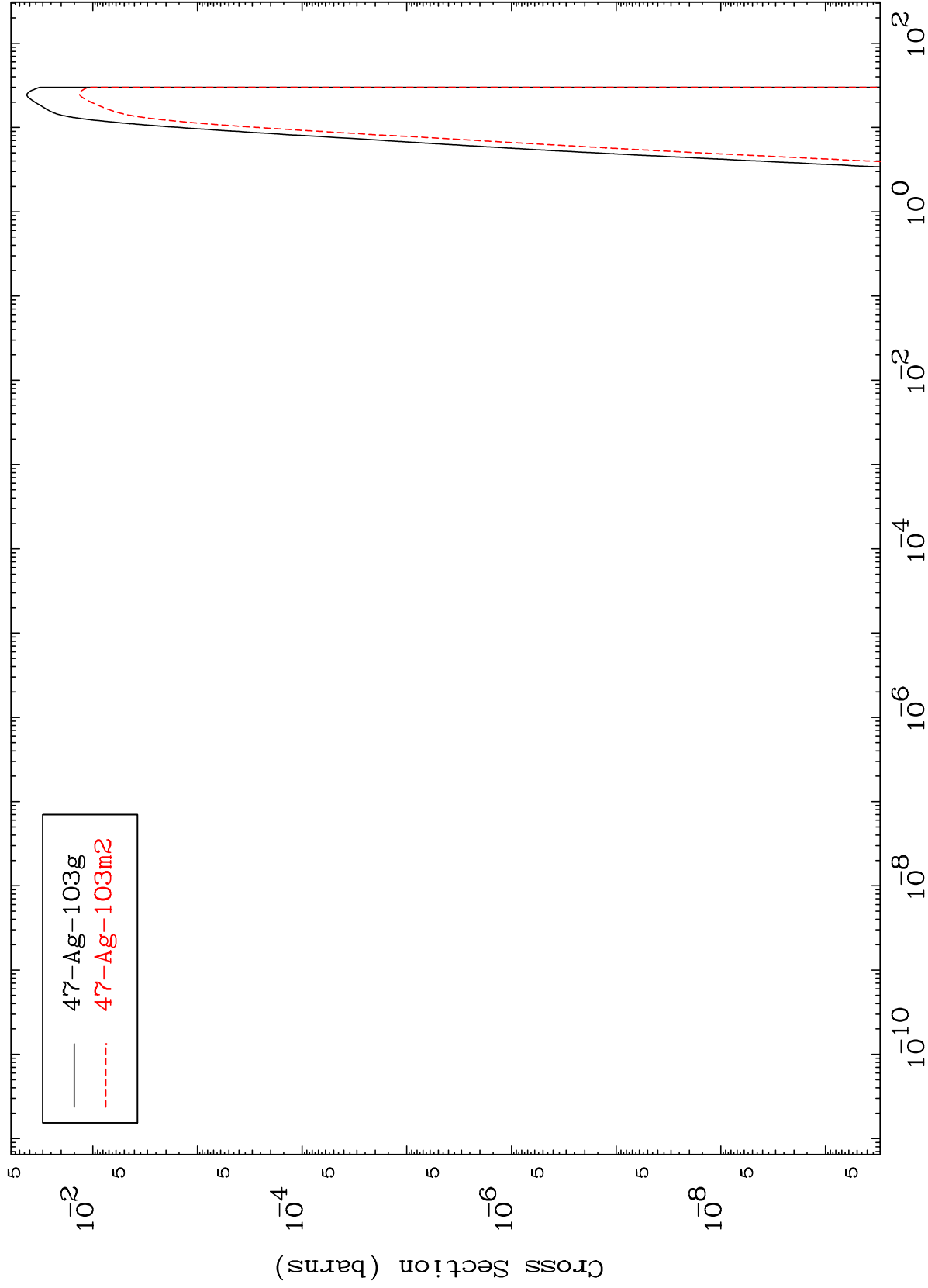
50-Sn-107

MAT 5010

(n,p)  $\alpha$

50-Sn-107

Radionuclide Production Cross Section



90

Incident Energy (MeV)

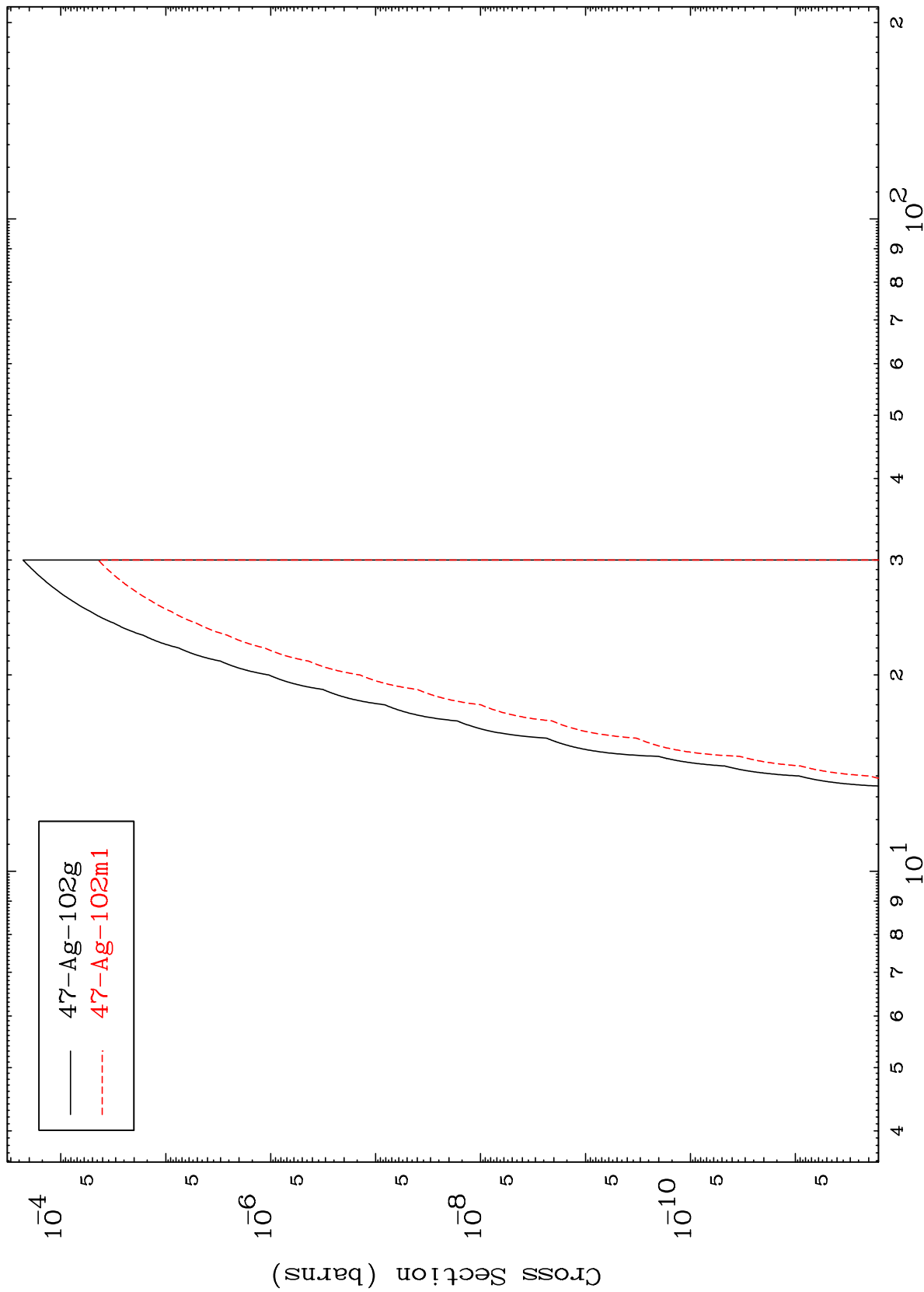
50-Sn-107

MAT 5010

(n,d)  $\alpha$

50-Sn-107

Radionuclide Production Cross Section



91

Incident Energy (MeV)

50-Sn-107