

Program EVALPLOT  
(Version 2021-1)

by

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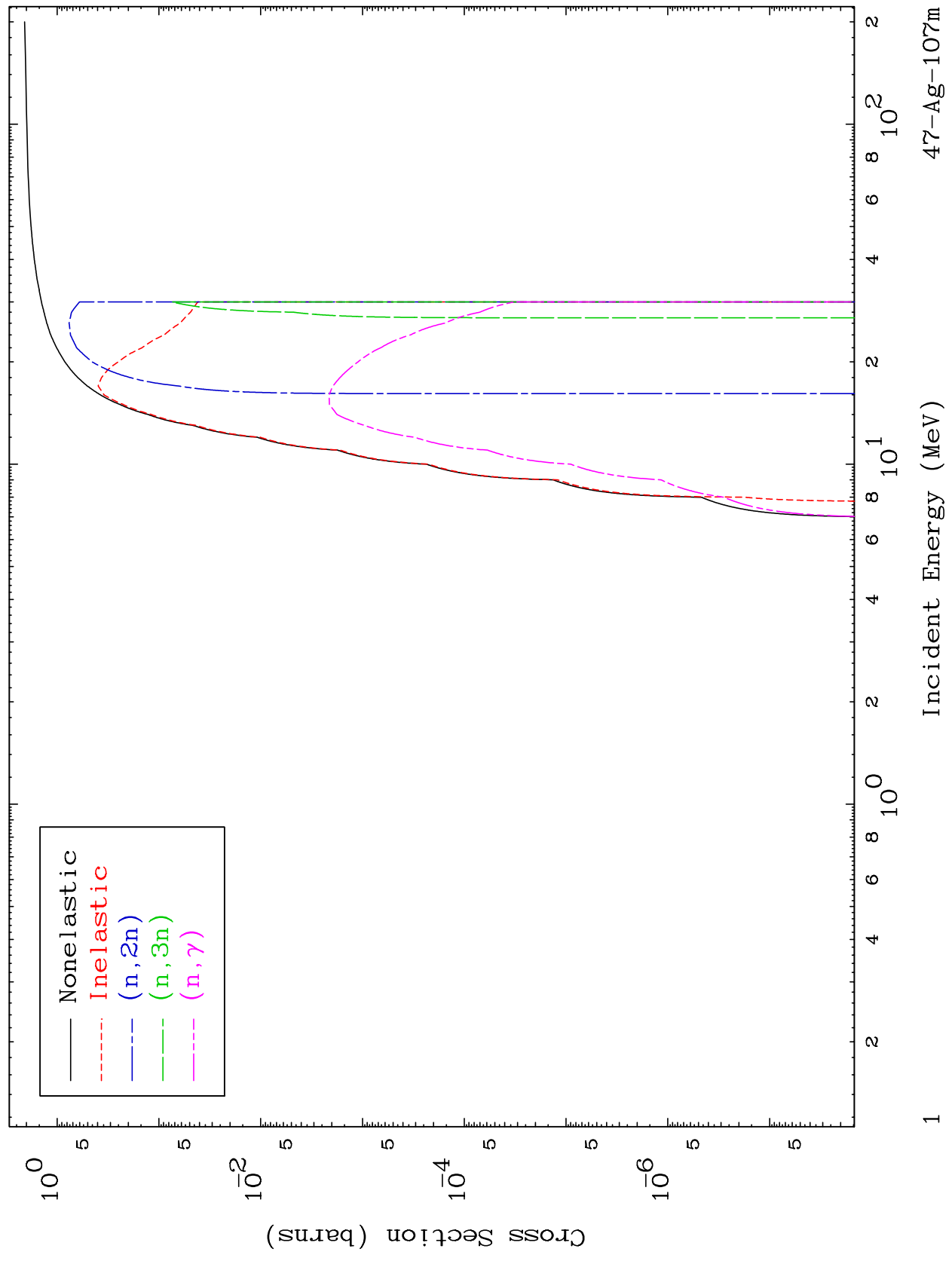
Press Mouse Button to Start

MAT 4726

0 Kelvin

$\alpha$  Major

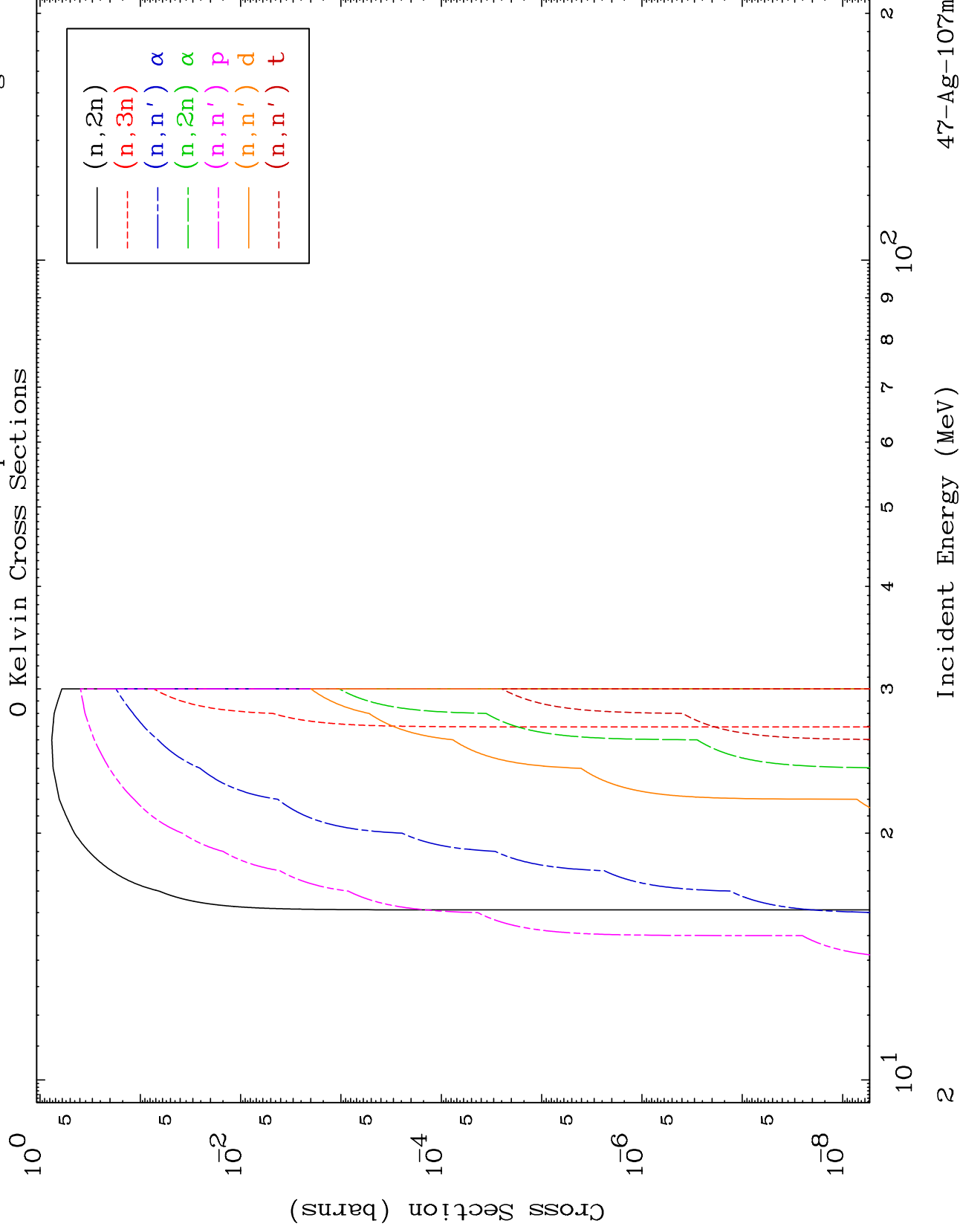
47-Ag-107m



MAT 4726

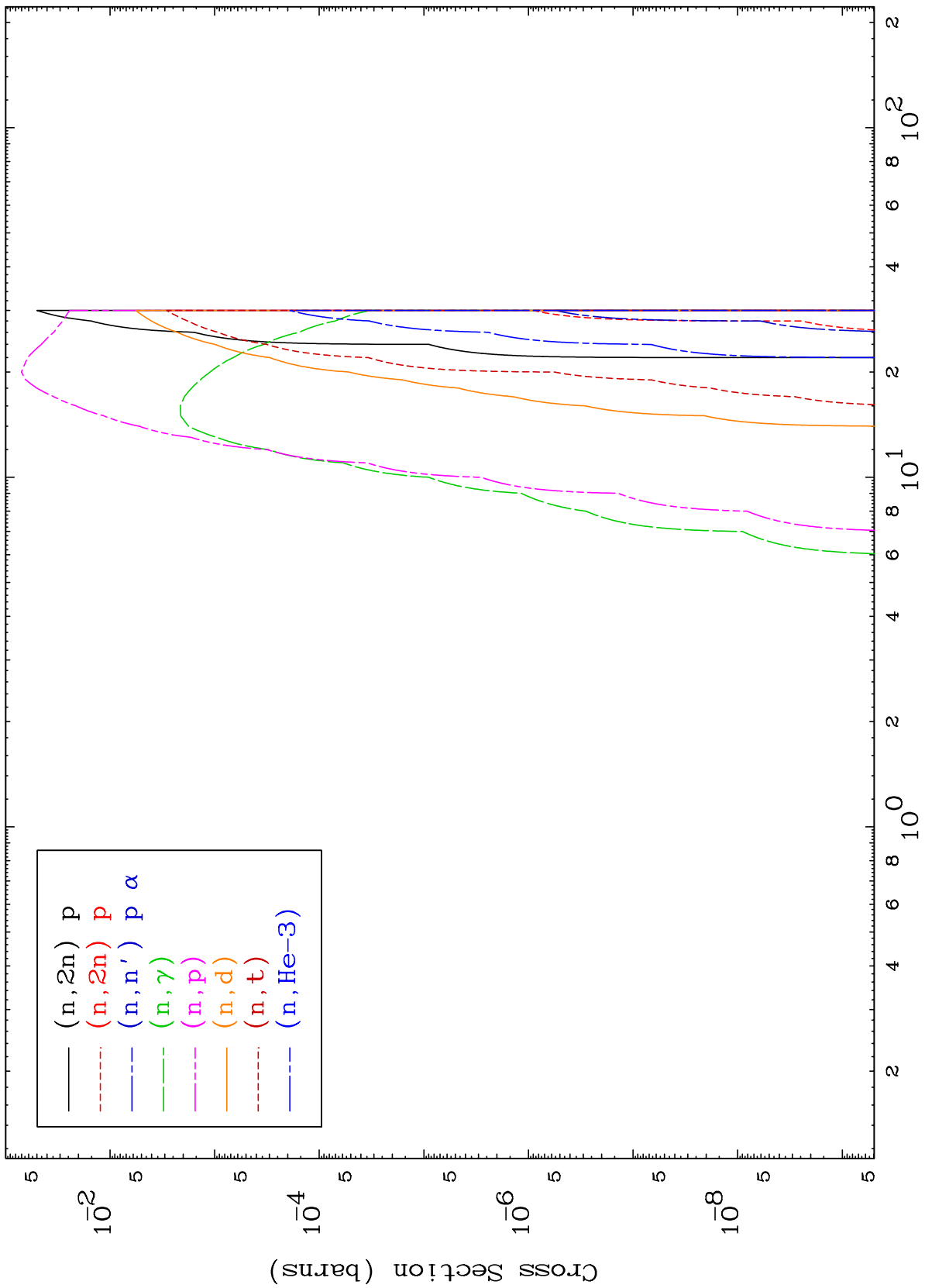
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

47-Ag-107m



47-Ag-107m

0 Kelvin Cross Sections

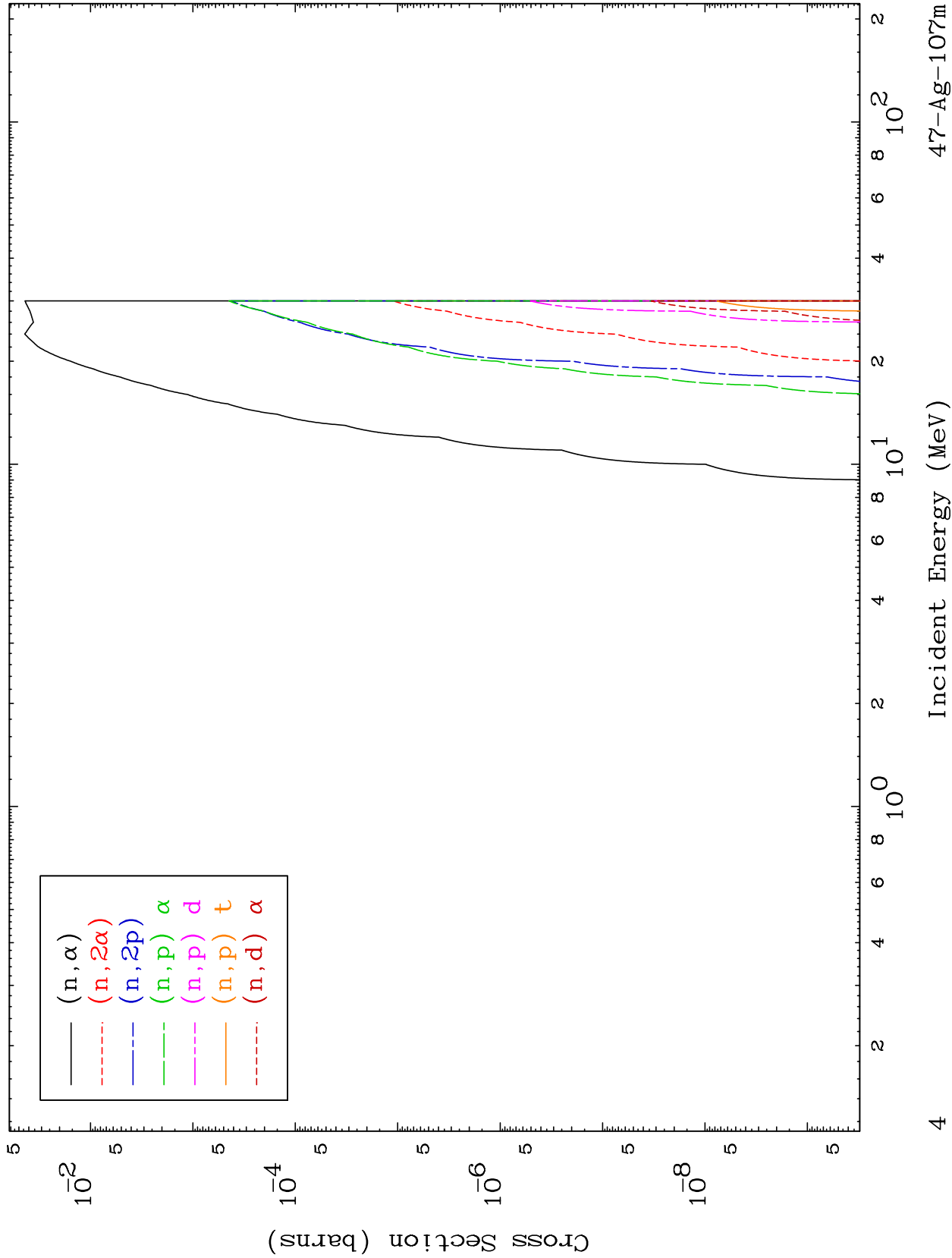


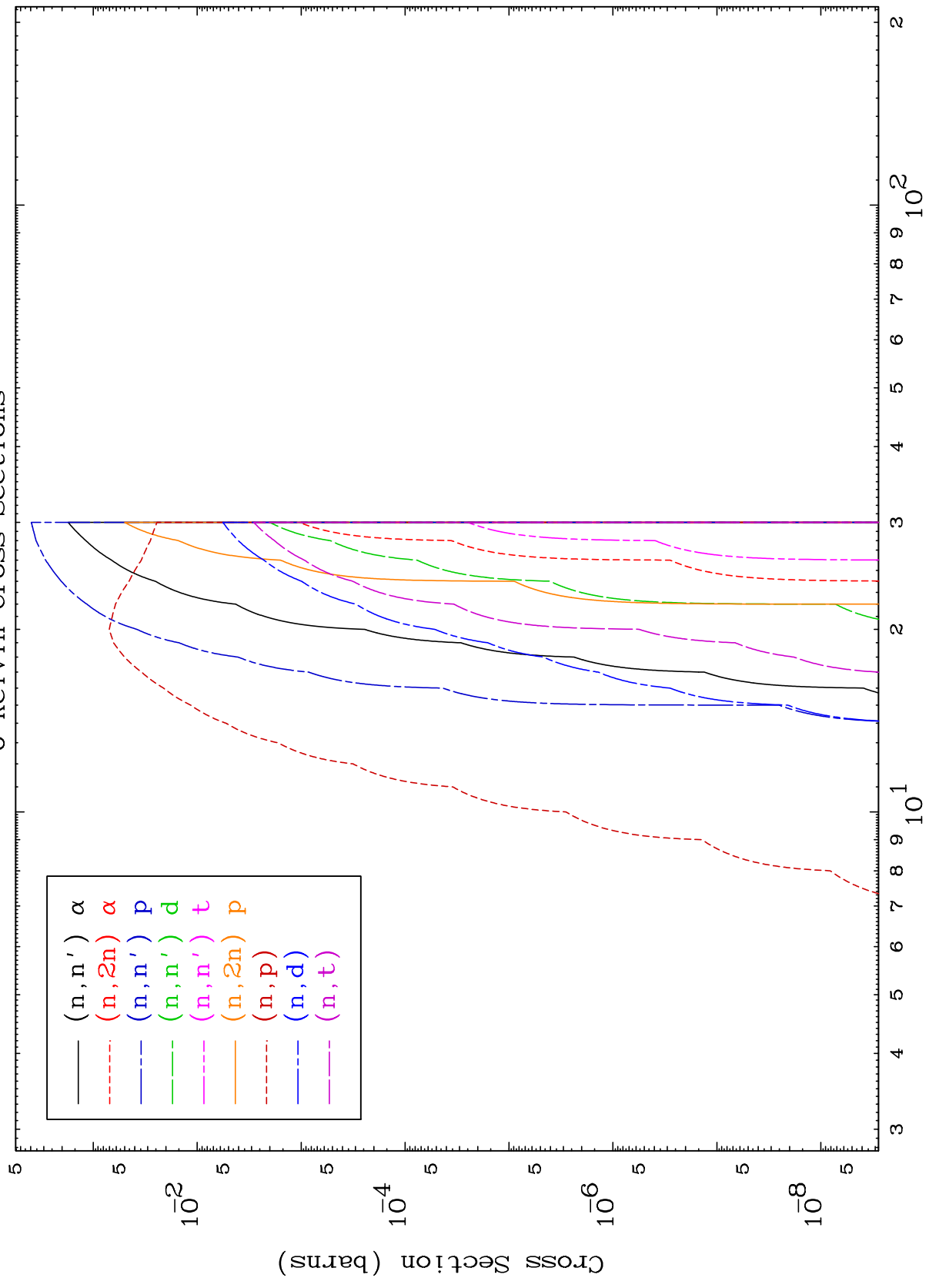
Incident Energy (MeV)

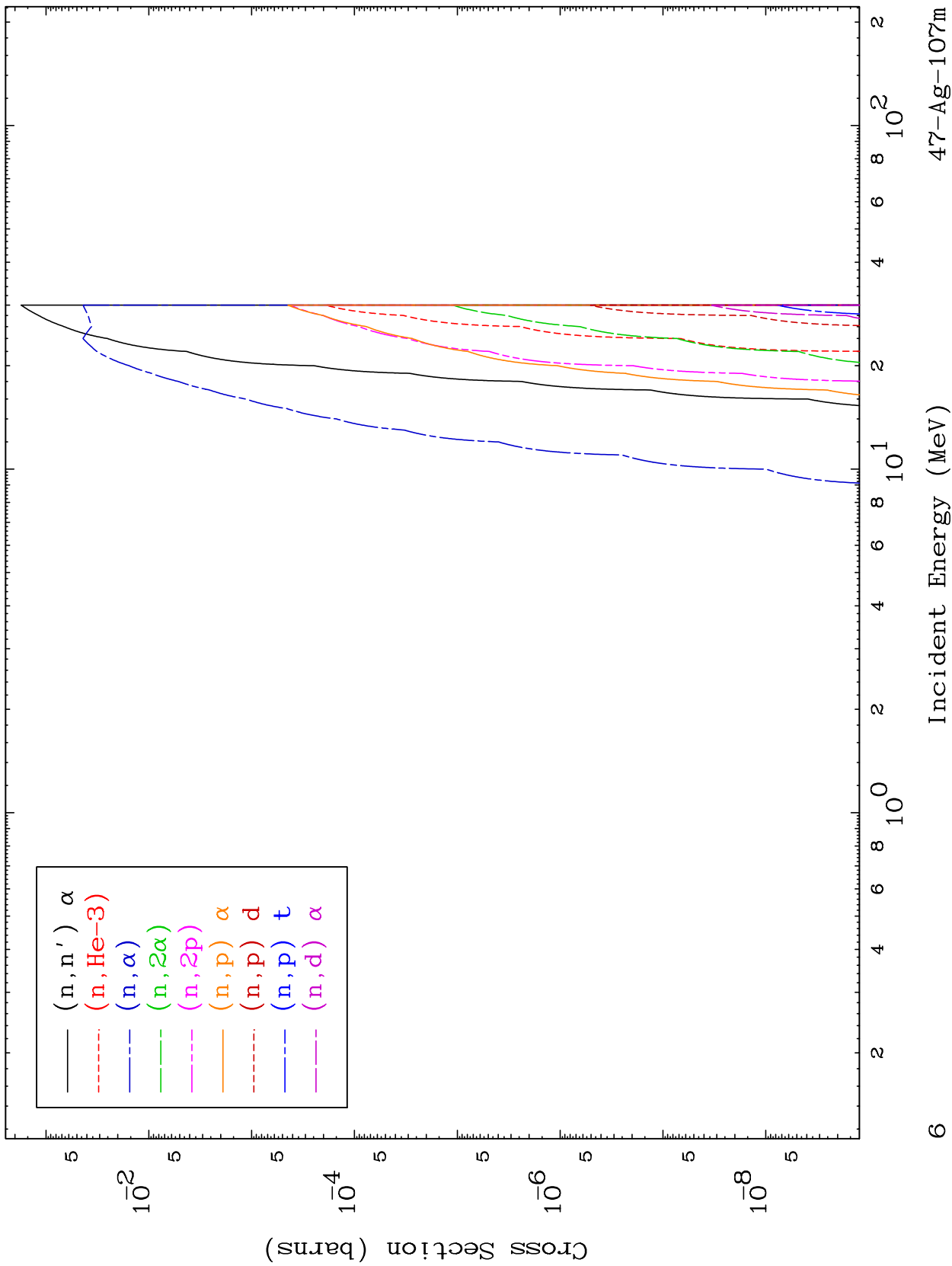
MAT 4726

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

47-Ag-107m





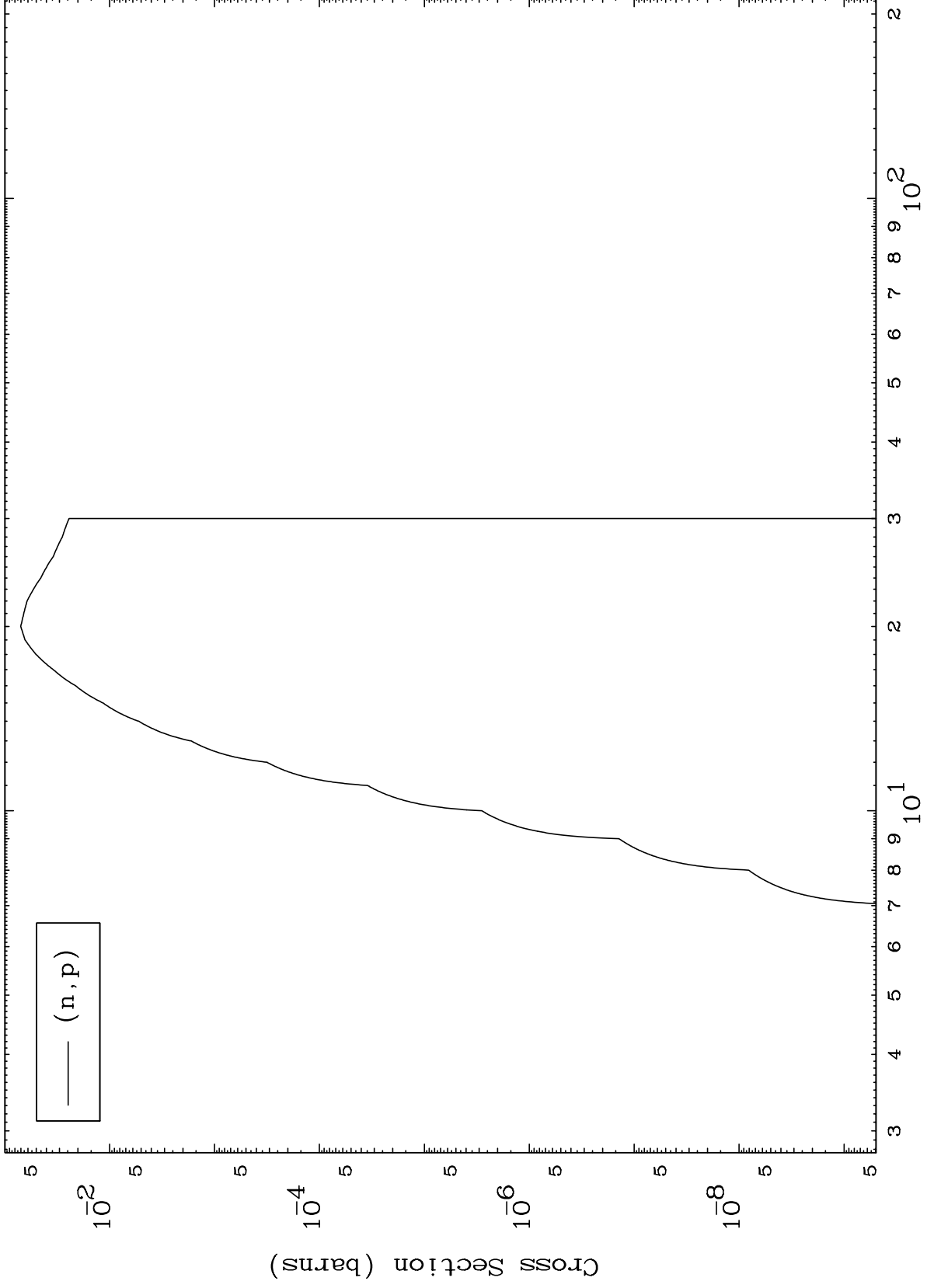


MAT 4726

( $\alpha, p$ ) Levels

47-Ag-107m

0 Kelvin Cross Sections



7

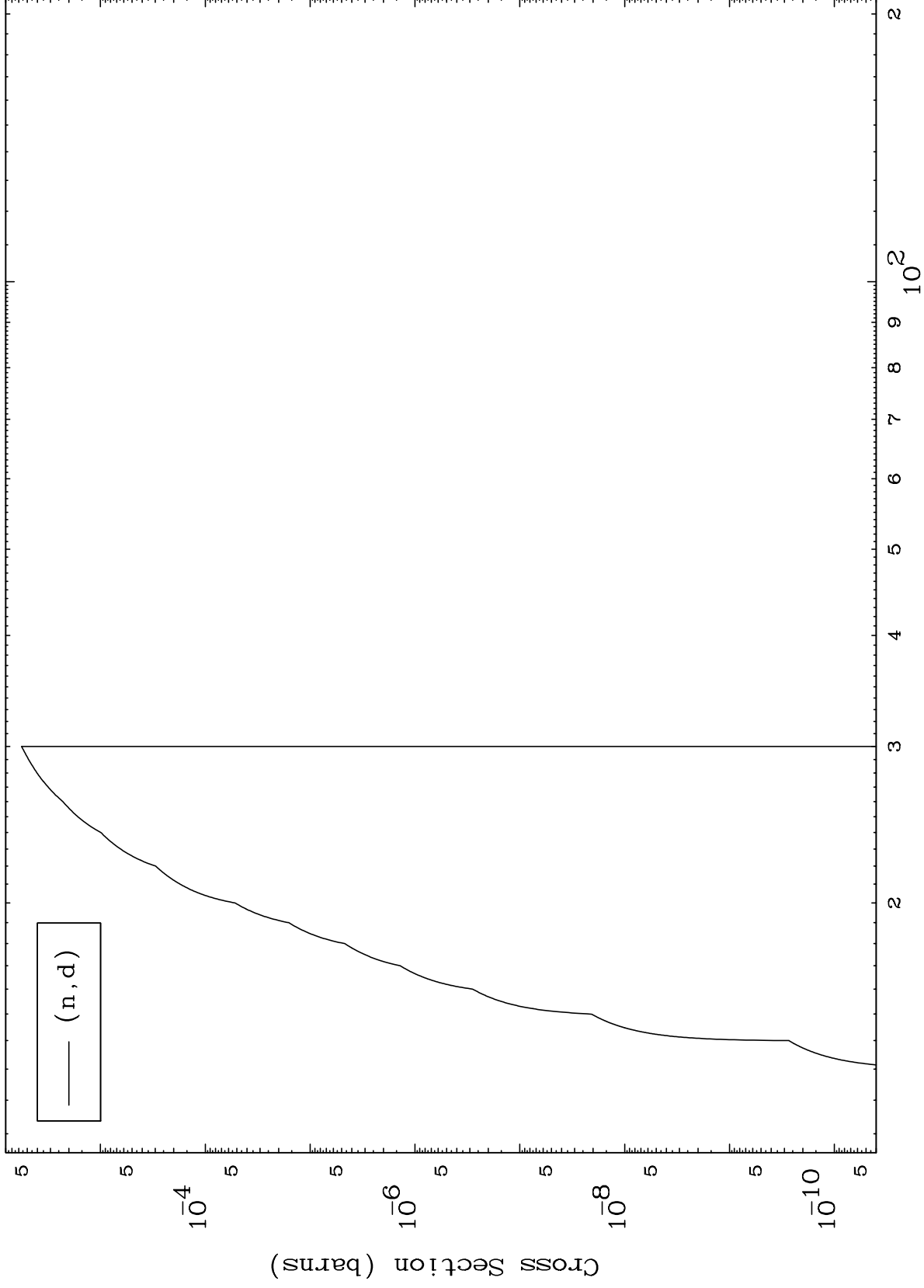
Incident Energy (MeV)

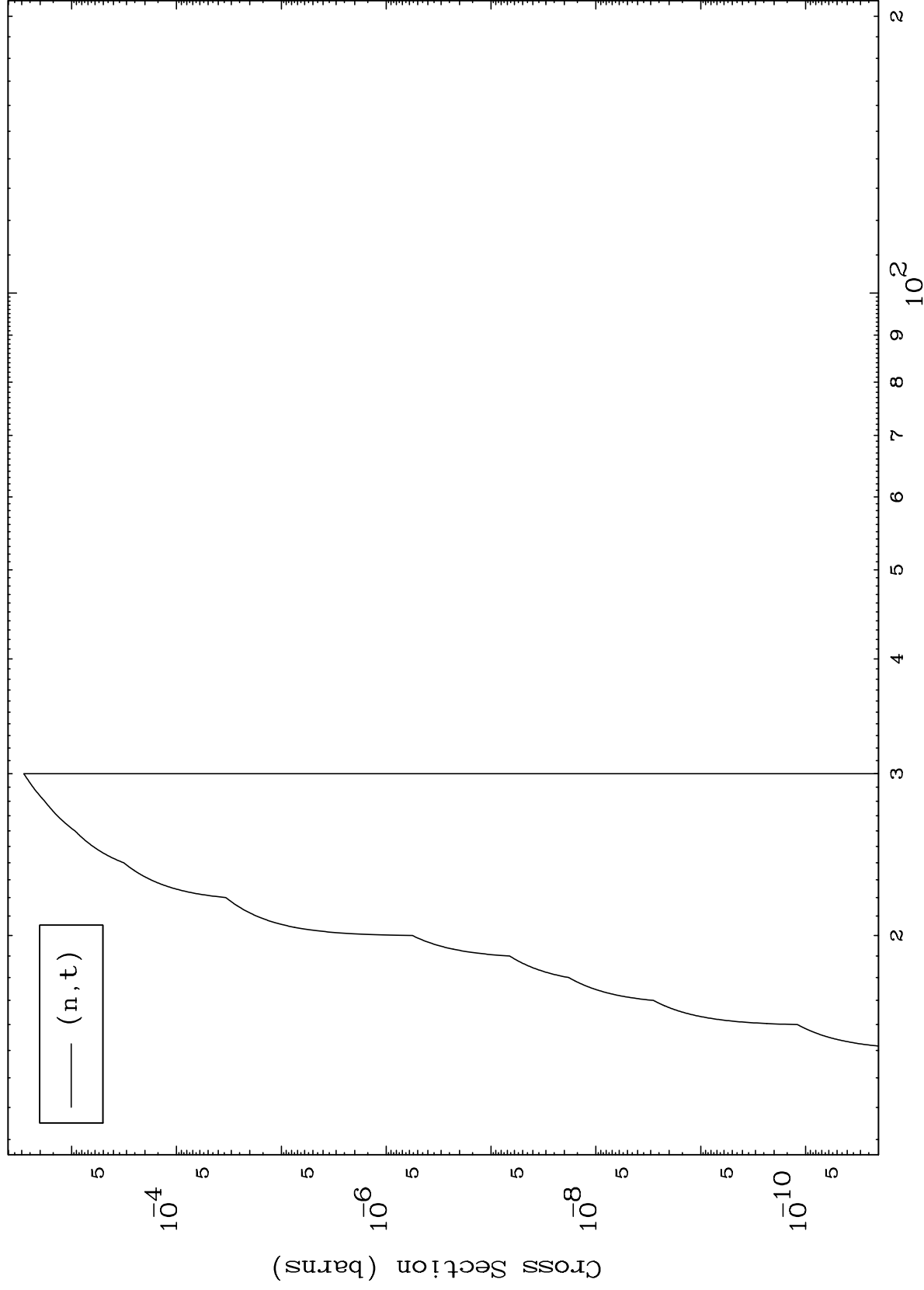
47-Ag-107m

MAT 4726

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

47-Ag-107m

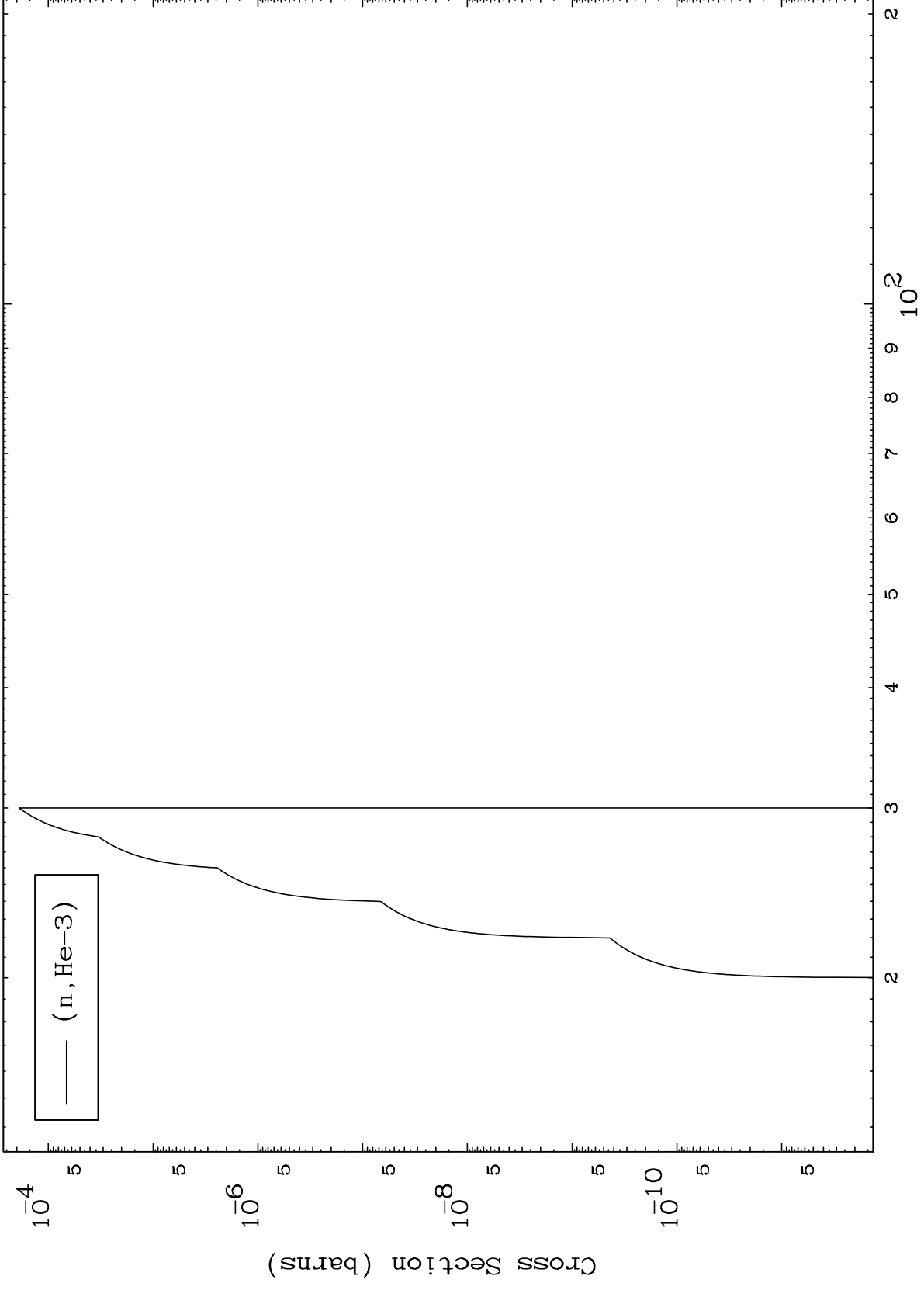




MAT 4726

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

47-Ag-107m



10

Incident Energy (MeV)

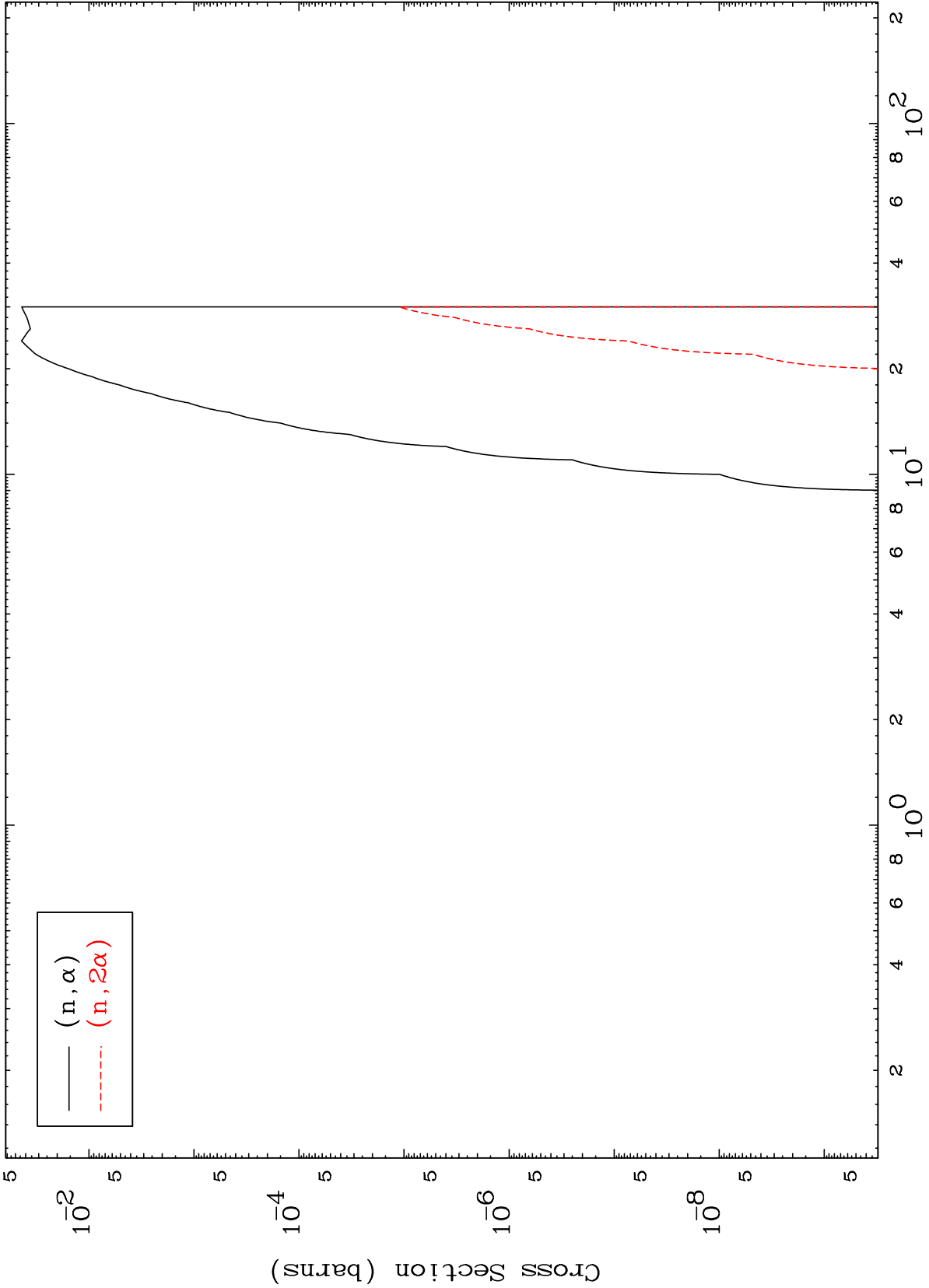
47-Ag-107m

MAT 4726

( $\alpha, \alpha$ ) Levels

47-Ag-107m

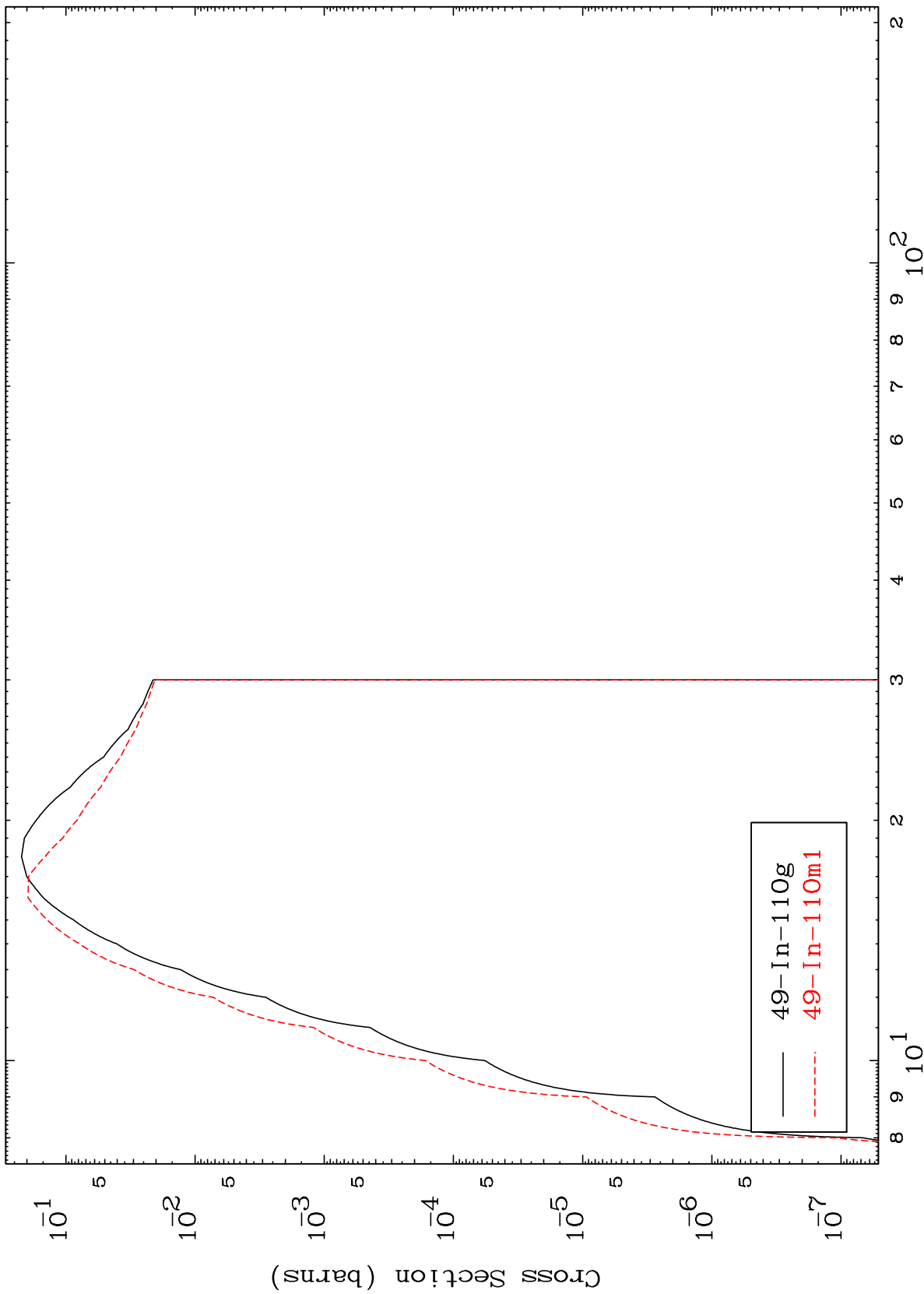
0 Kelvin Cross Sections



MAT 4726

47-Ag-107m

Inelastic  
Radionuclide Production Cross Section



47-Ag-107m

Incident Energy (MeV)

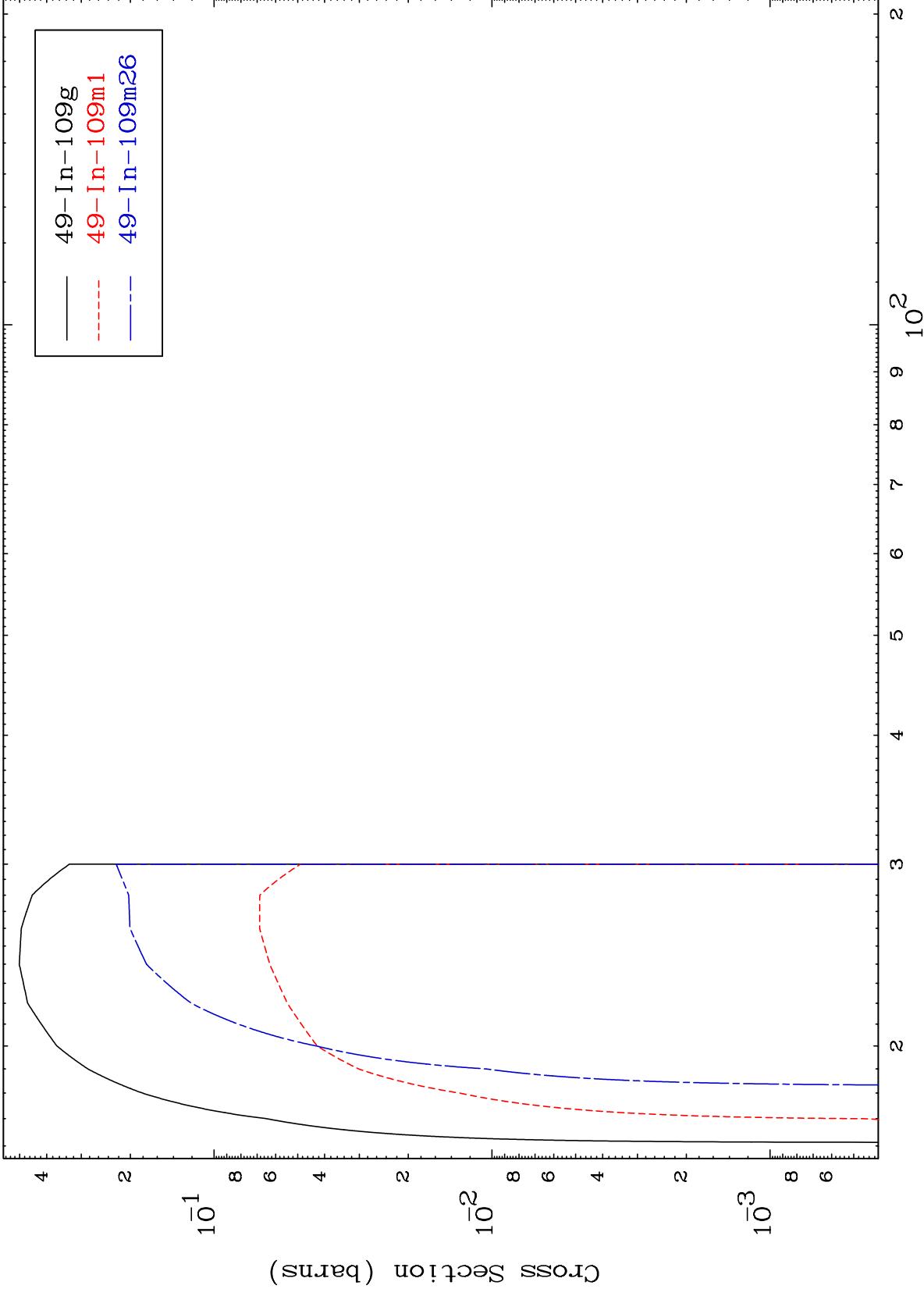
12

MAT 4726

(n,2n)

47-Ag-107m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

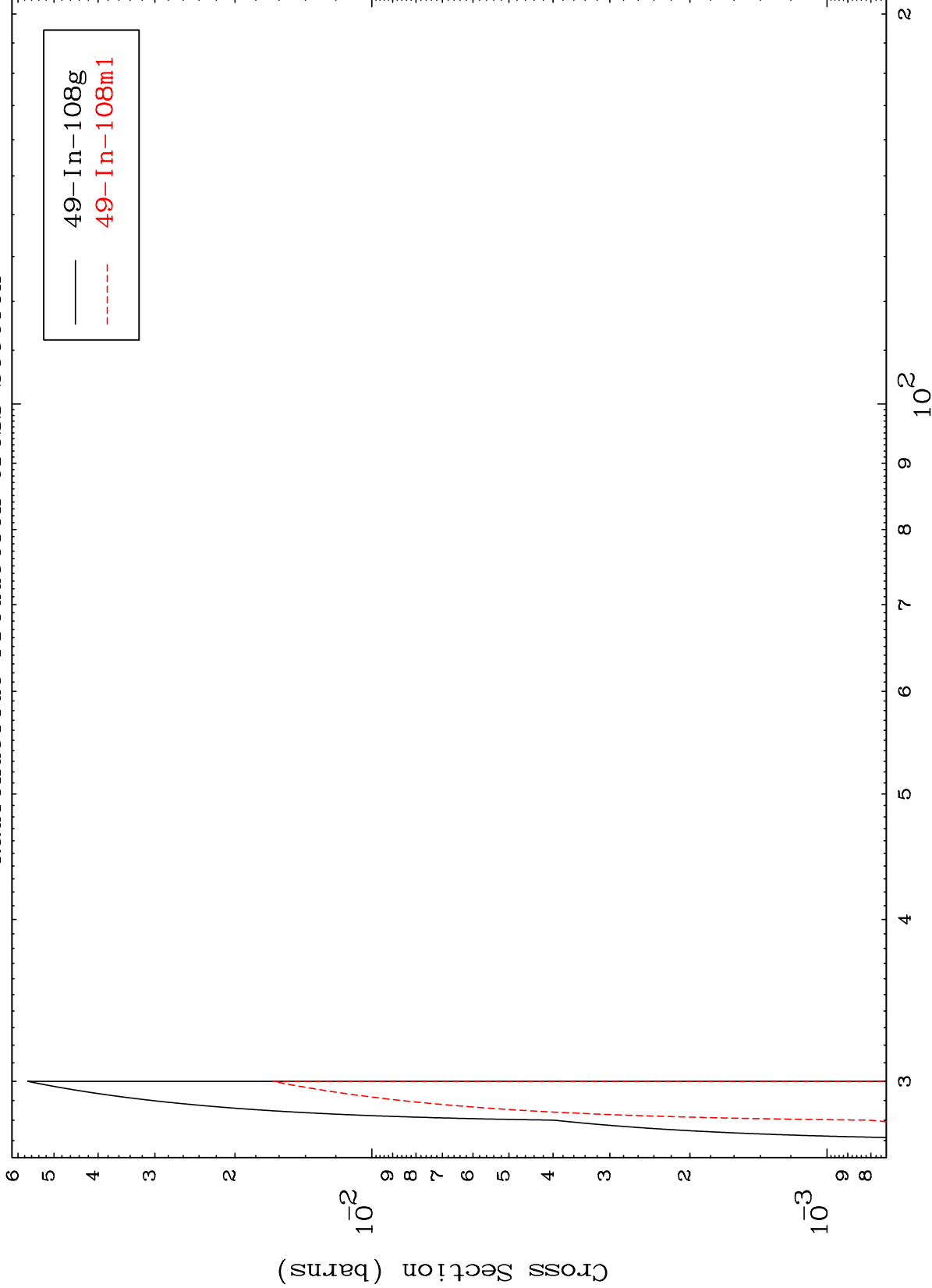
47-Ag-107m

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(n,3n)

47-Ag-107m

Radionuclide Production Cross Section



49-In-108g  
49-In-108m1

14

Incident Energy (MeV)

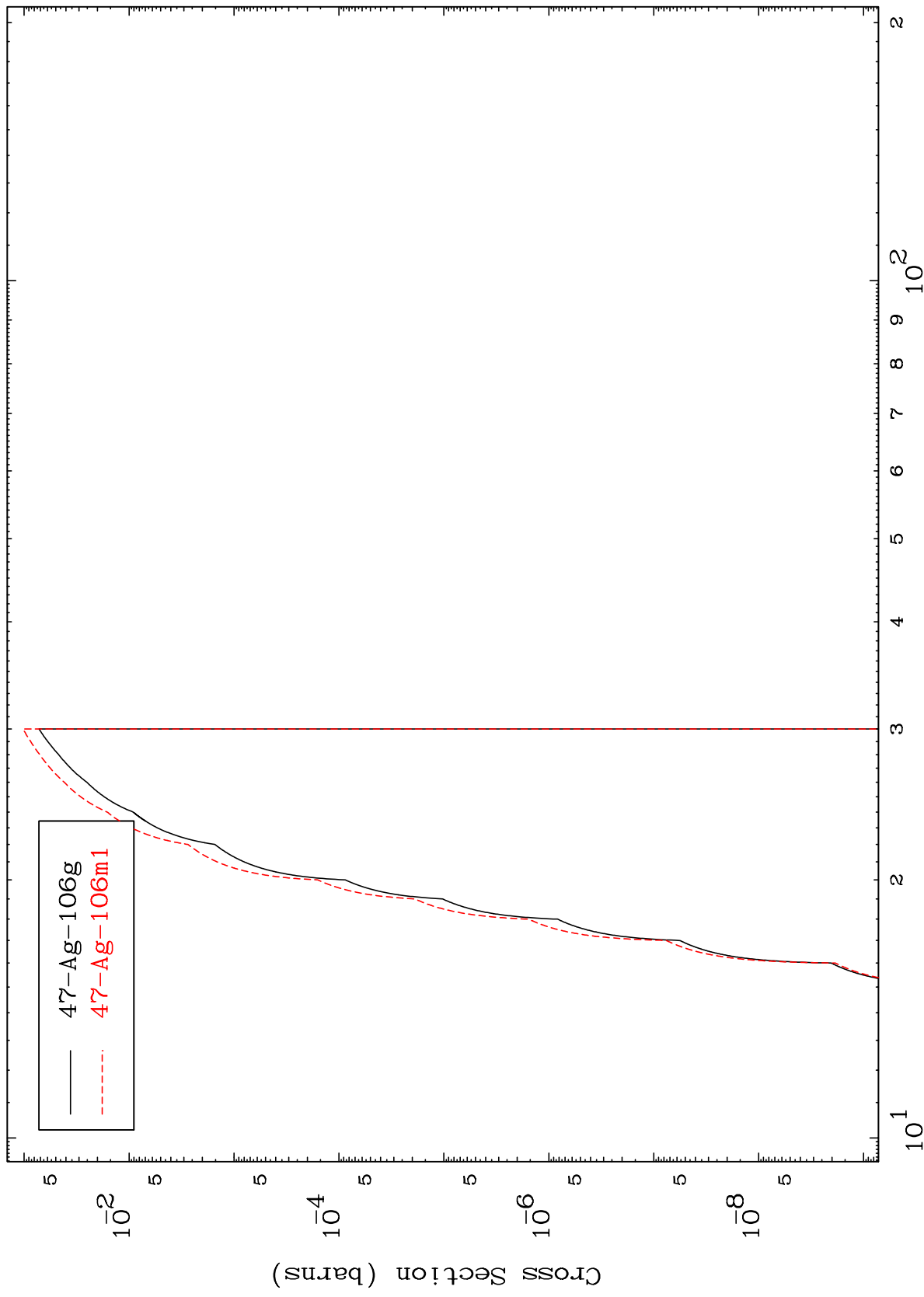
47-Ag-107m

MAT 4726

47-Ag-107m

(n,n')  $\alpha$

Radionuclide Production Cross Section



47-Ag-107m

Incident Energy (MeV)

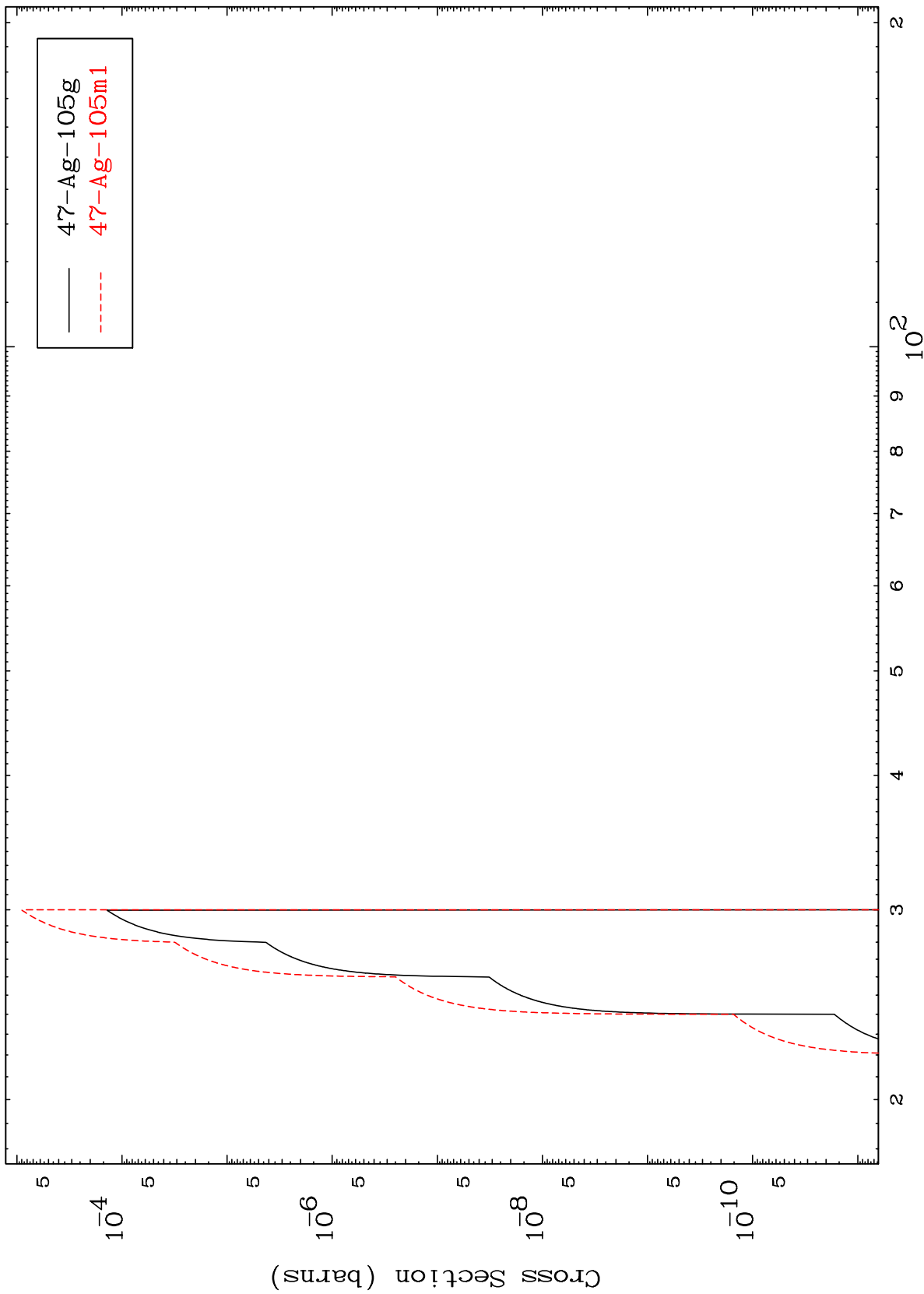
15

MAT 4726

(n,2n)  $\alpha$

47-Ag-107m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

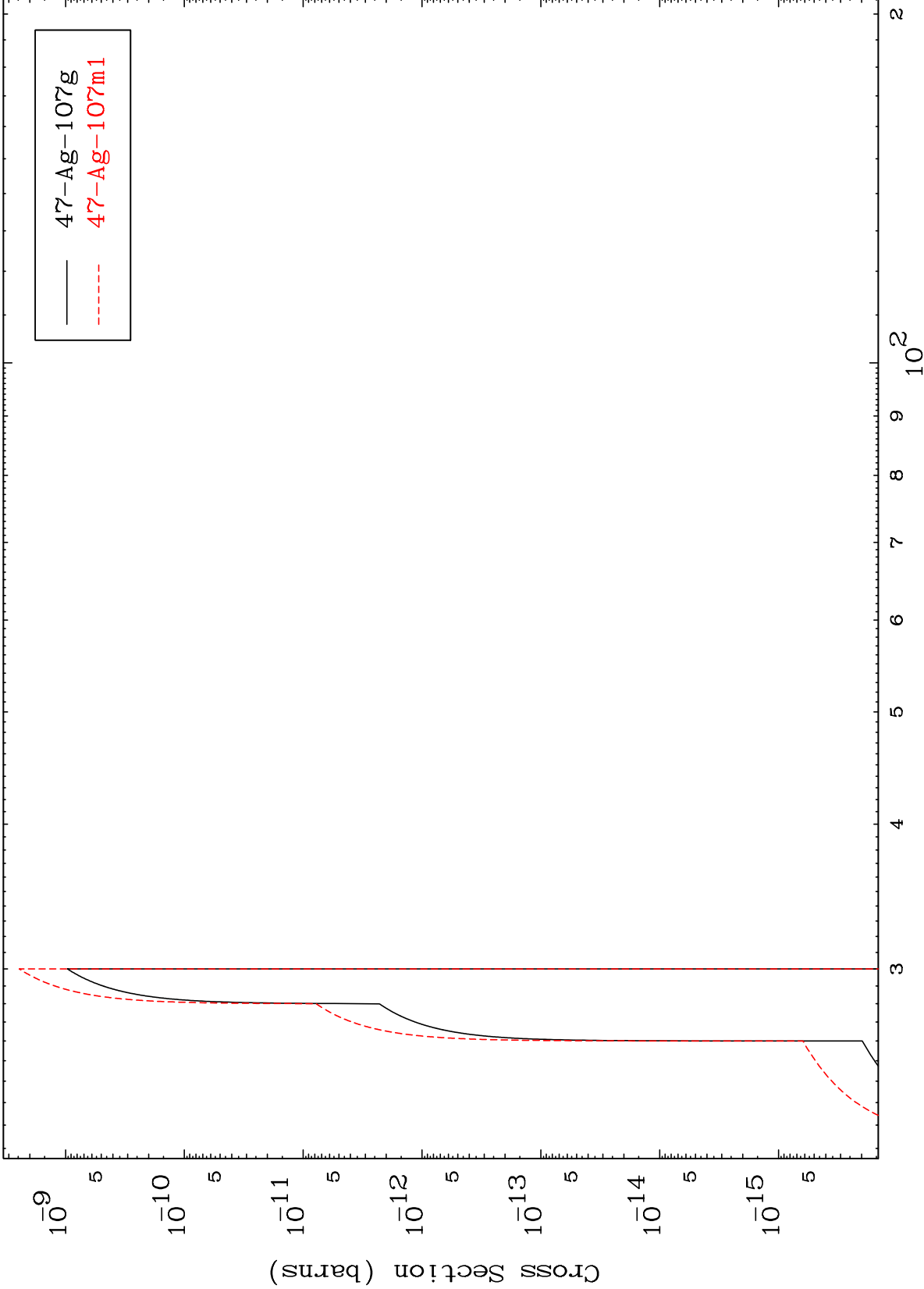
47-Ag-107m

MAT 4726

(n,n') He-3

47-Ag-107m

Radionuclide Production Cross Section



17

Incident Energy (MeV)

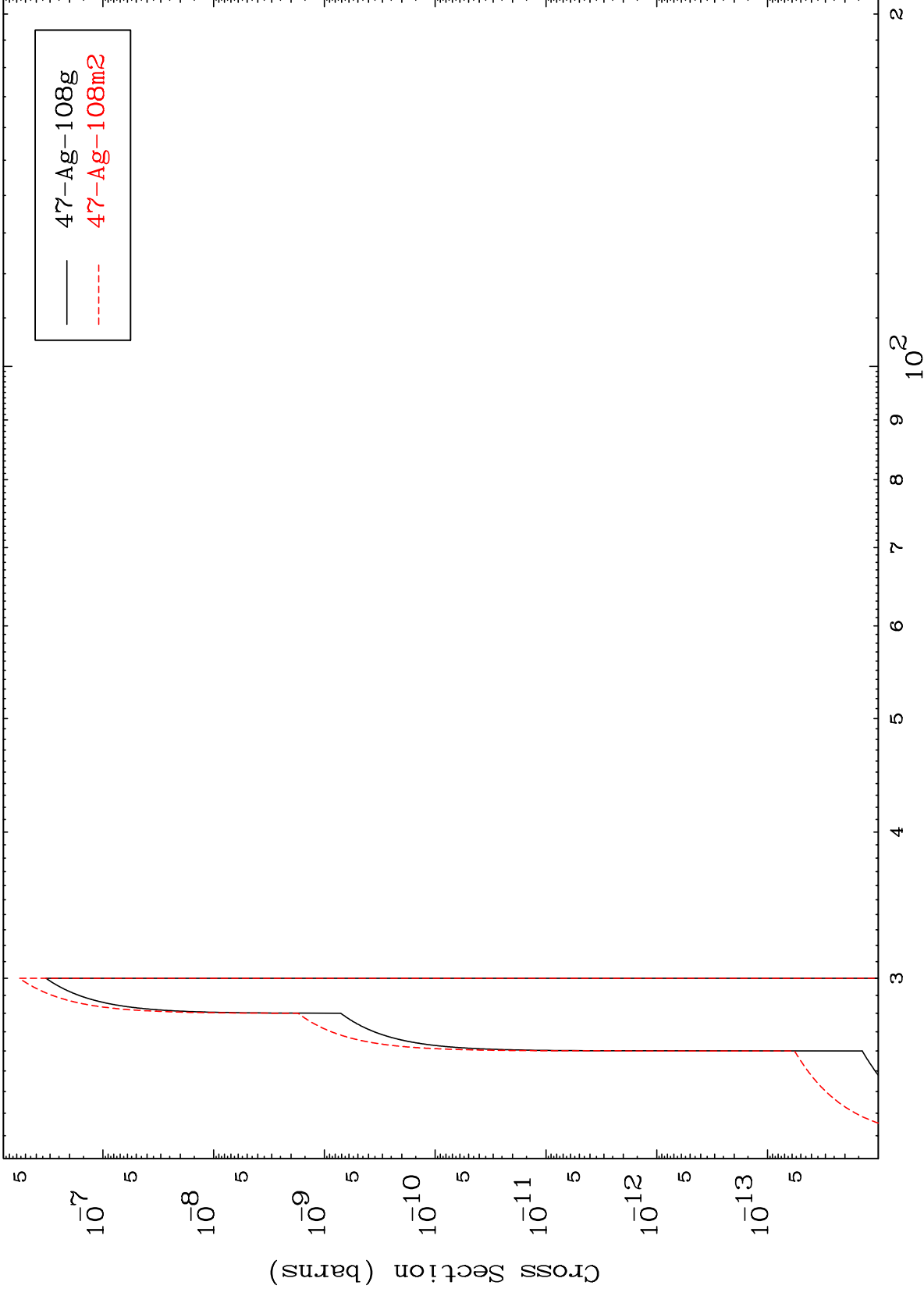
47-Ag-107m

MAT 4726

(n,2n) p

47-Ag-107m

Radionuclide Production Cross Section



18

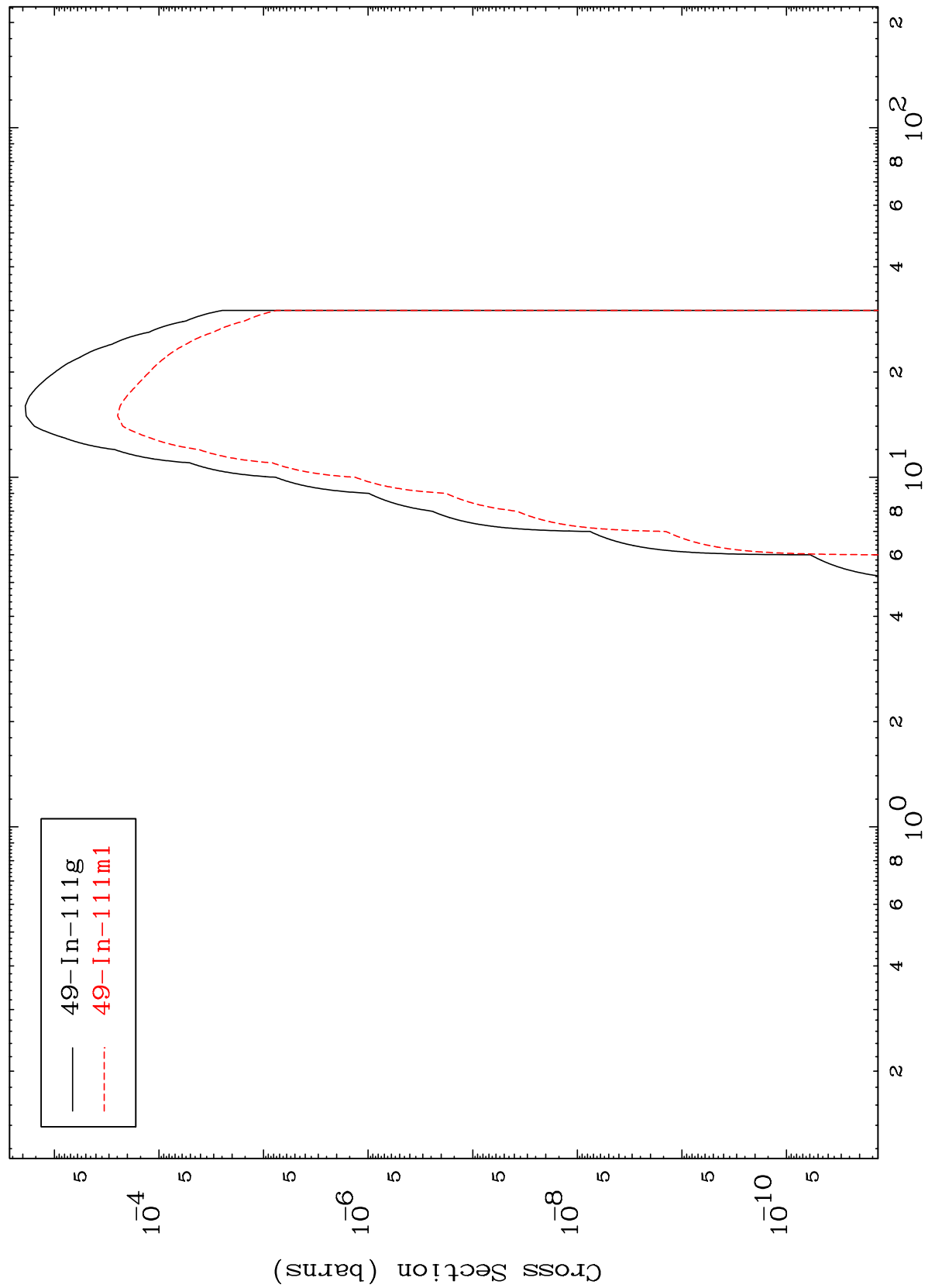
Incident Energy (MeV)

47-Ag-107m

MAT 4726

47-Ag-107m

Radionuclide Production Cross Section  
(n,  $\gamma$ )



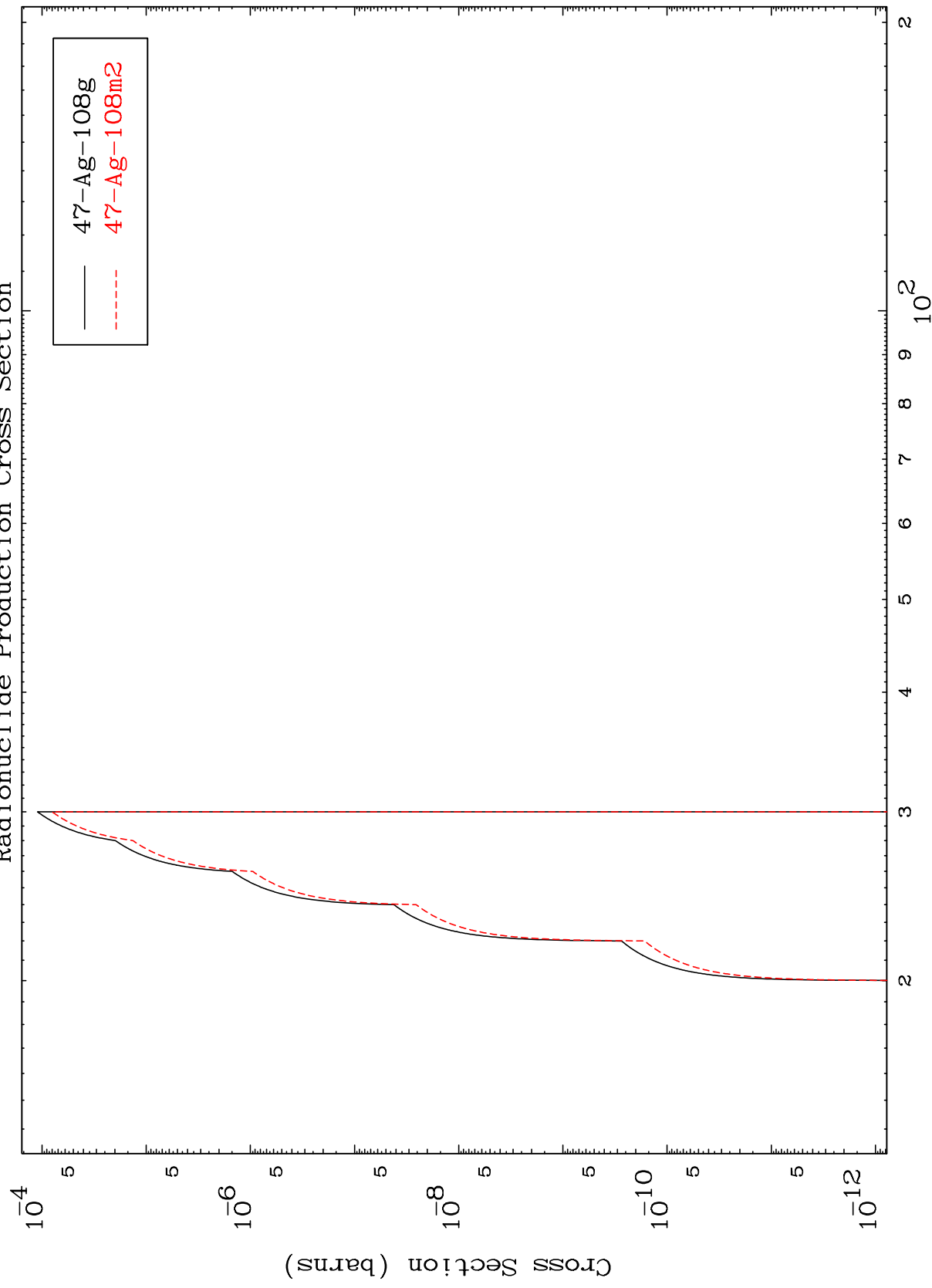
— 49-In-111g  
- - - 49-In-111m1

MAT 4726

47-Ag-107m

(n,He-3)

Radionuclide Production Cross Section



20

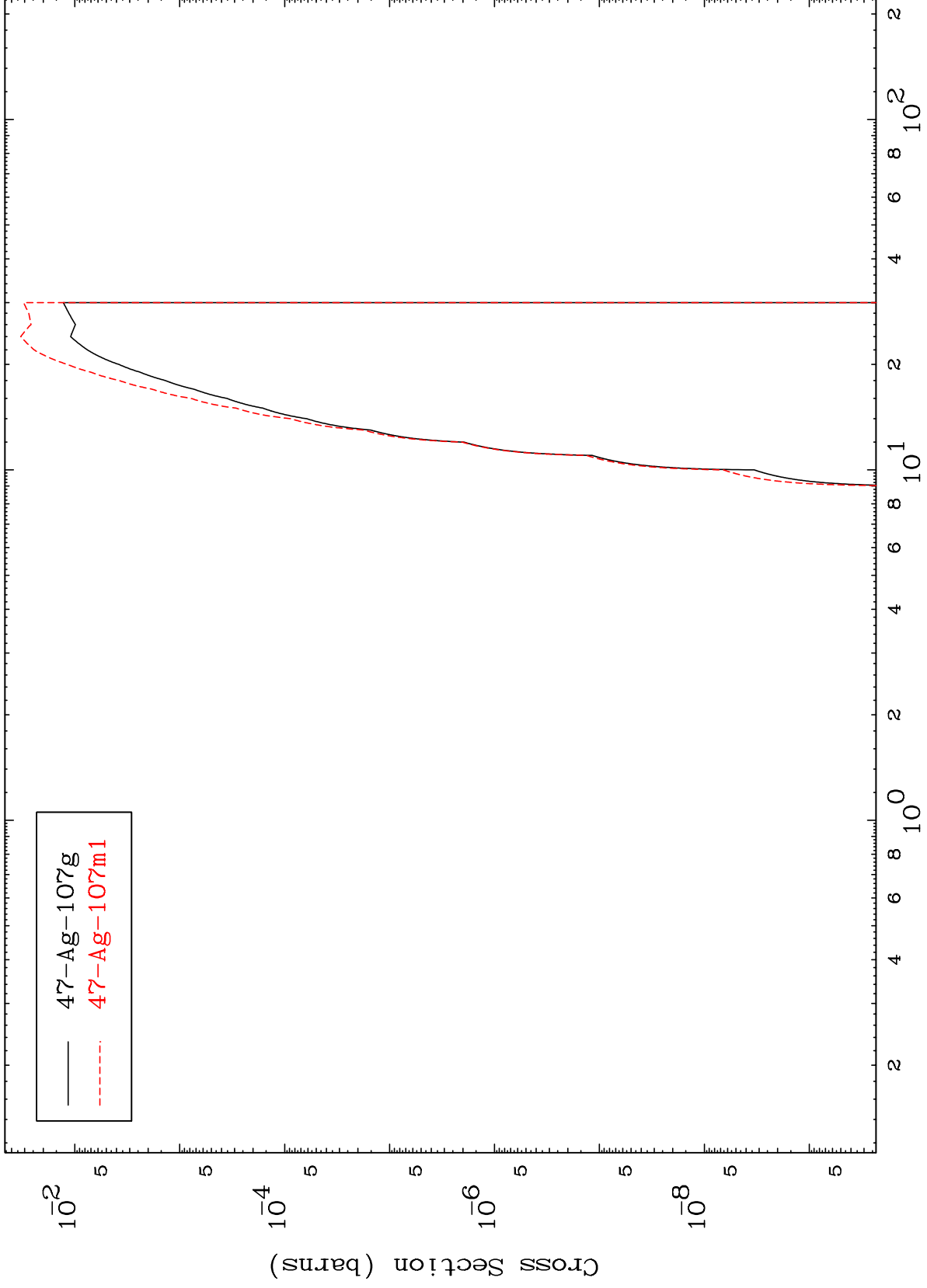
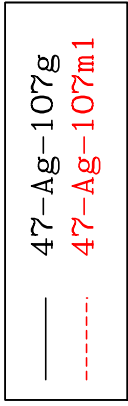
Incident Energy (MeV)

47-Ag-107m

MAT 4726

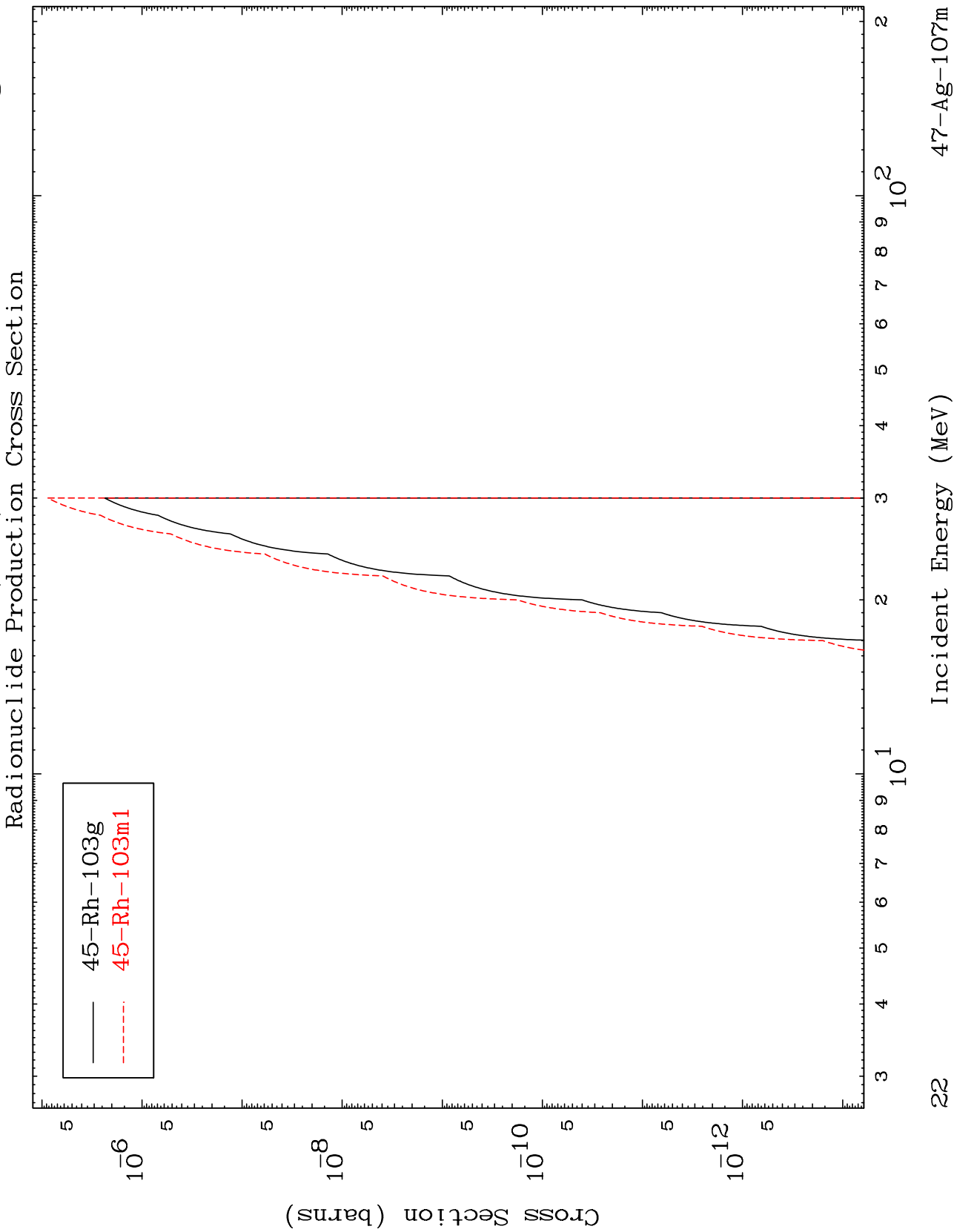
47-Ag-107m

Radionuclide Production Cross Section



MAT 4726

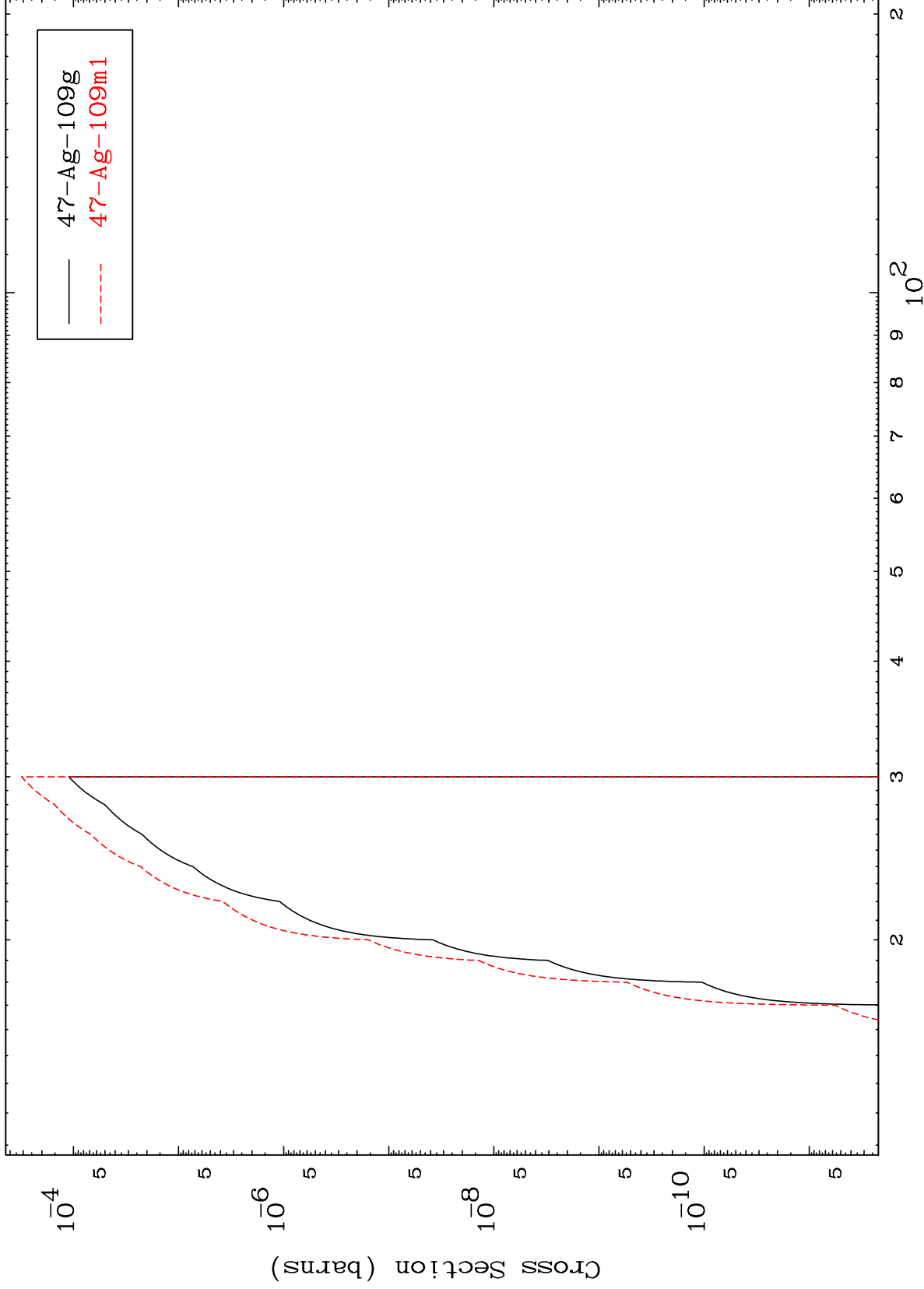
47-Ag-107m



MAT 4726

47-Ag-107m

(n,2p)  
Radionuclide Production Cross Section



47-Ag-107m

Incident Energy (MeV)

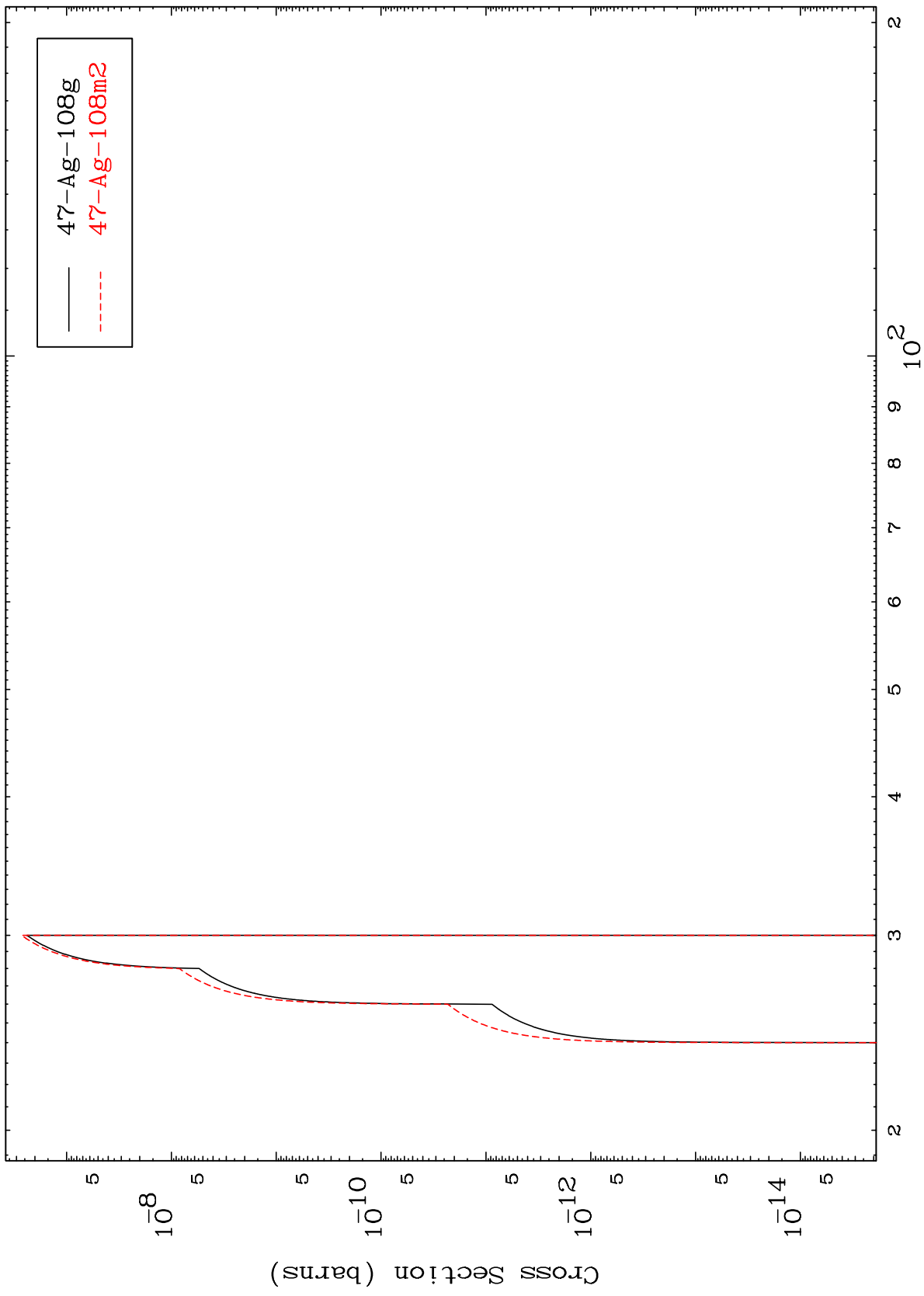
23

MAT 4726

(n,p) d

47-Ag-107m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

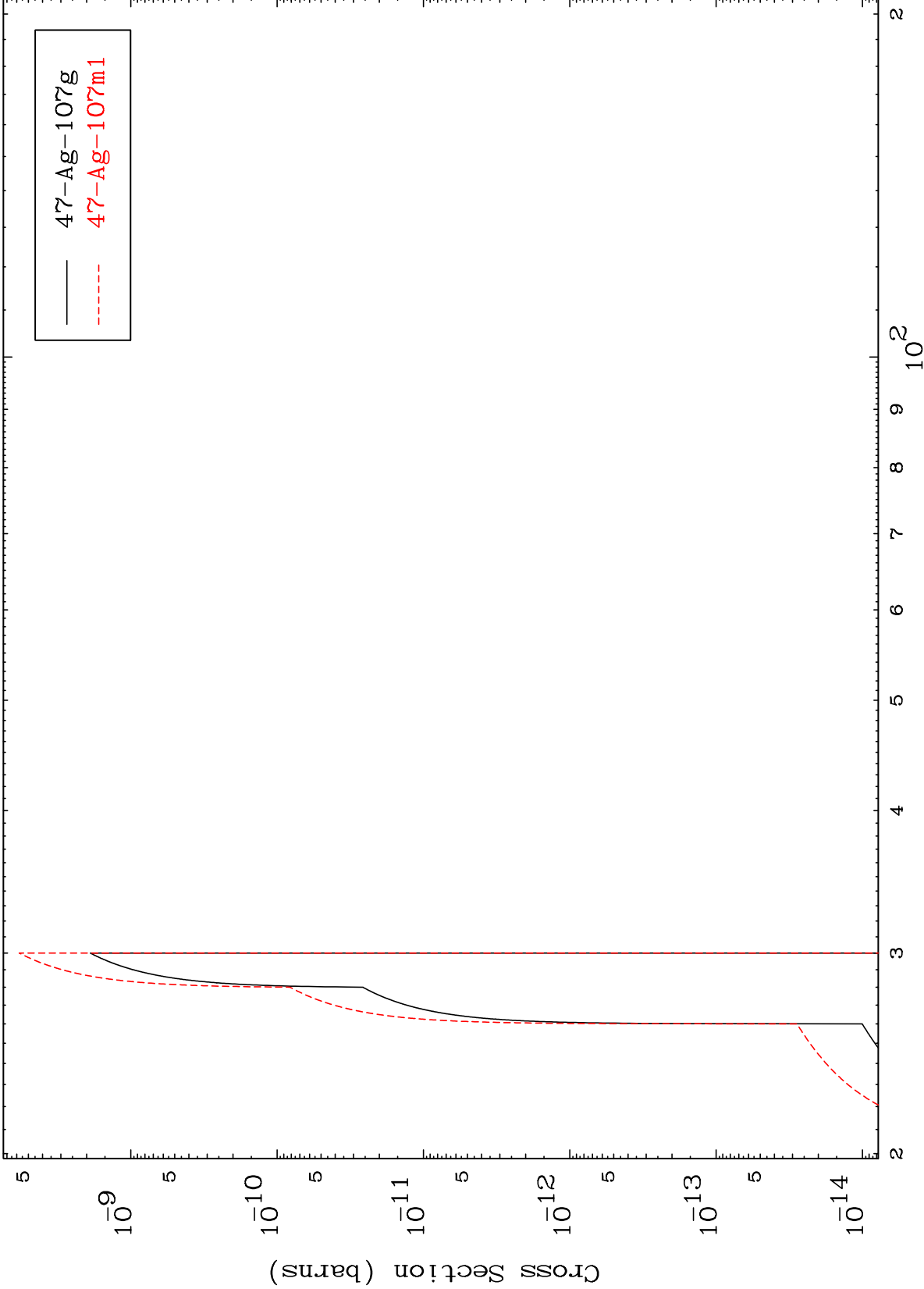
47-Ag-107m

MAT 4726

(n,p) t

47-Ag-107m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

47-Ag-107m