

Program EVALPLOT  
(Version 2021-1)

by

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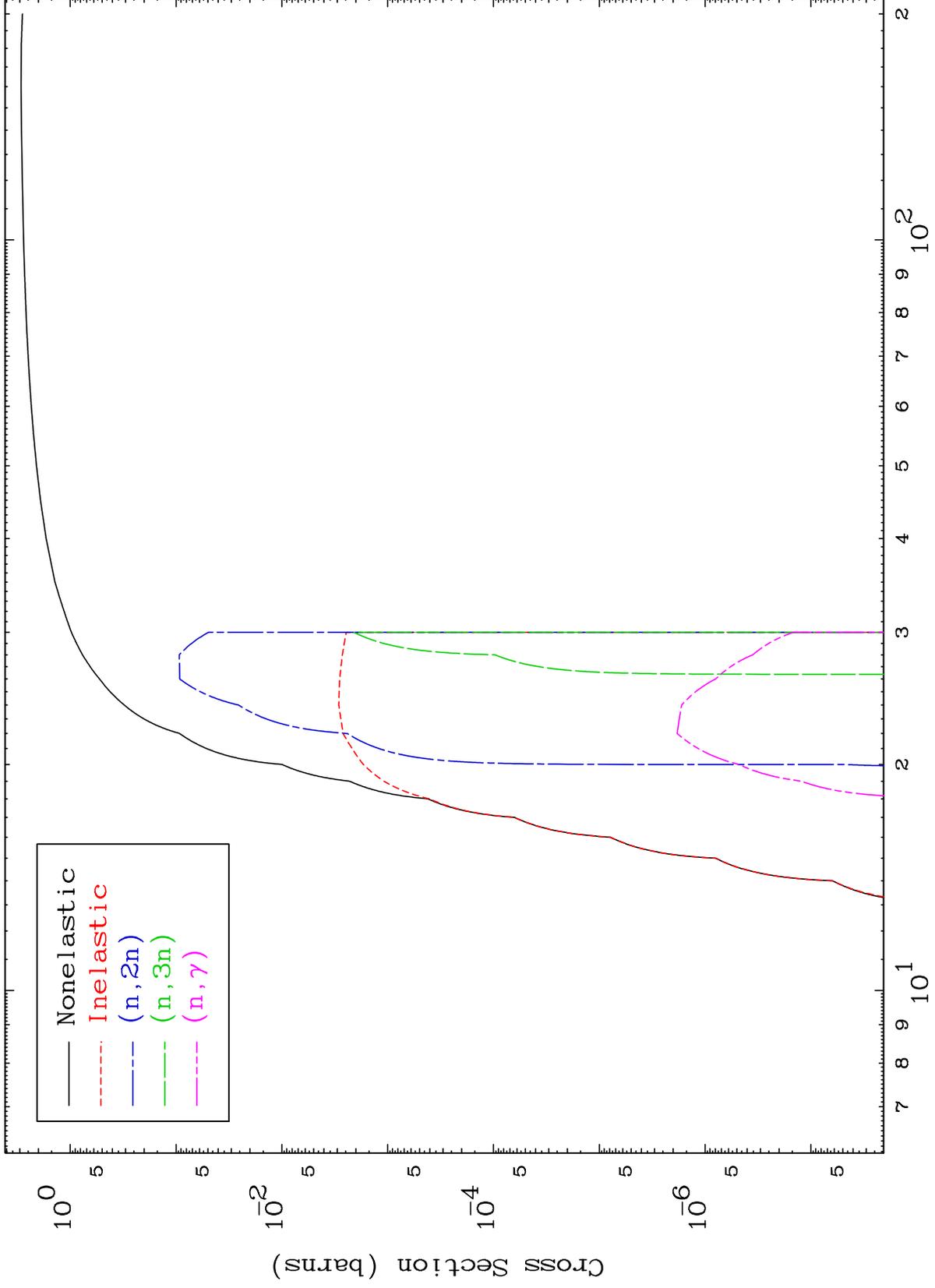
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Press Mouse Button to Start

MAT 8922

$\alpha$  Major  
0 Kelvin Cross Sections

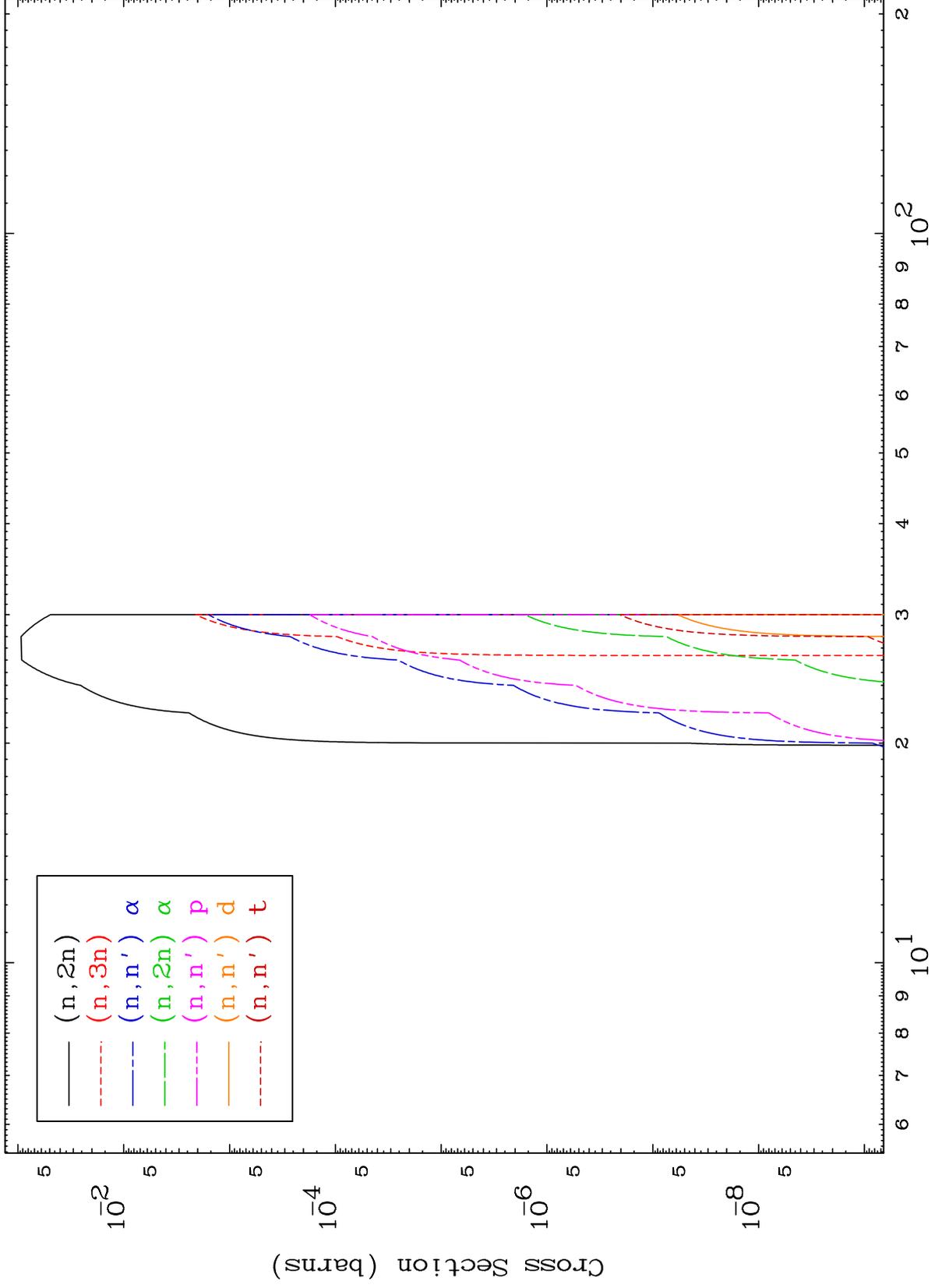
89-Ac-224



MAT 8922

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

89-Ac-224



2

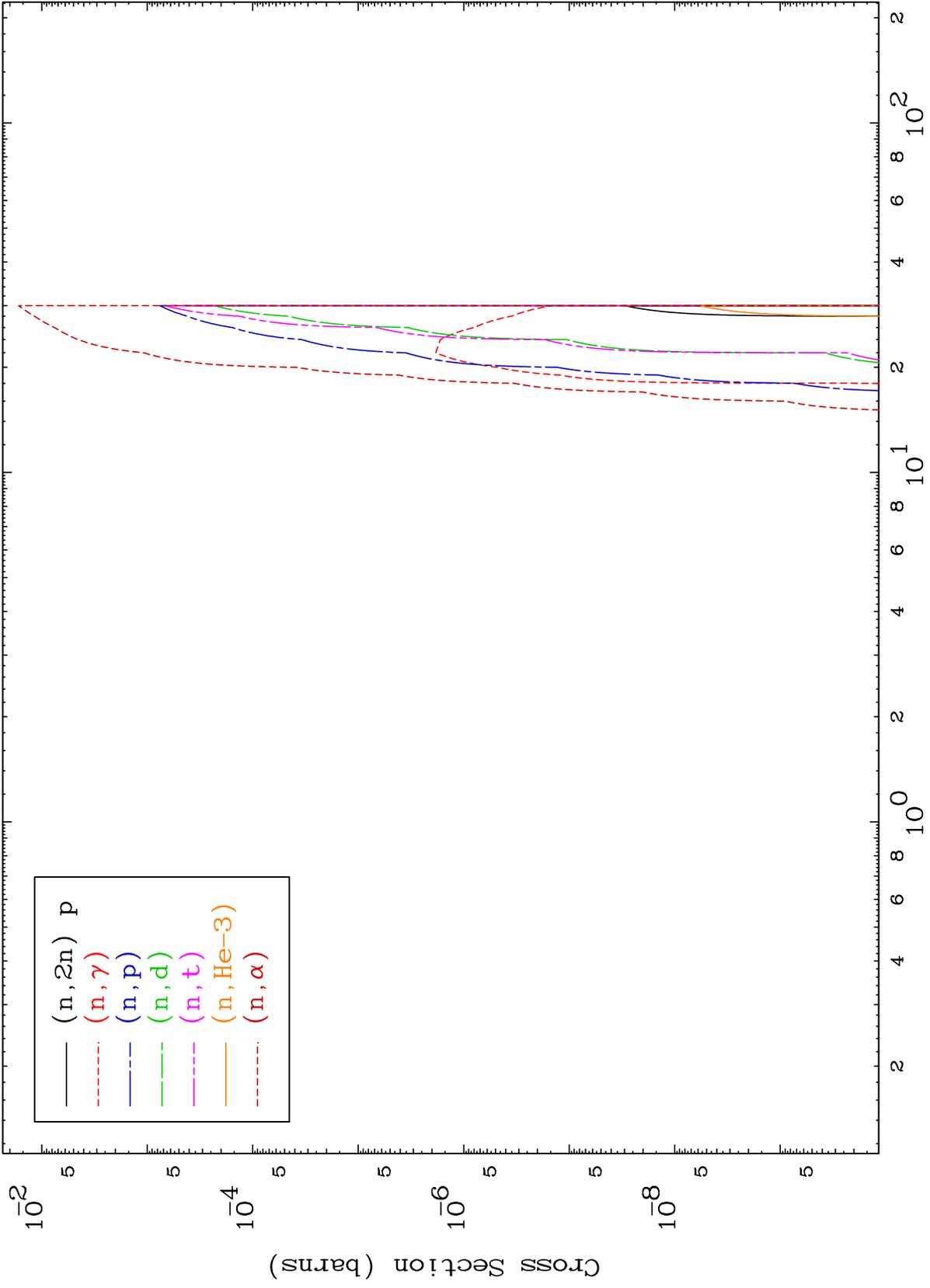
Incident Energy (MeV)

89-Ac-224

MAT 8922

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

89-Ac-224

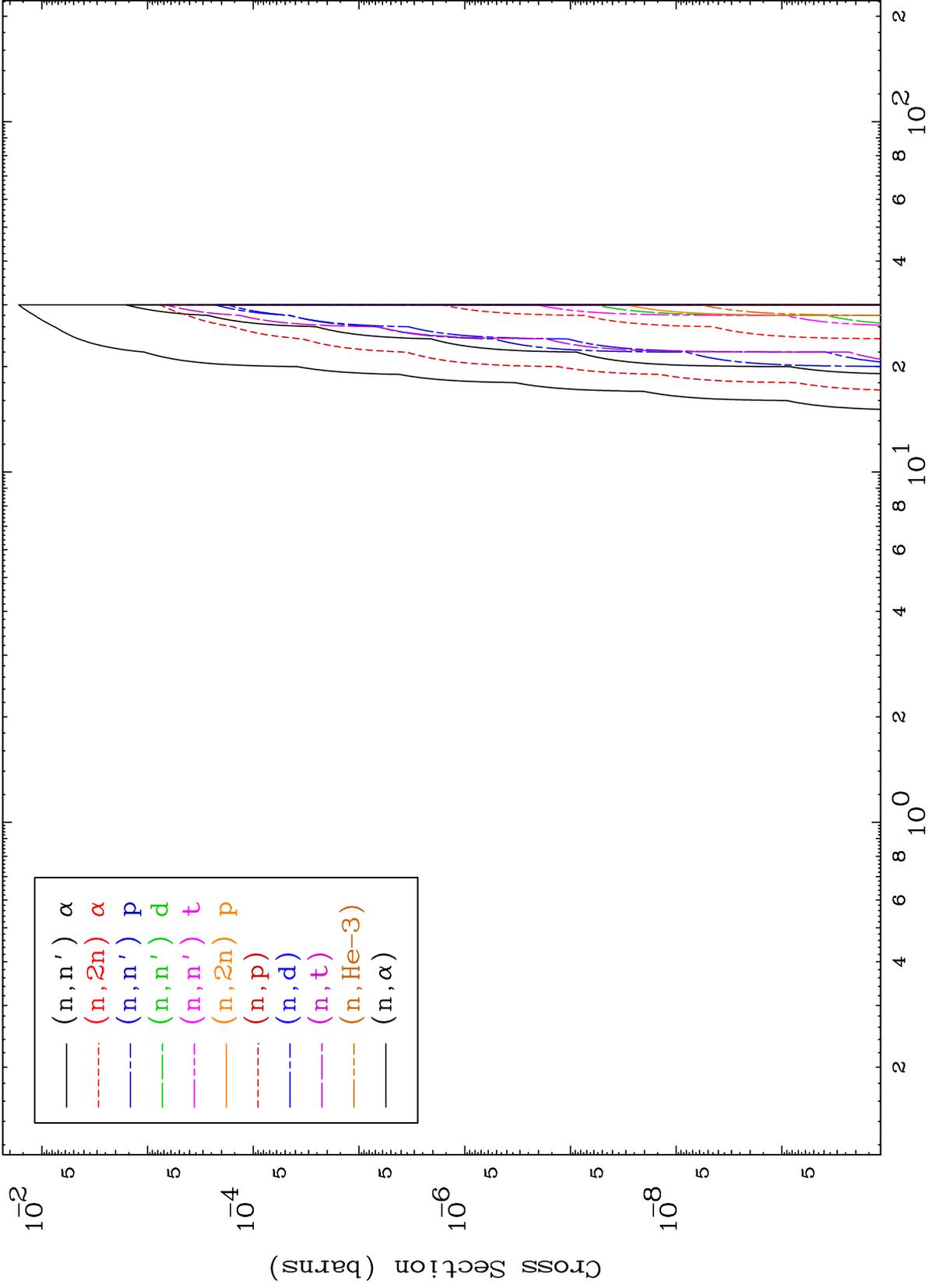


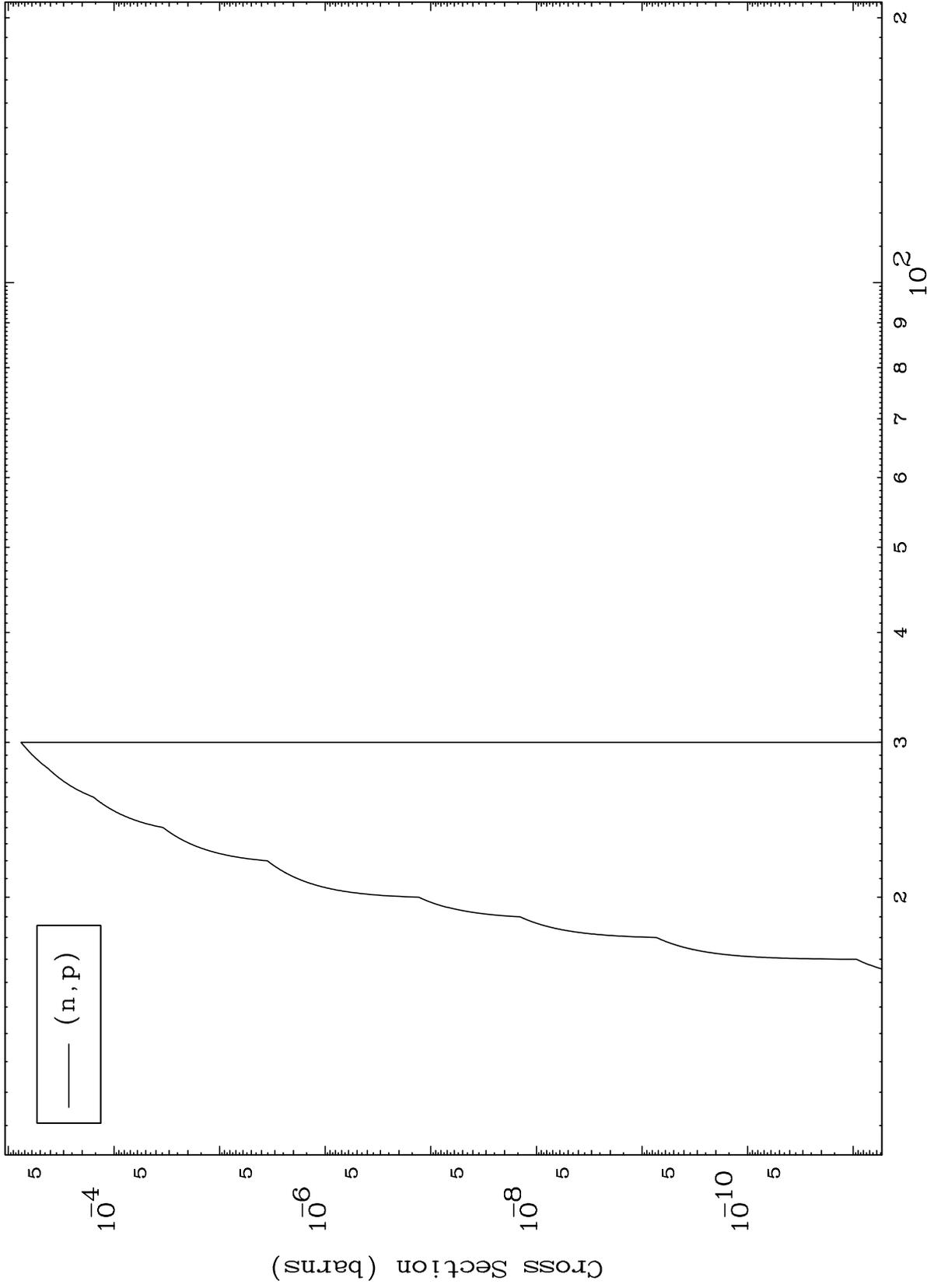
MAT 8922

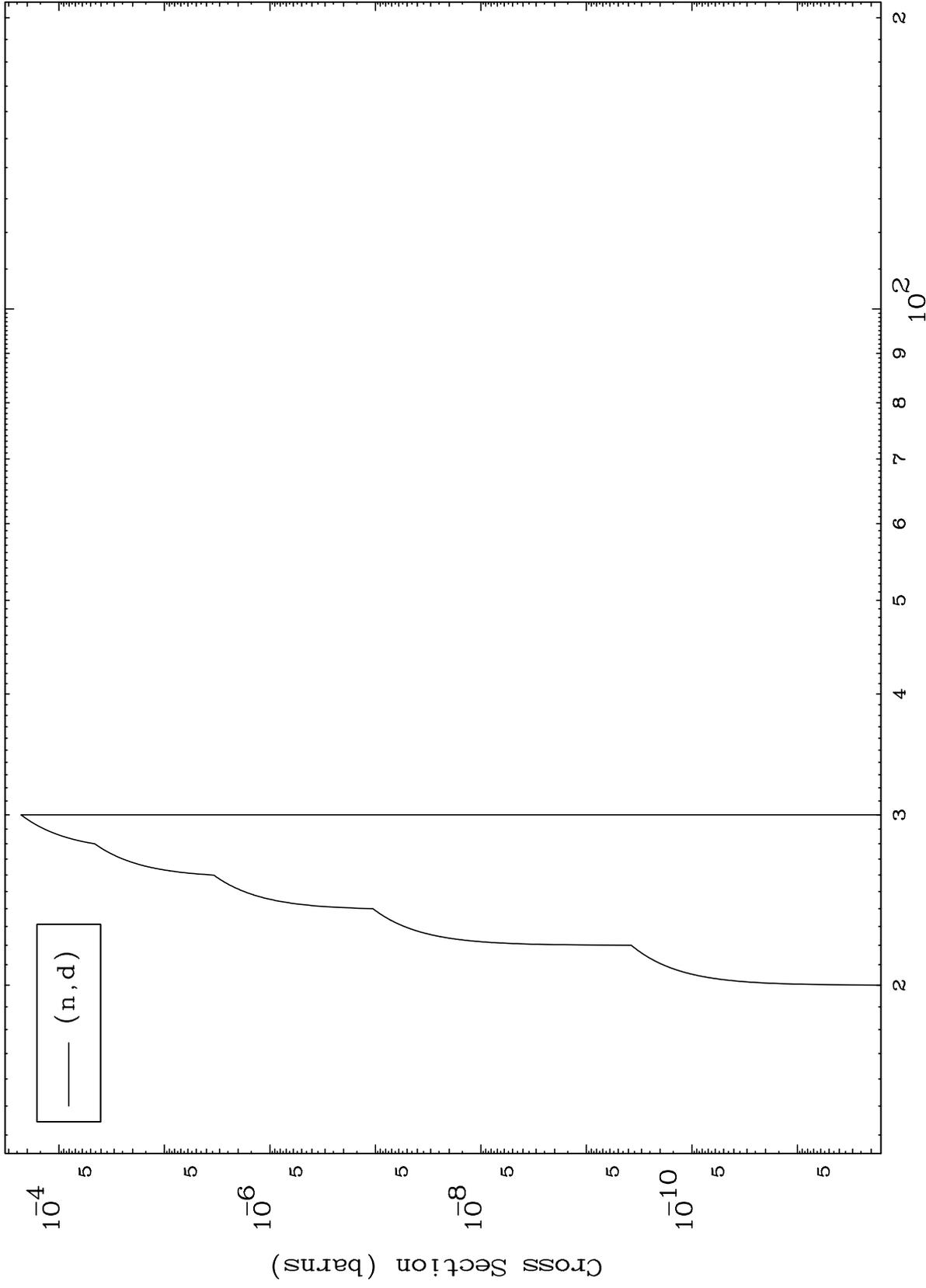
$\alpha$  Charged Particle

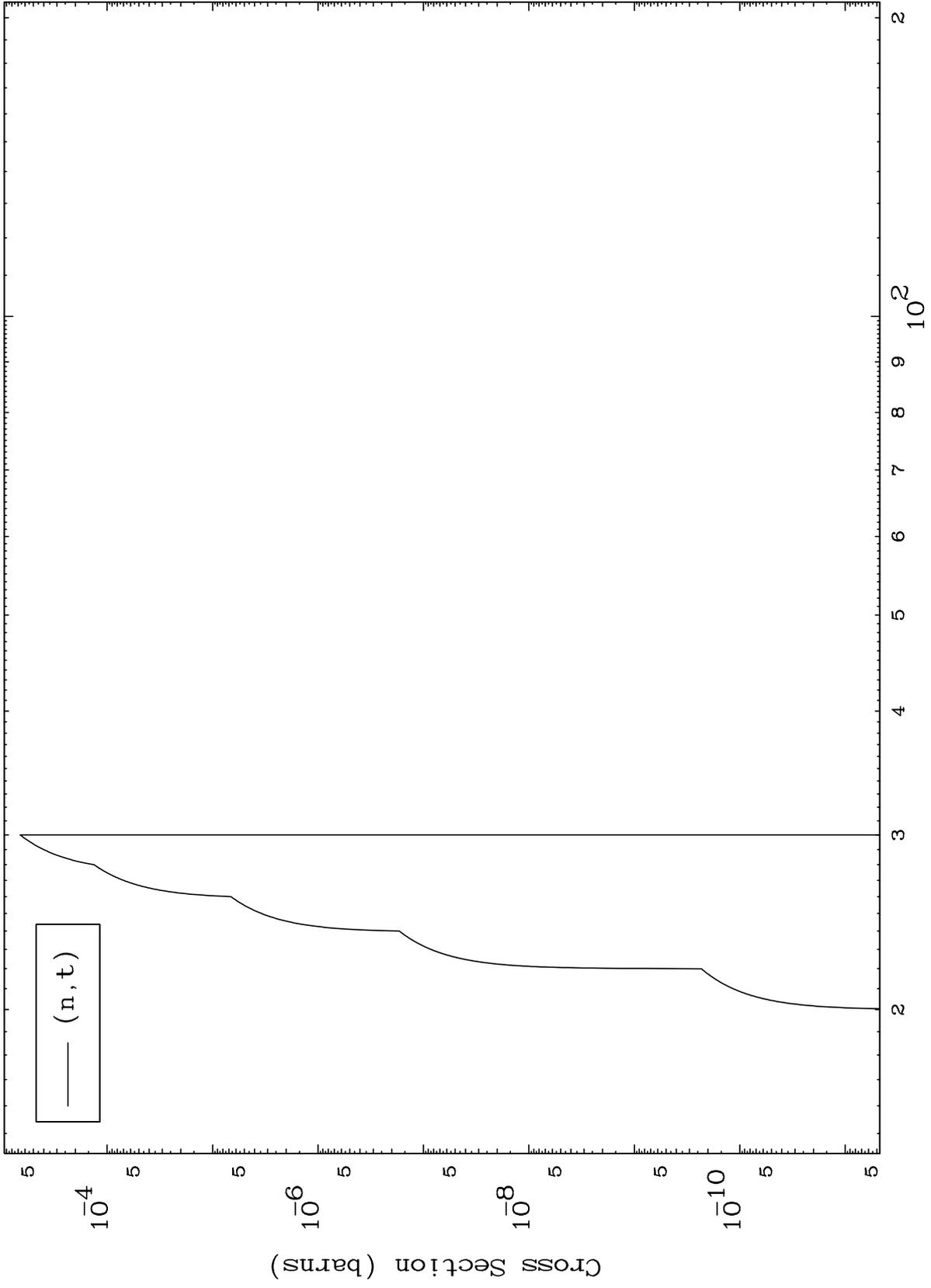
89-Ac-224

0 Kelvin Cross Sections

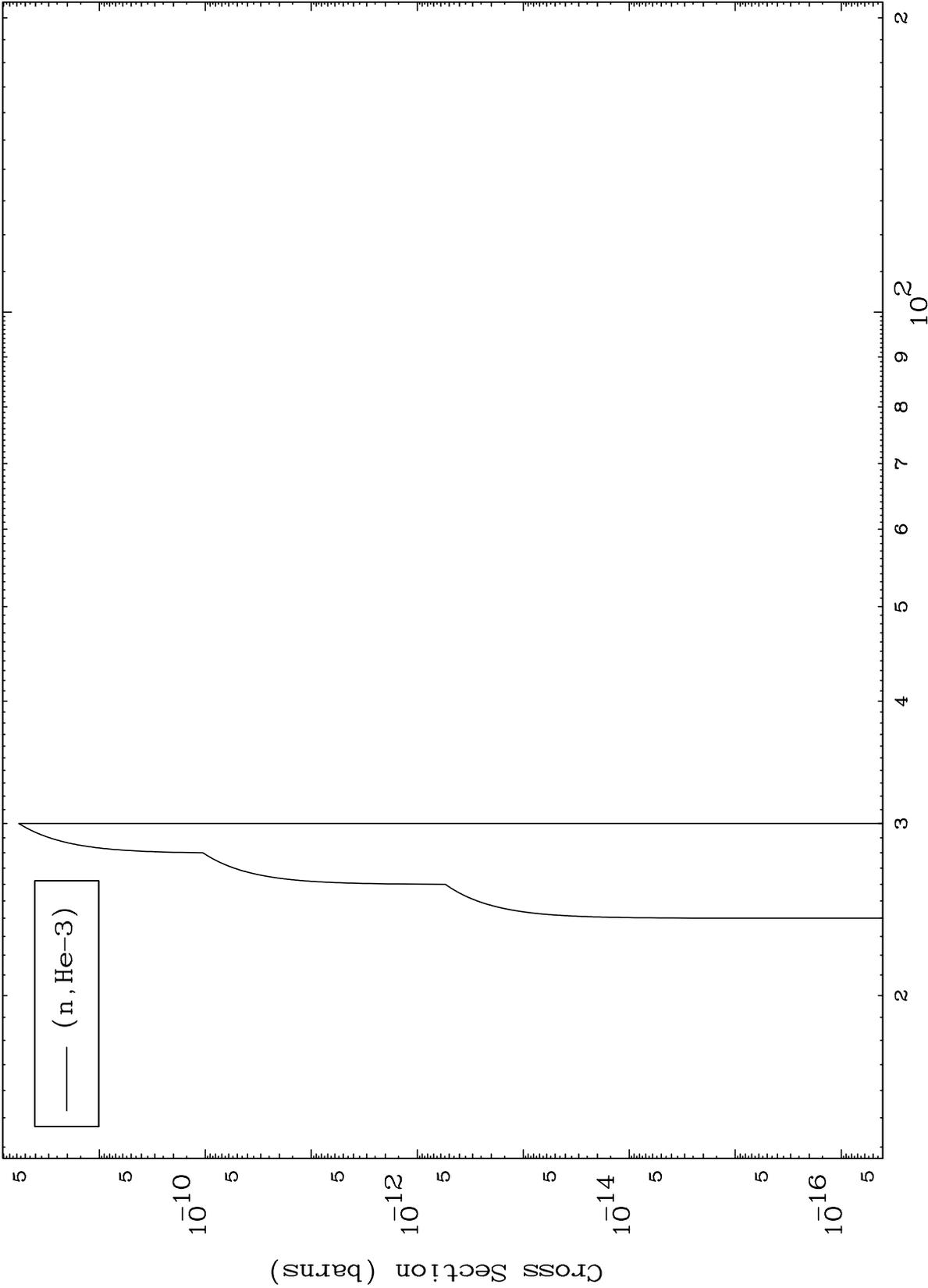








( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

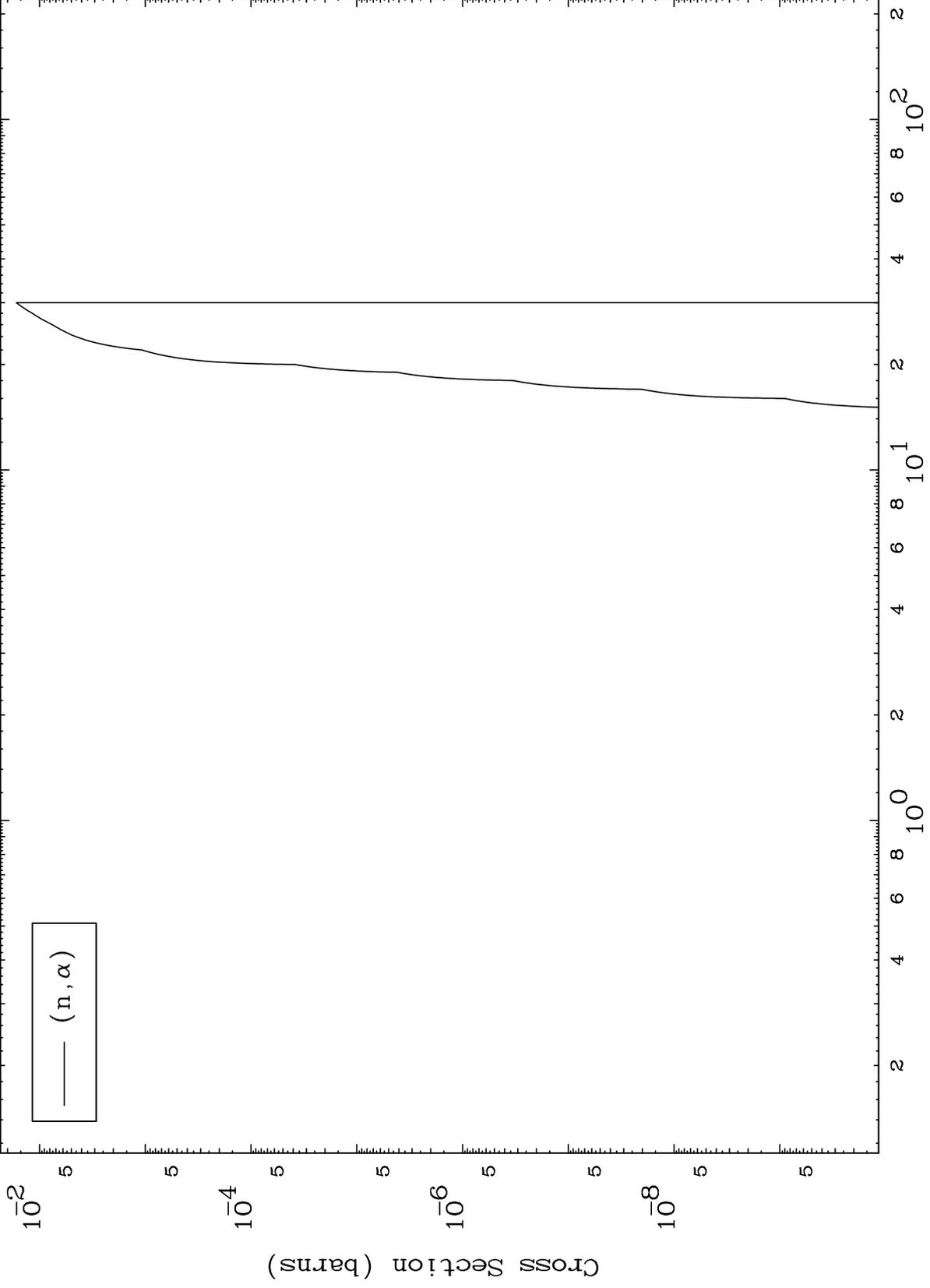


MAT 8922

( $\alpha, \alpha$ ) Levels

89-Ac-224

0 Kelvin Cross Sections

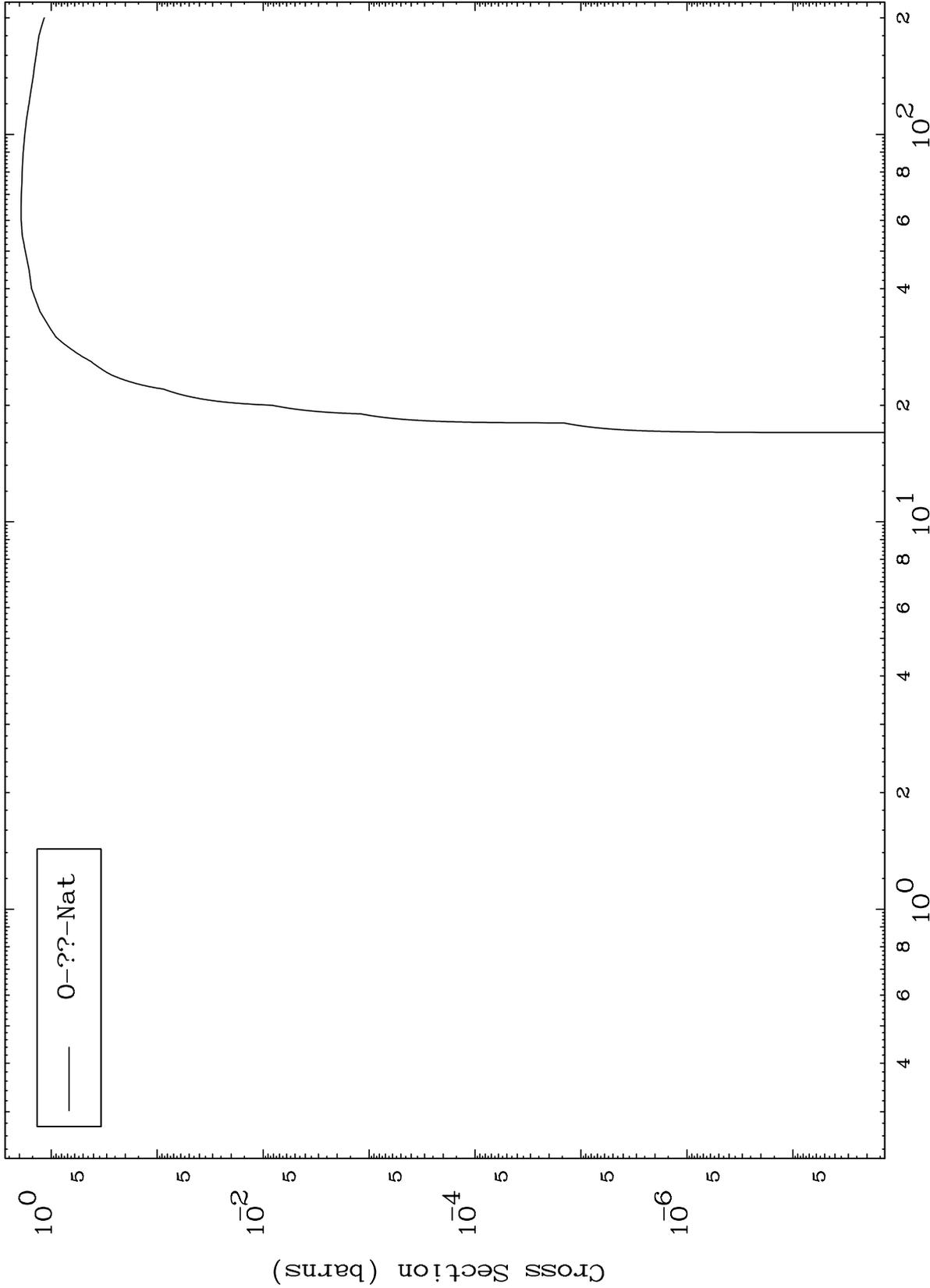


MAT 8922

Fission

89-Ac-224

Radionuclide Production Cross Section



10

Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,2n)  $\alpha$

89-Ac-224

Radionuclide Production Cross Section

