

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

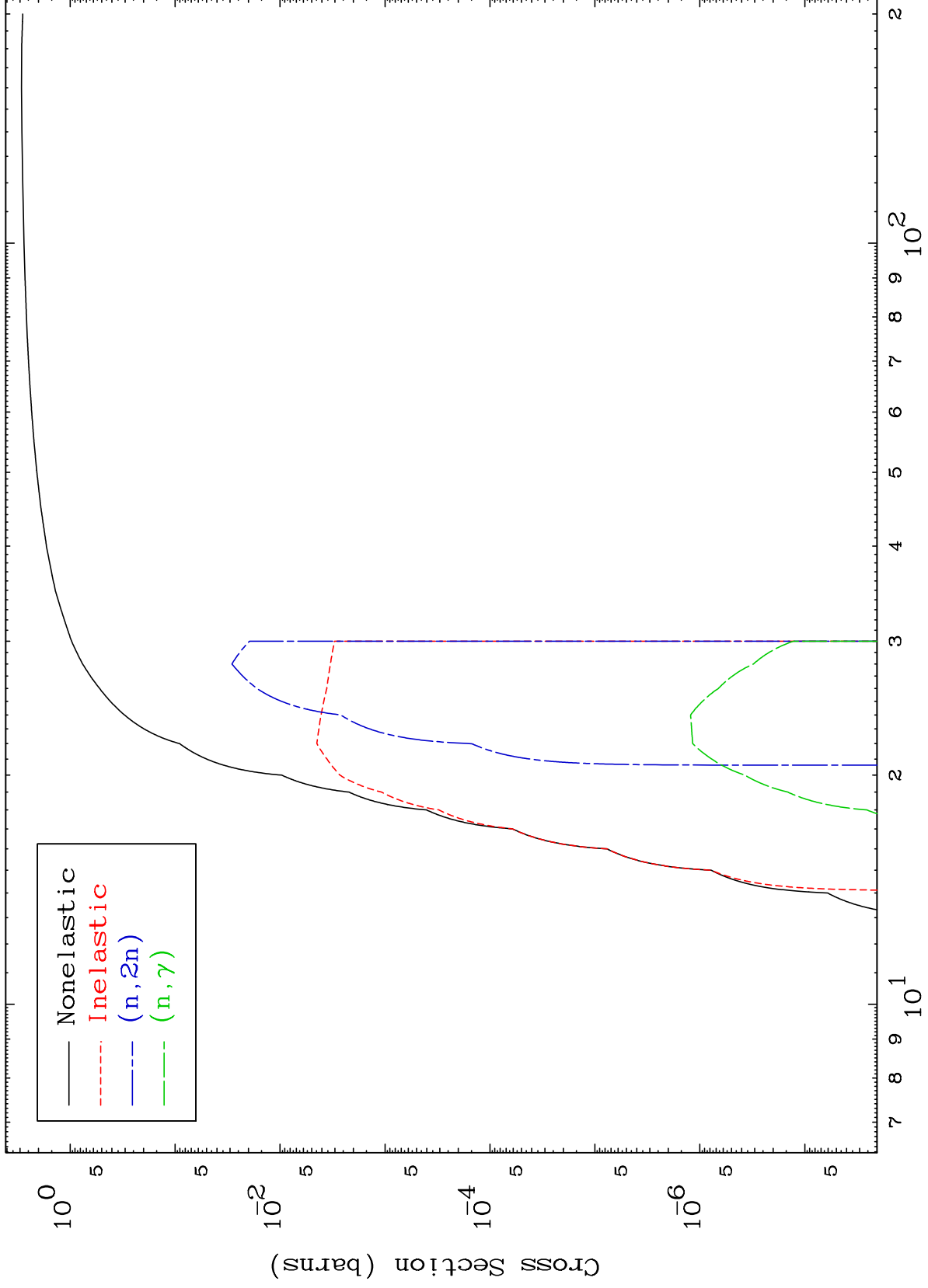
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Press Mouse Button to Start

MAT 8919

$\alpha$  Major  
0 Kelvin Cross Sections

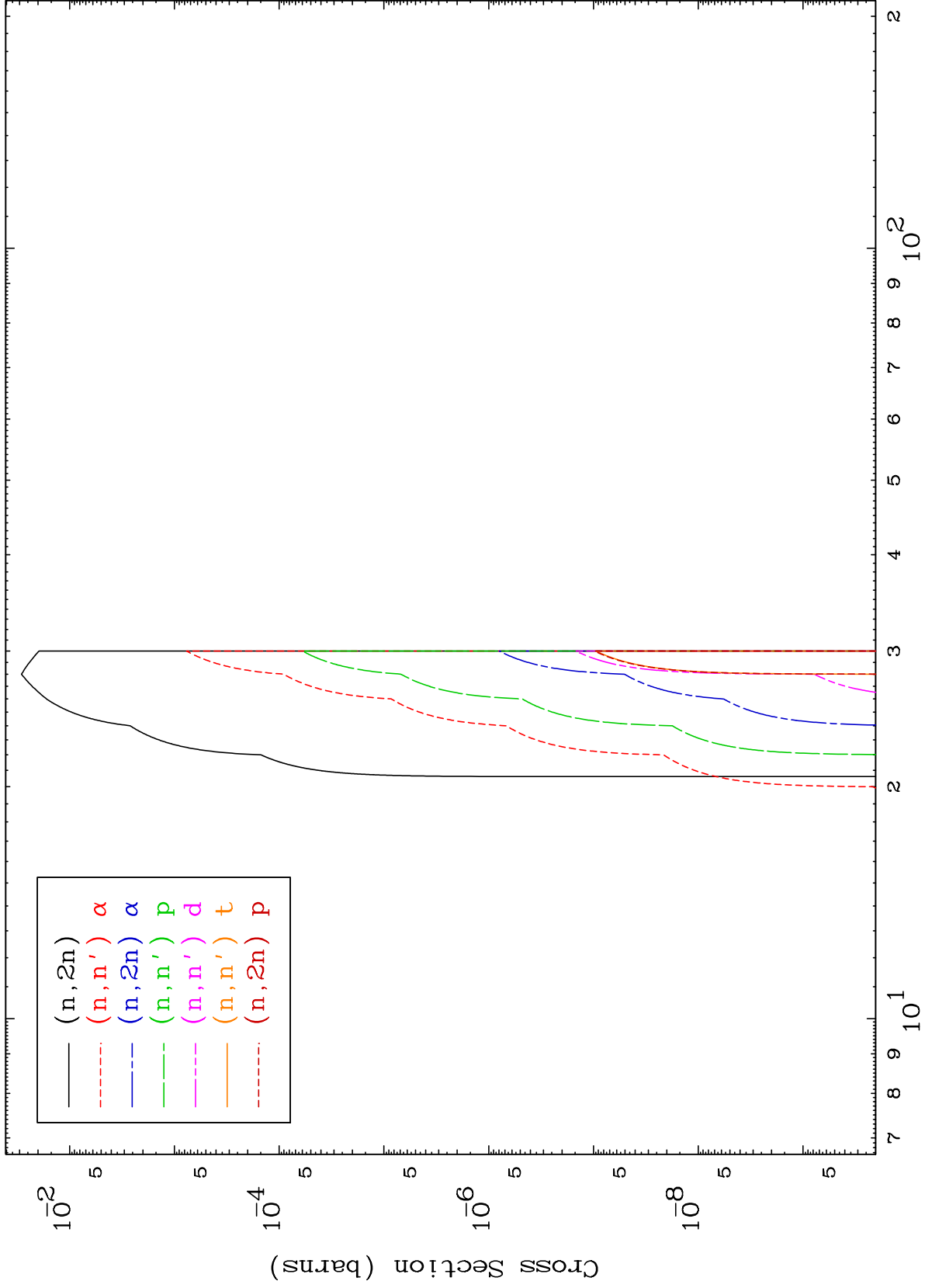
89-Ac-223

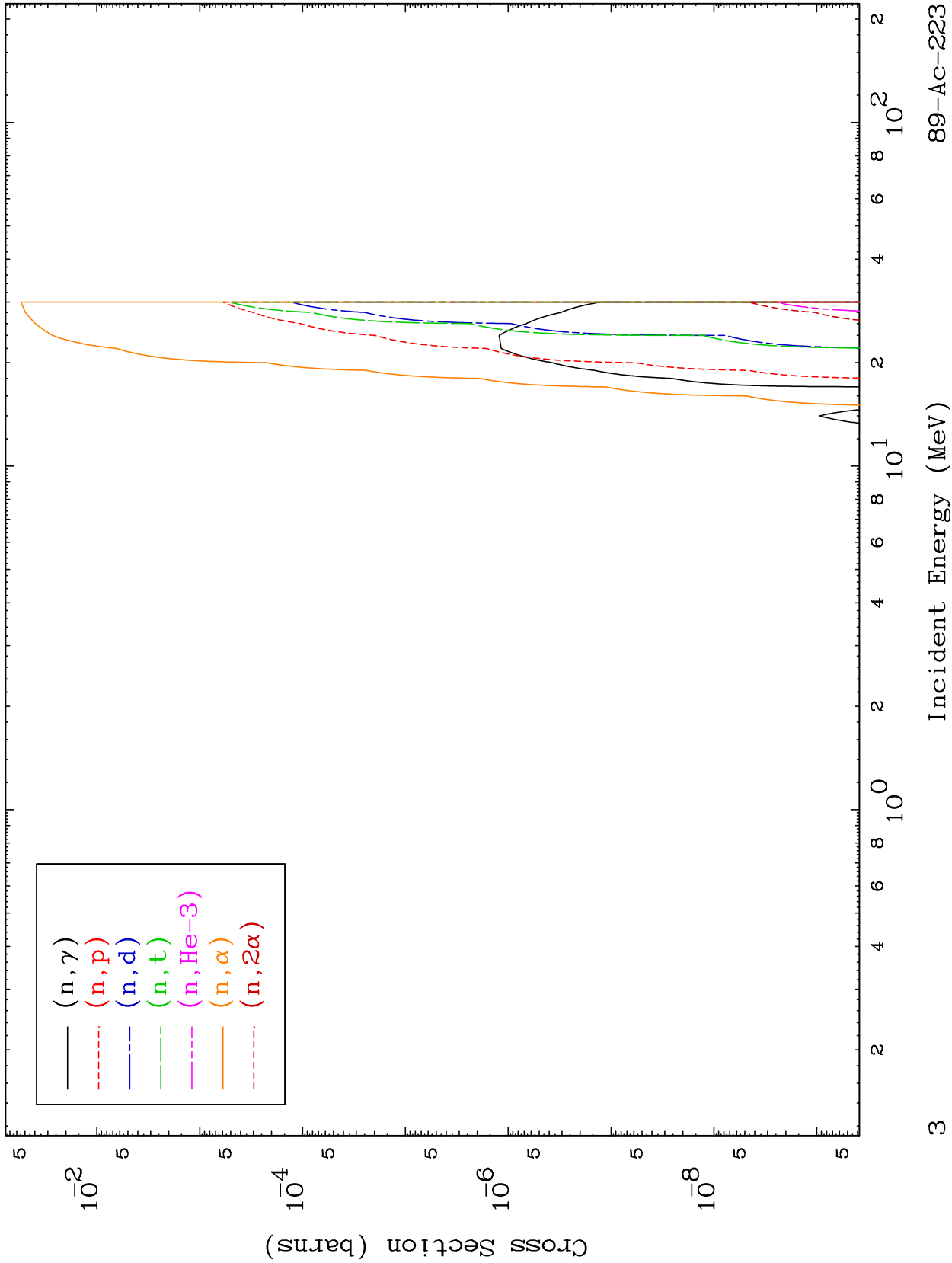


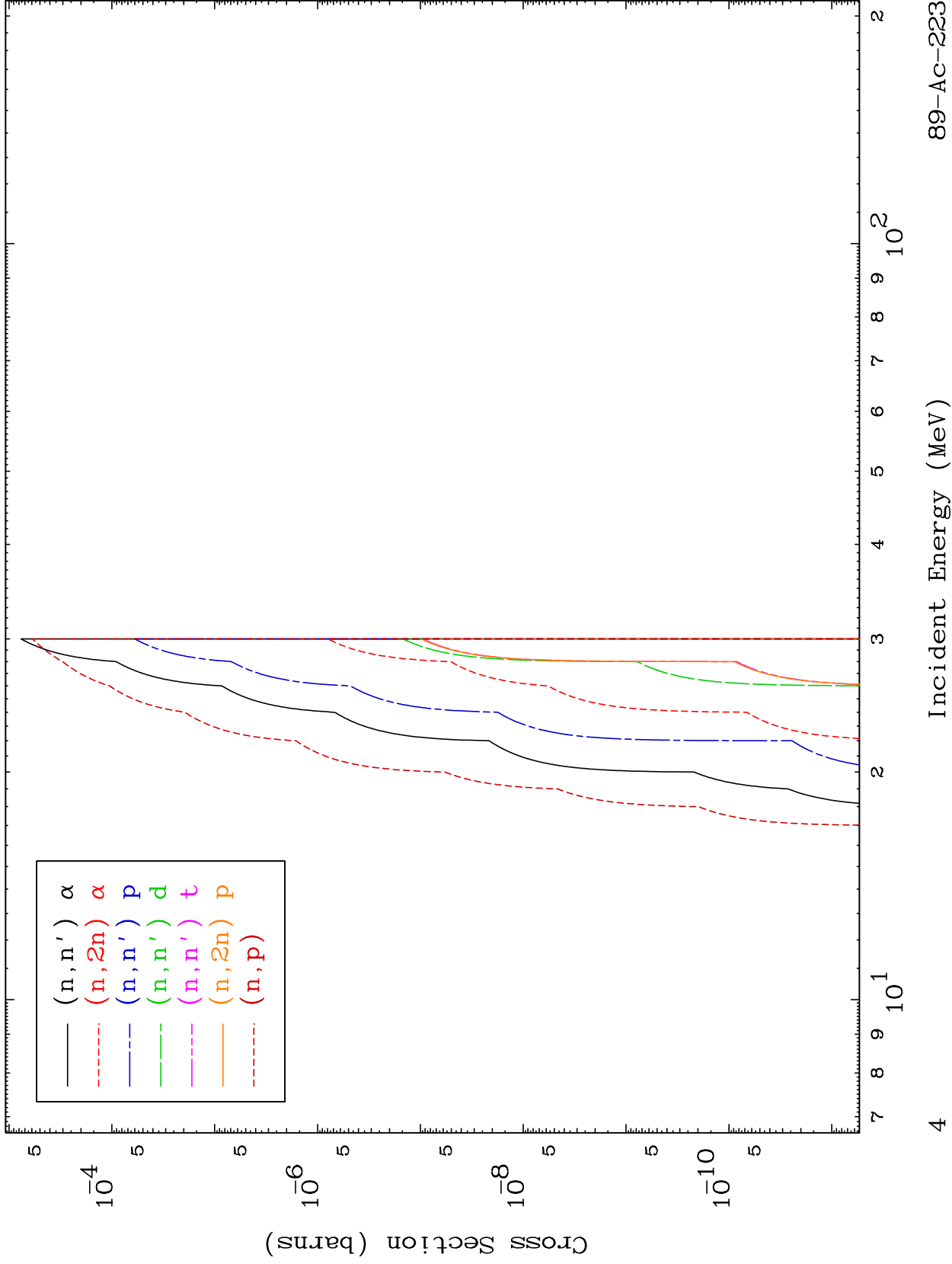
1

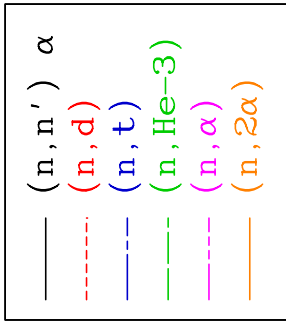
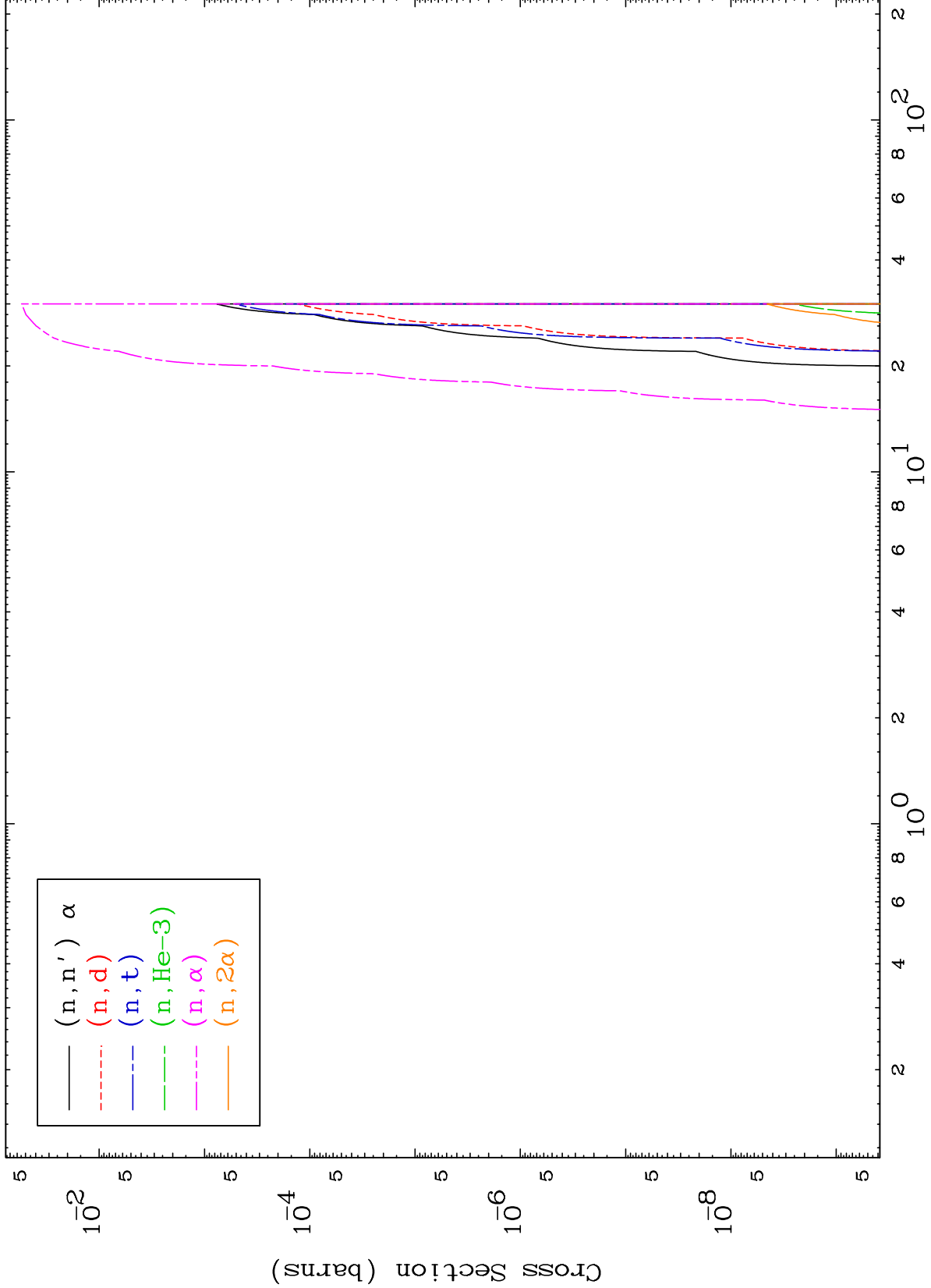
Incident Energy (MeV)

89-Ac-223





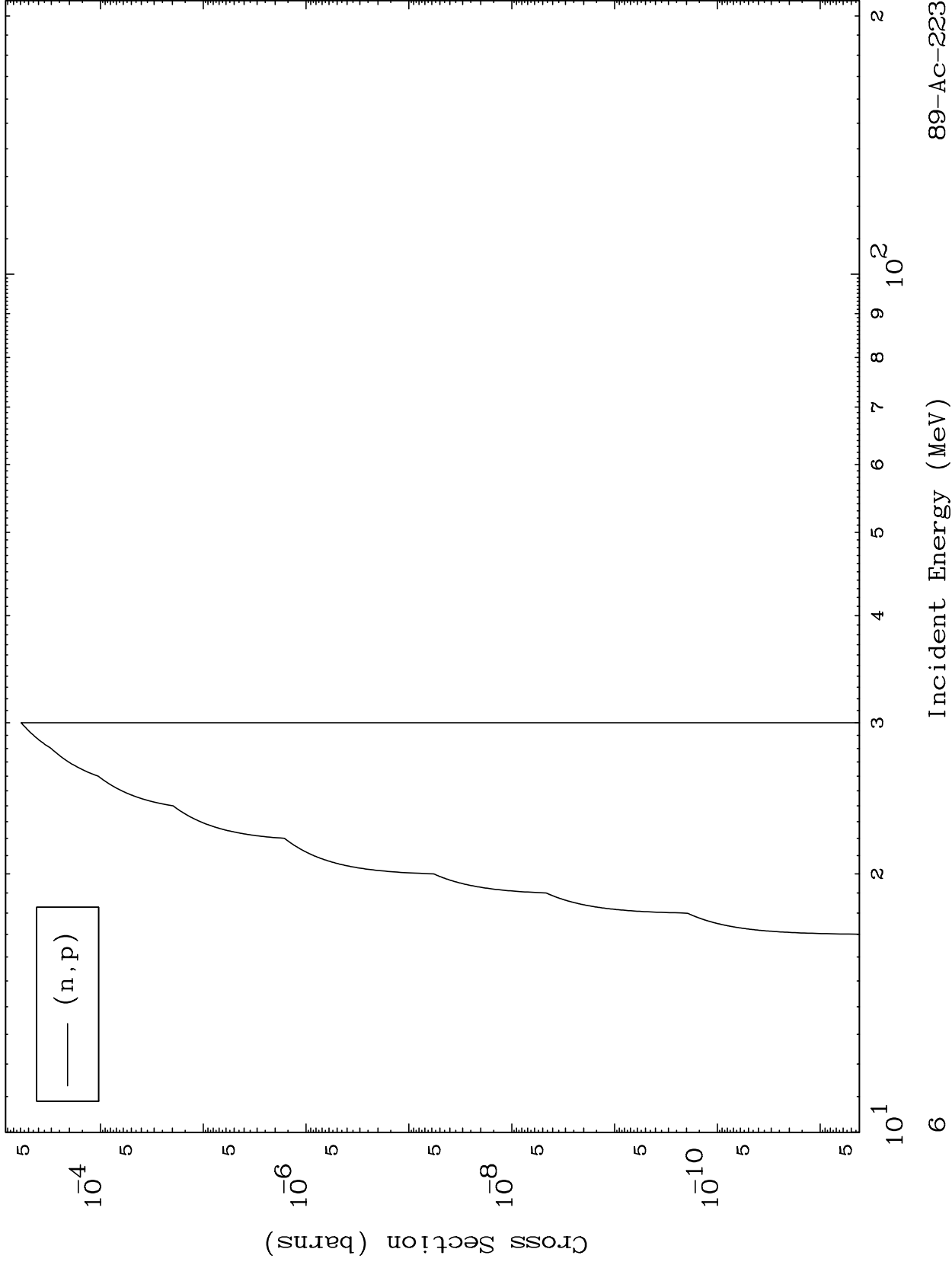




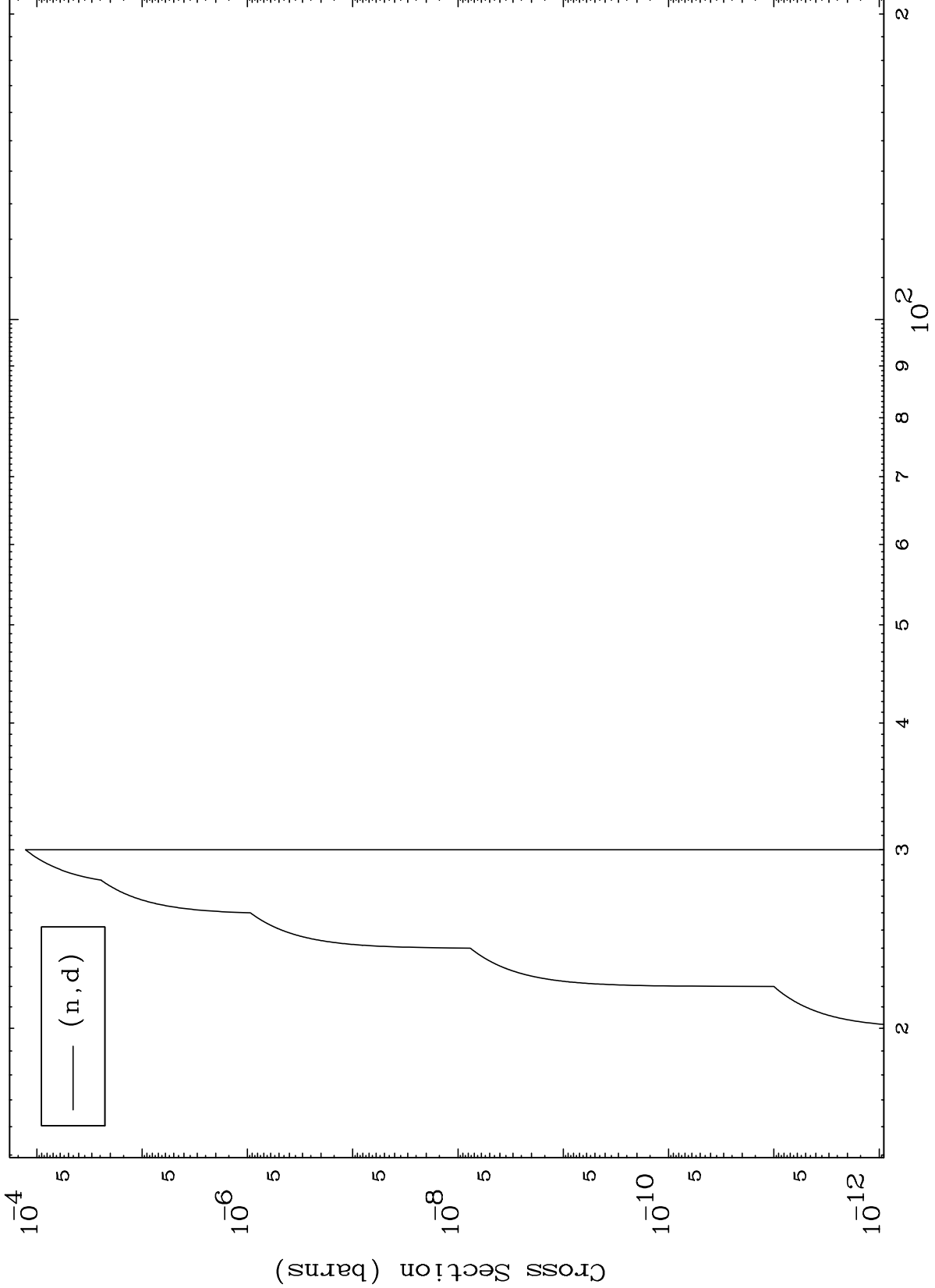
MAT 8919

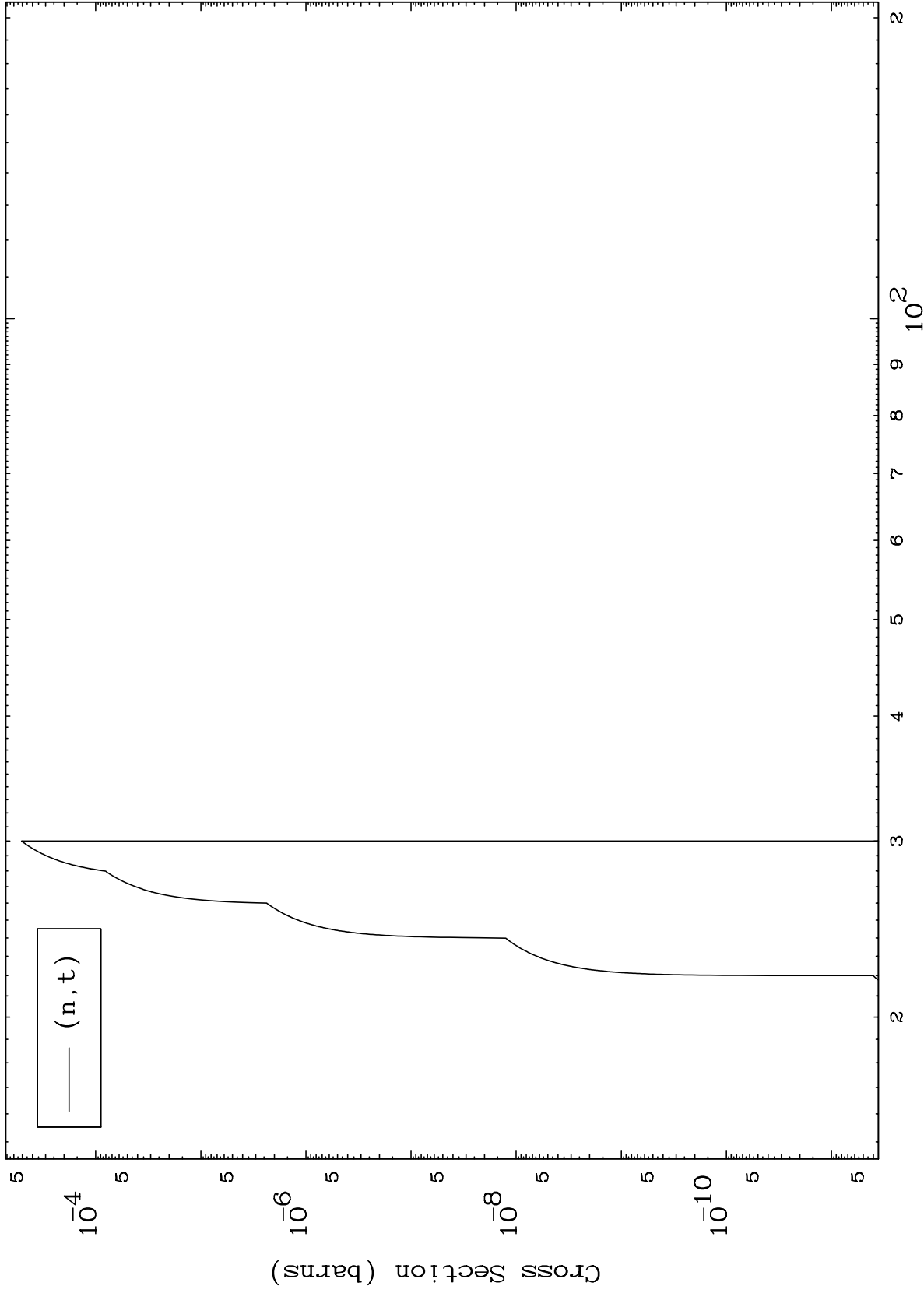
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

89-Ac-223

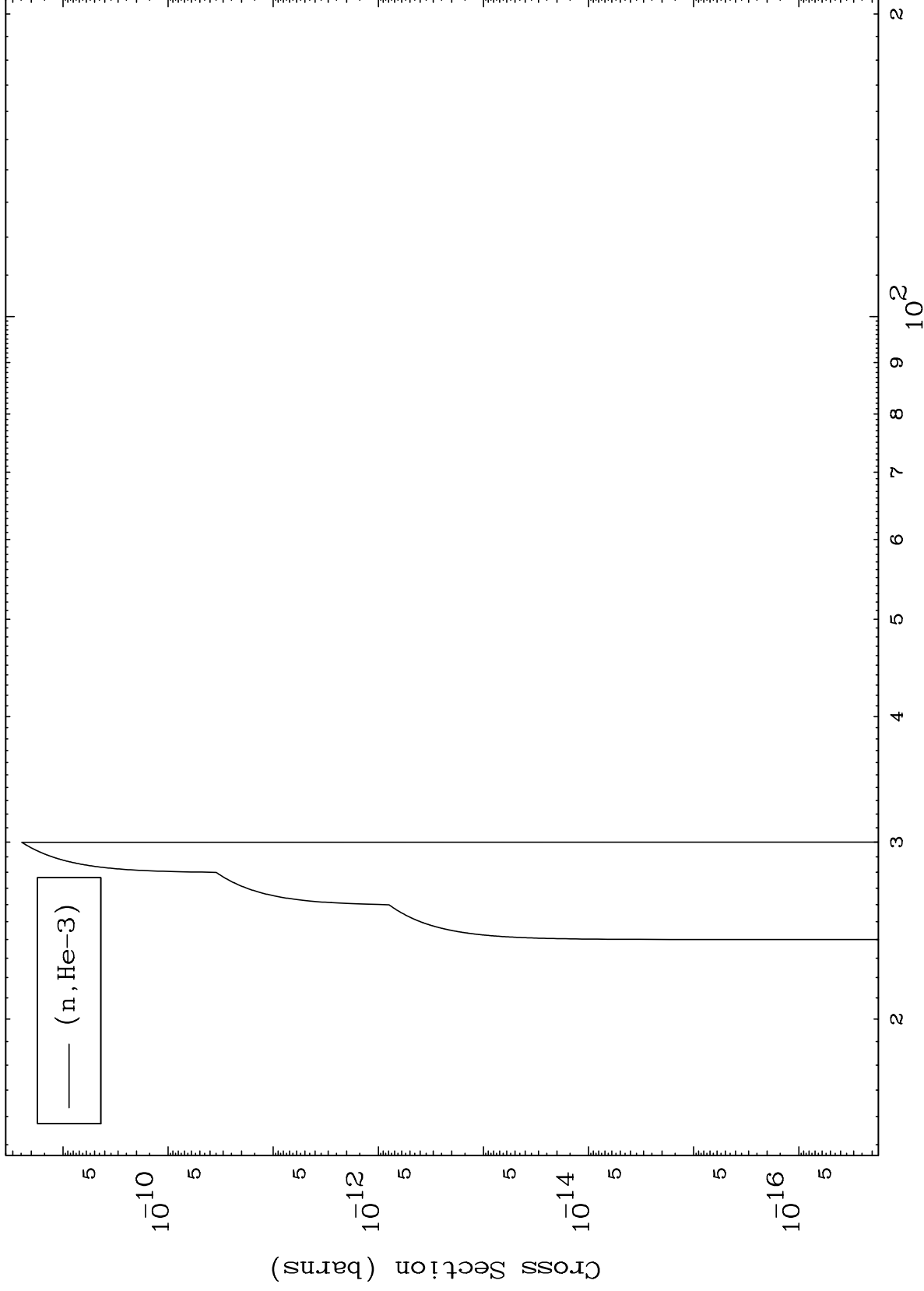


89-Ac-223





0 Kelvin Cross Sections

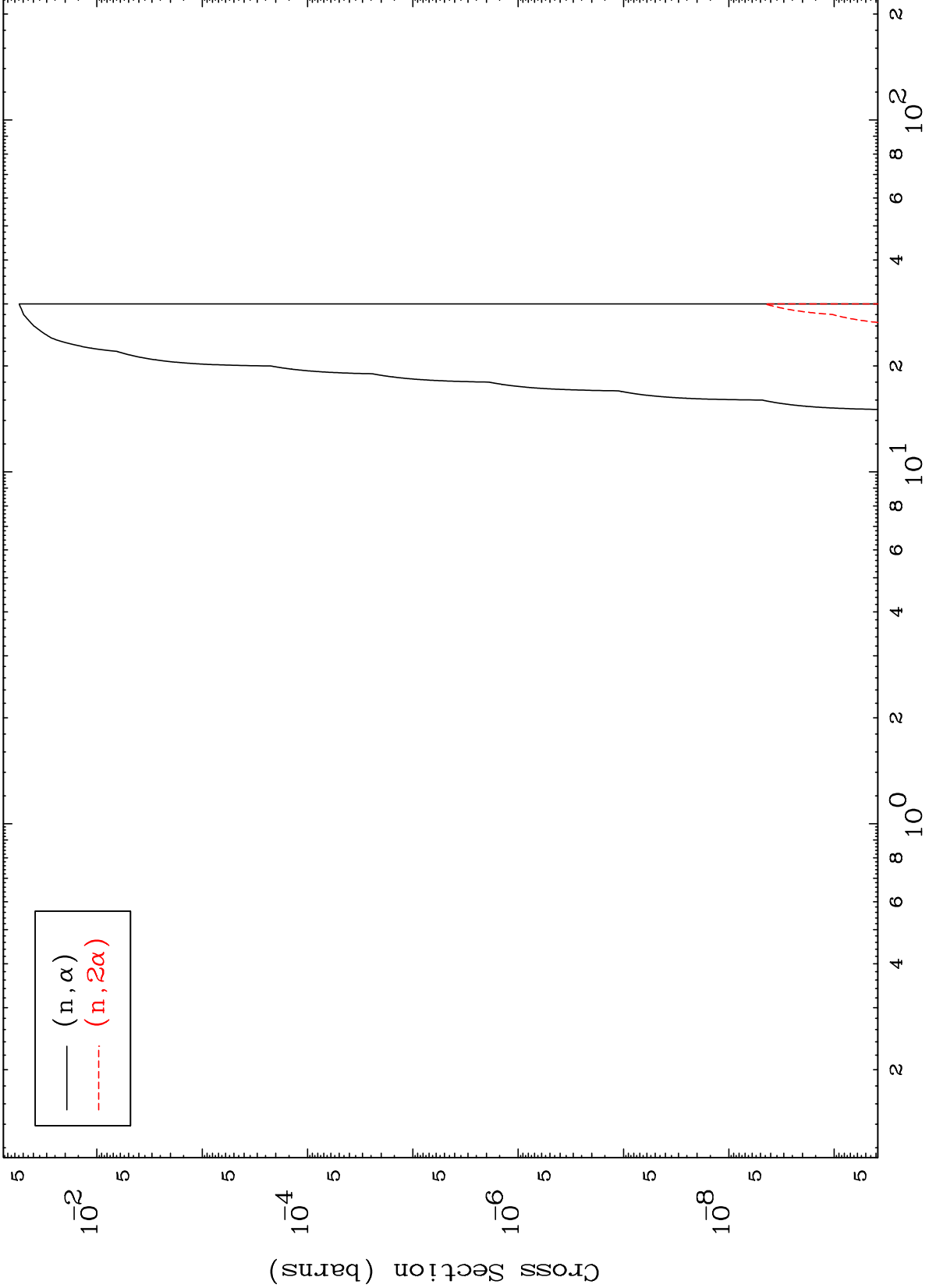


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( $\alpha, \alpha$ ) Levels

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0 Kelvin Cross Sections



10

Incident Energy (MeV)

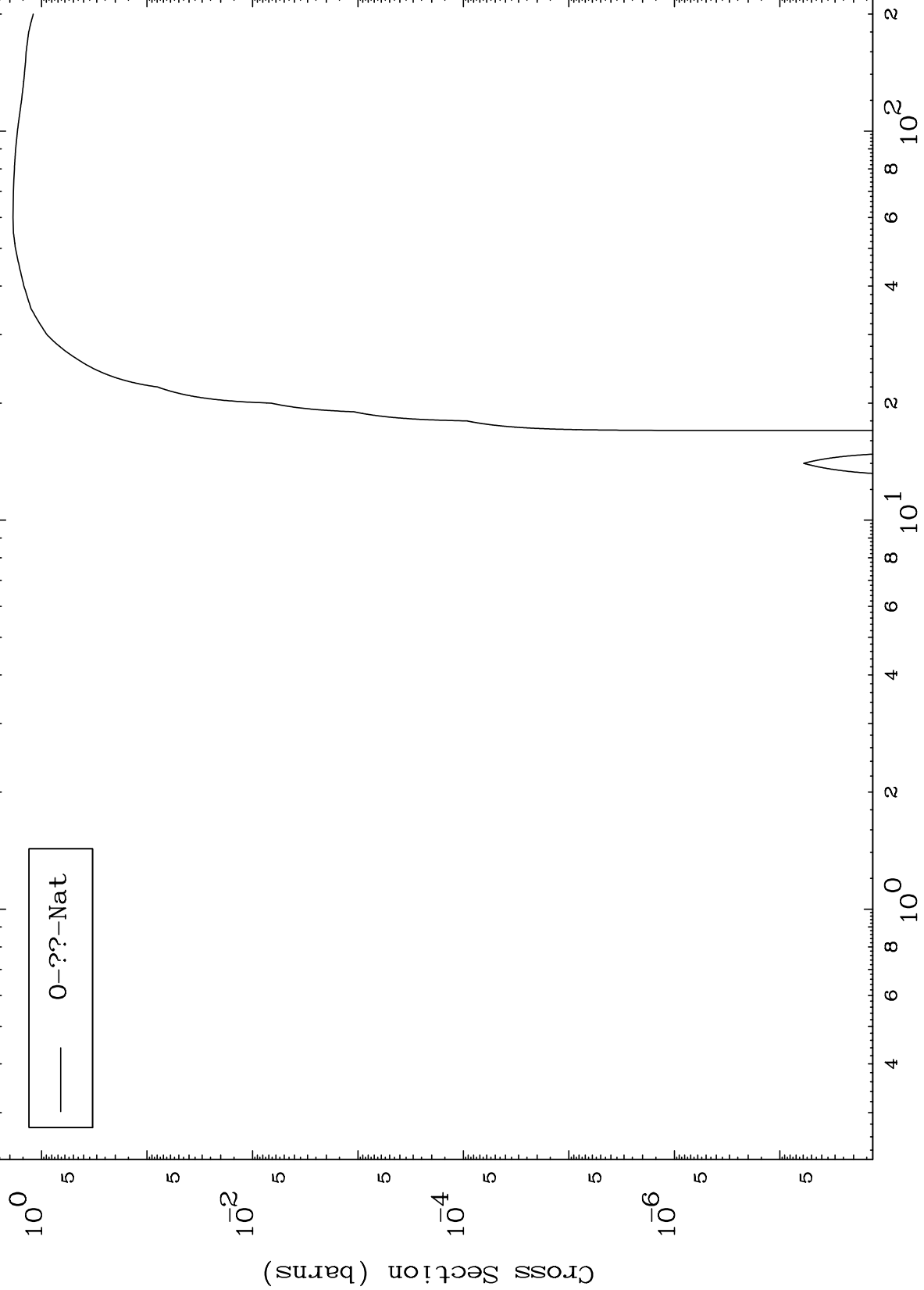
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Fission

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Radionuclide Production Cross Section

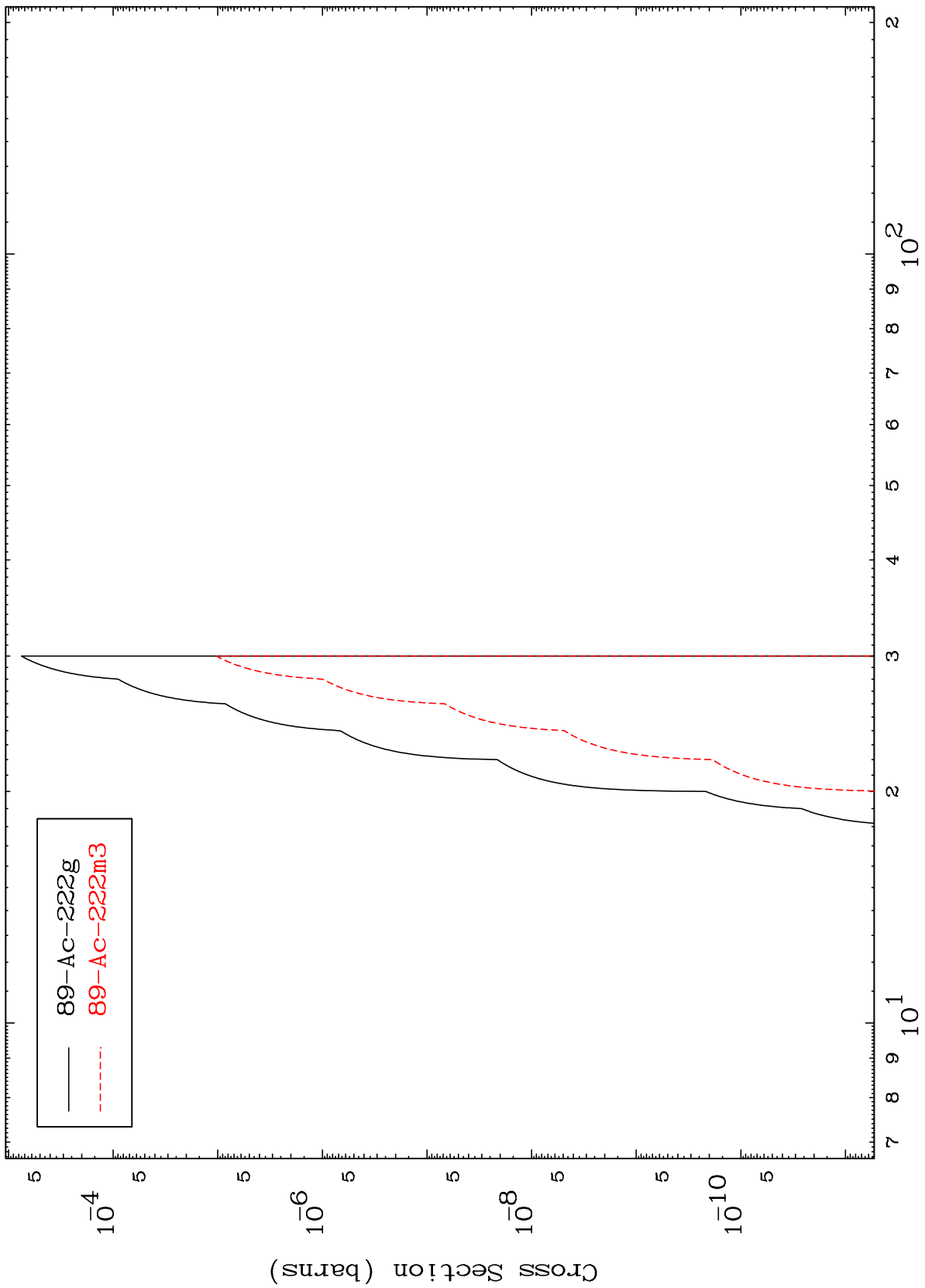


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(n,n')  $\alpha$

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Radionuclide Production Cross Section



12

Incident Energy (MeV)

89-Ac-223