

Program EVALPLOT
(Version 2021-1)

by

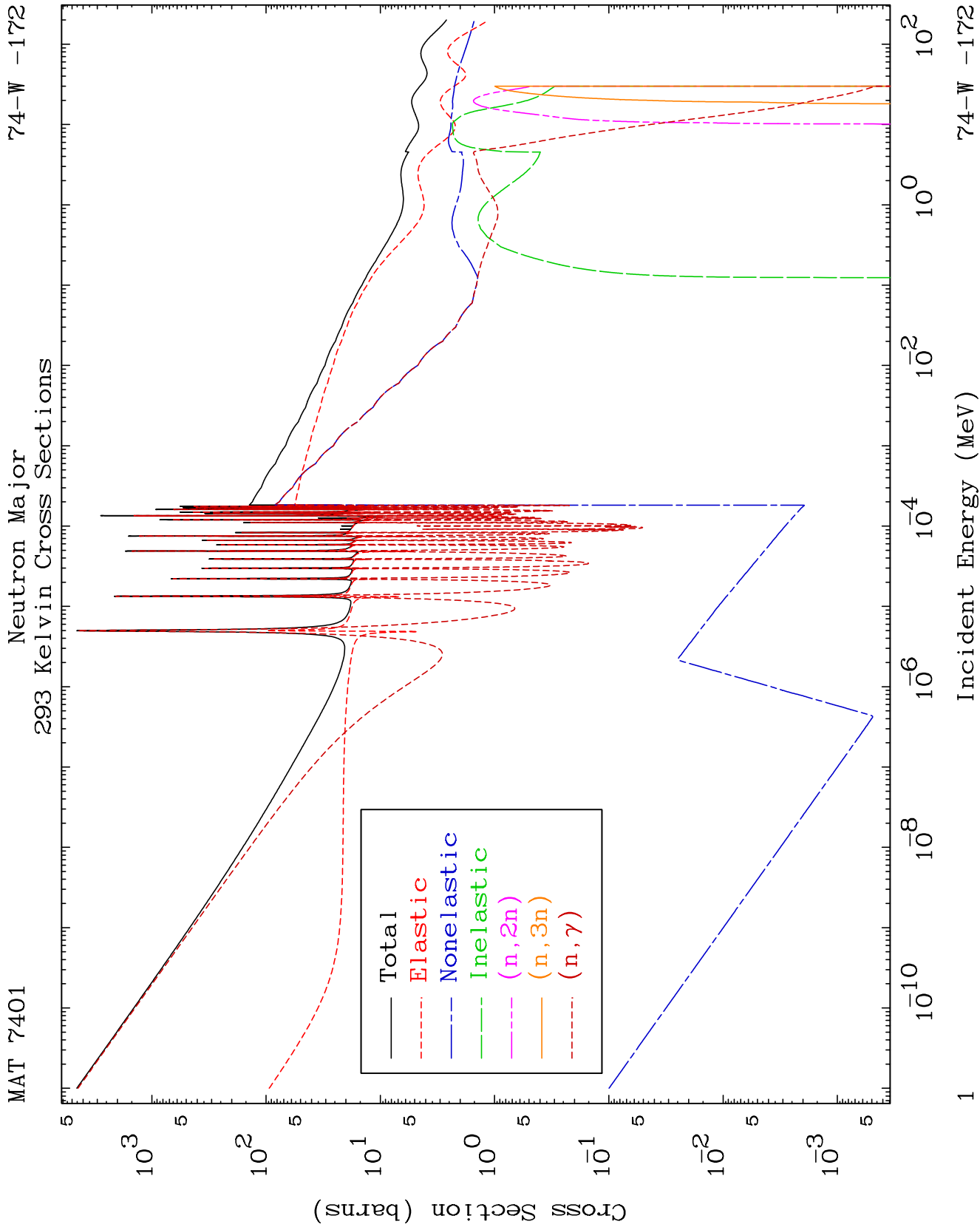
Dermott E. Cullen
(Present Contact Information)

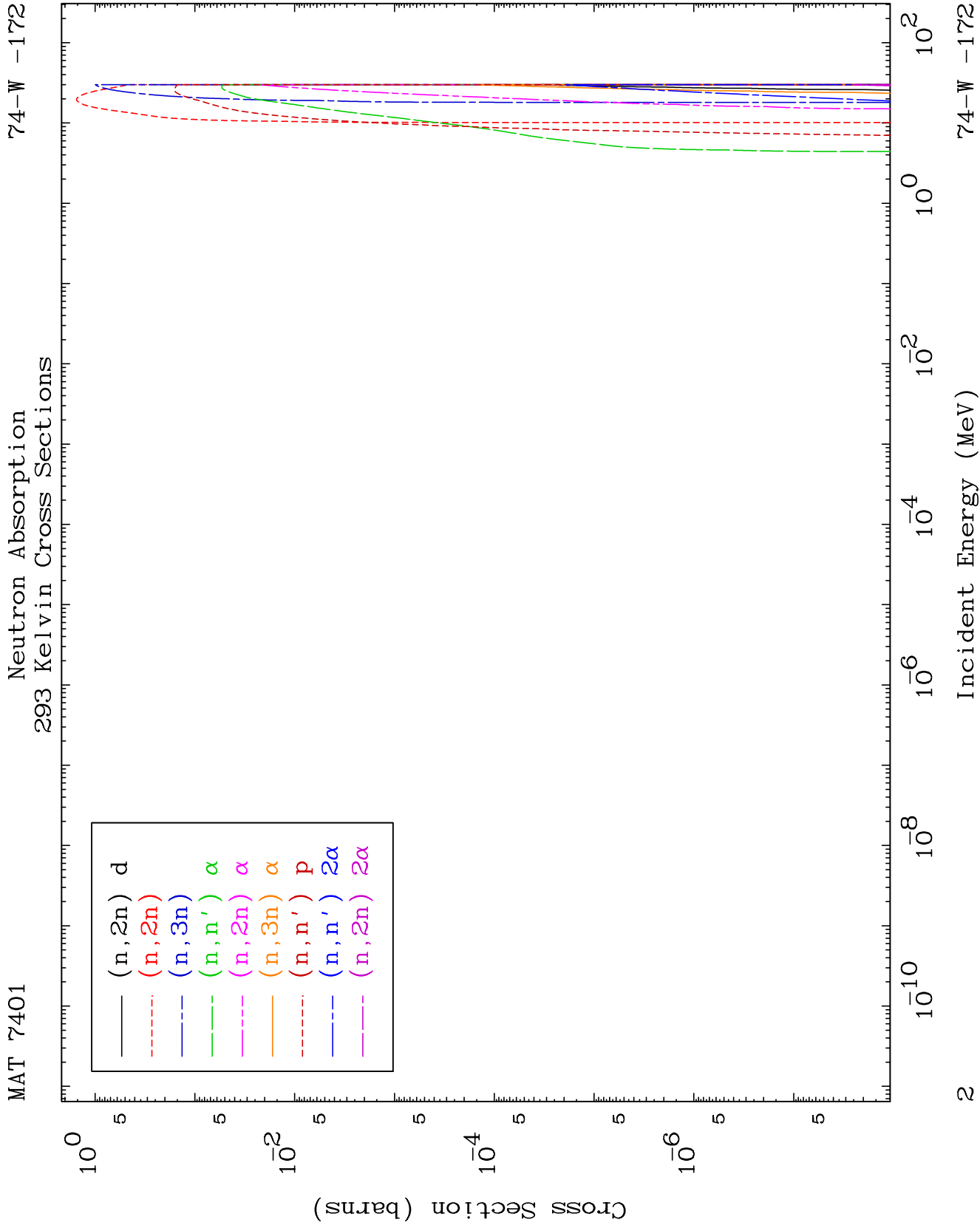
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

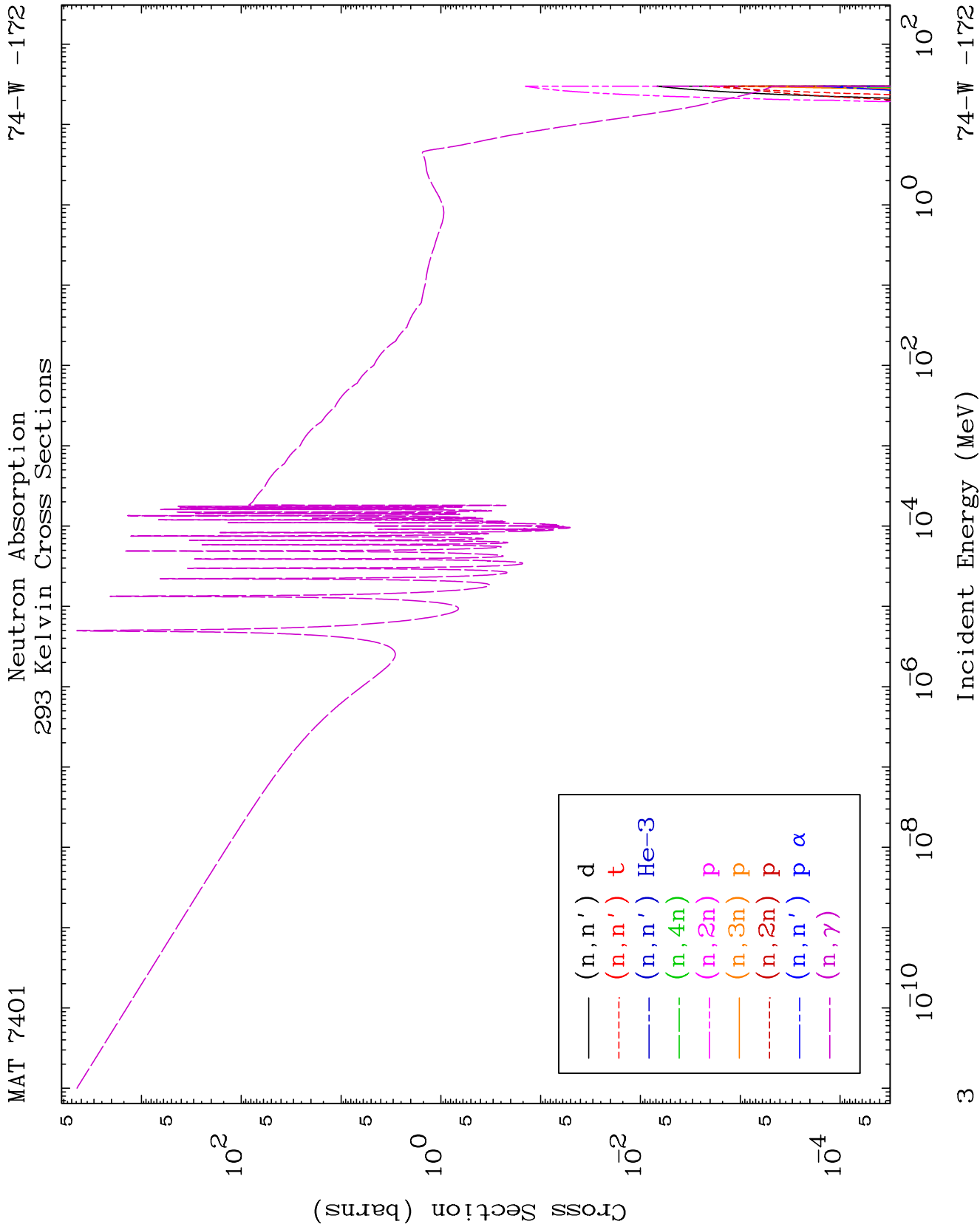
Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start



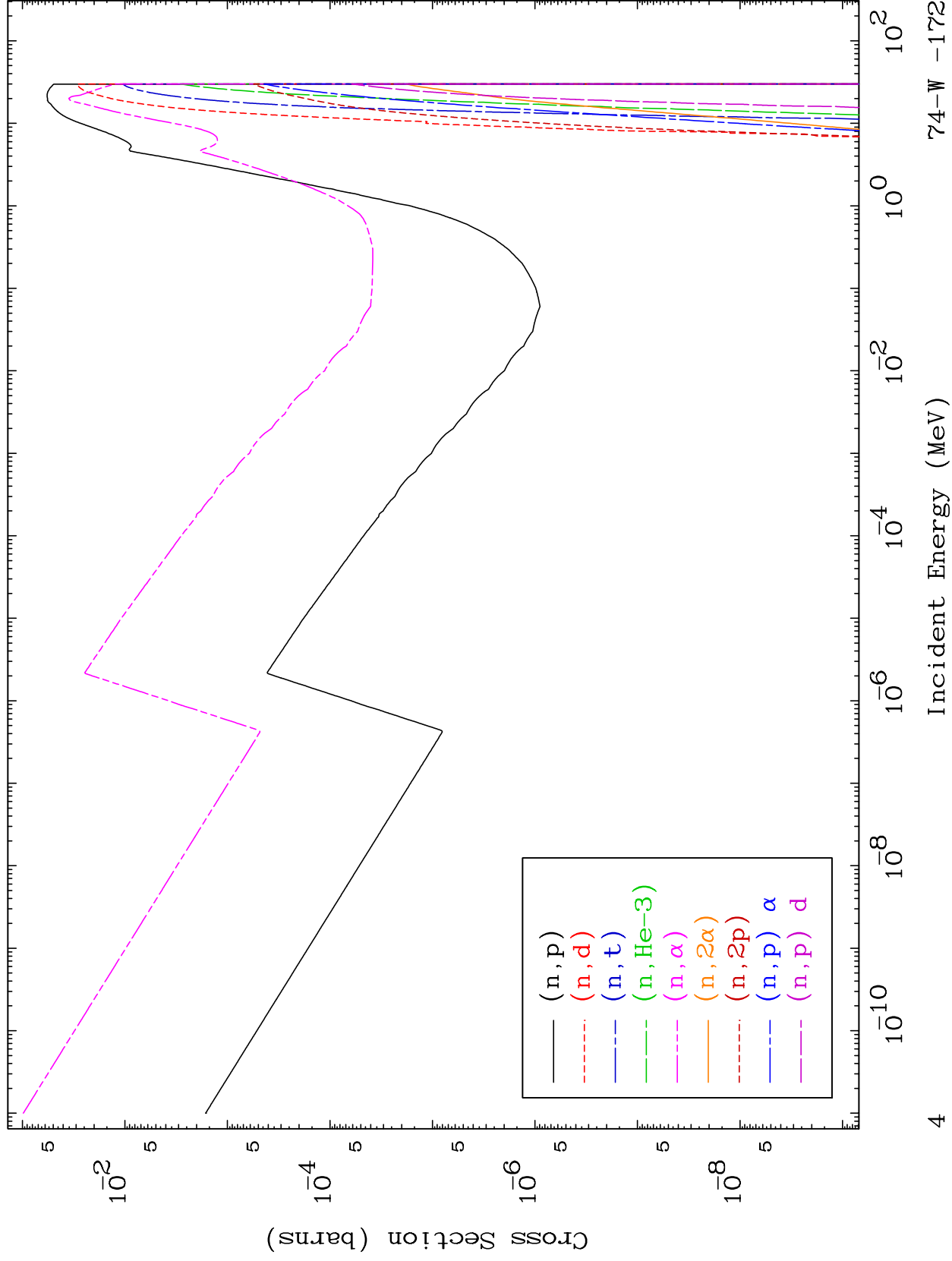


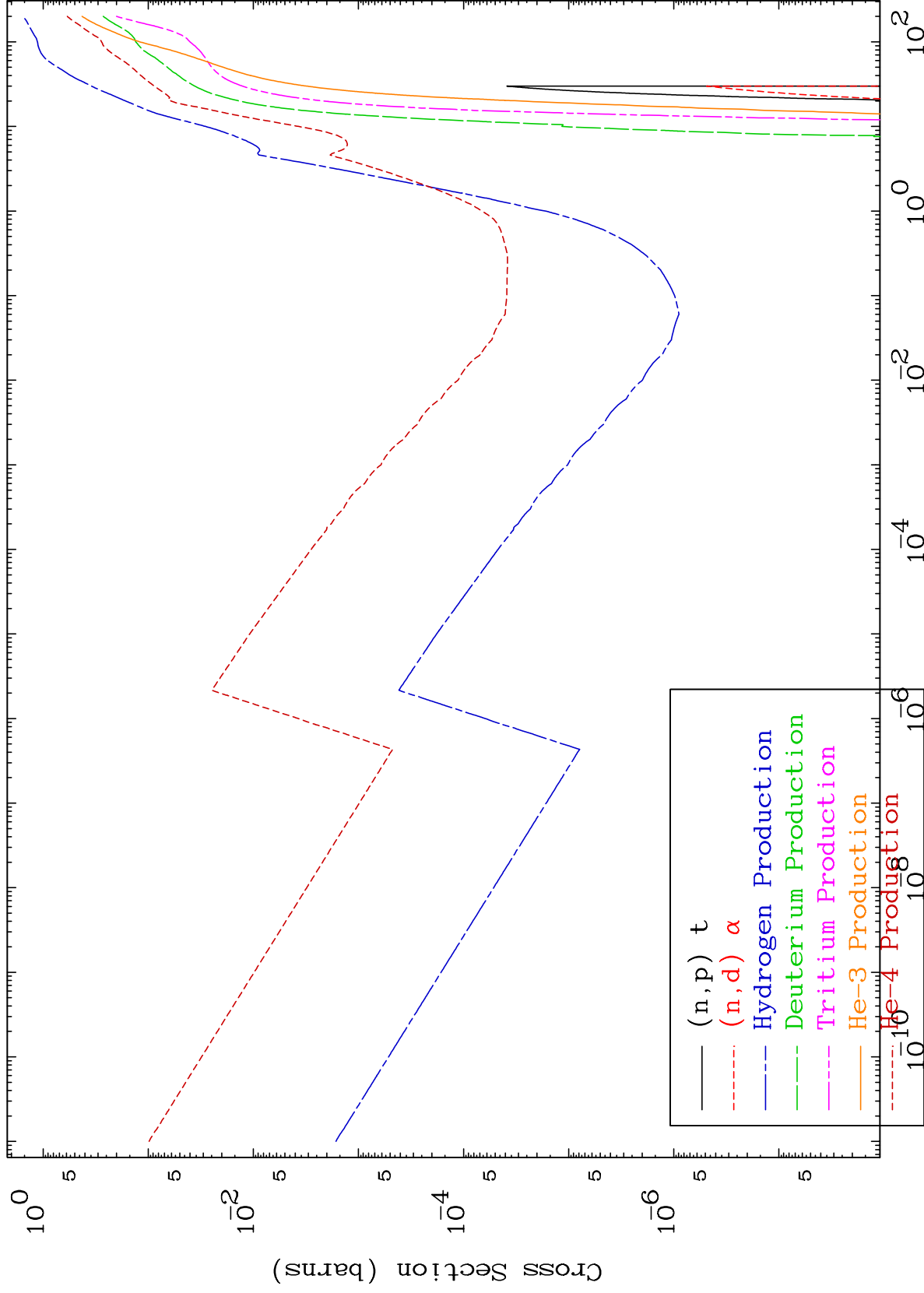


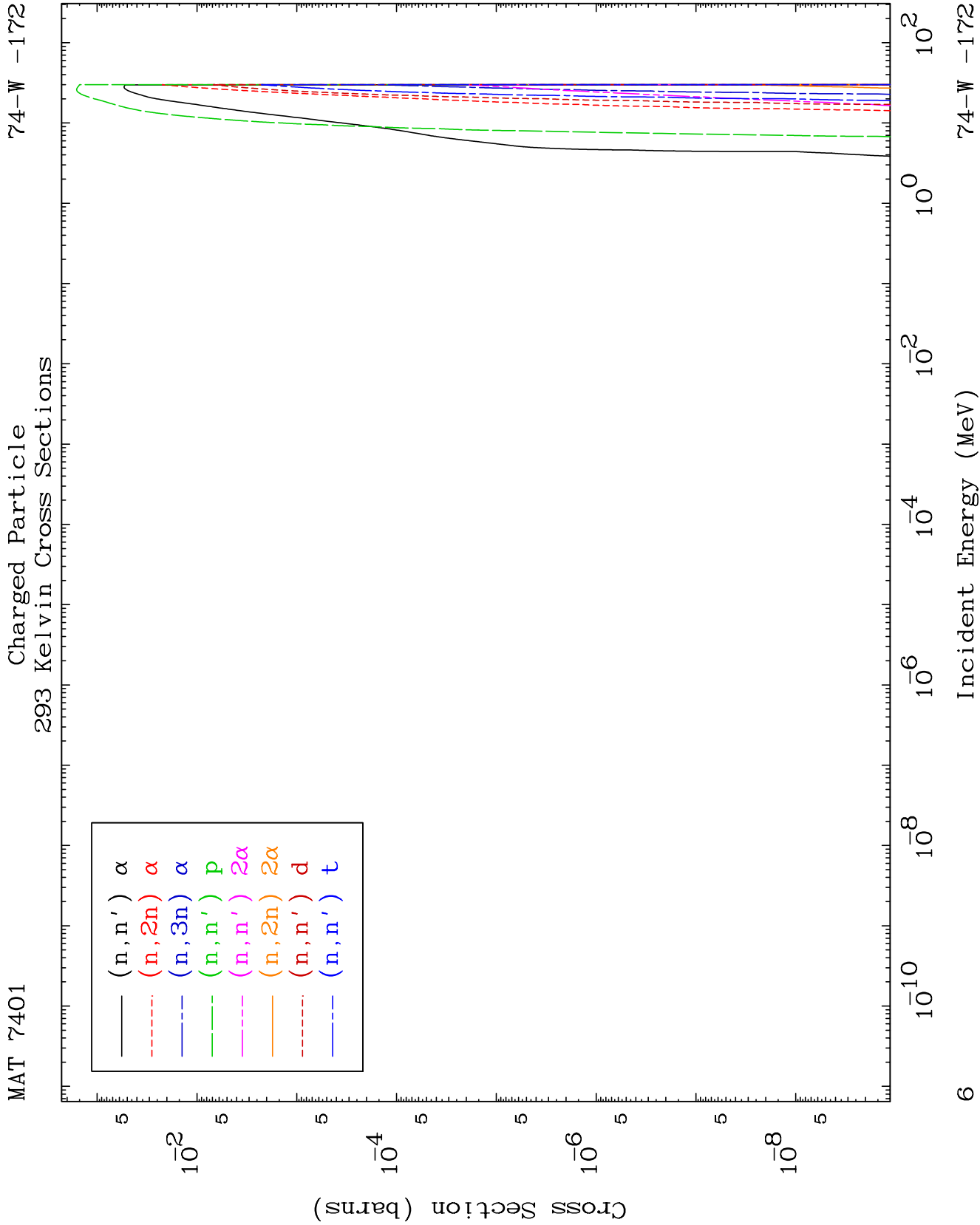
MAT 7401

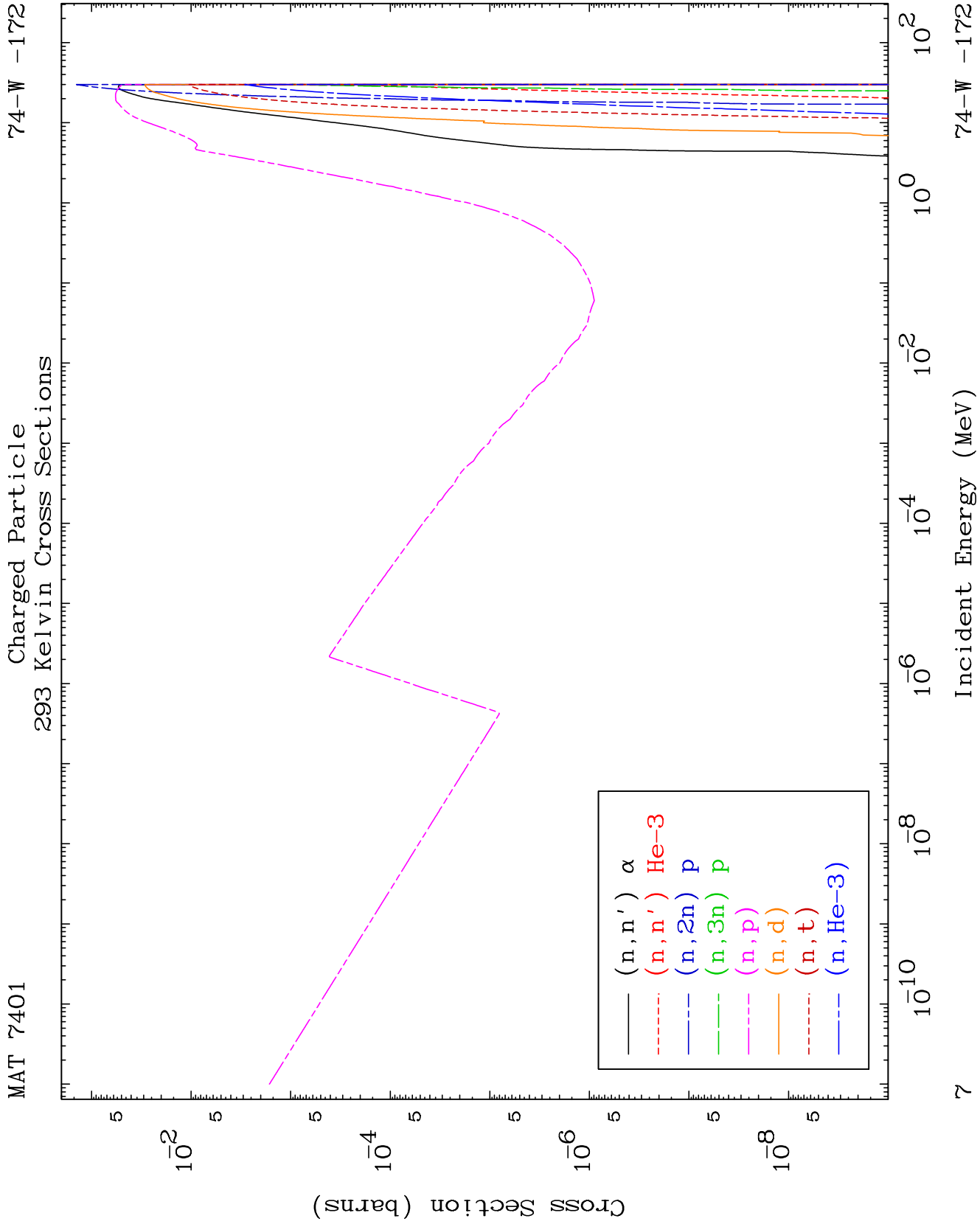
Neutron Absorption
293 Kelvin Cross Sections

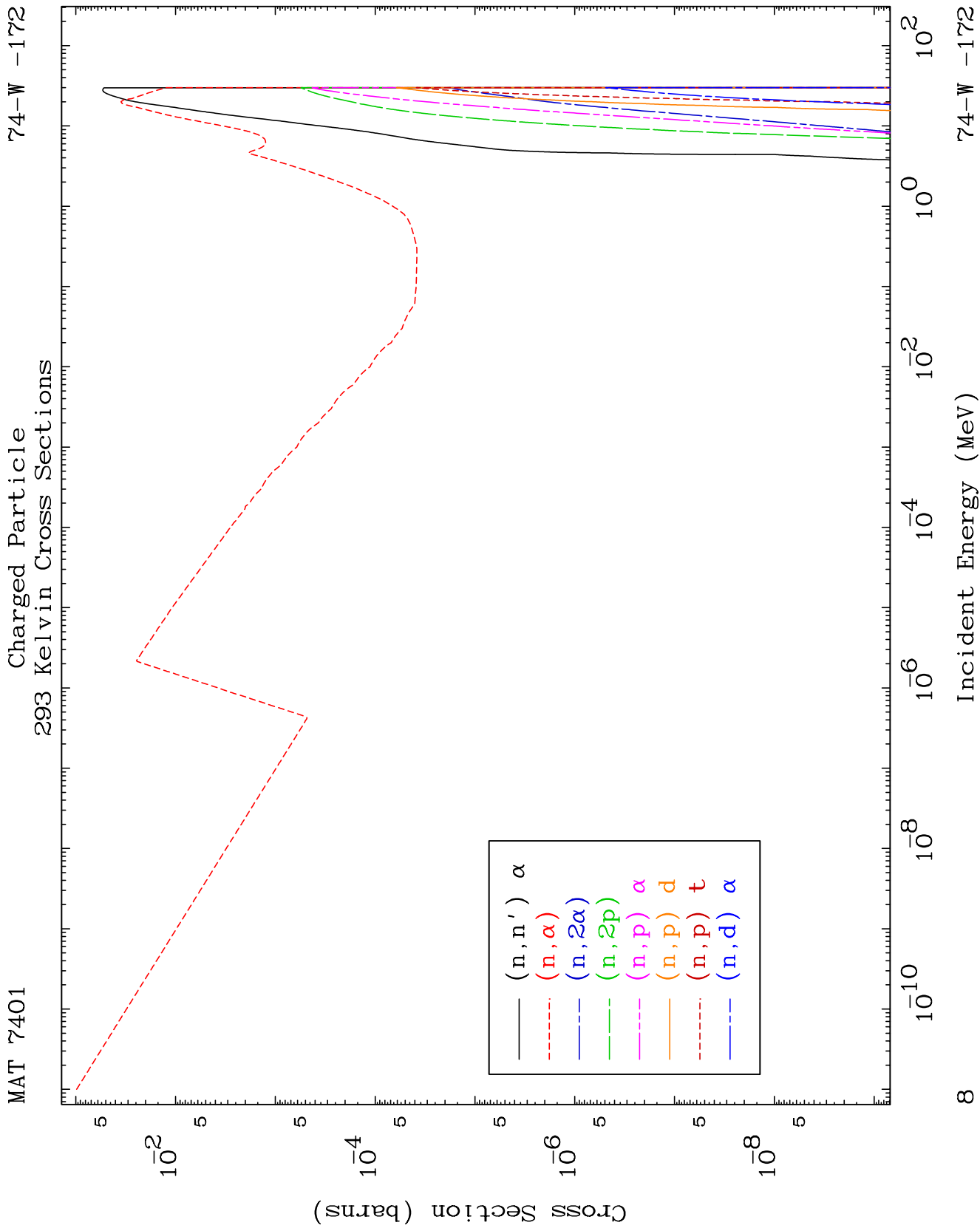
74-W -172

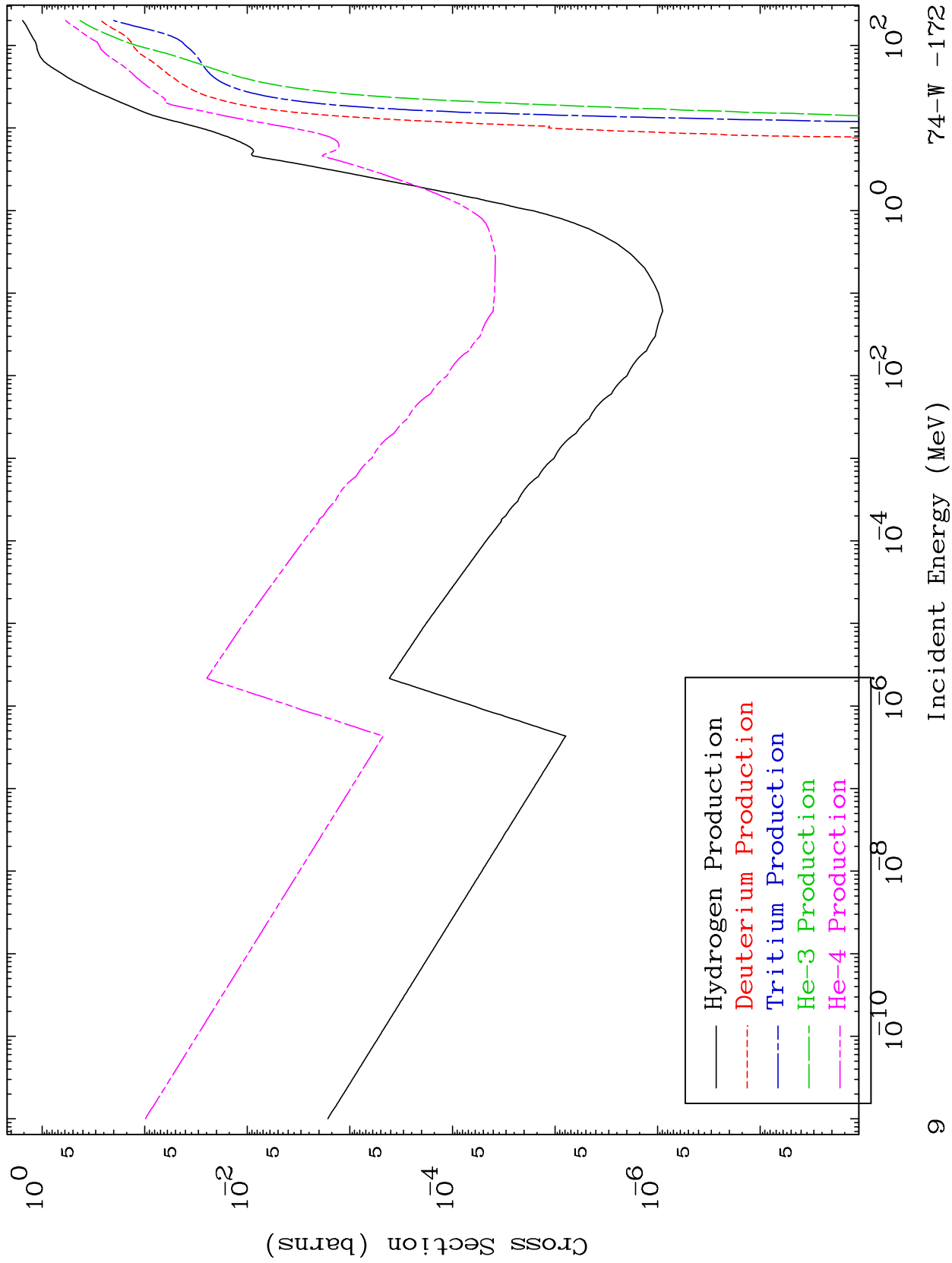








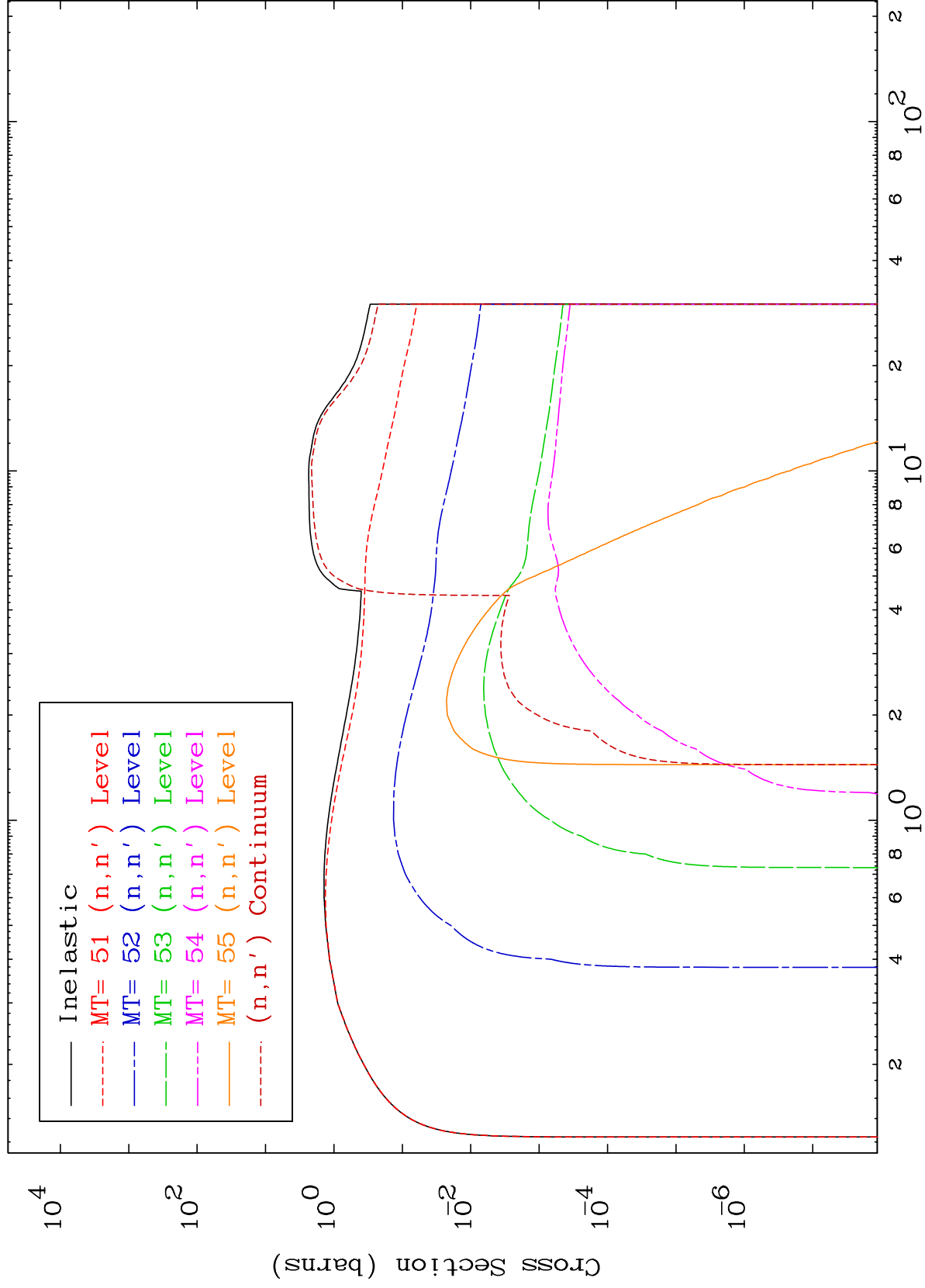




MAT 7401

(n,n') Levels
293 Kelvin Cross Sections

74-W -172



10

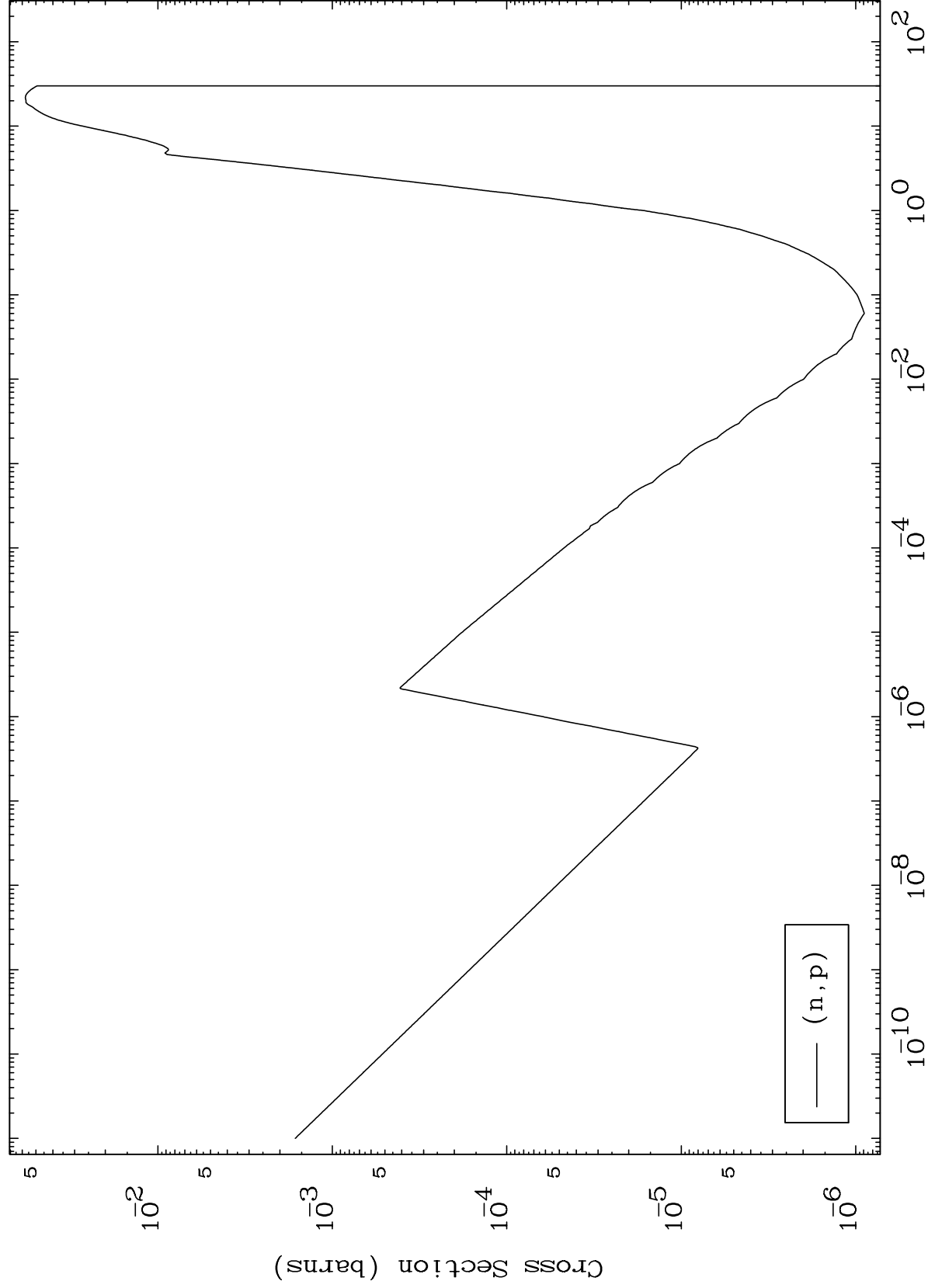
Incident Energy (MeV)

74-W -172

MAT 7401

(n,p) Levels
293 Kelvin Cross Sections

74-W -172



11

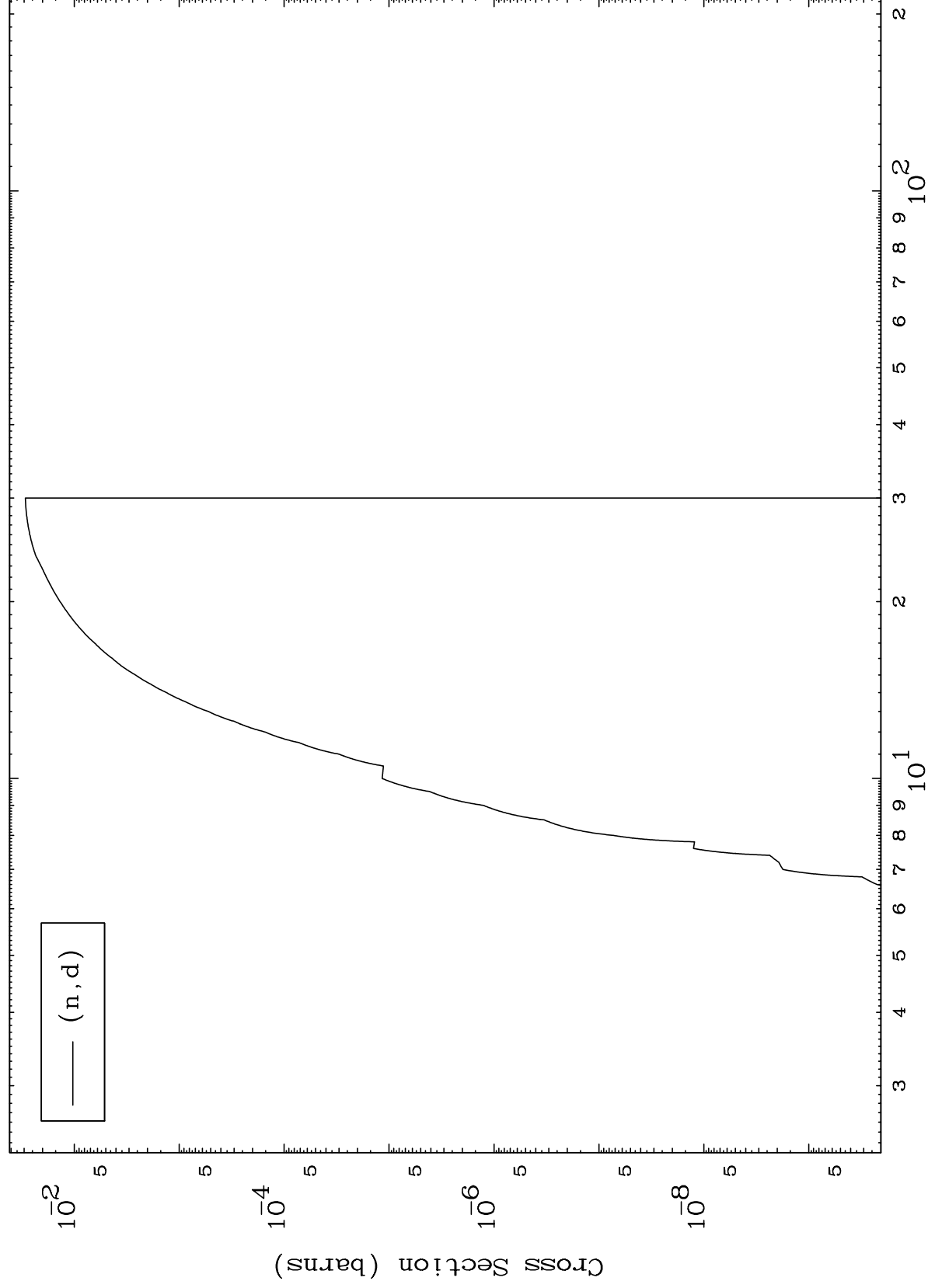
Incident Energy (MeV)

74-W -172

MAT 7401

(n,d) Levels
293 Kelvin Cross Sections

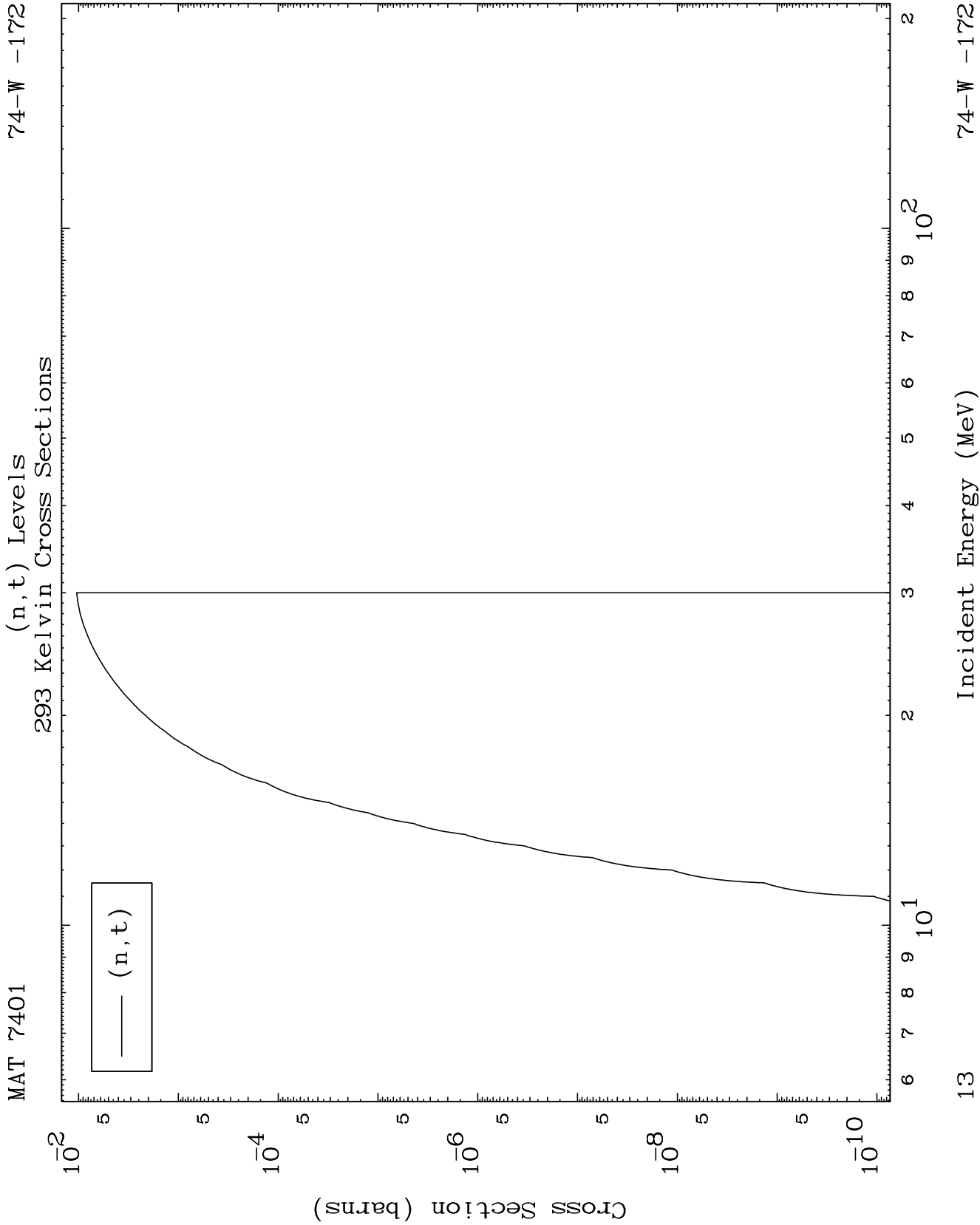
74-W -172



12

Incident Energy (MeV)

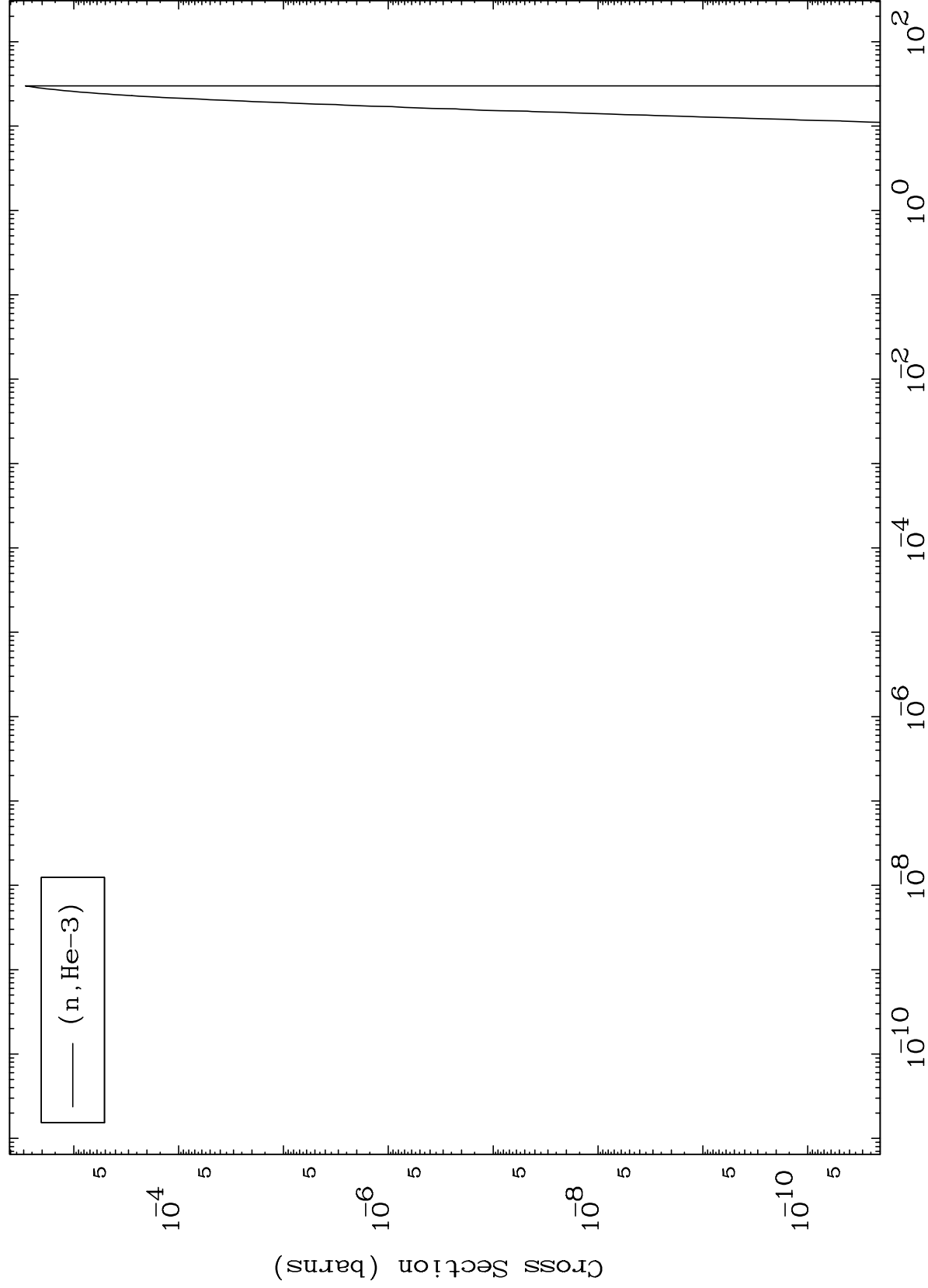
74-W -172



MAT 7401

(n,He3) Levels
293 Kelvin Cross Sections

74-W -172



14

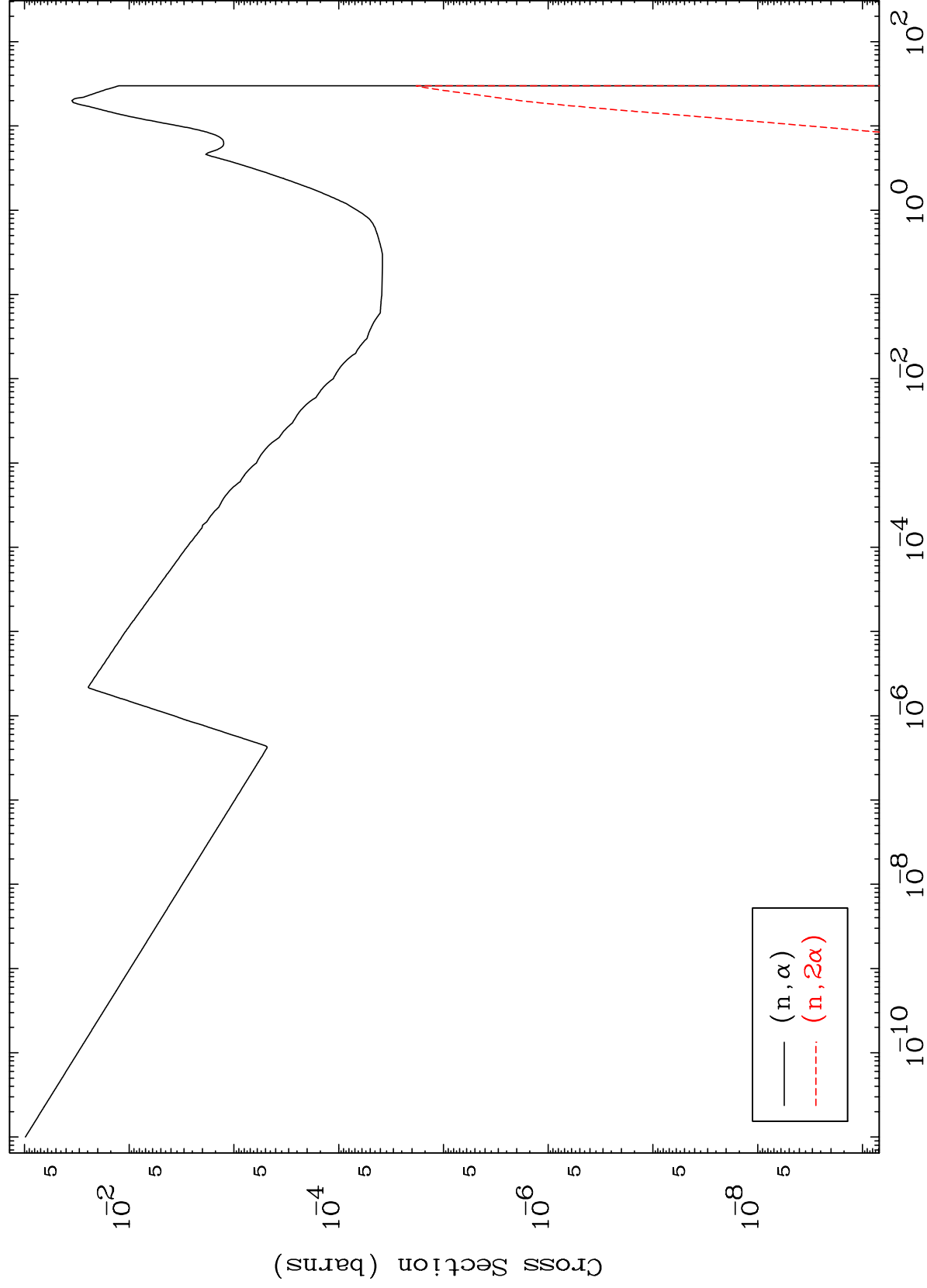
Incident Energy (MeV)

74-W -172

MAT 7401

(n, α) Levels
293 Kelvin Cross Sections

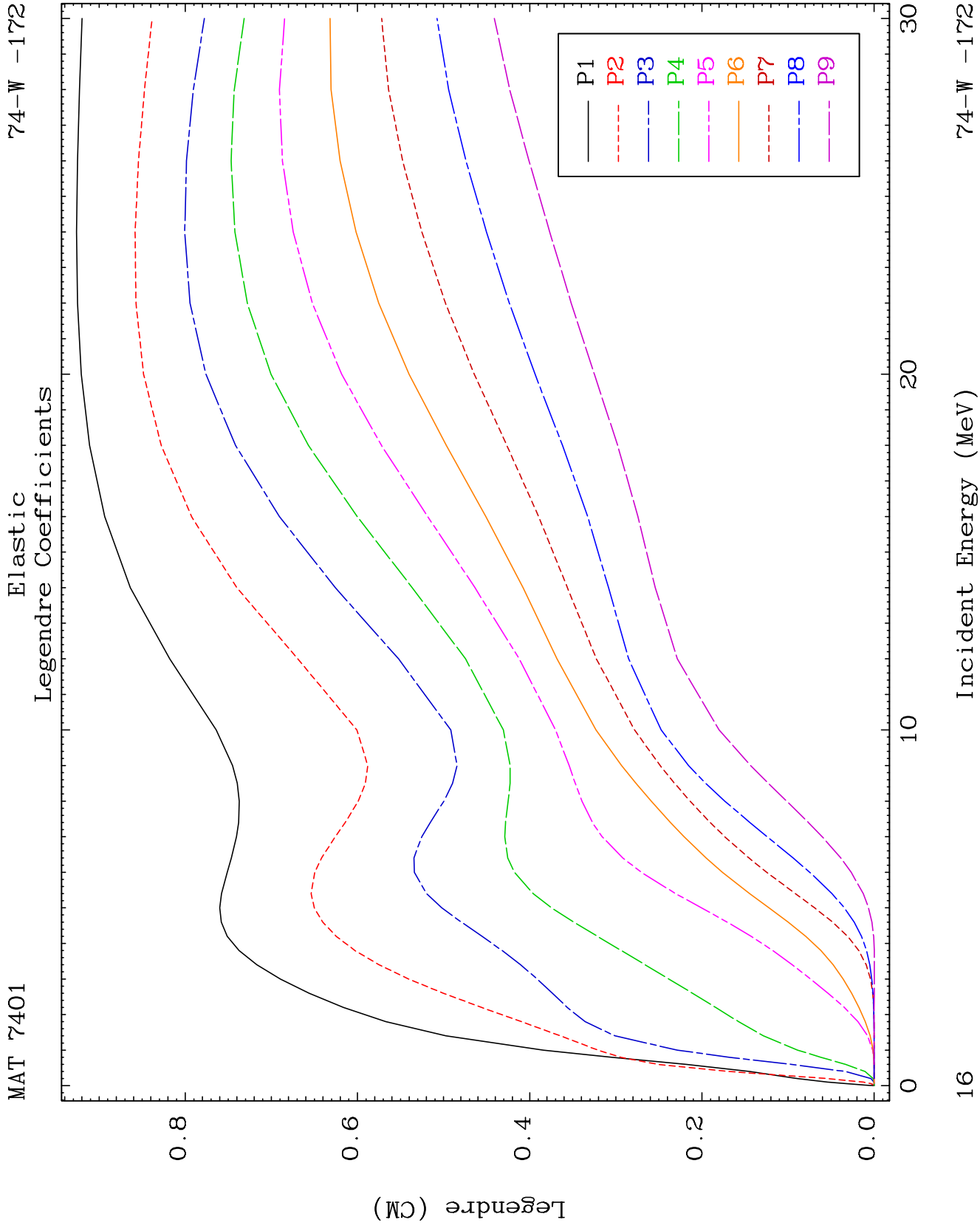
74-W -172

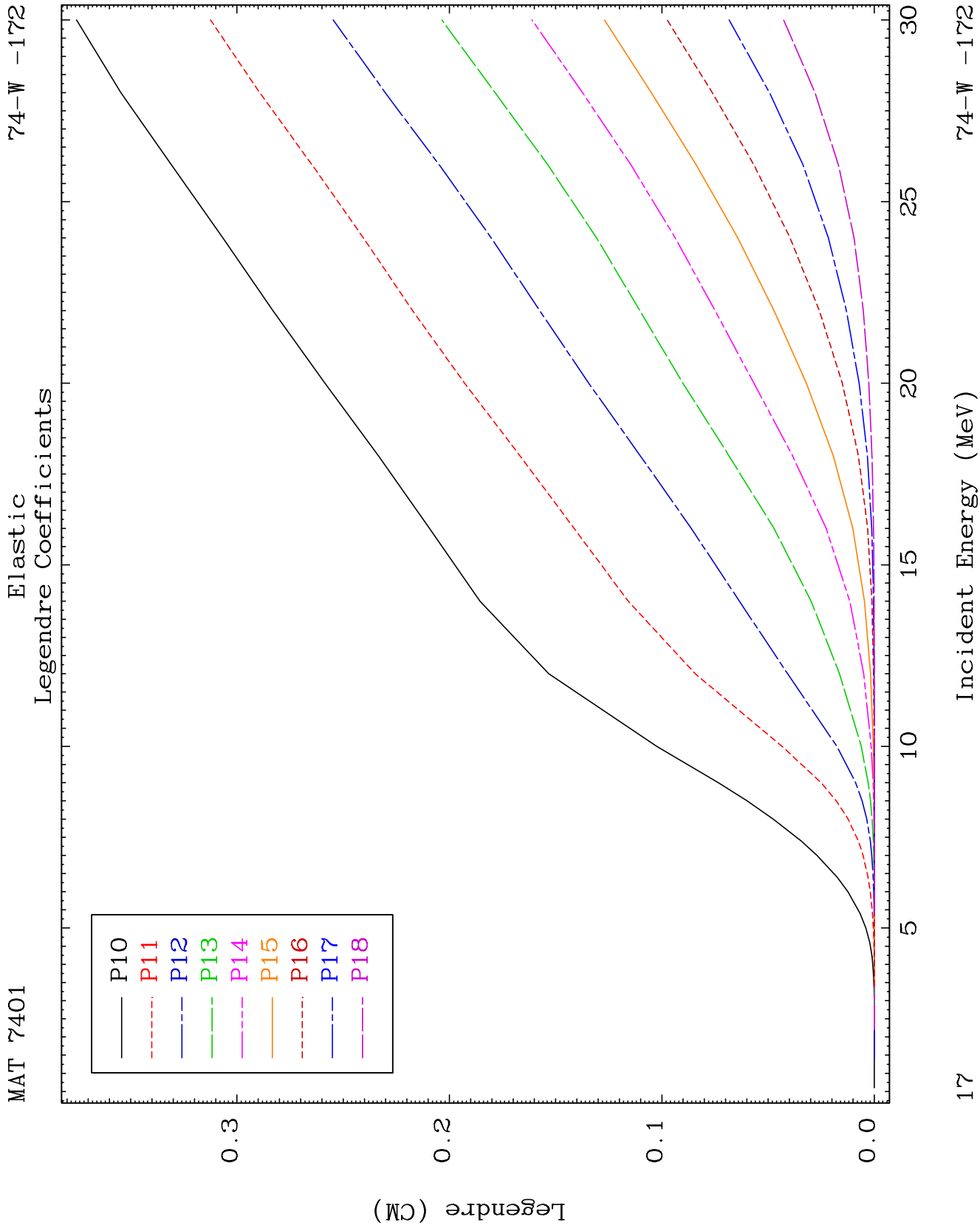


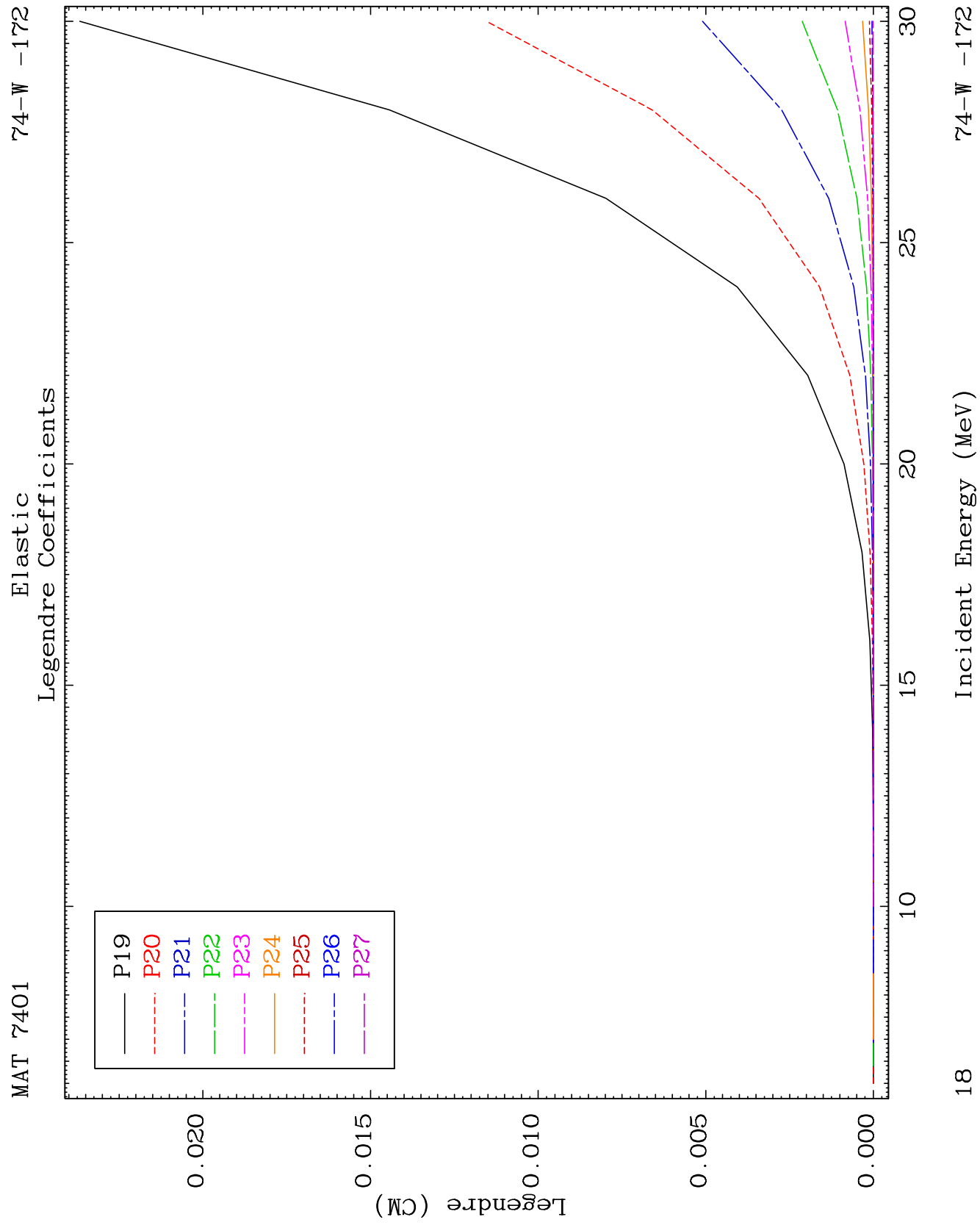
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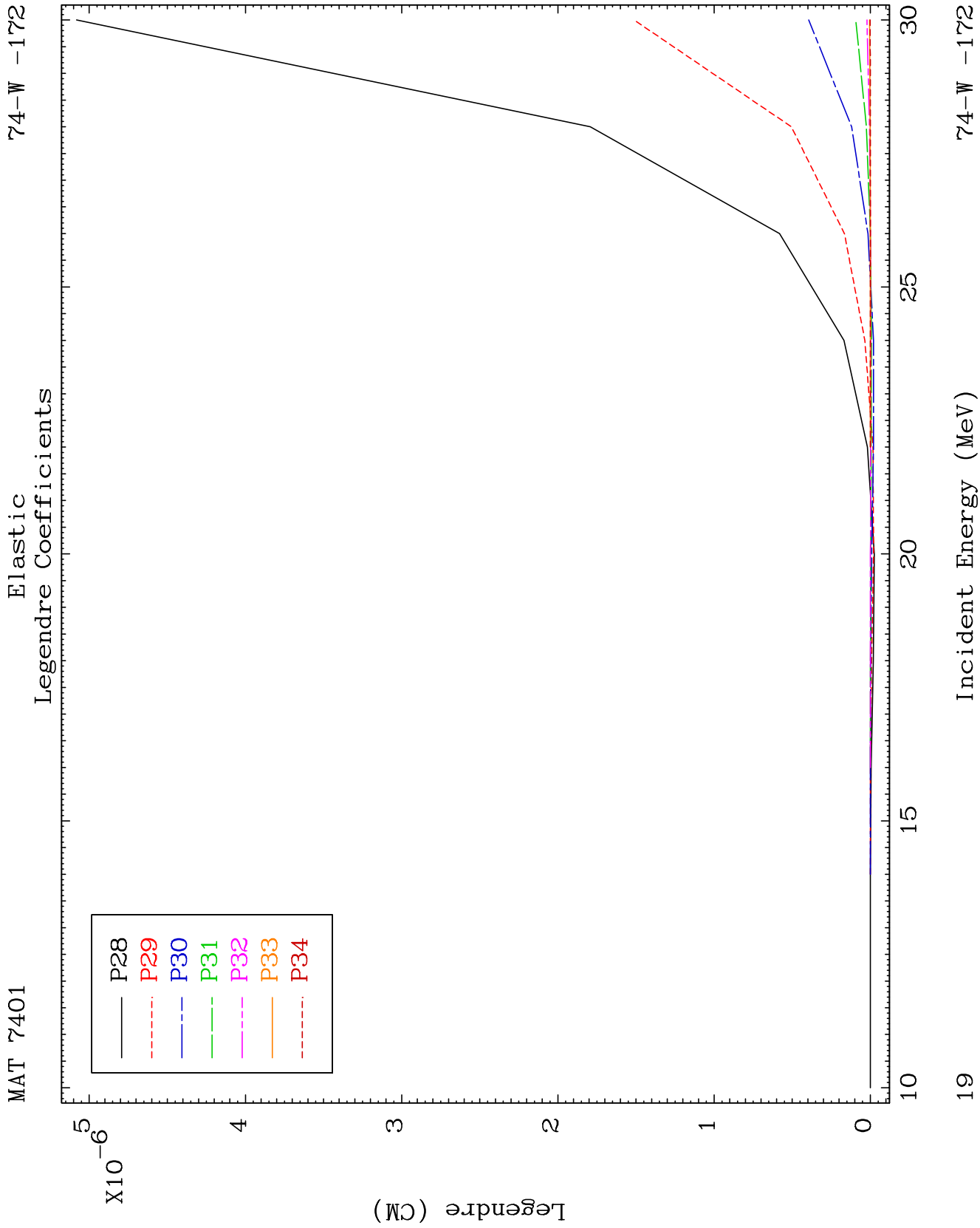
Incident Energy (MeV)

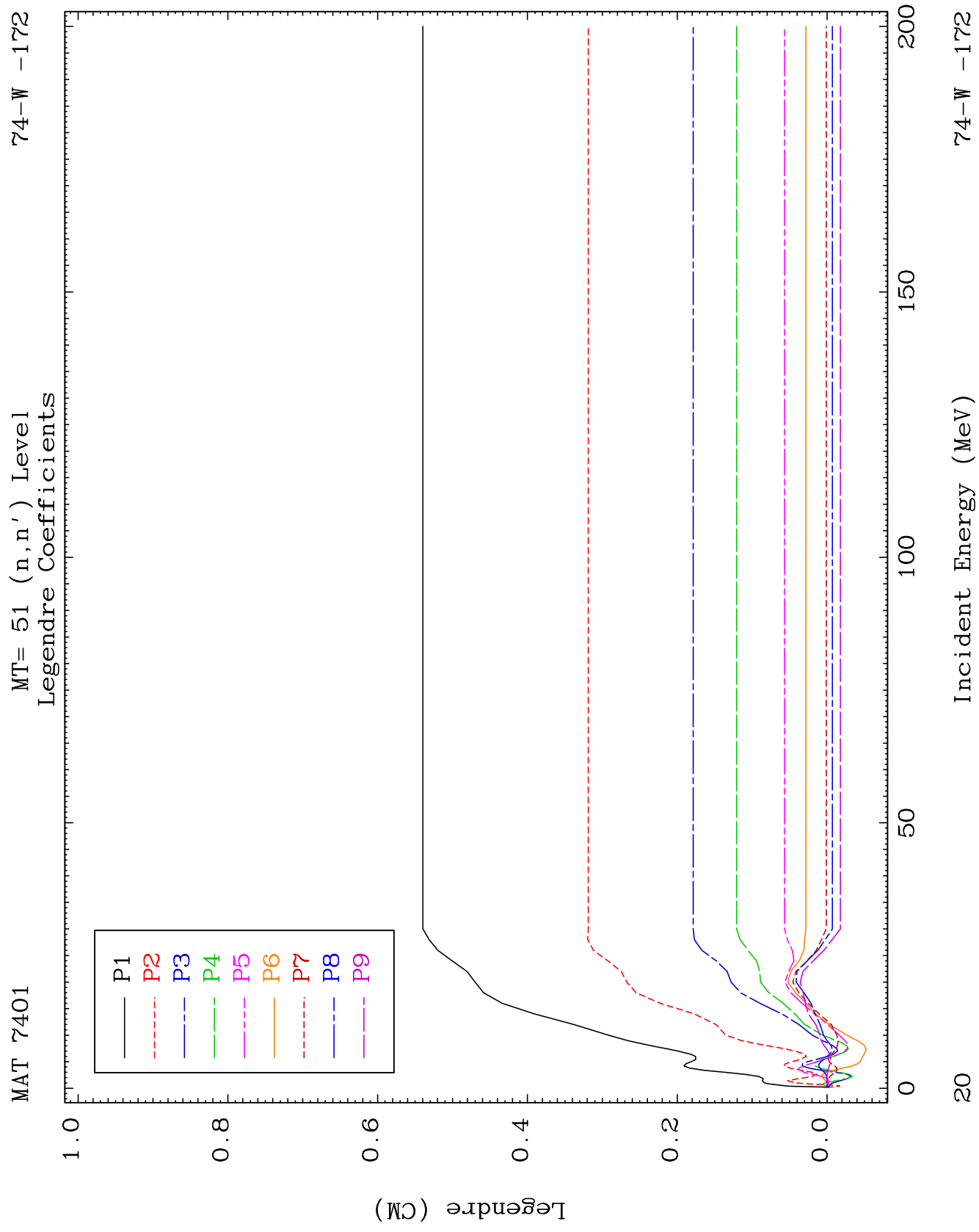
74-W -172







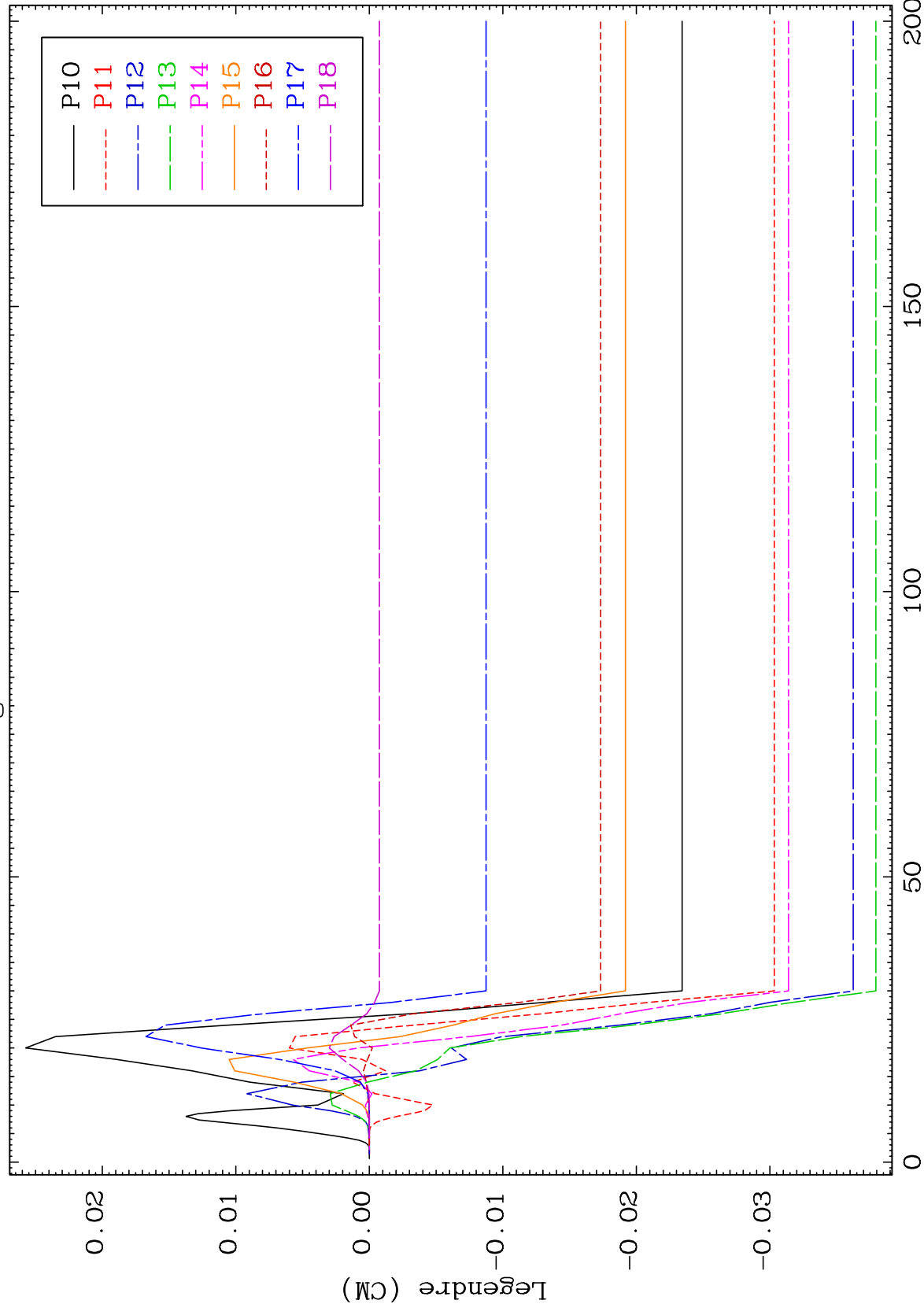




MAT 7401

MT= 51 (n, n') Level
Legendre Coefficients

74-W-172



21

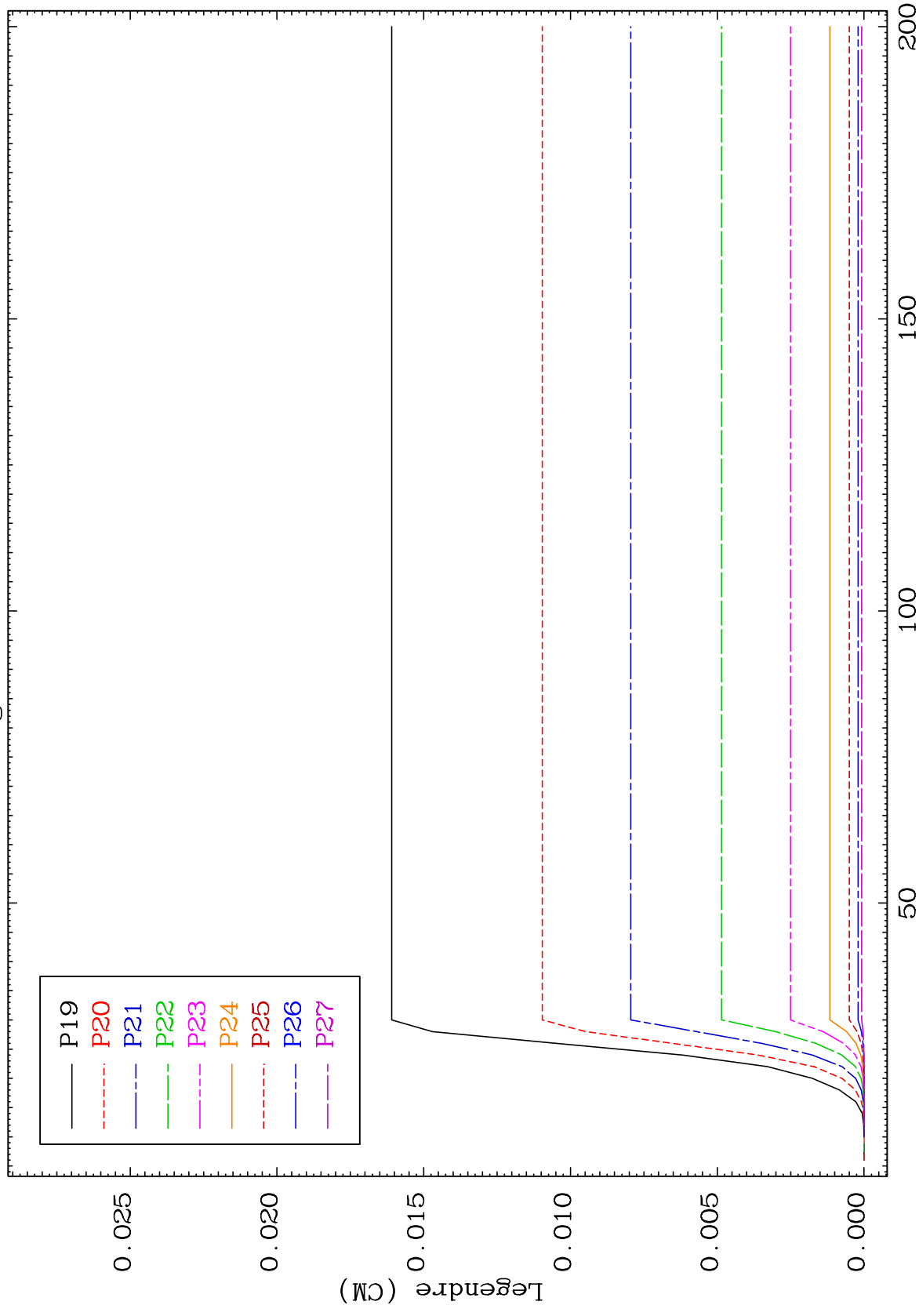
Incident Energy (MeV)

74-W -172

MAT 7401

MT= 51 (n, n') Level
Legendre Coefficients

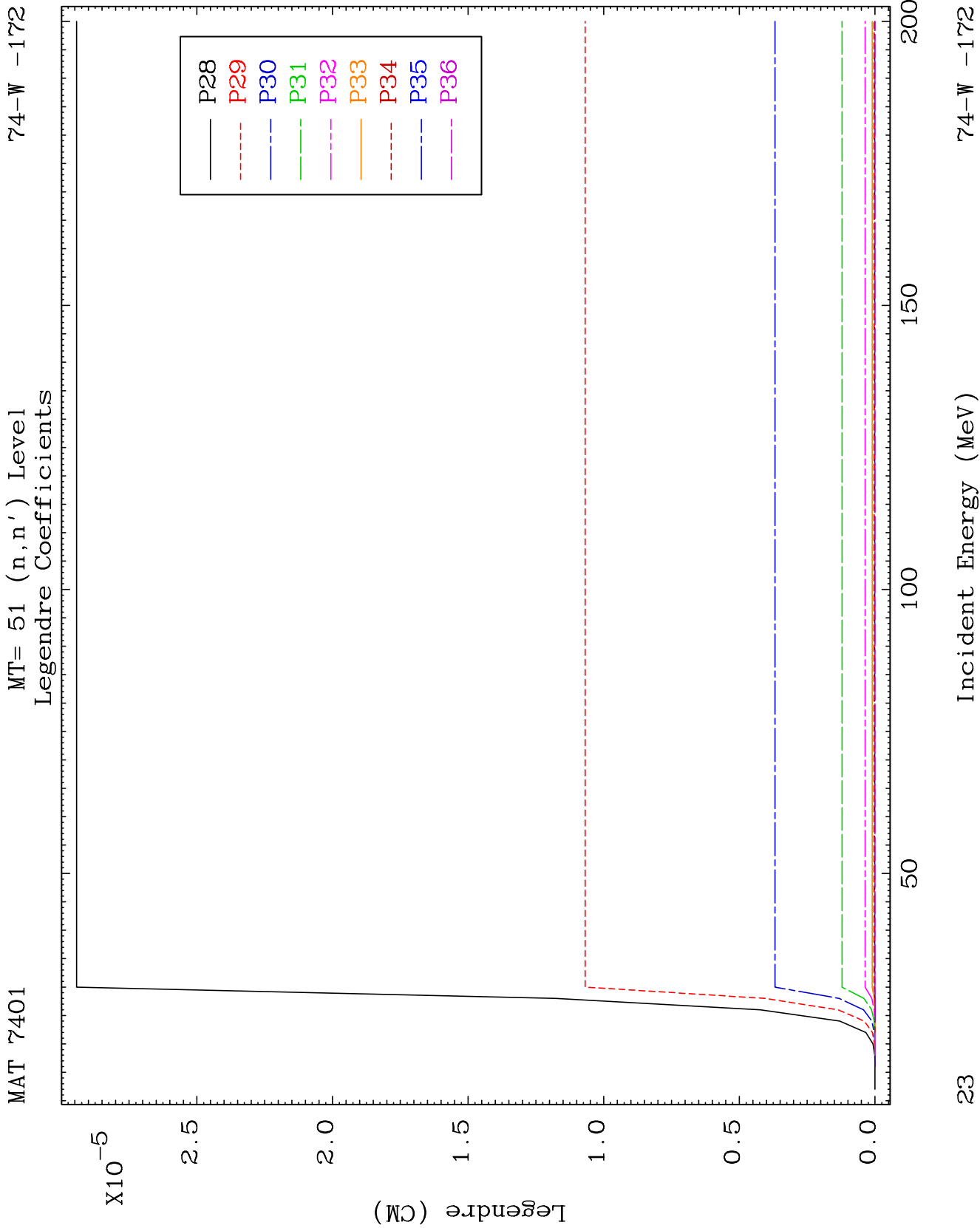
24-W -172

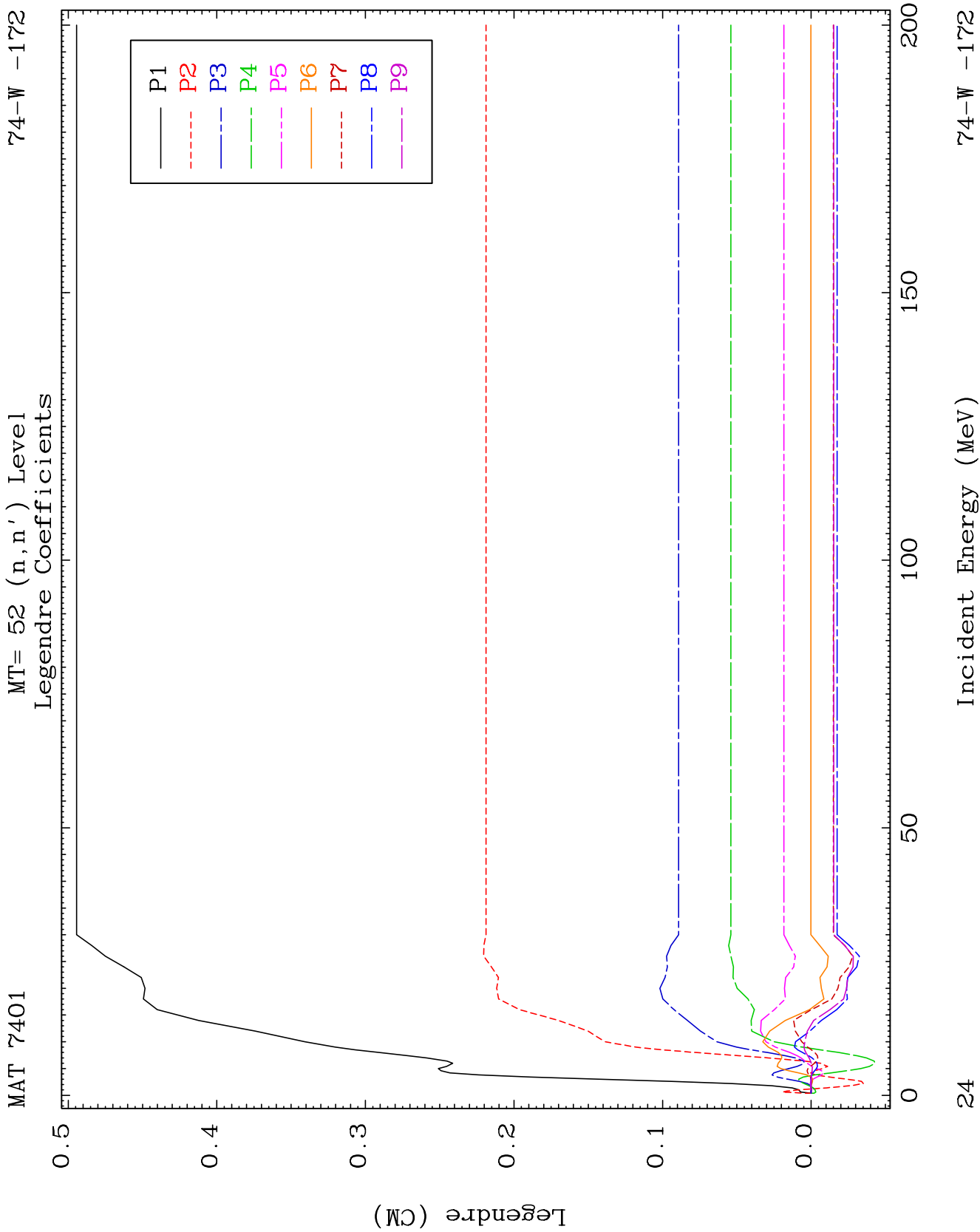


22

Incident Energy (MeV)

74-W-172

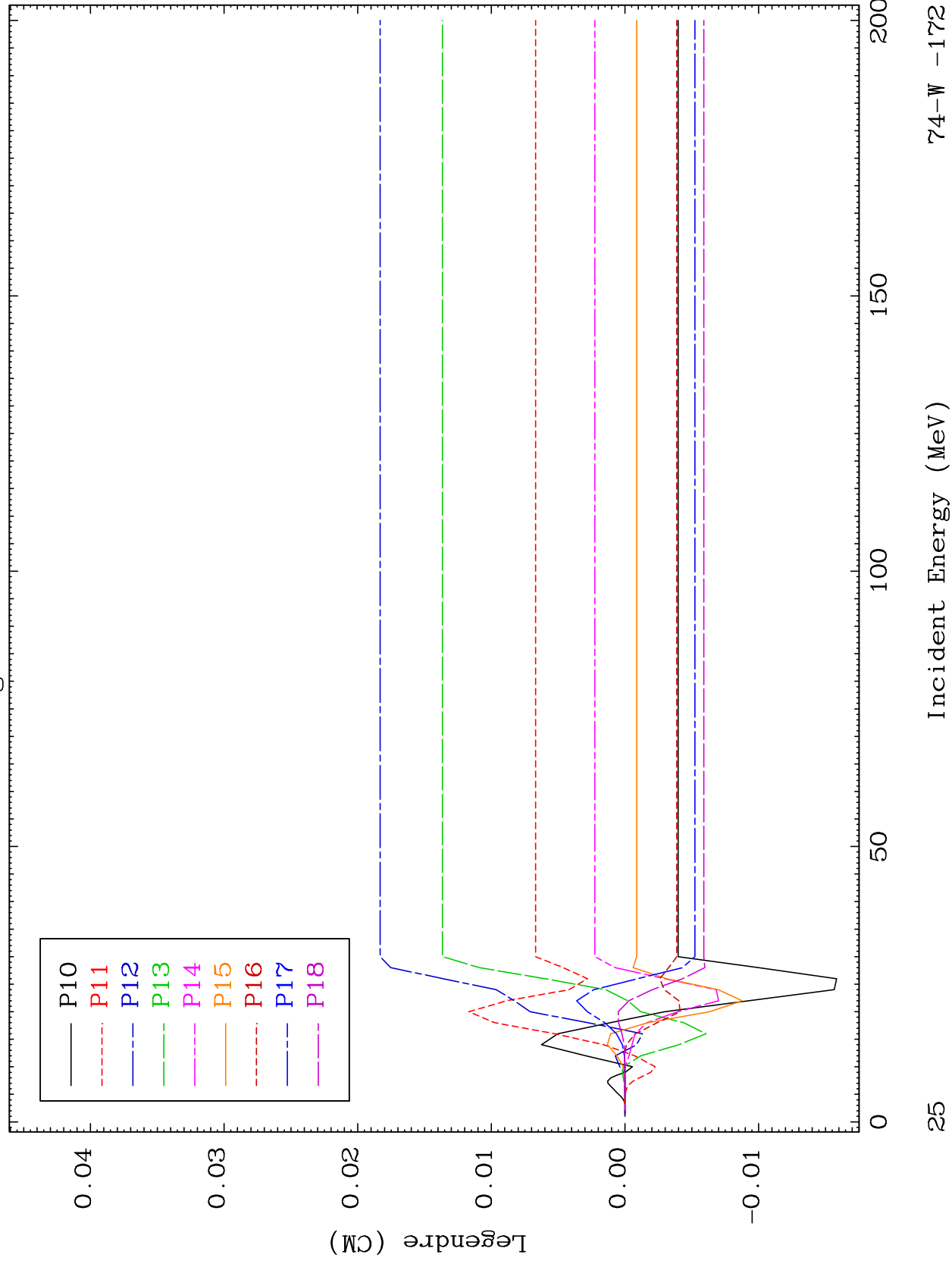


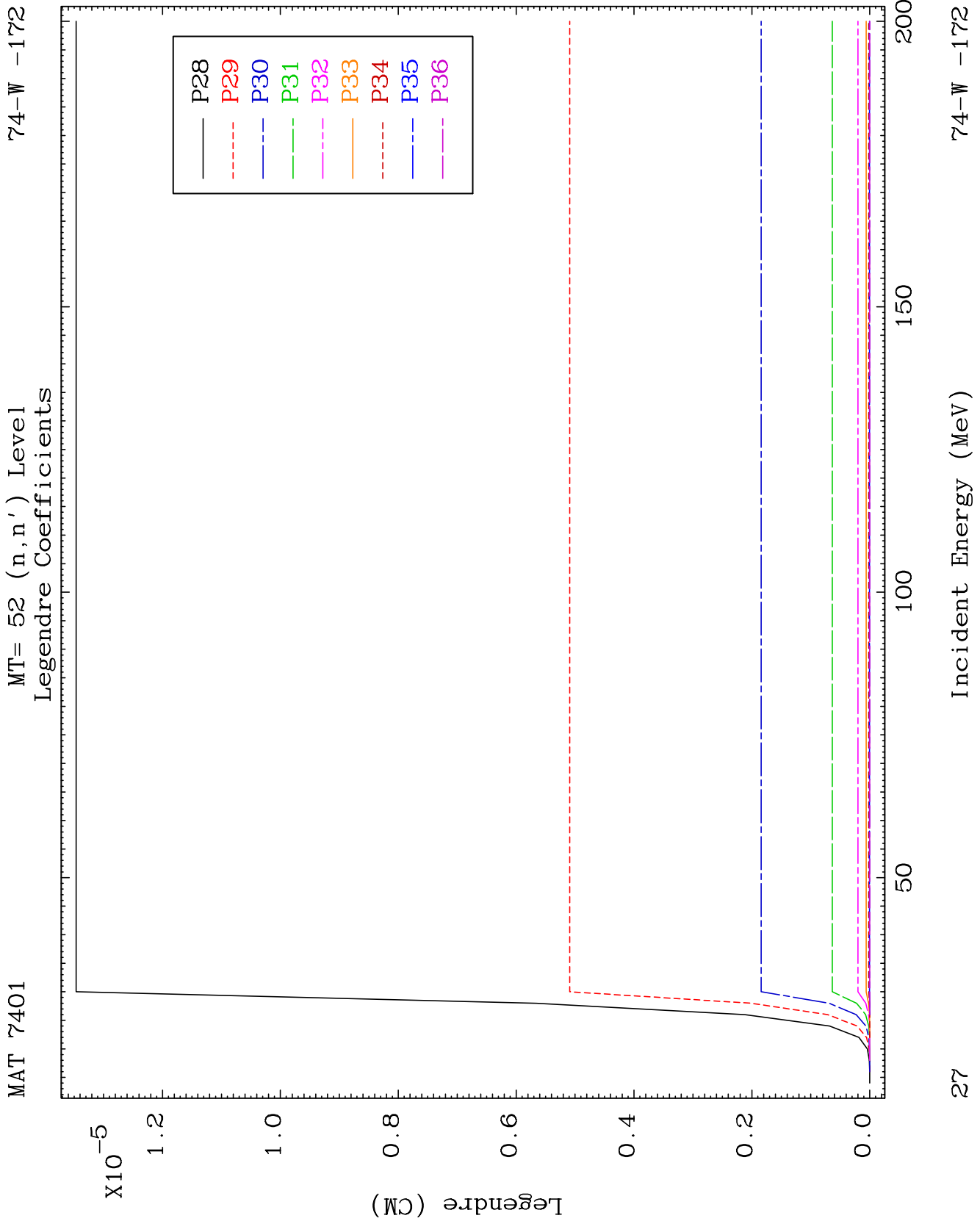


MAT 7401

MT= 52 (n, n') Level
Legendre Coefficients

24-W -172

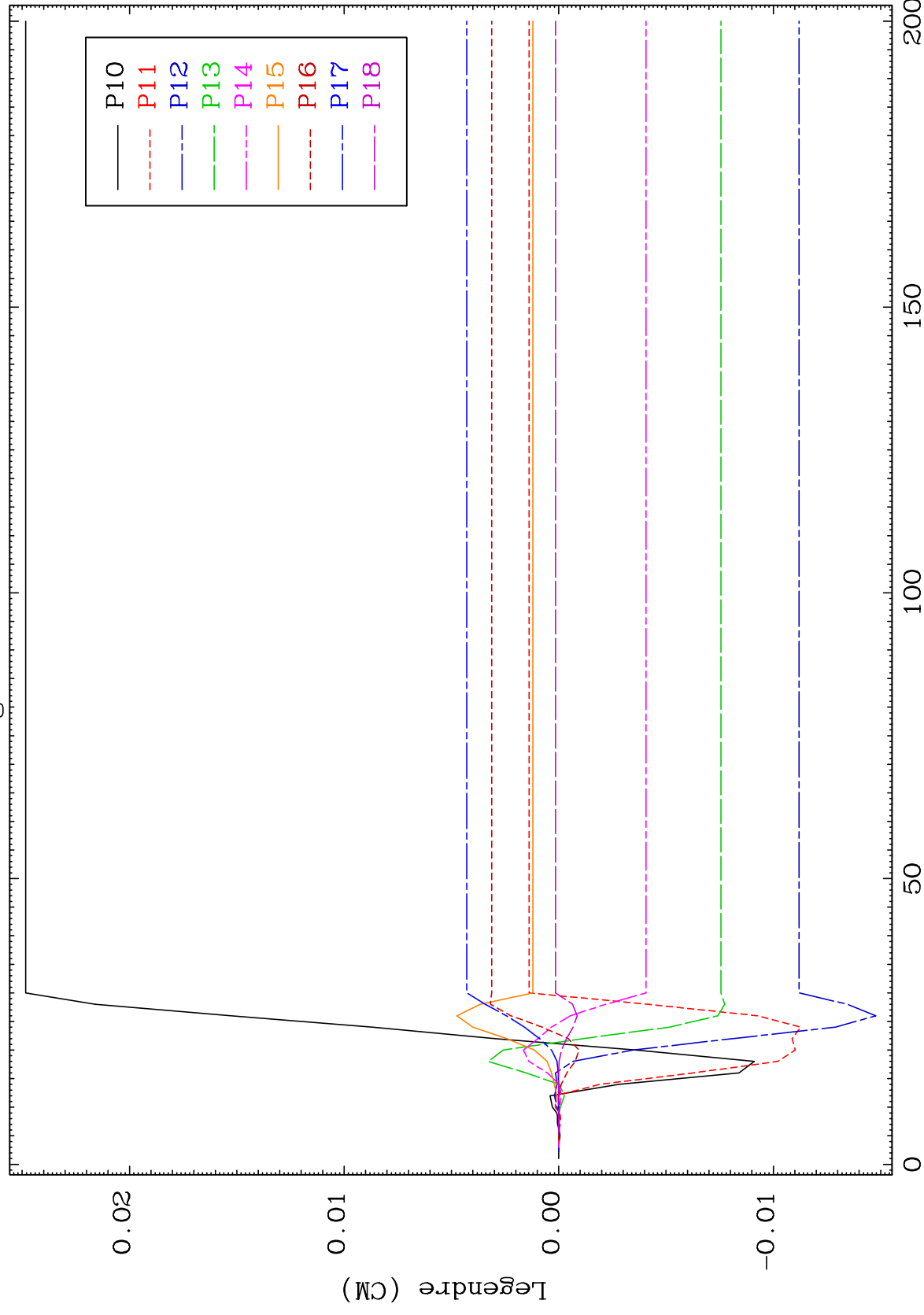




MAT 7401

MT= 53 (n, n') Level
Legendre Coefficients

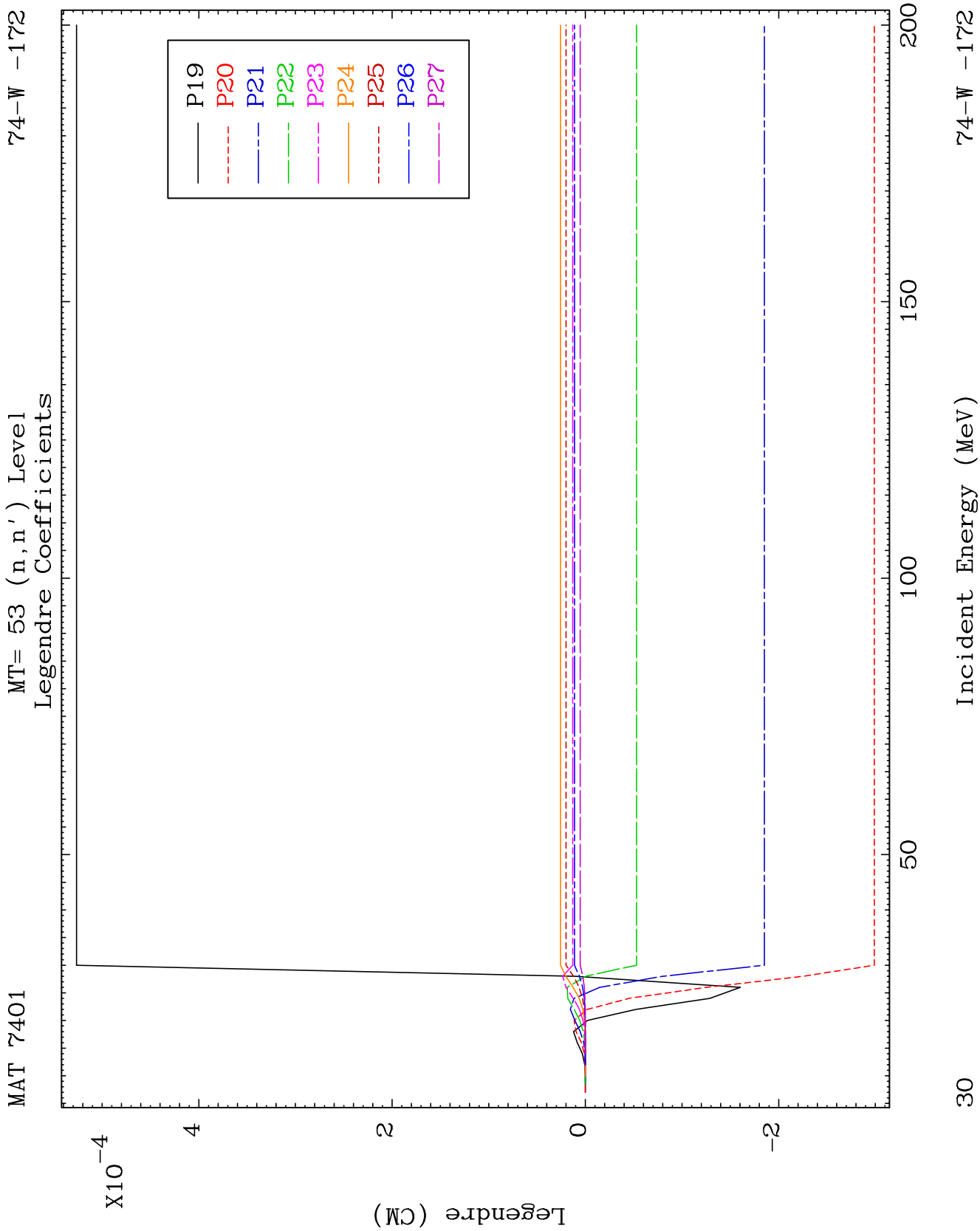
24-W -172

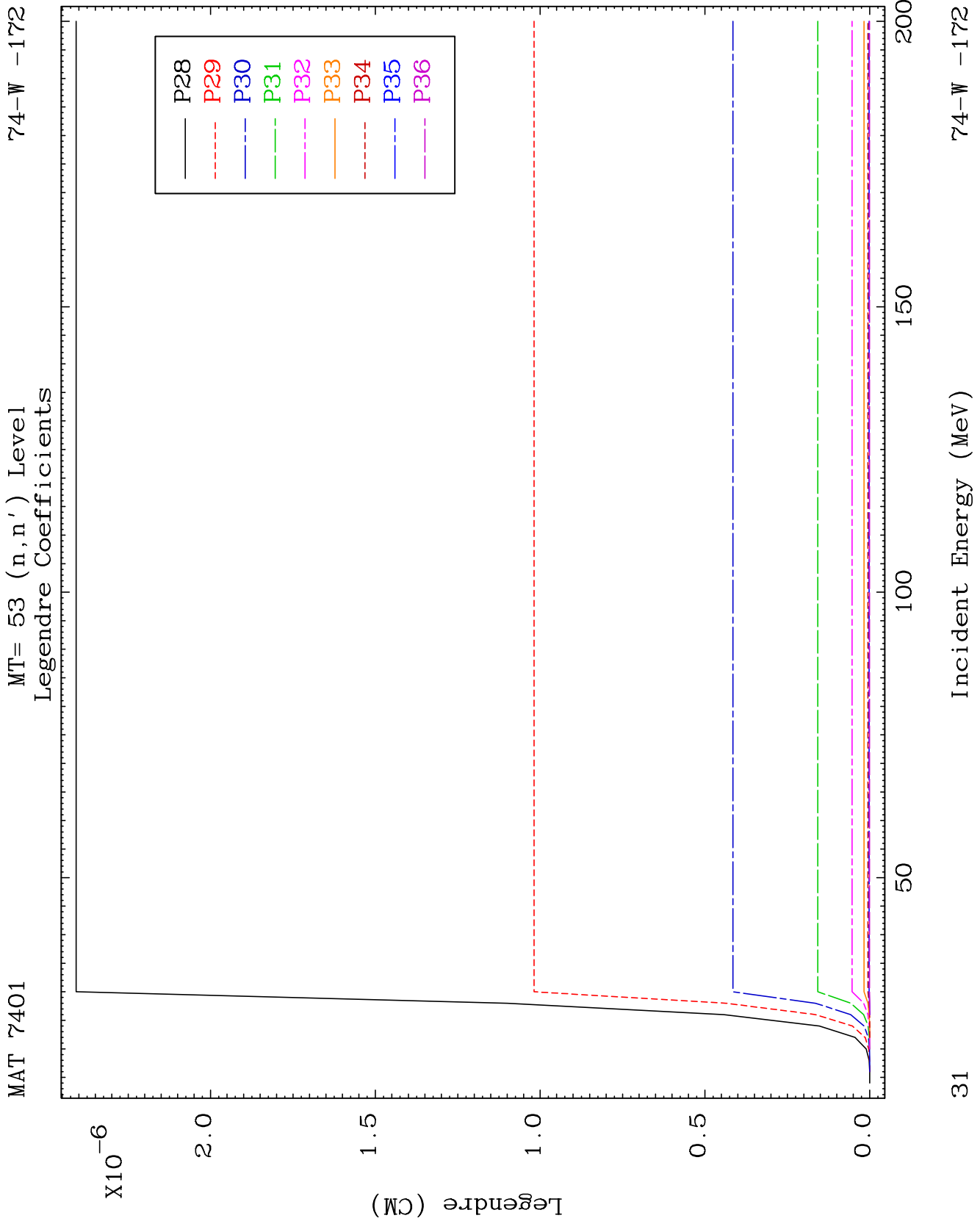


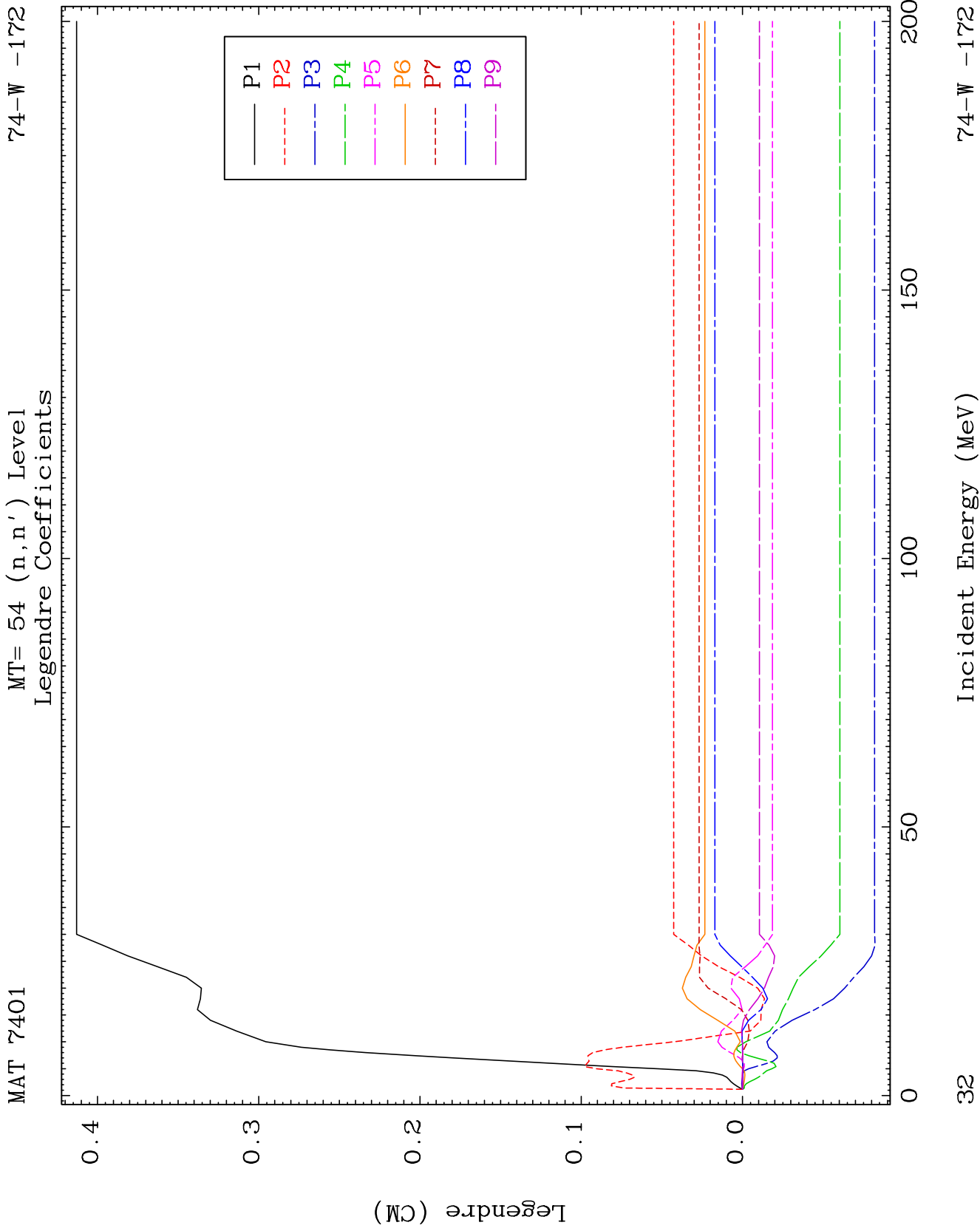
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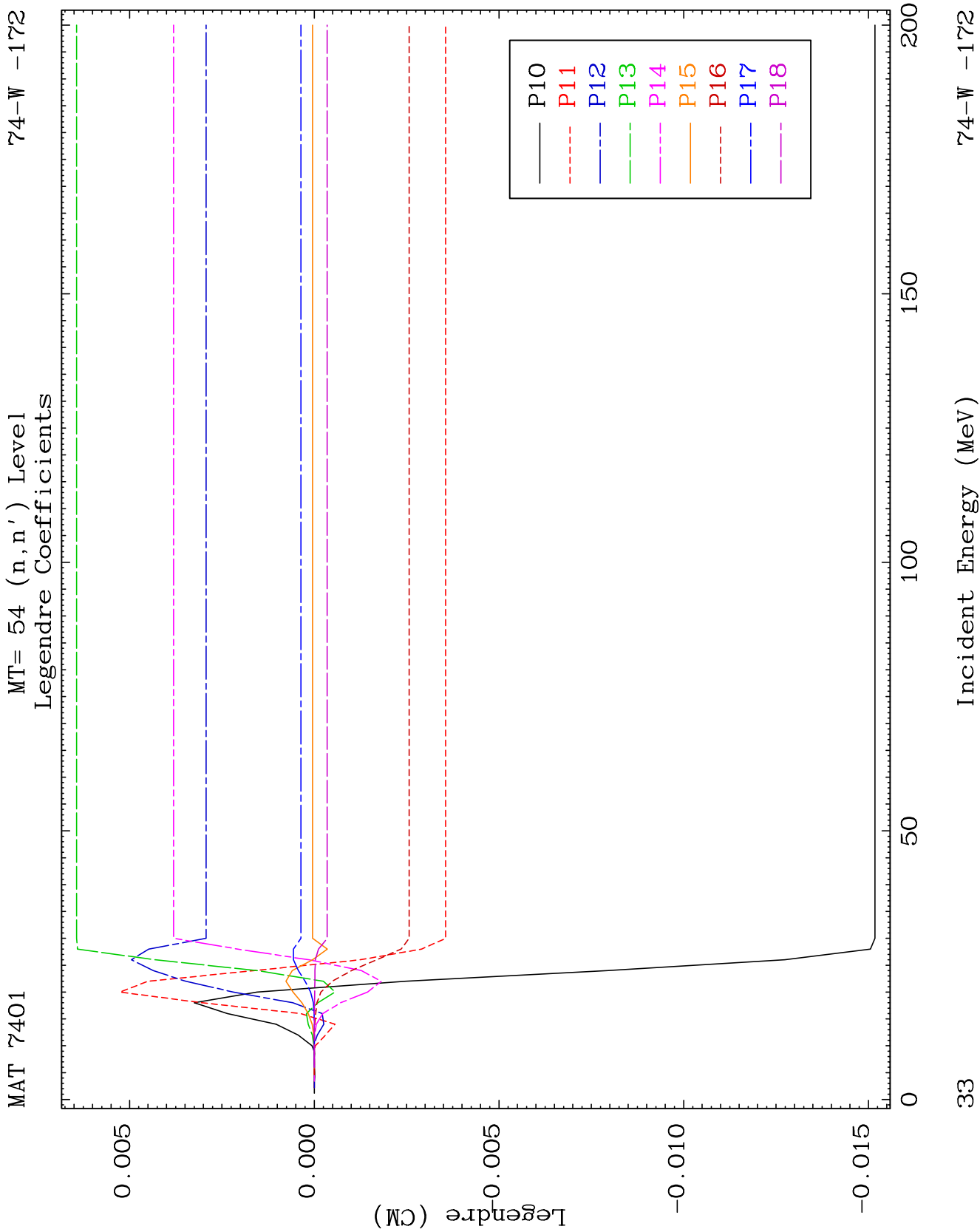
Incident Energy (MeV)

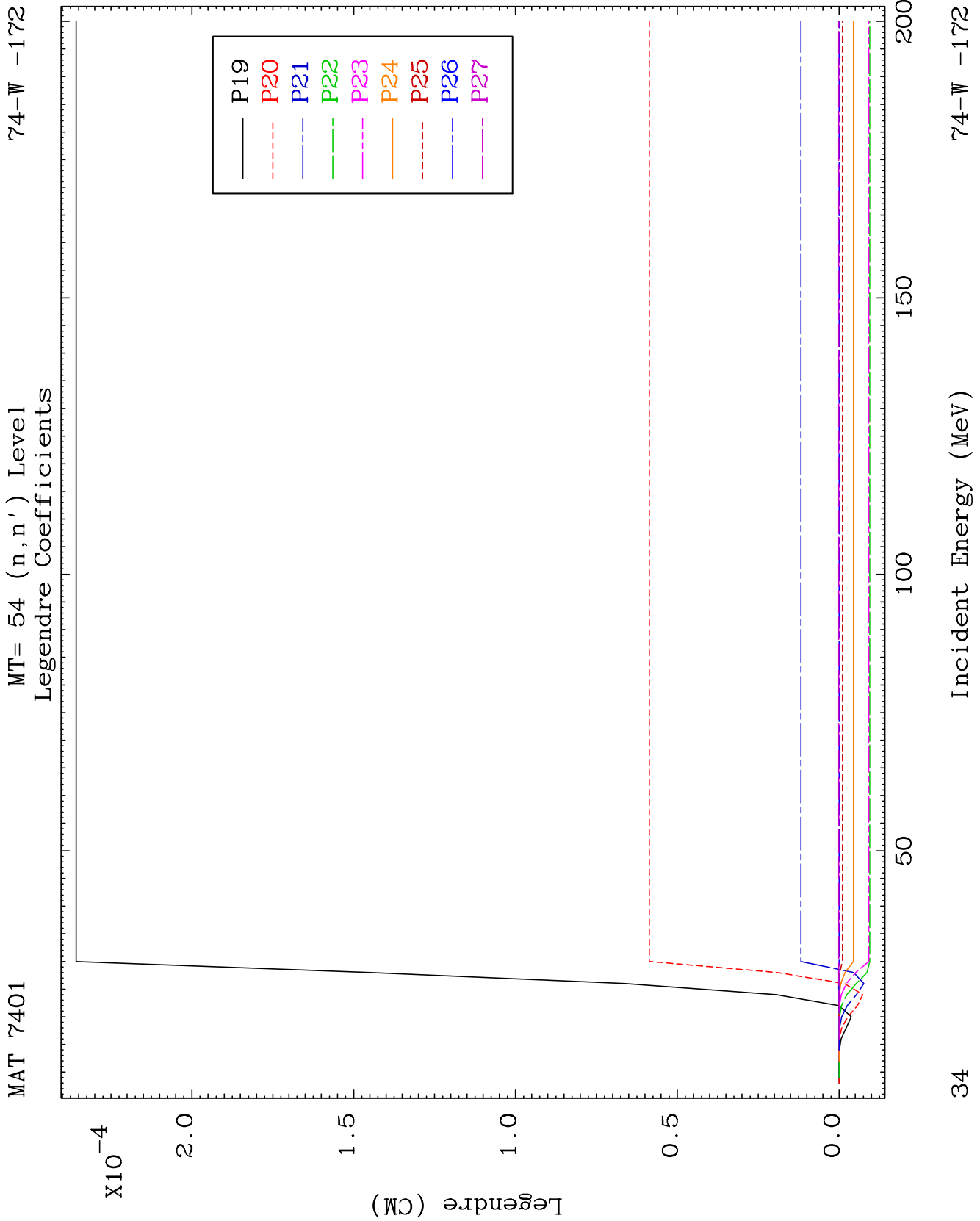
74-W -172

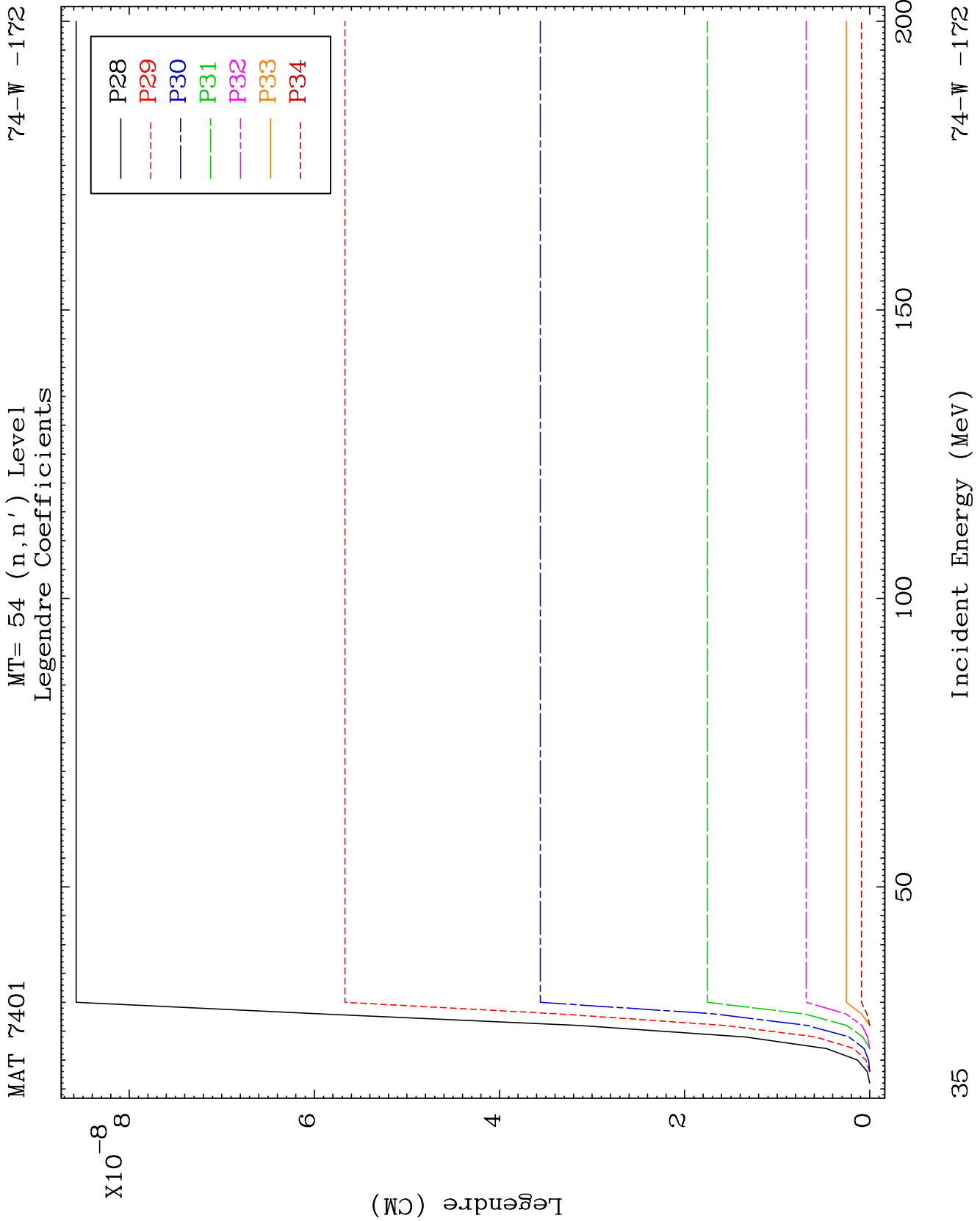


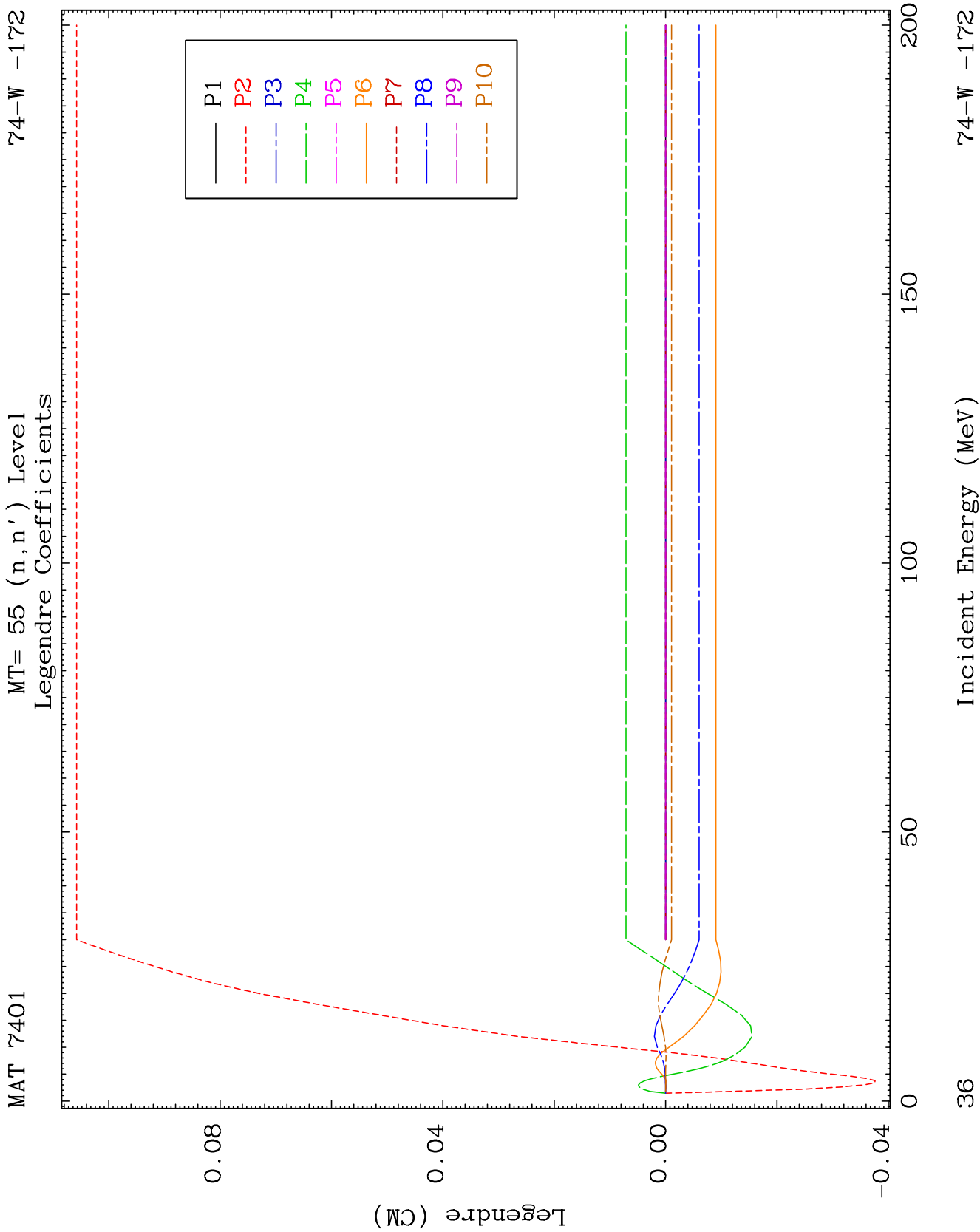


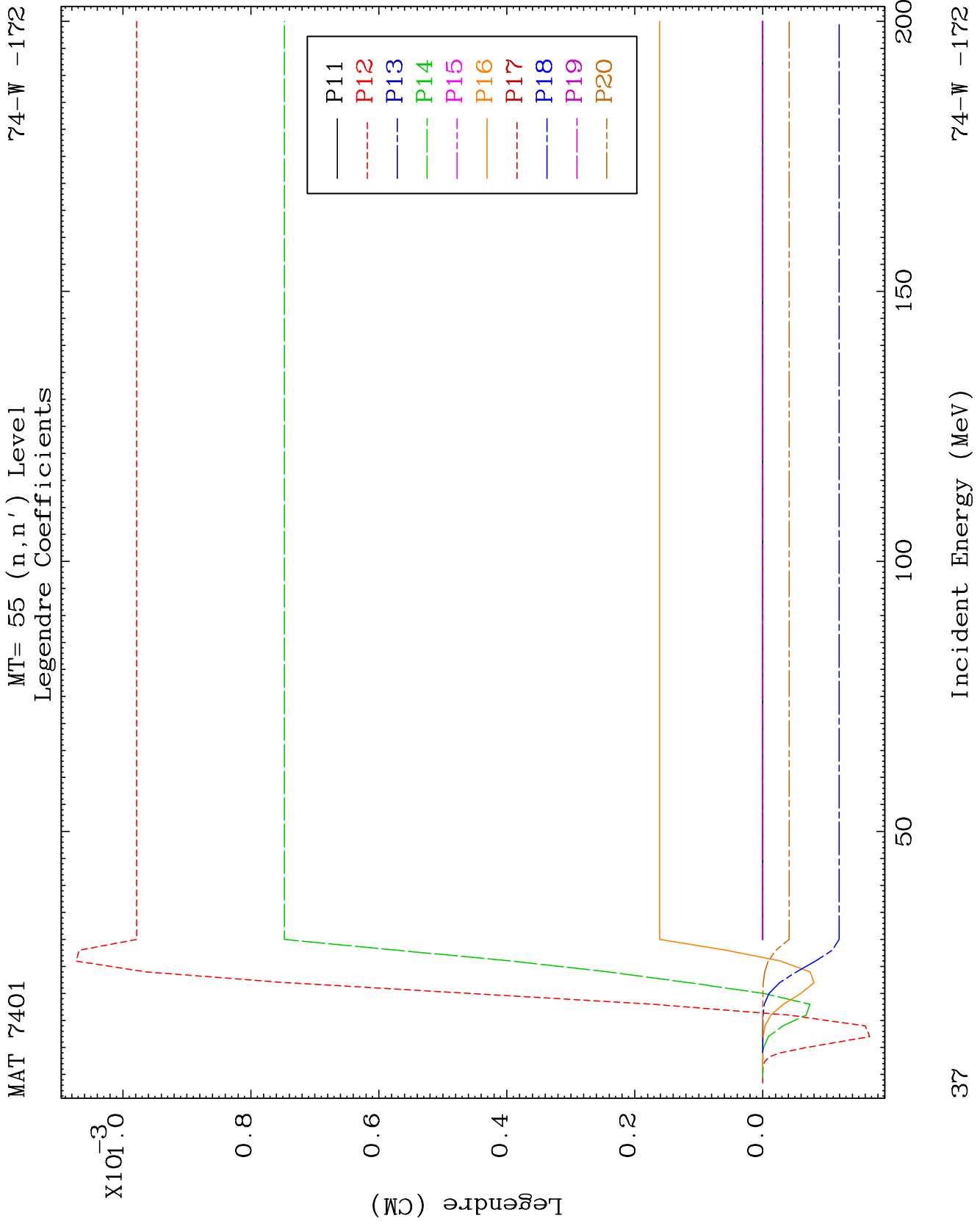


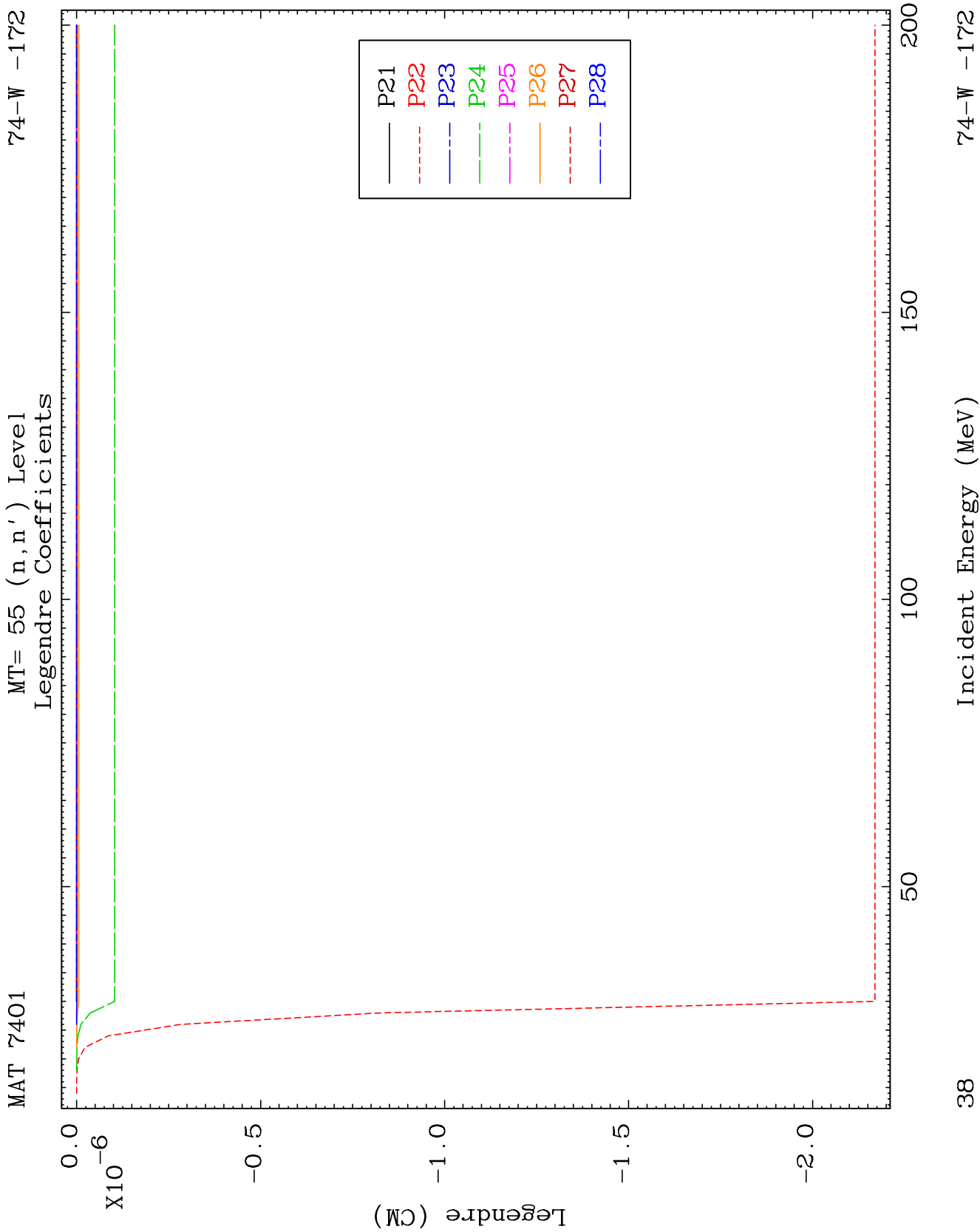








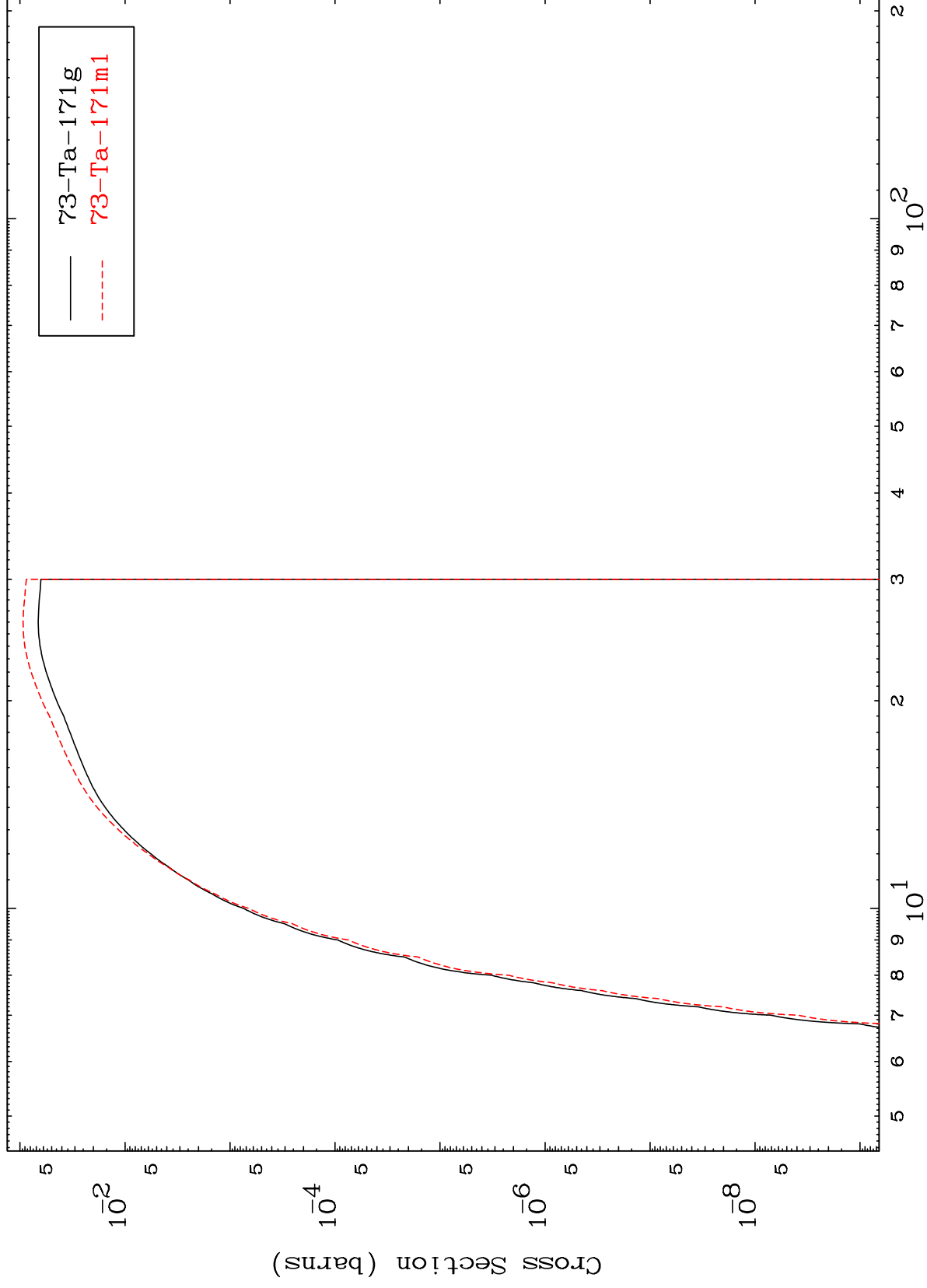




MAT 7401

74-W -172

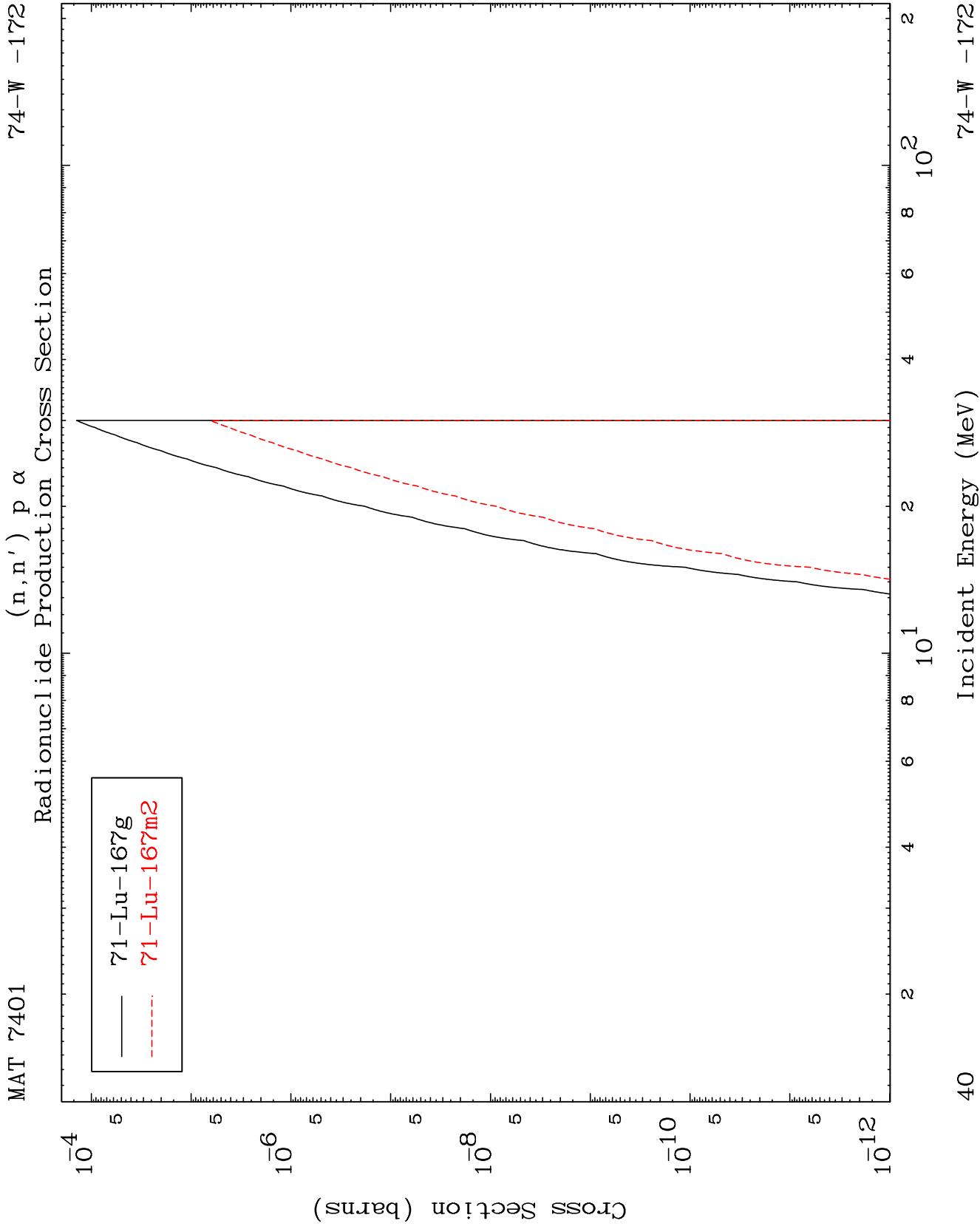
(n,n') p
Radionuclide Production Cross Section



39

Incident Energy (MeV)

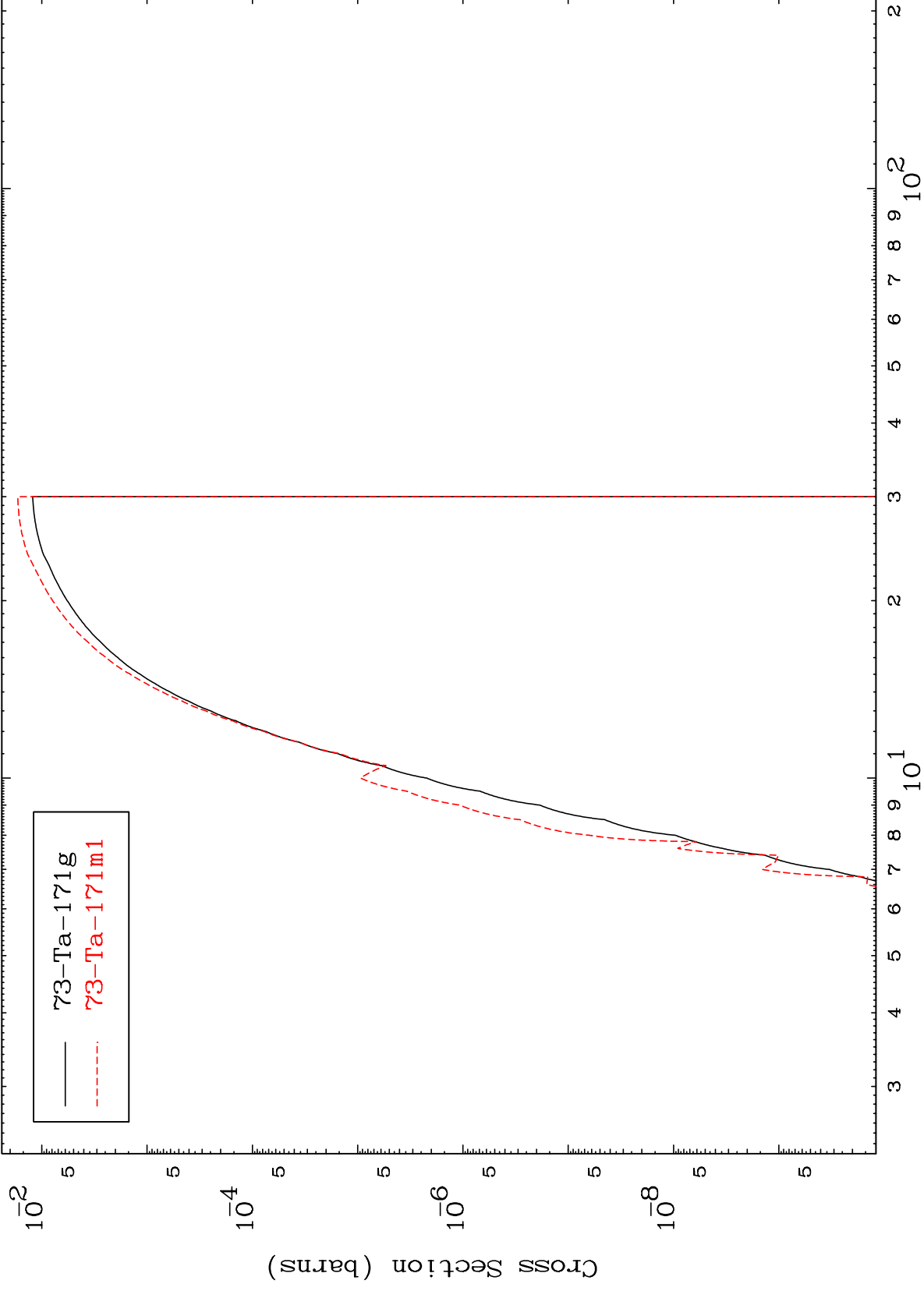
74-W -172



MAT 7401

74-W -172

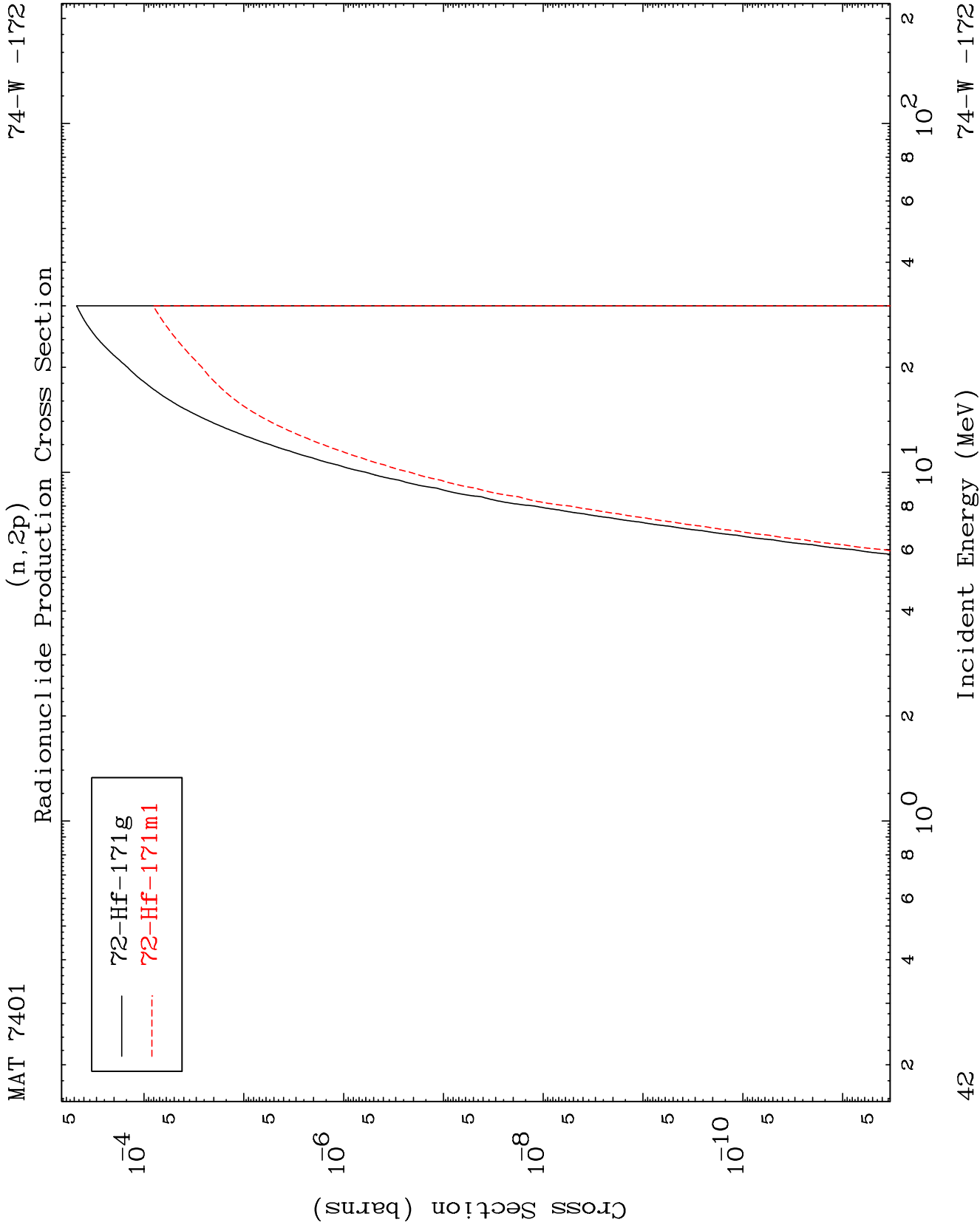
Radionuclide Production Cross Section
(n,d)



41

Incident Energy (MeV)

74-W -172

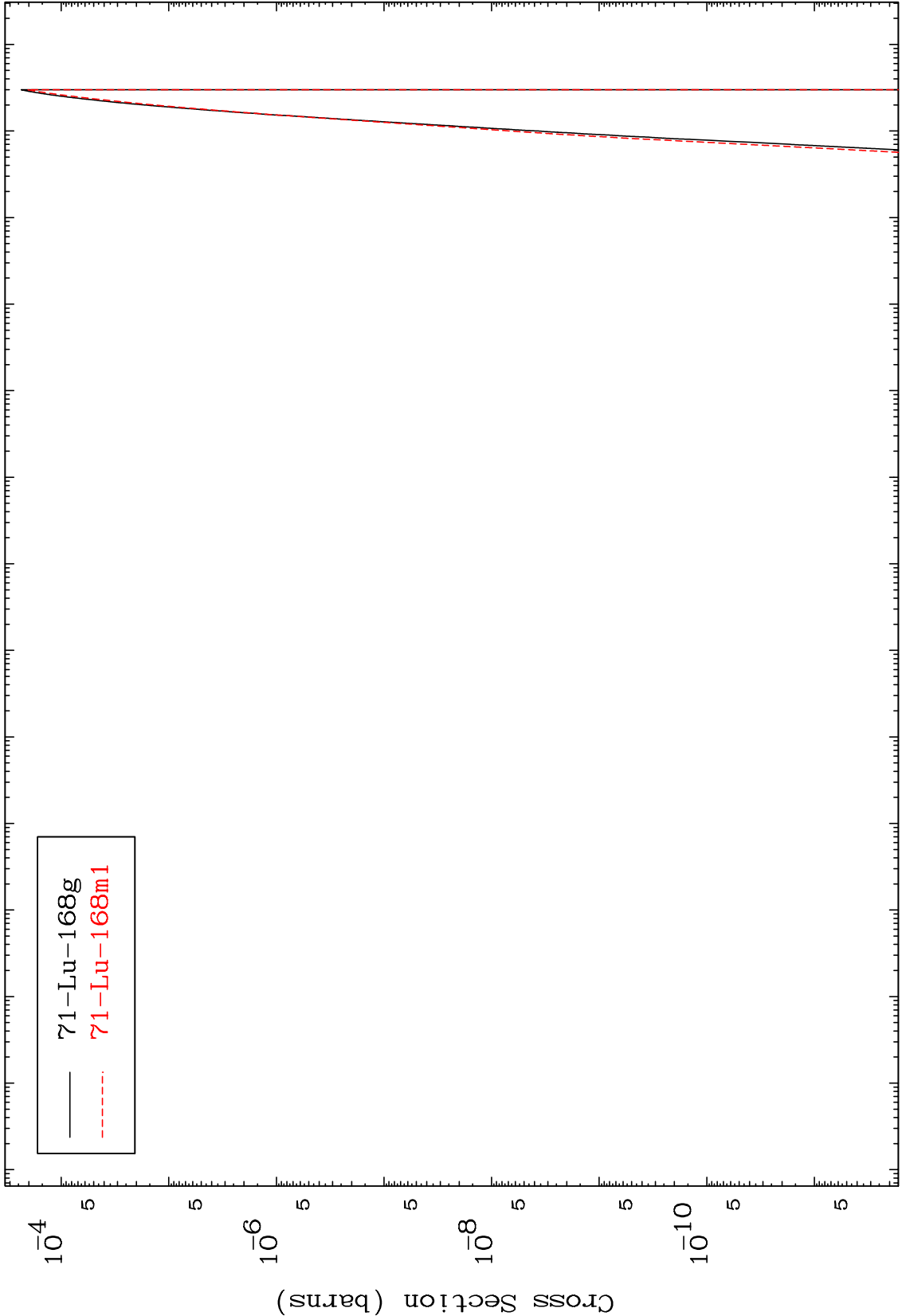


MAT 7401

(n,p) α

74-W -172

Radionuclide Production Cross Section



43

Incident Energy (MeV)

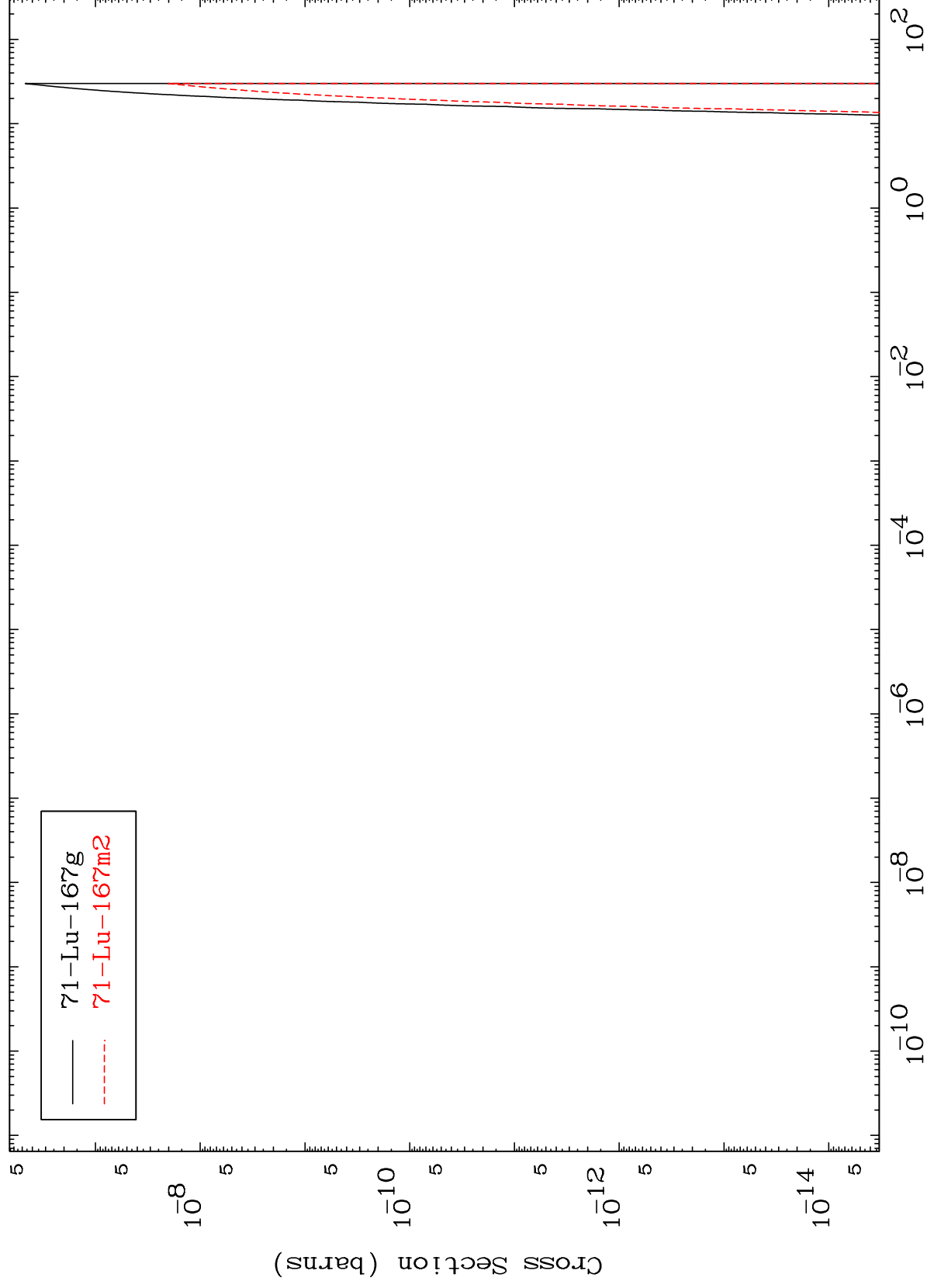
74-W -172

MAT 7401

(n,d) α

74-W -172

Radionuclide Production Cross Section



44

Incident Energy (MeV)

74-W -172