

Program EVALPLOT  
(Version 2021-1)

by

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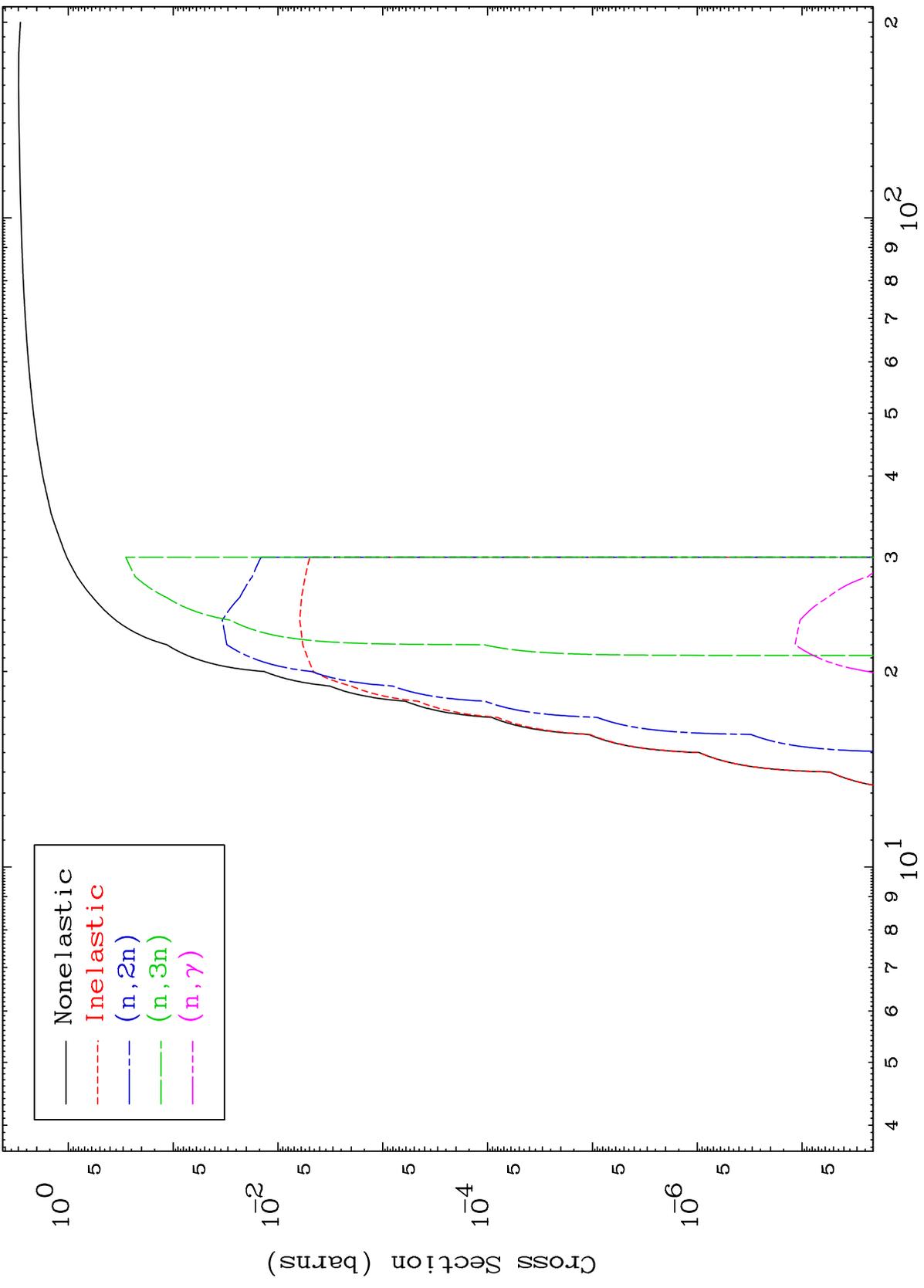
Press Mouse Button to Start

MAT 8949

$\alpha$  Major

89-Ac-233

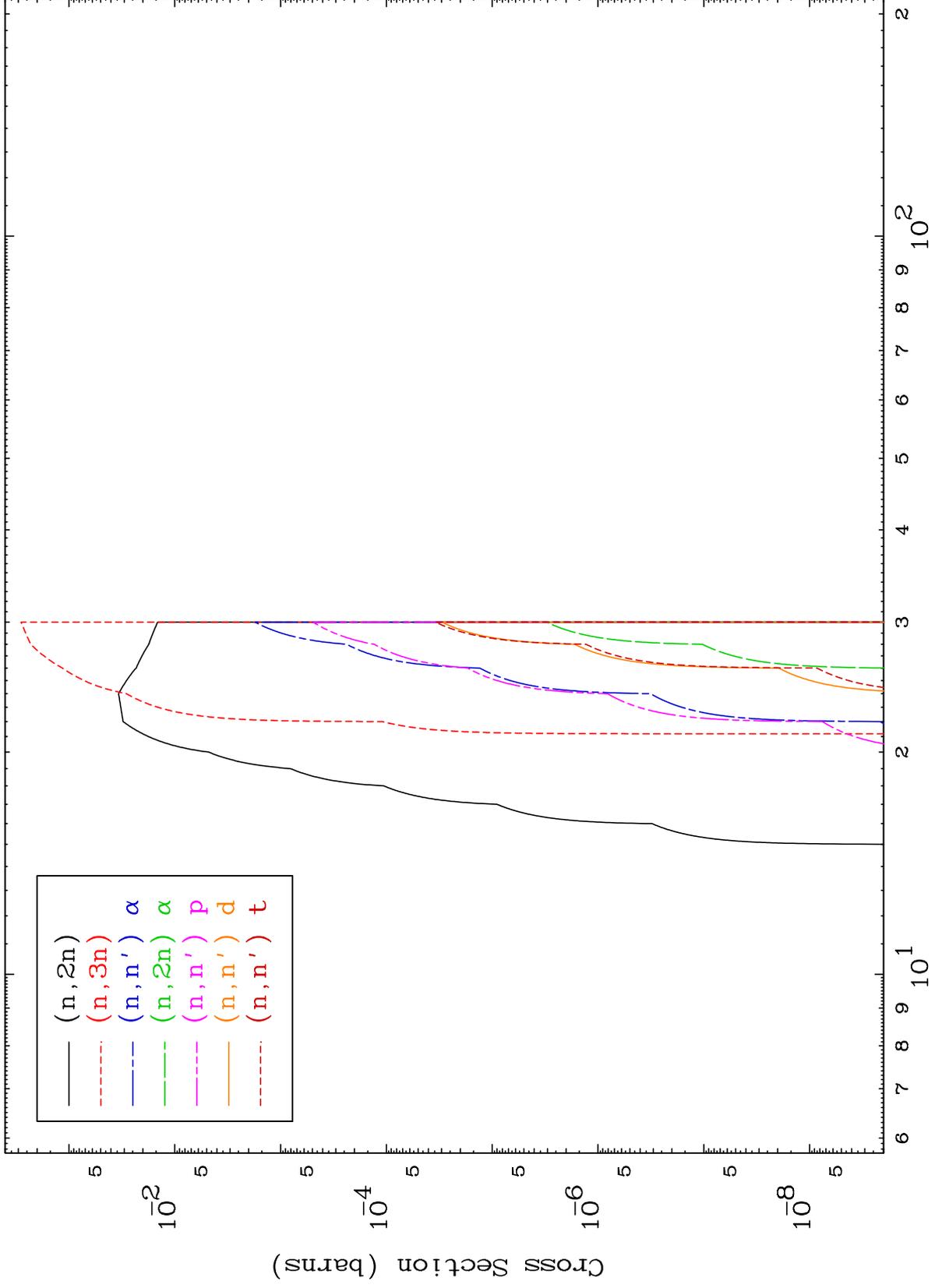
0 Kelvin Cross Sections



1

Incident Energy (MeV)

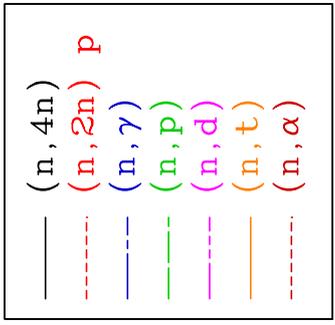
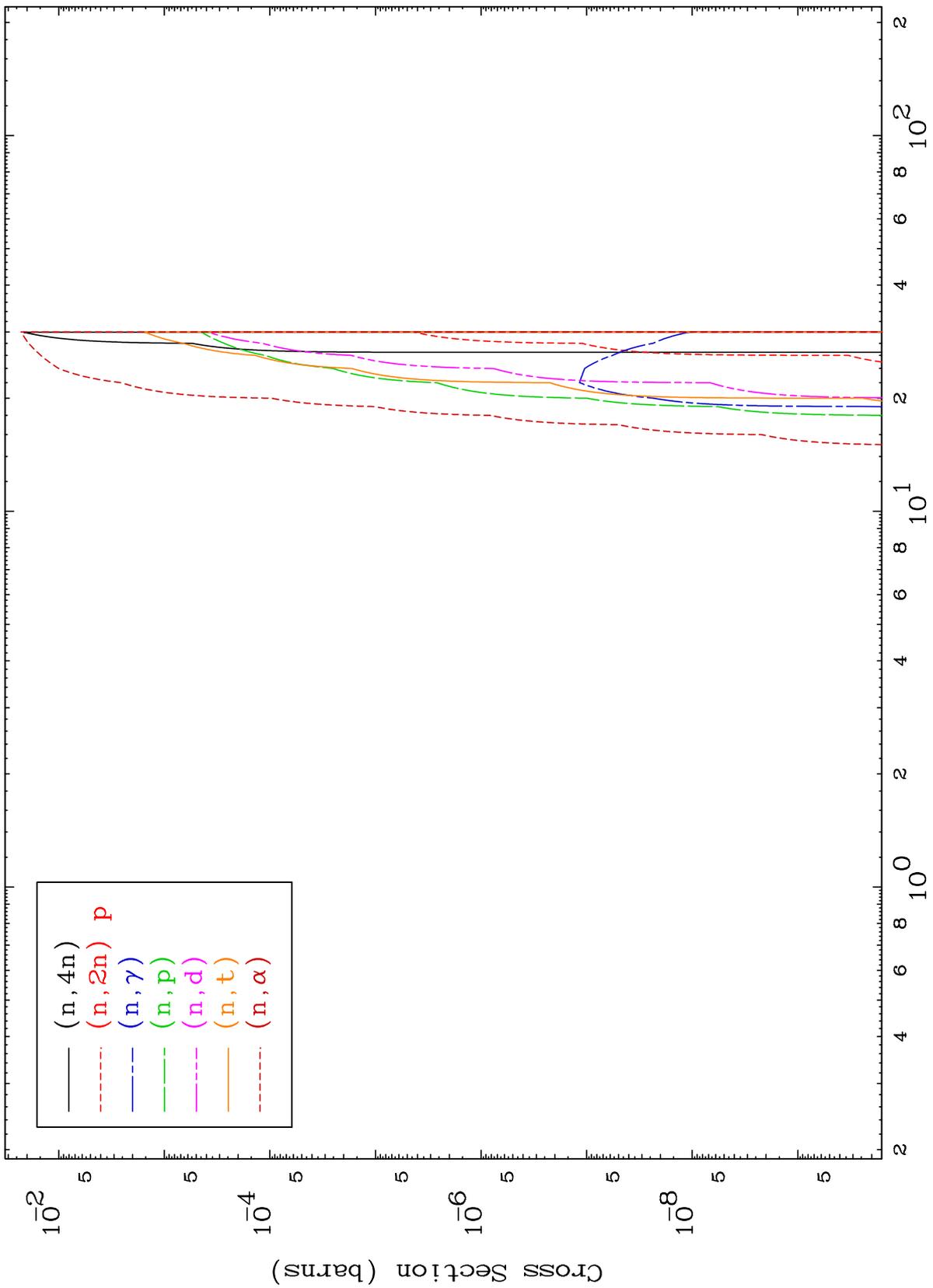
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$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

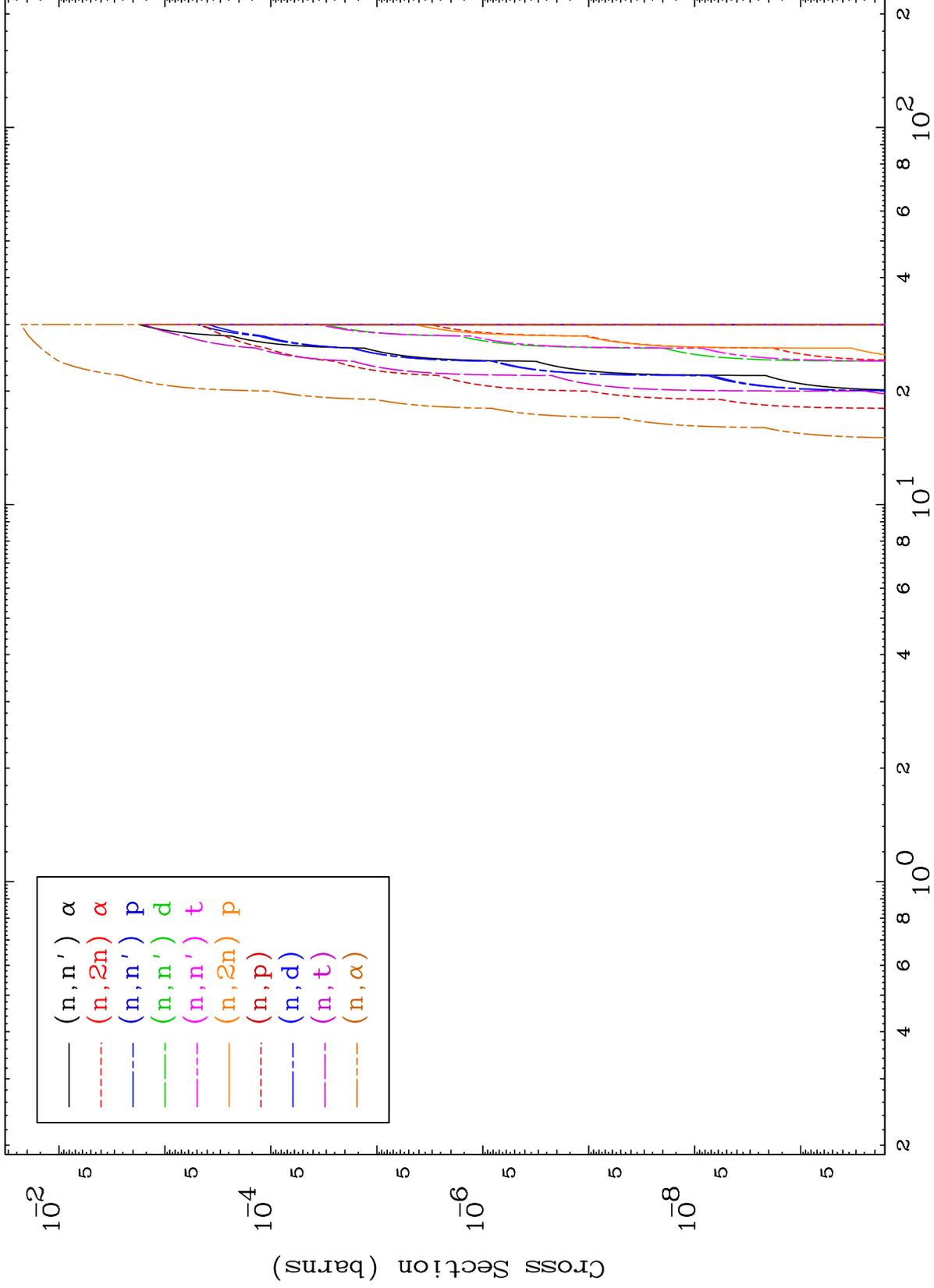
89-Ac-233



3

Incident Energy (MeV)

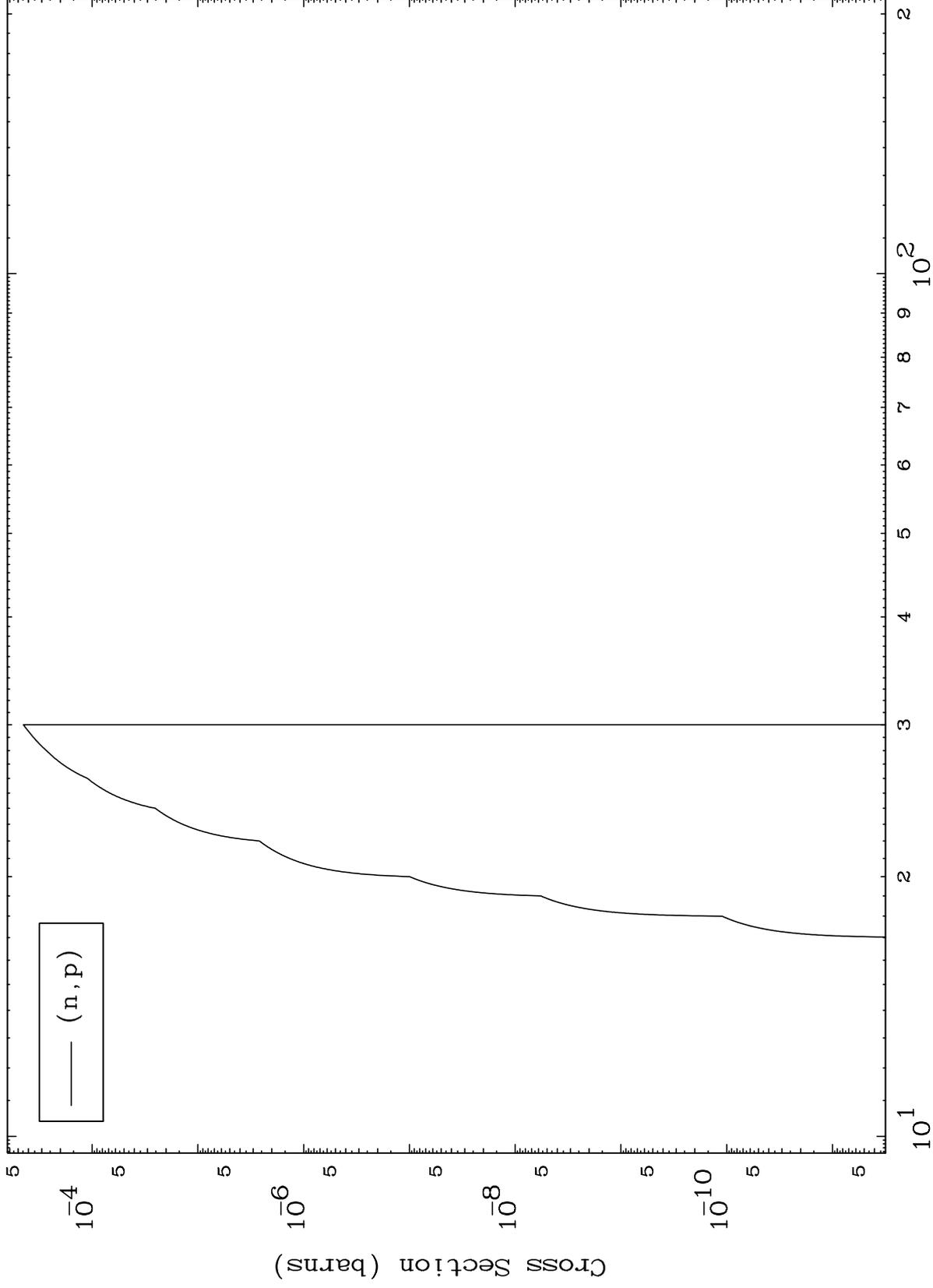
89-Ac-233



MAT 8949

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

89-Ac-233



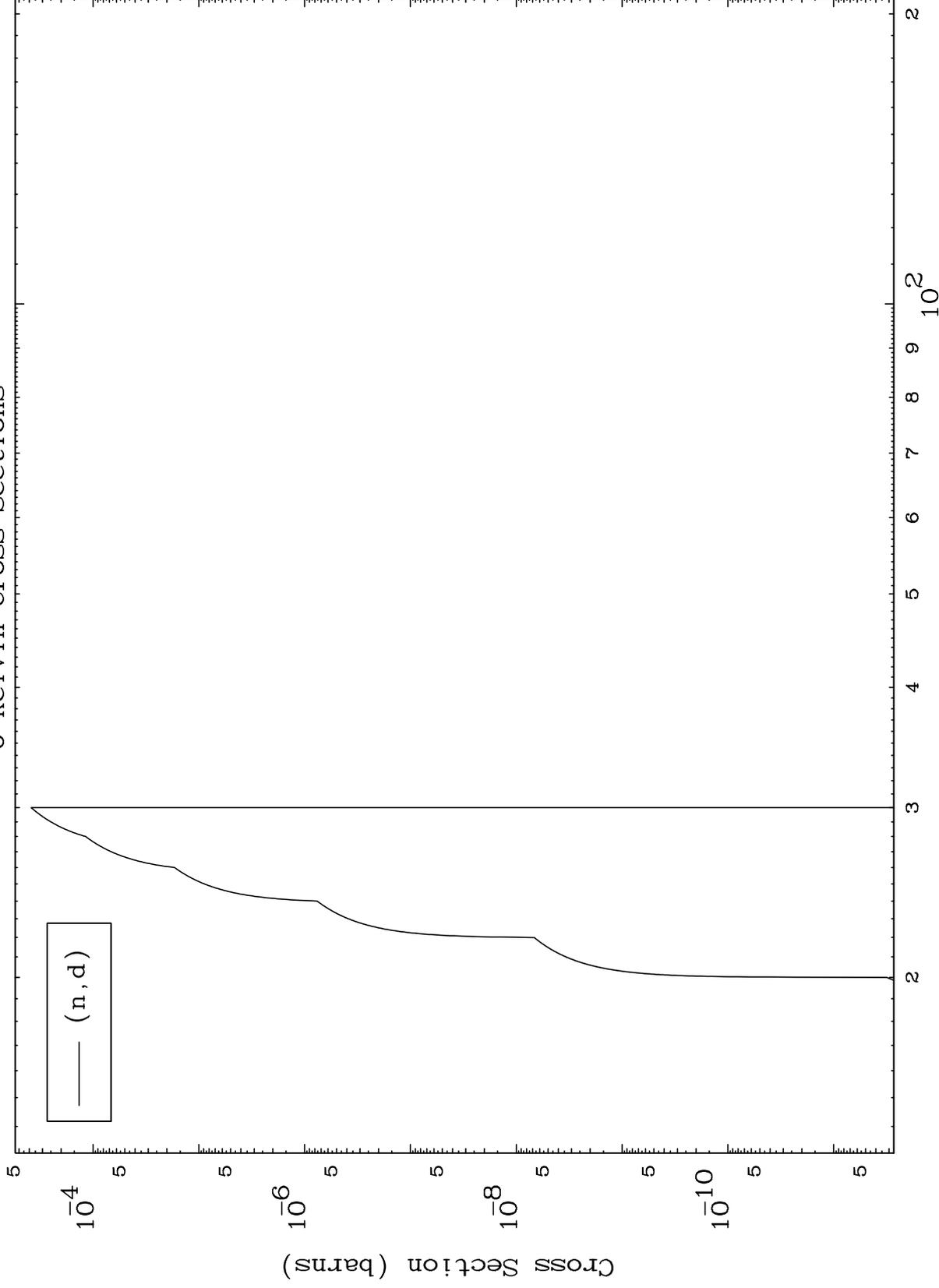
89-Ac-233

Incident Energy (MeV)

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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

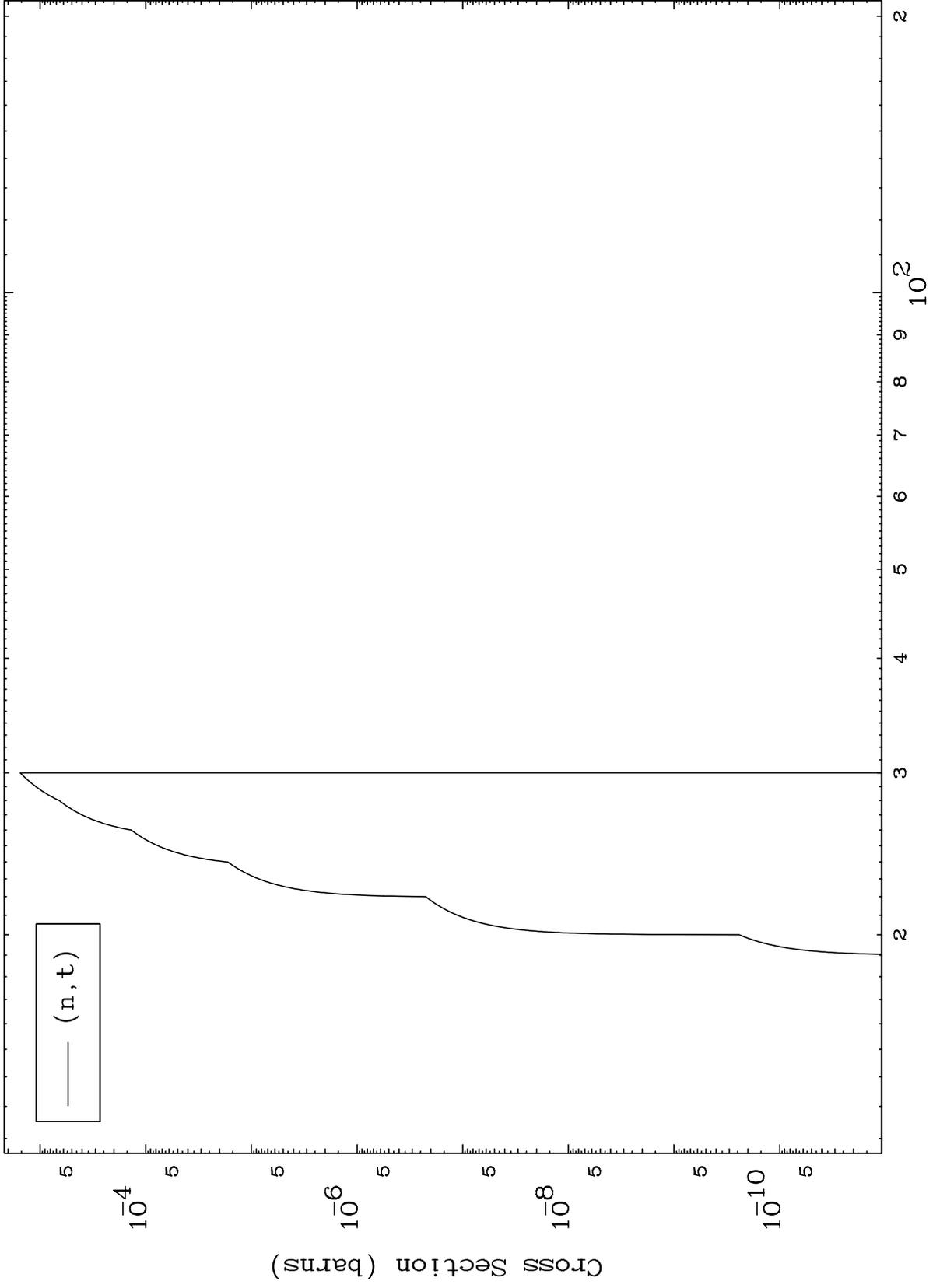
89-Ac-233



6

Incident Energy (MeV)

89-Ac-233

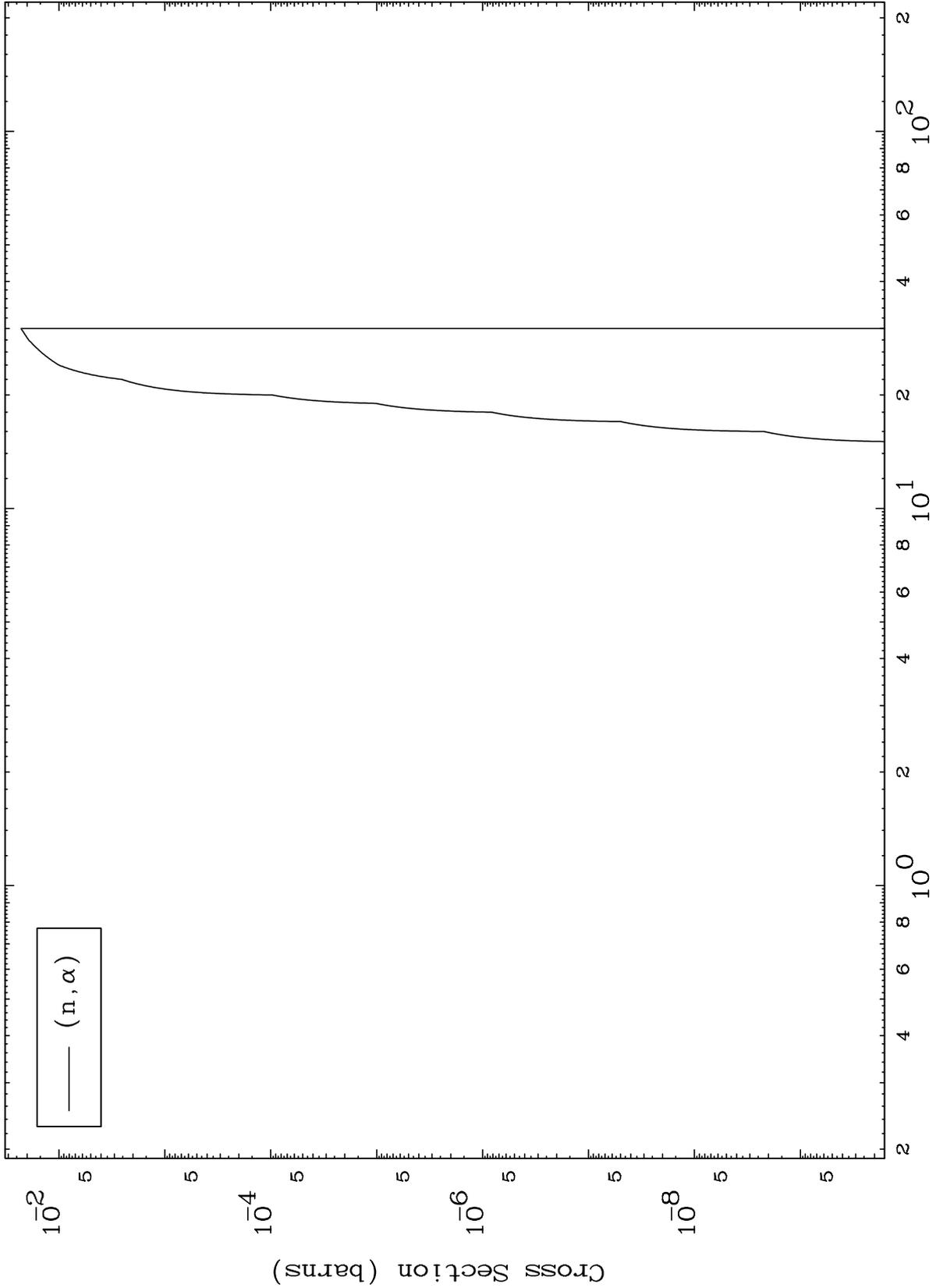


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( $\alpha, \alpha$ ) Levels

89-Ac-233

0 Kelvin Cross Sections



8

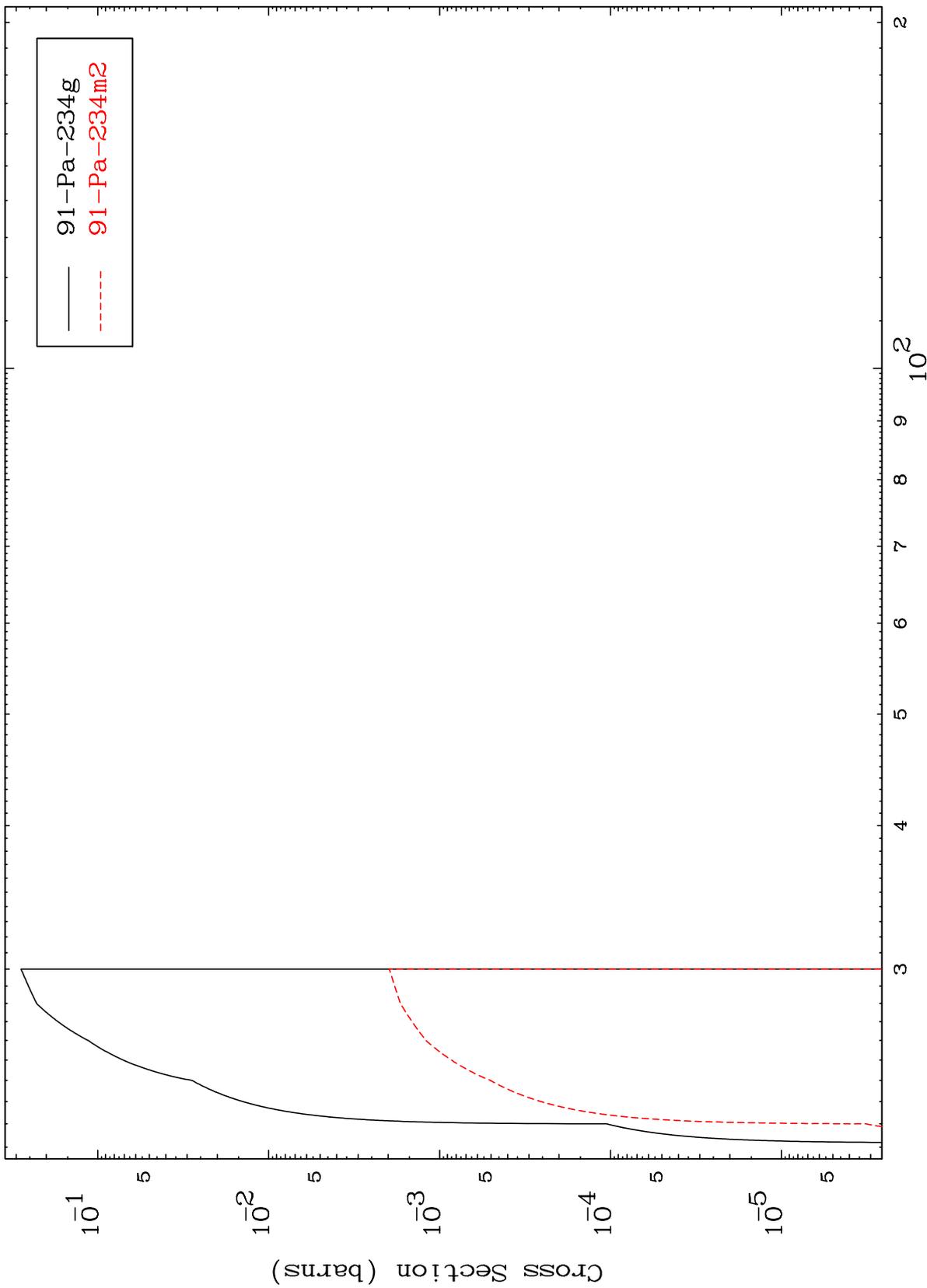
Incident Energy (MeV)

89-Ac-233

MAT 8949

89-Ac-233

(n,3n)  
Radionuclide Production Cross Section



9

Incident Energy (MeV)

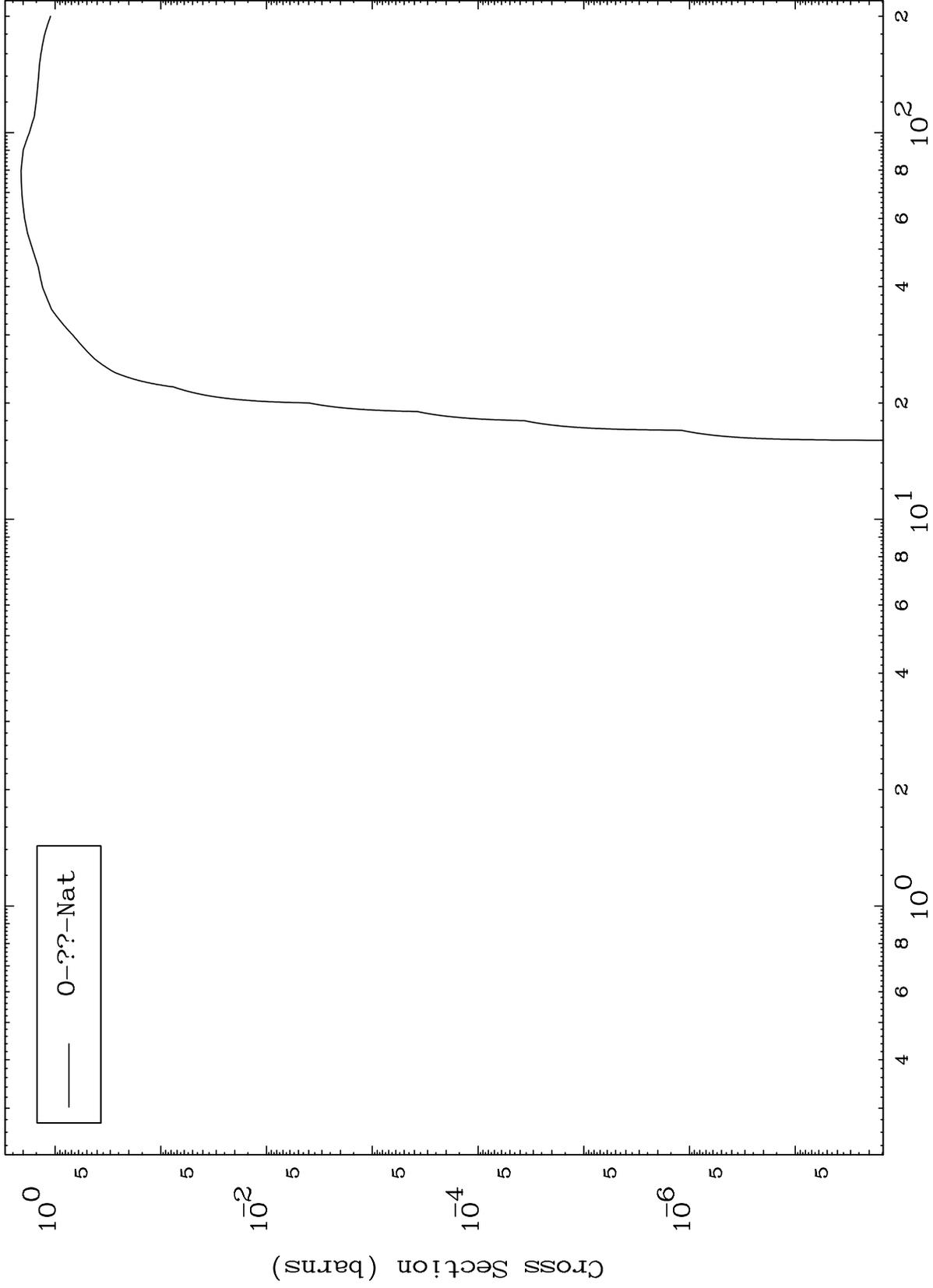
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Fission

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Radionuclide Production Cross Section



10

Incident Energy (MeV)

89-Ac-233