

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

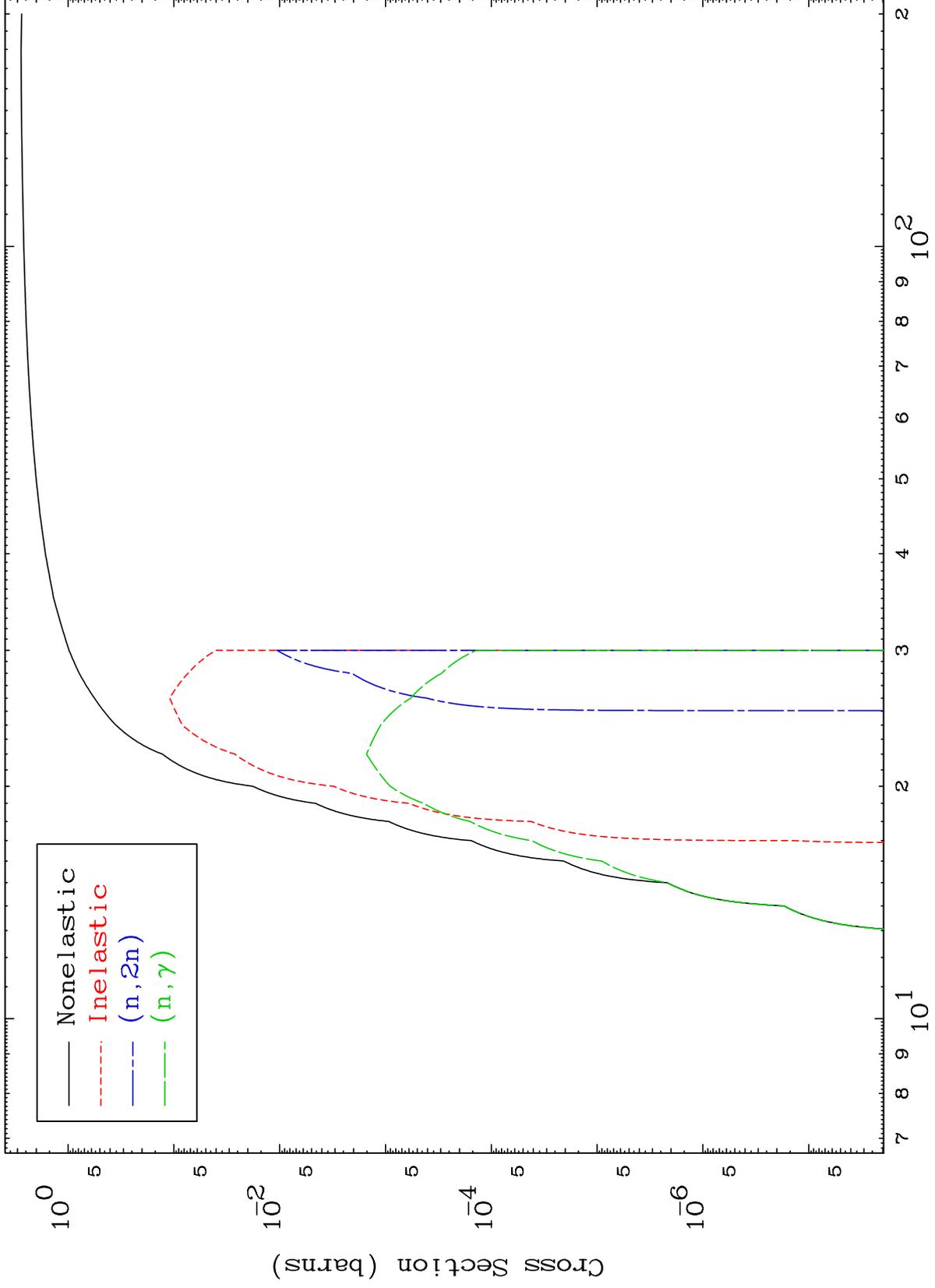
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Press Mouse Button to Start

MAT 8525

0 Kelvin Cross Sections

85-At-203

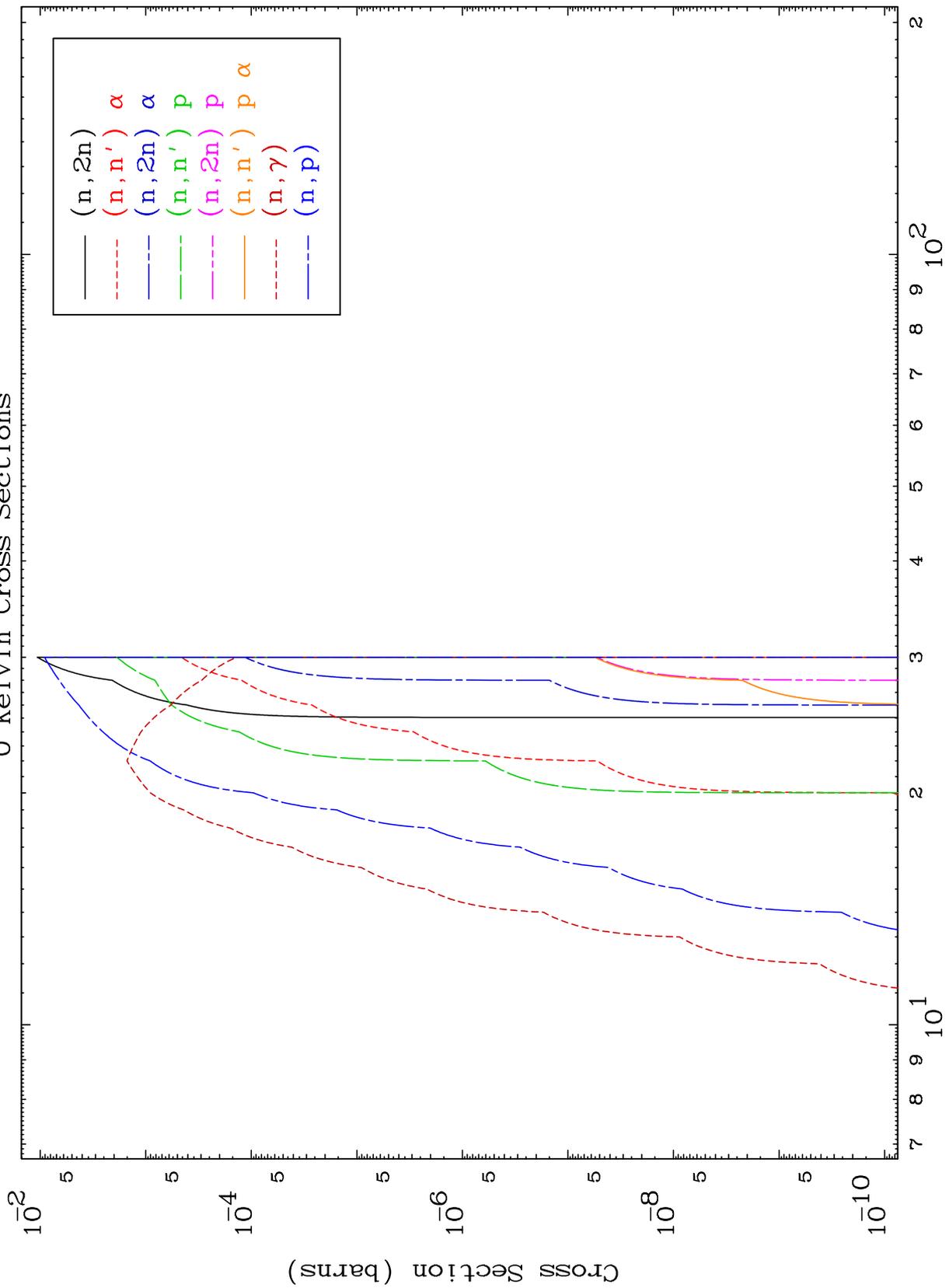


1

Incident Energy (MeV)

85-At-203

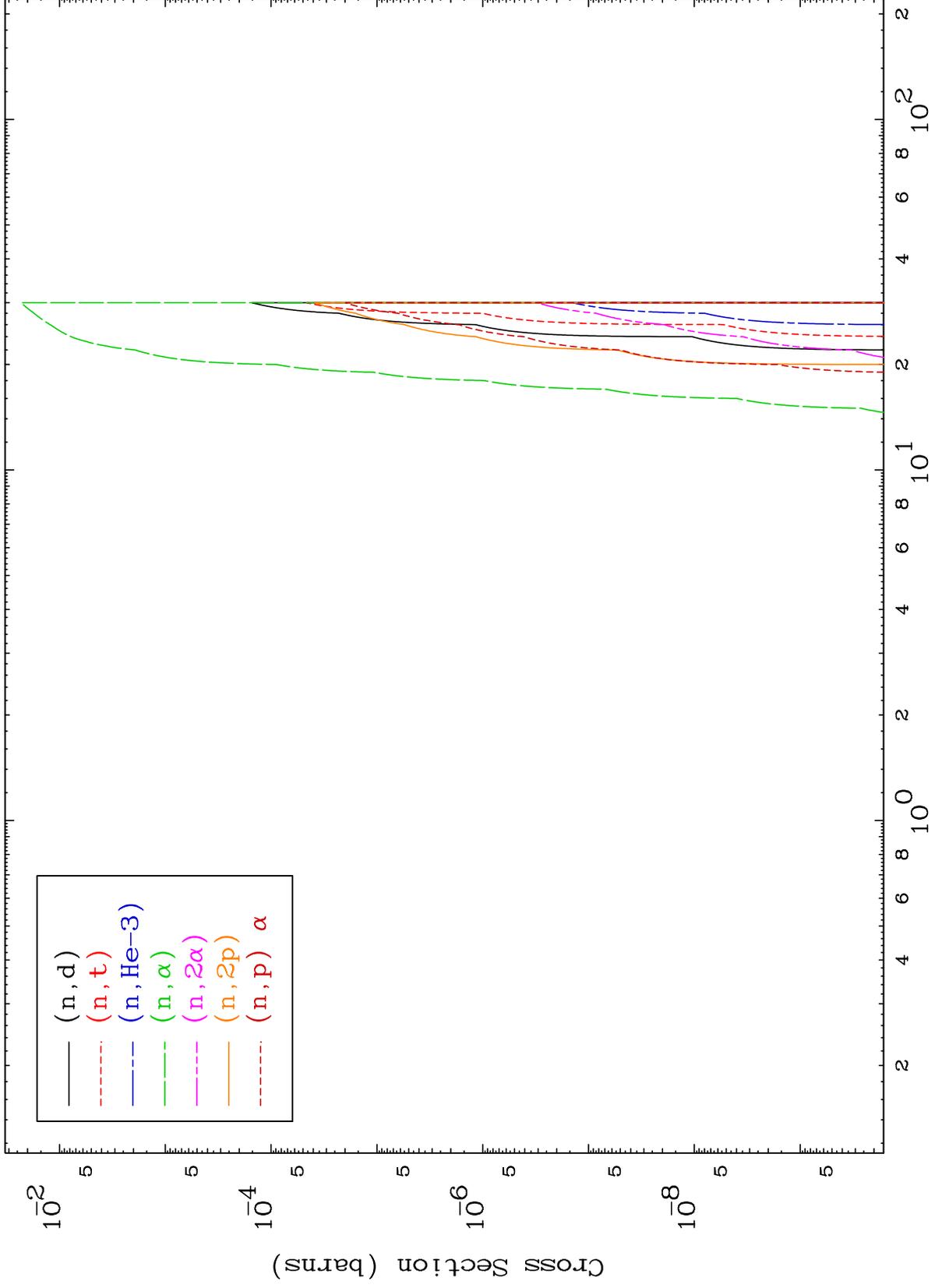
α Neutron Absorption
0 Kelvin Cross Sections

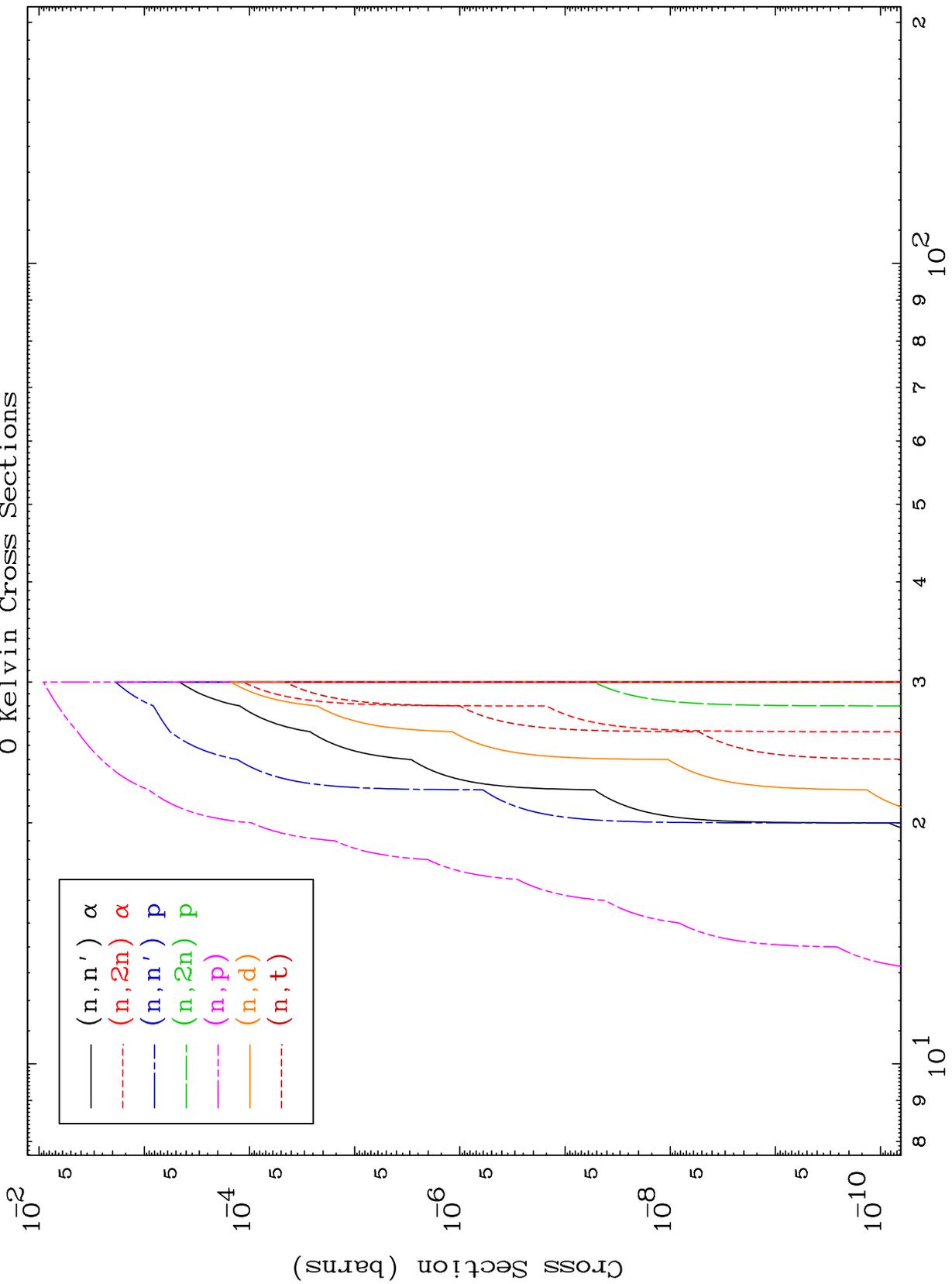


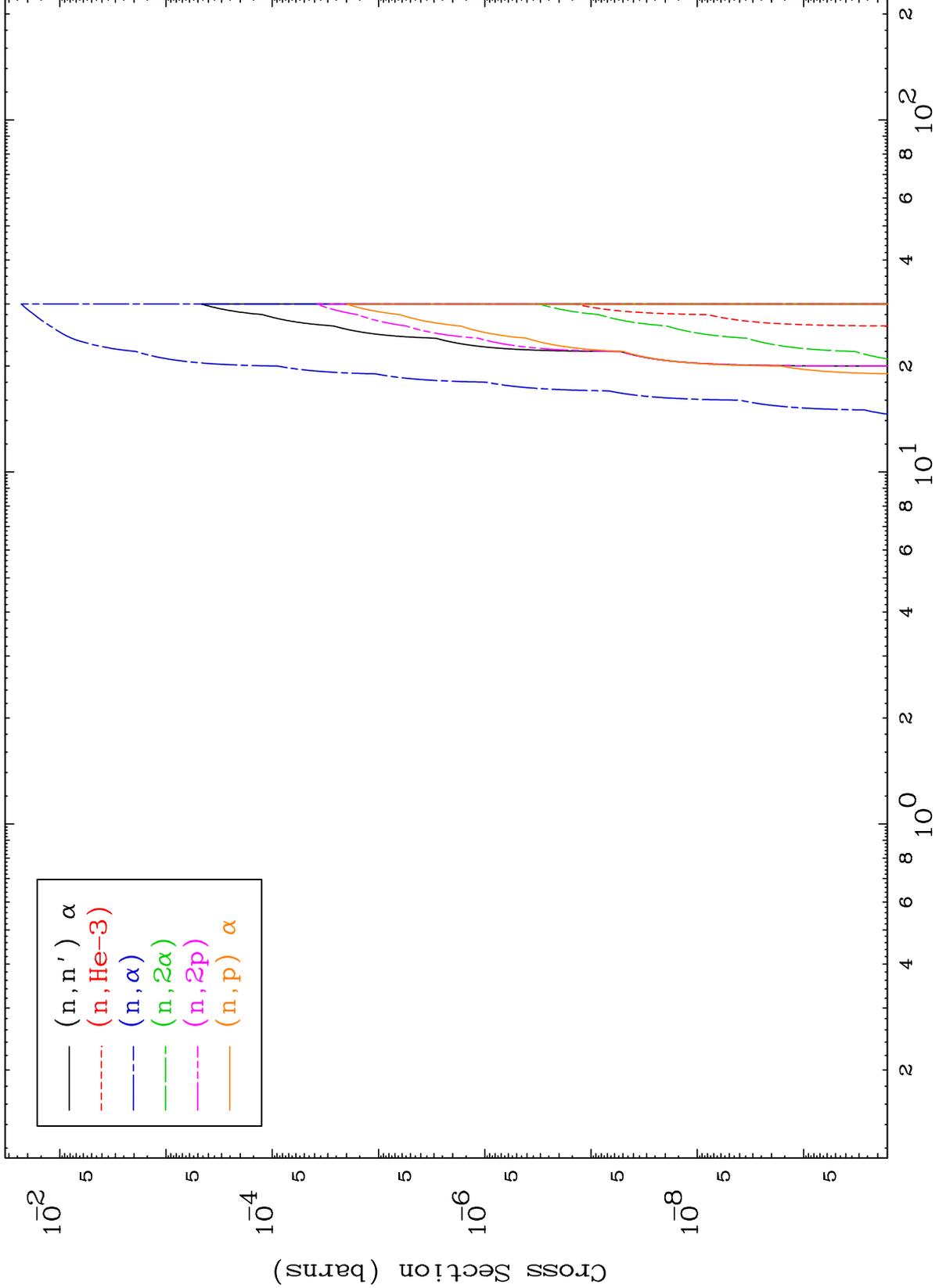
MAT 8525

α Neutron Absorption
0 Kelvin Cross Sections

85-At-203



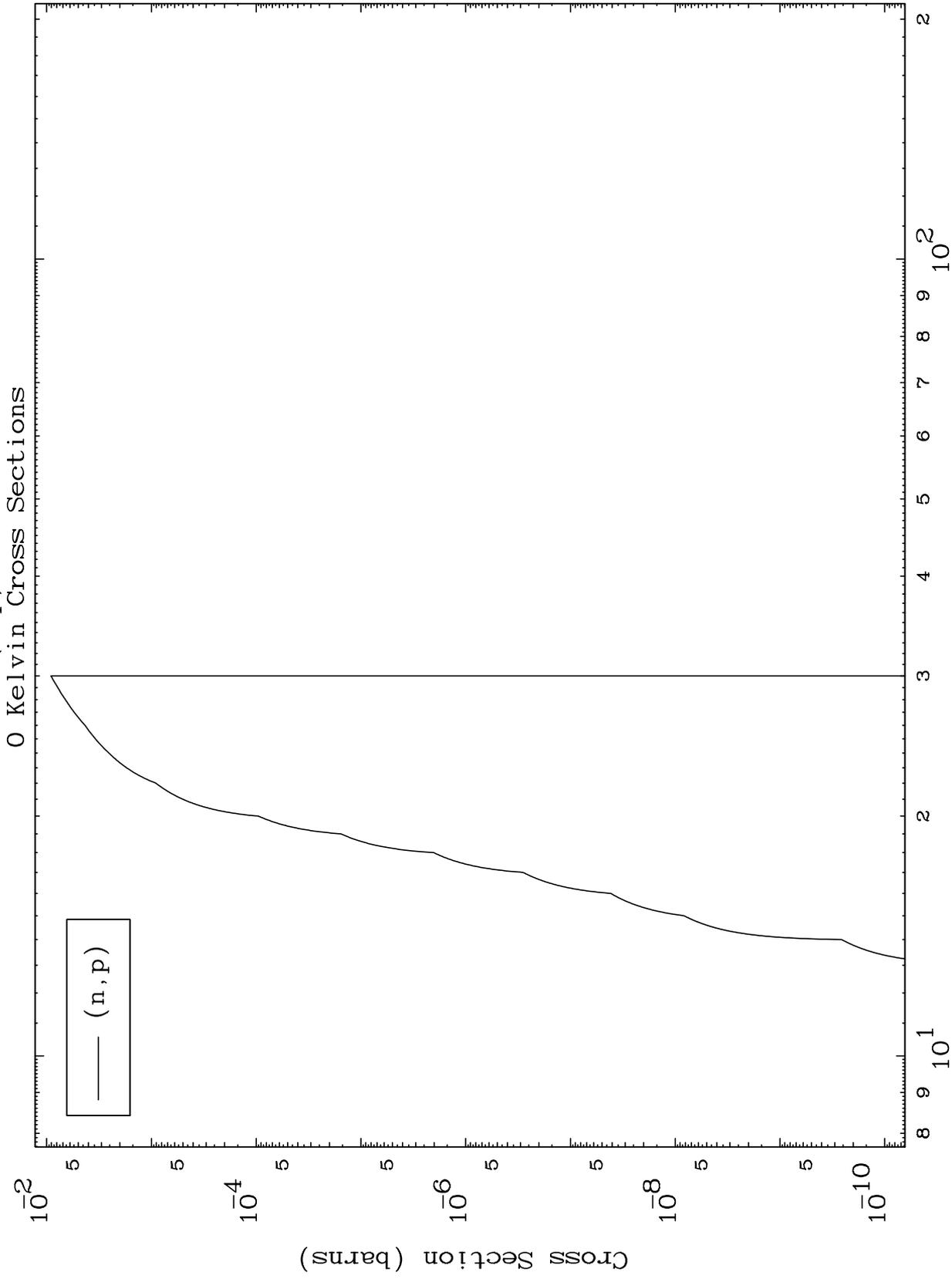




MAT 8525

(α, p) Levels

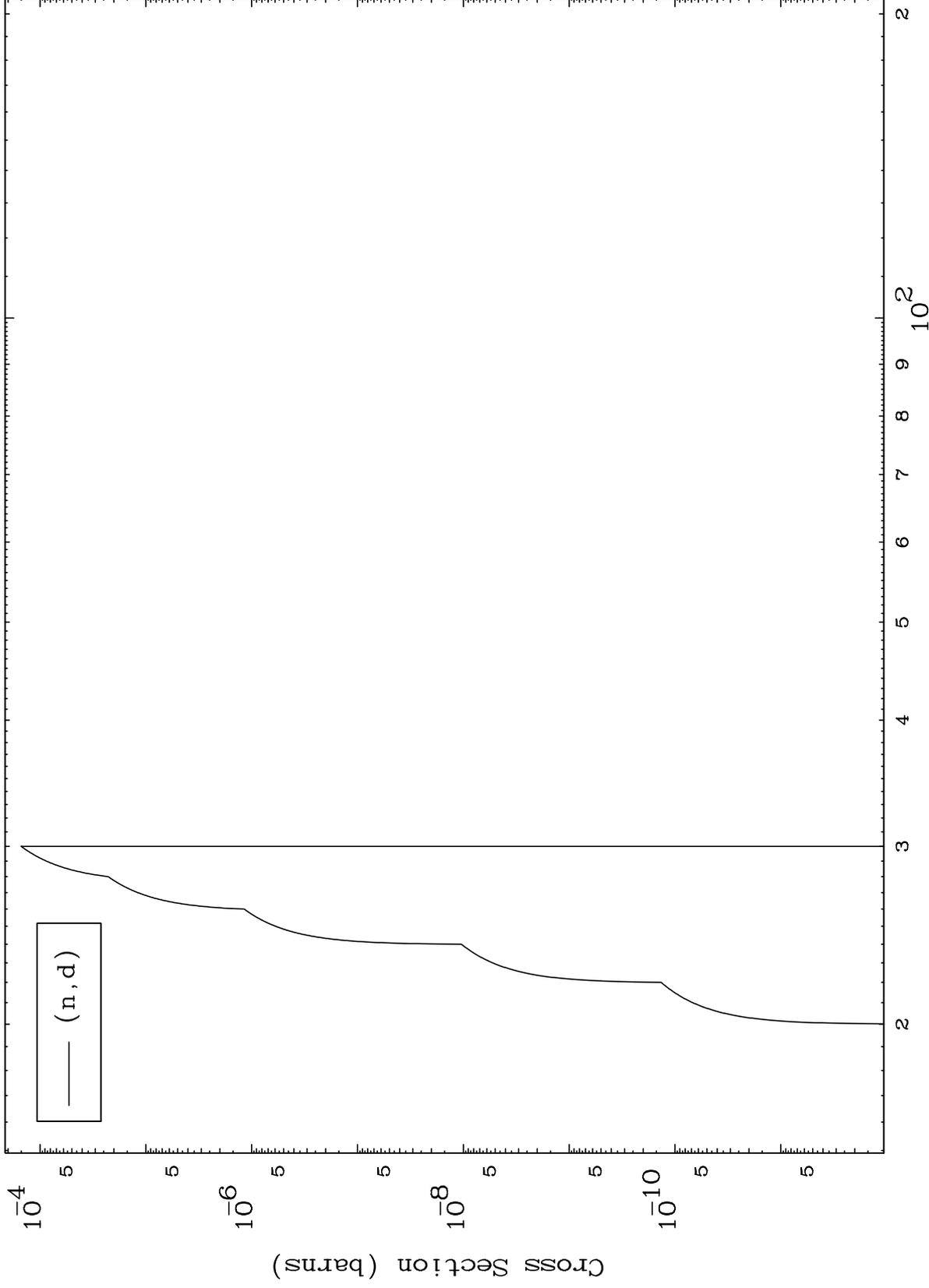
85-At-203

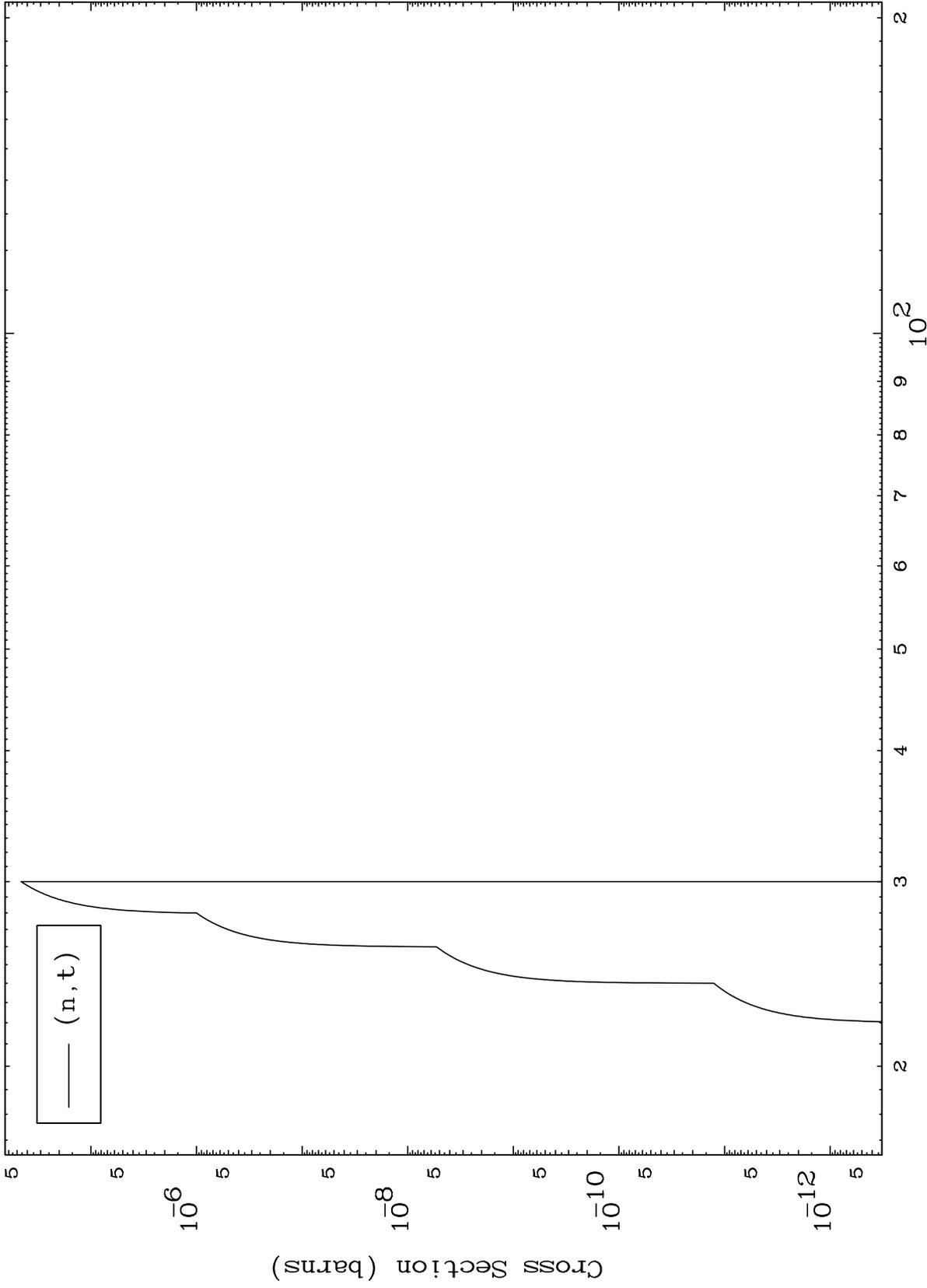


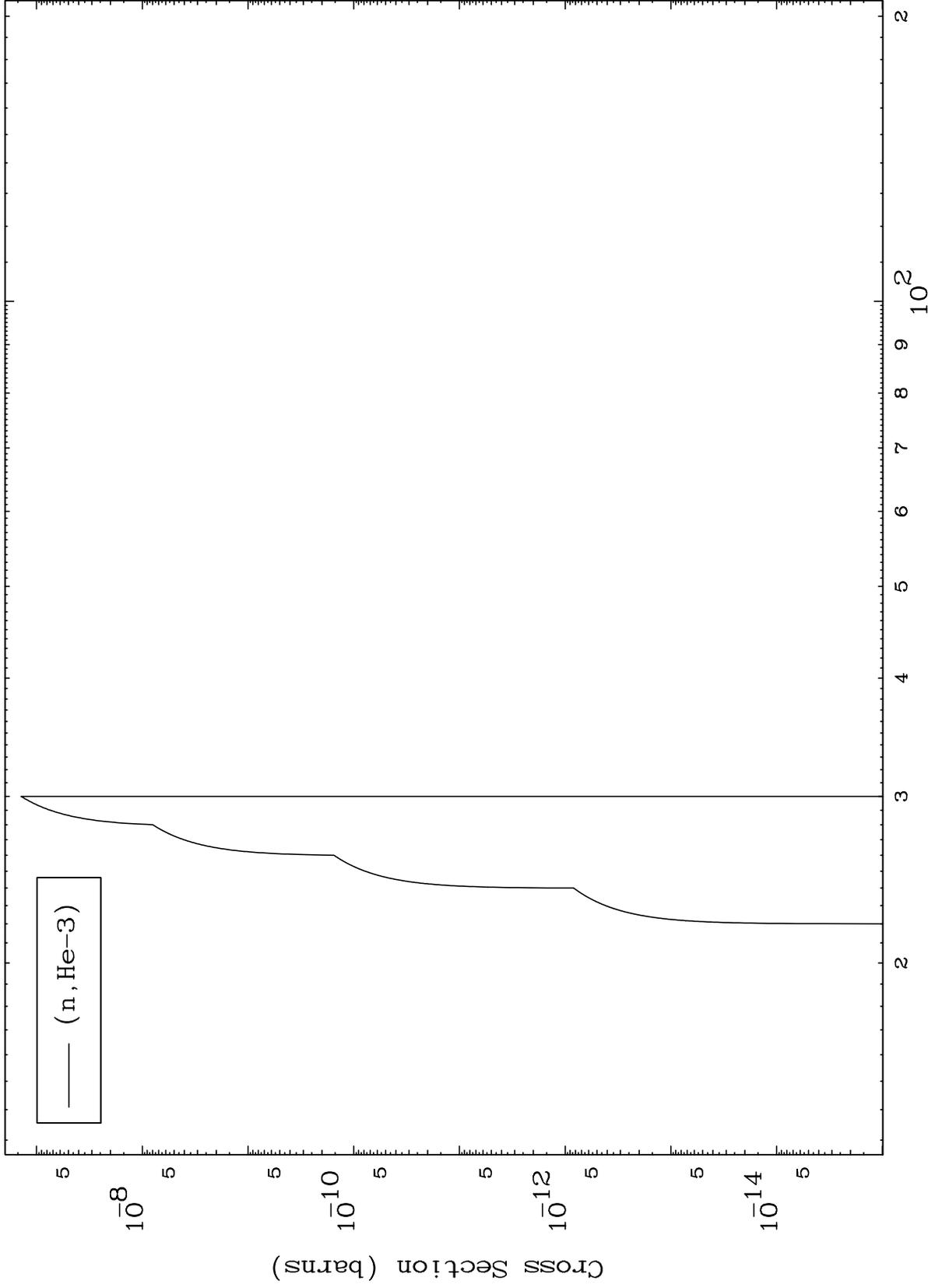
85-At-203

Incident Energy (MeV)

6





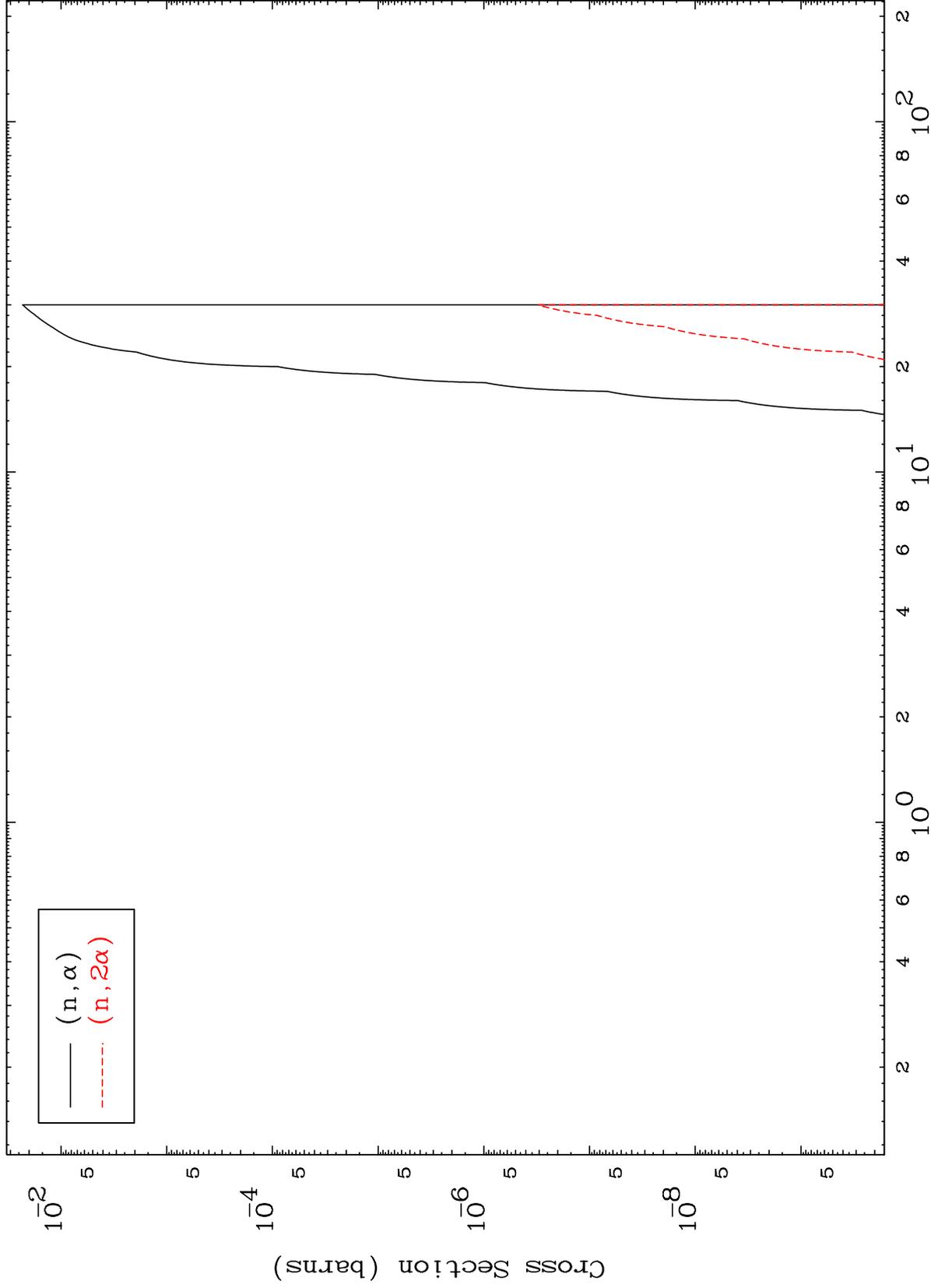


MAT 8525

(α, α) Levels

85-At-203

0 Kelvin Cross Sections

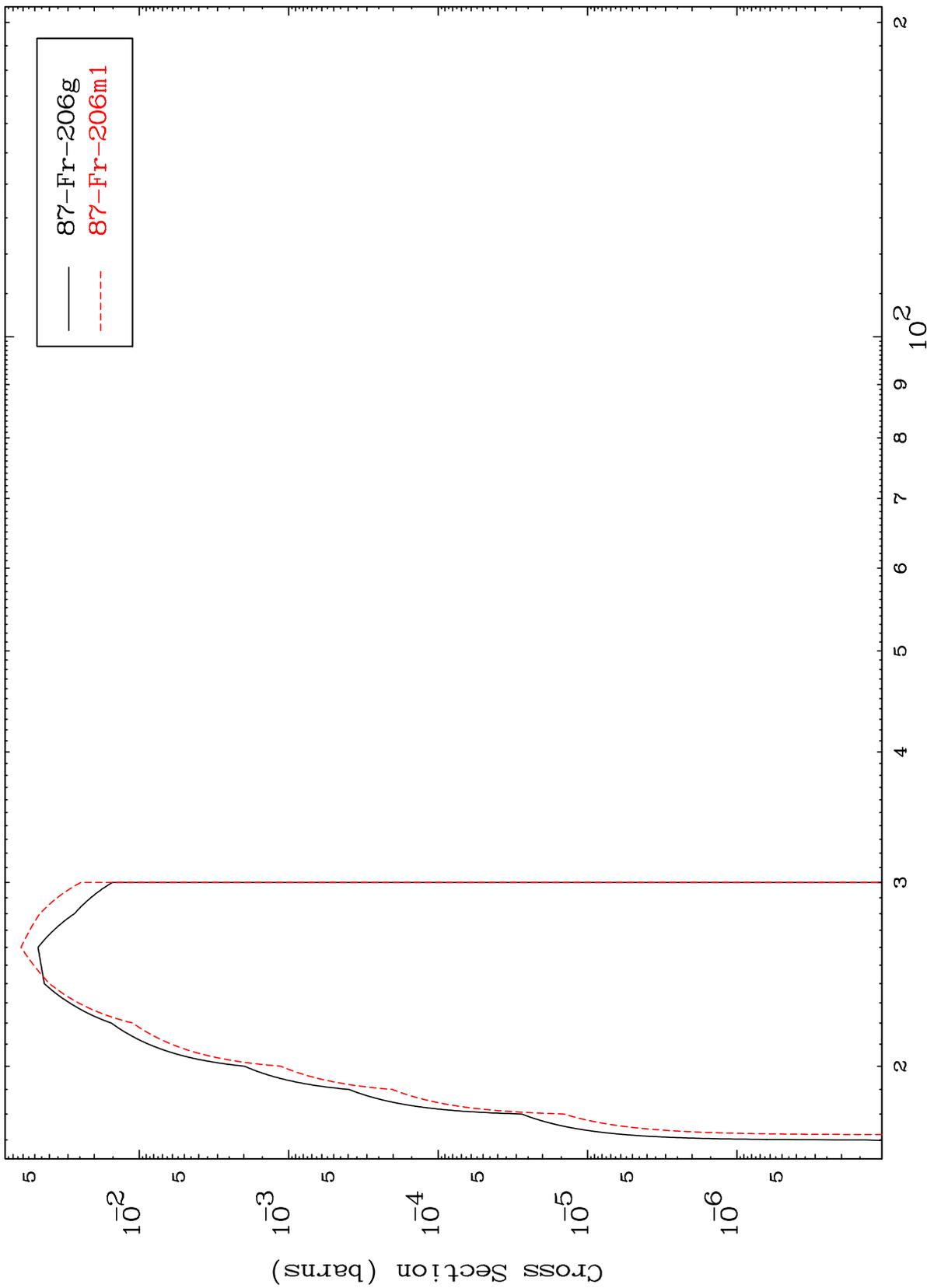


10

Incident Energy (MeV)

85-At-203

Inelastic
Radionuclide Production Cross Section

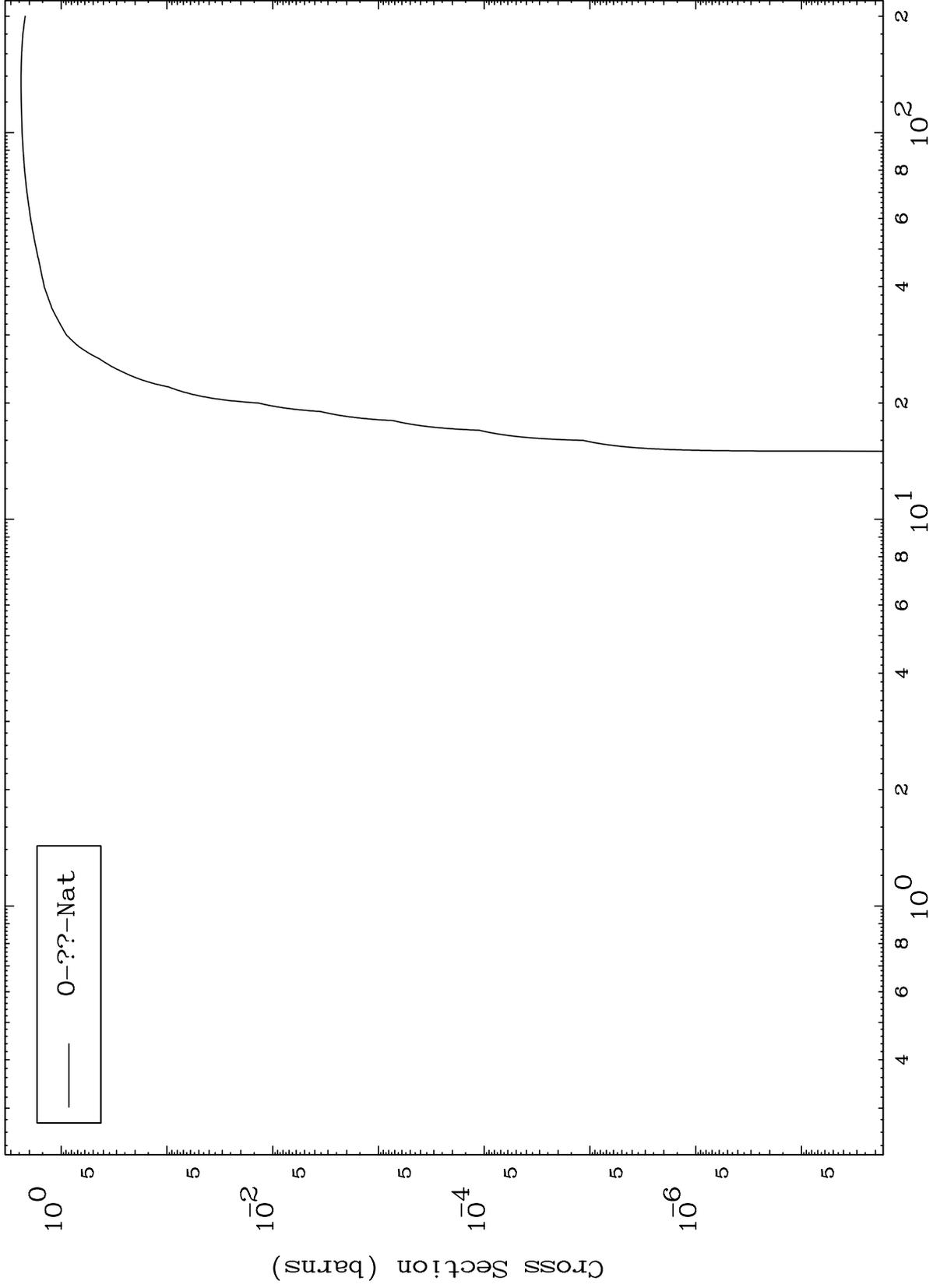


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Fission

85-At-203

Radionuclide Production Cross Section



12

Incident Energy (MeV)

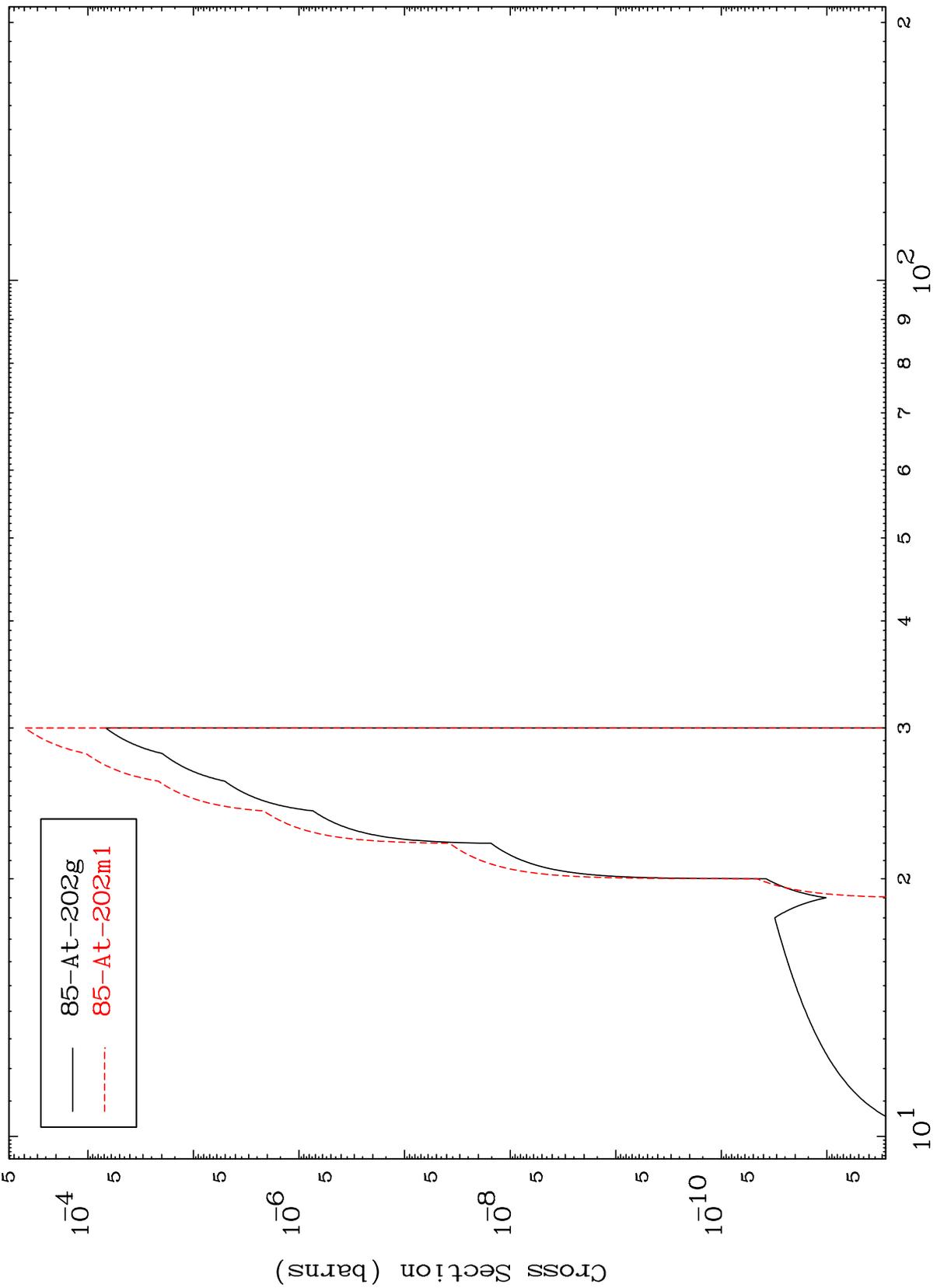
85-At-203

MAT 8525

(n,n') α

85-At-203

Radionuclide Production Cross Section

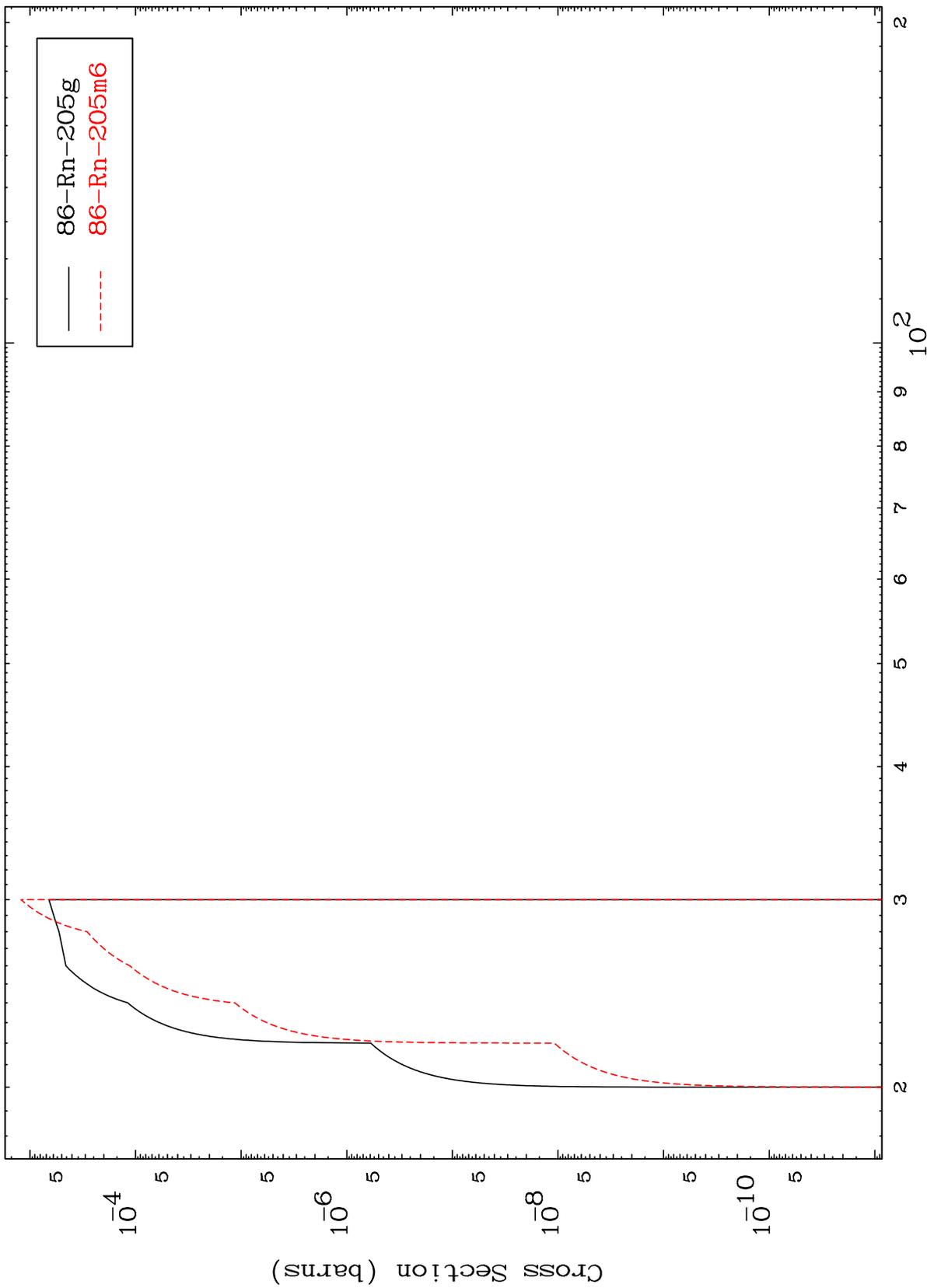


13

Incident Energy (MeV)

85-At-203

(n, n') p
Radionuclide Production Cross Section

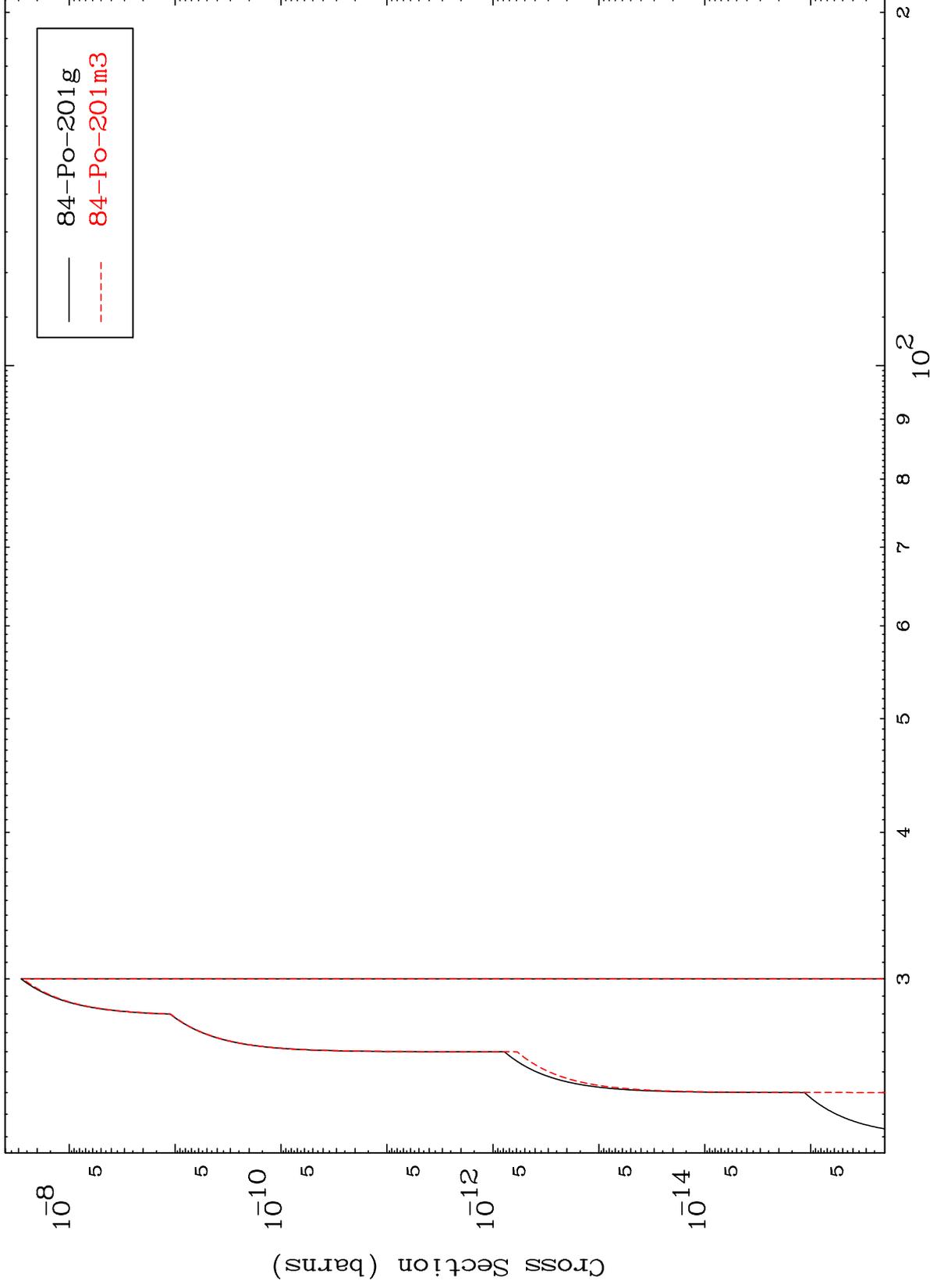


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(n,n') p α

85-At-203

Radionuclide Production Cross Section



15

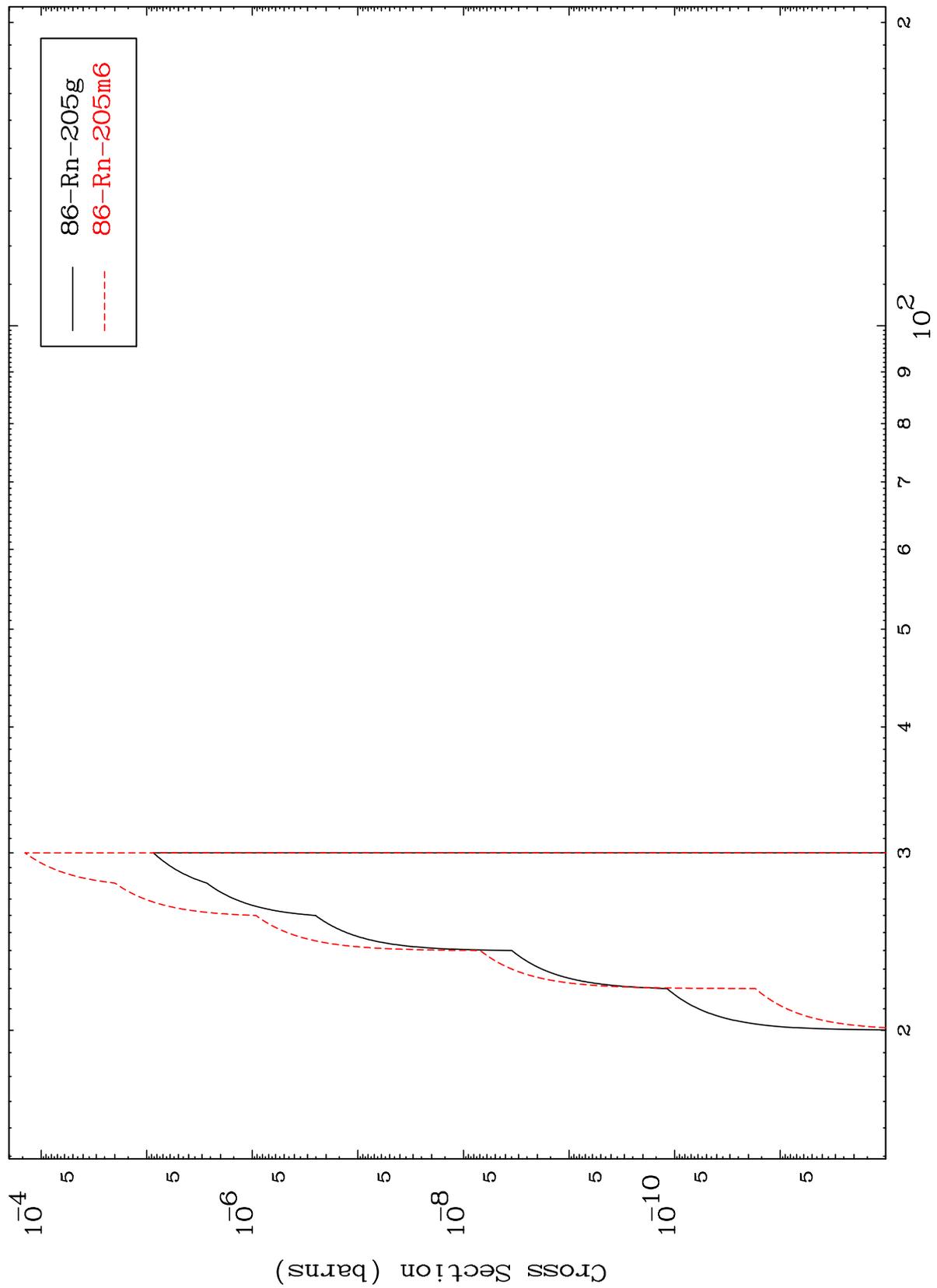
Incident Energy (MeV)

85-At-203

MAT 8525

85-At-203

(n,d)
Radionuclide Production Cross Section



16

85-At-203

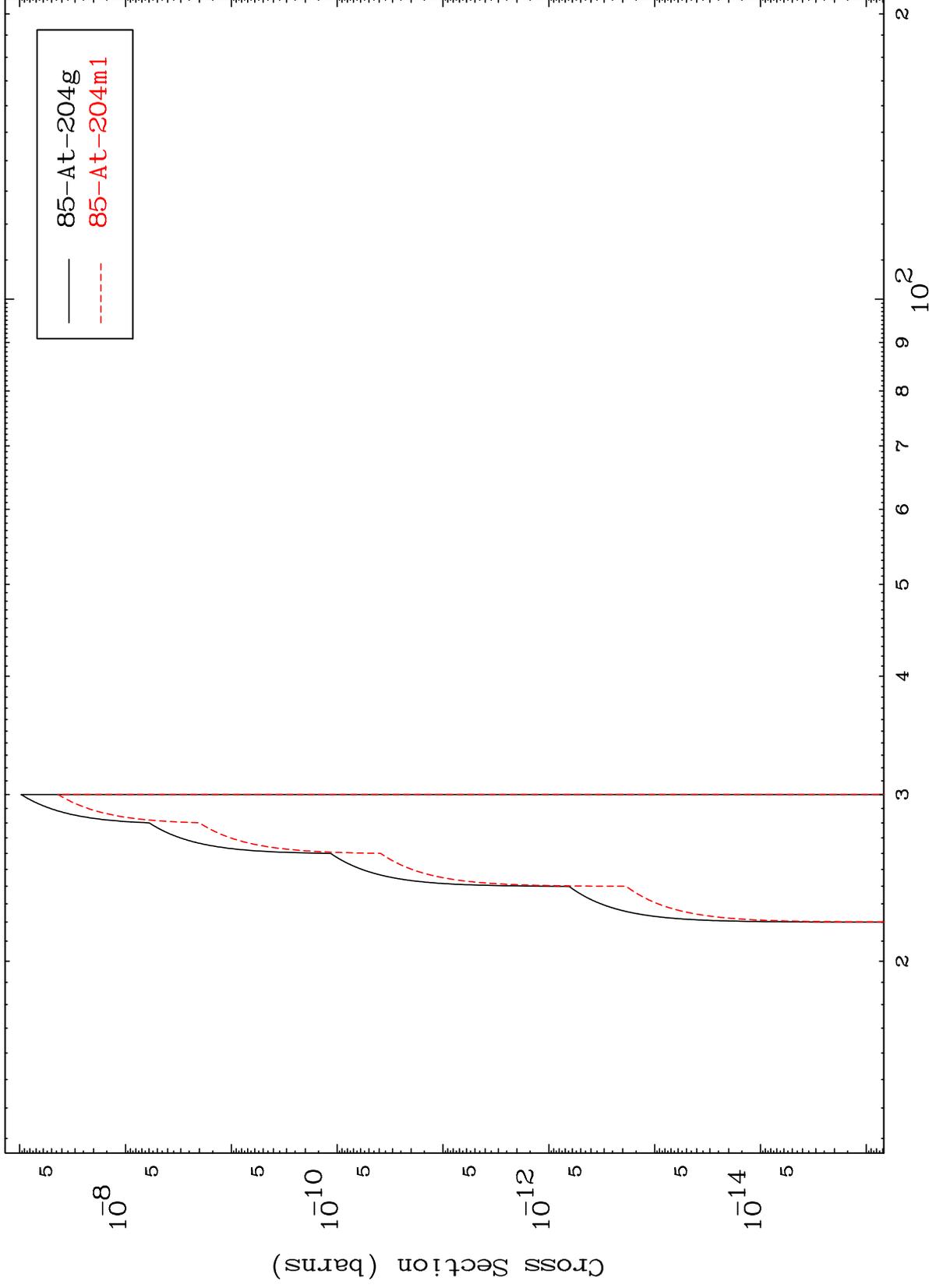
Incident Energy (MeV)

MAT 8525

(n,He-3)

85-At-203

Radionuclide Production Cross Section



17

Incident Energy (MeV)

85-At-203

MAT 8525

(n,2α)

85-At-203

Radionuclide Production Cross Section

