

Program EVALPLOT
(Version 2021-1)

by

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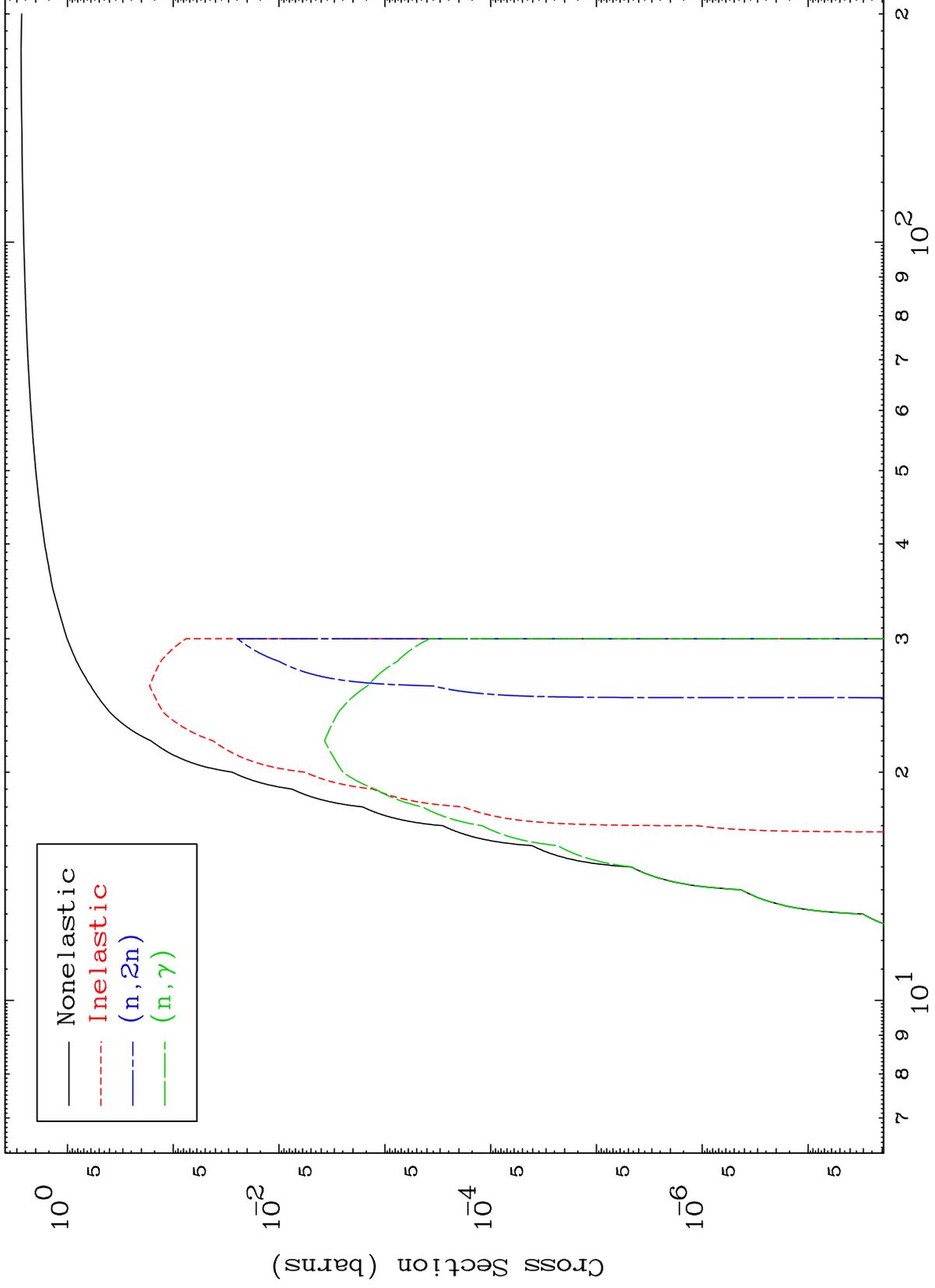
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Press Mouse Button to Start

MAT 8289

0 Kelvin Cross Sections

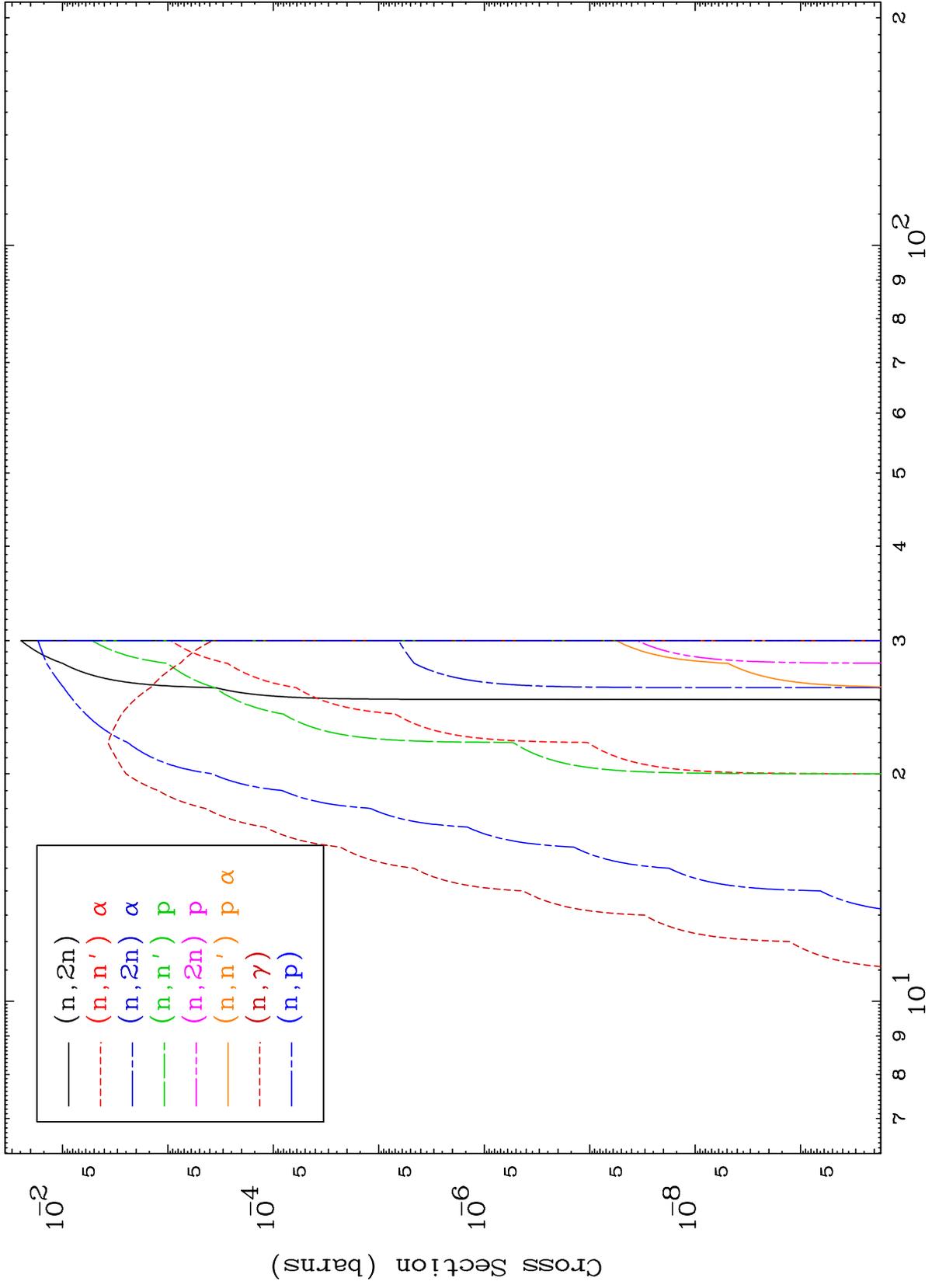
83-Bi-197



1

Incident Energy (MeV)

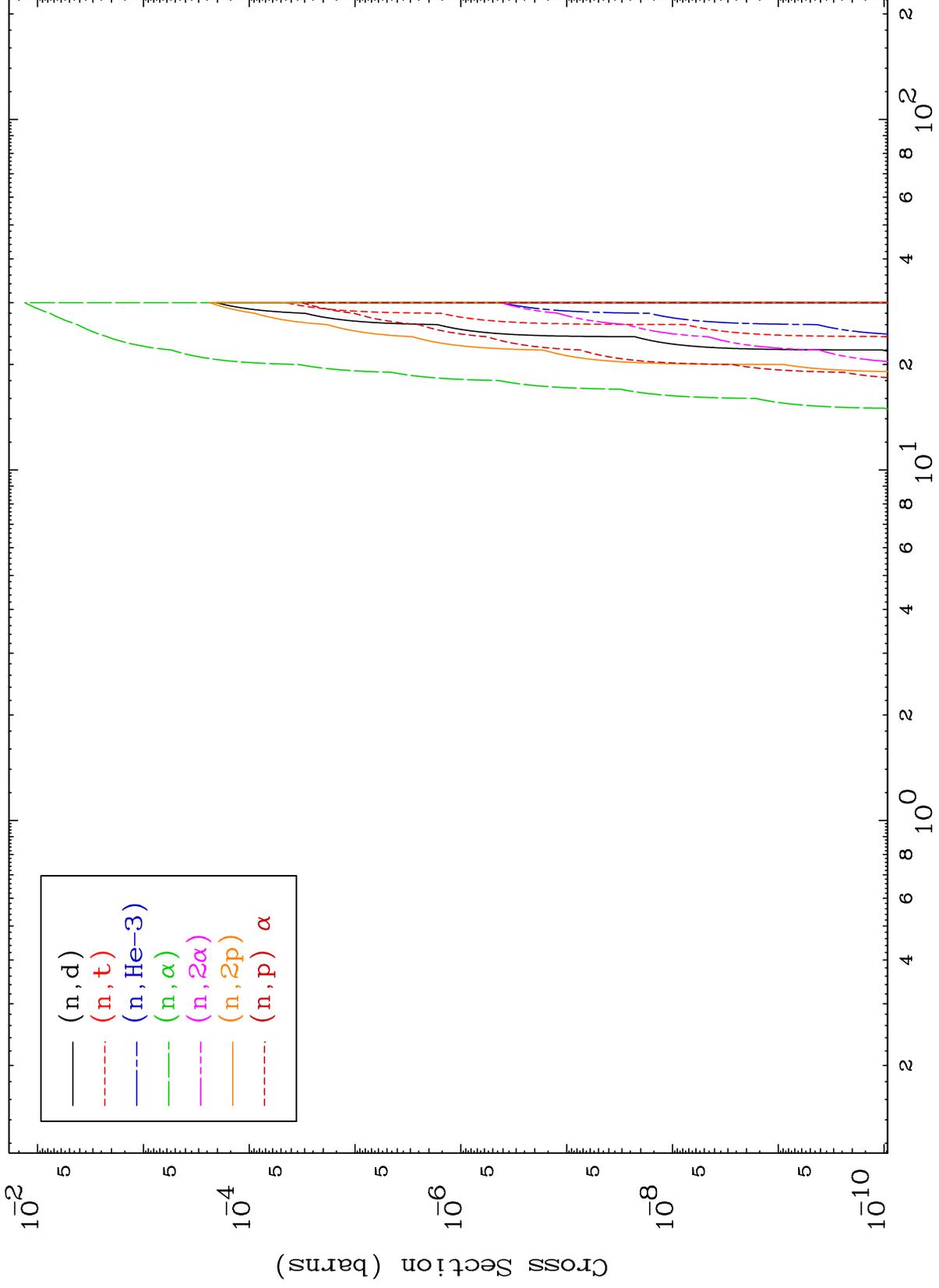
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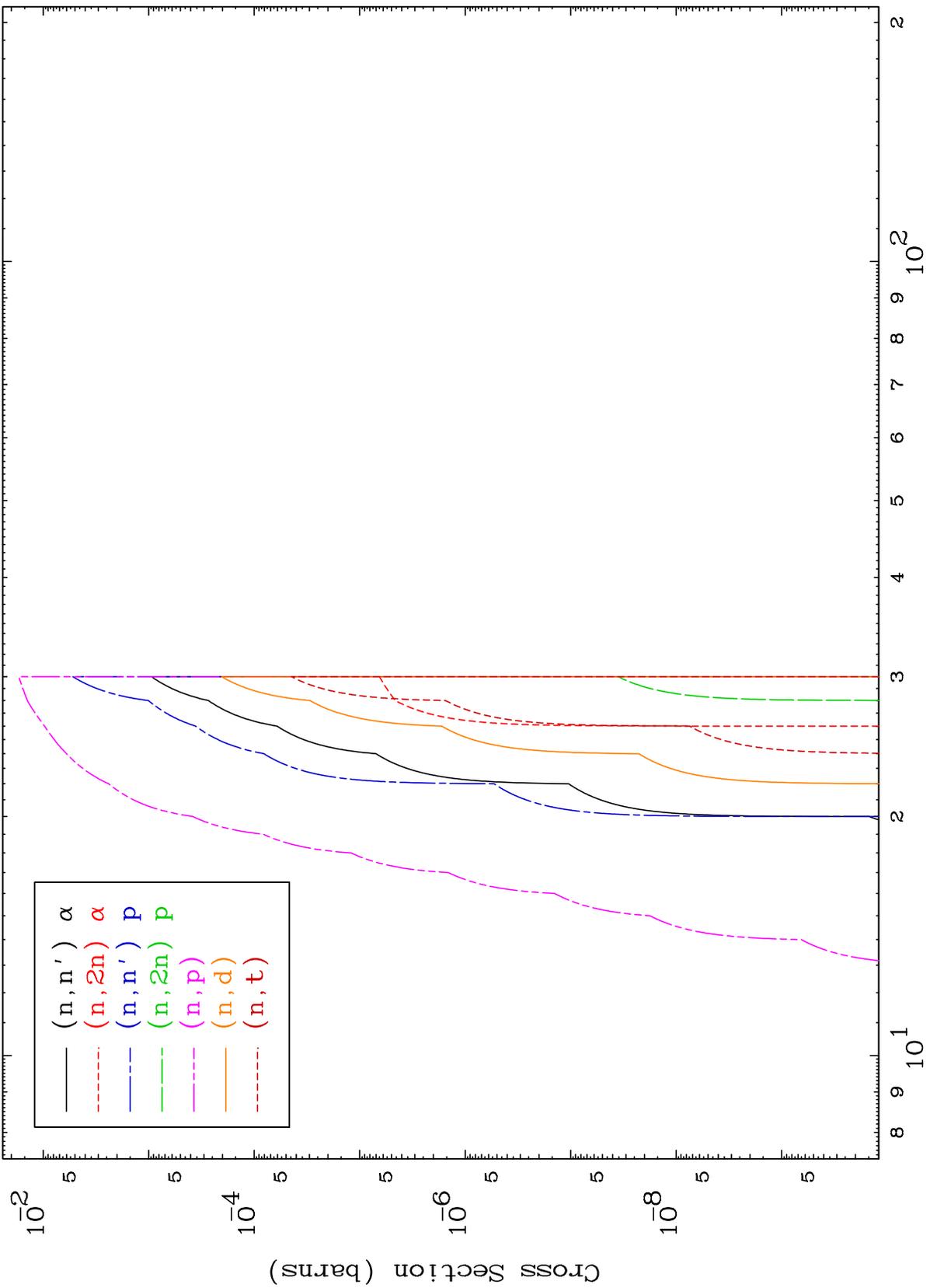


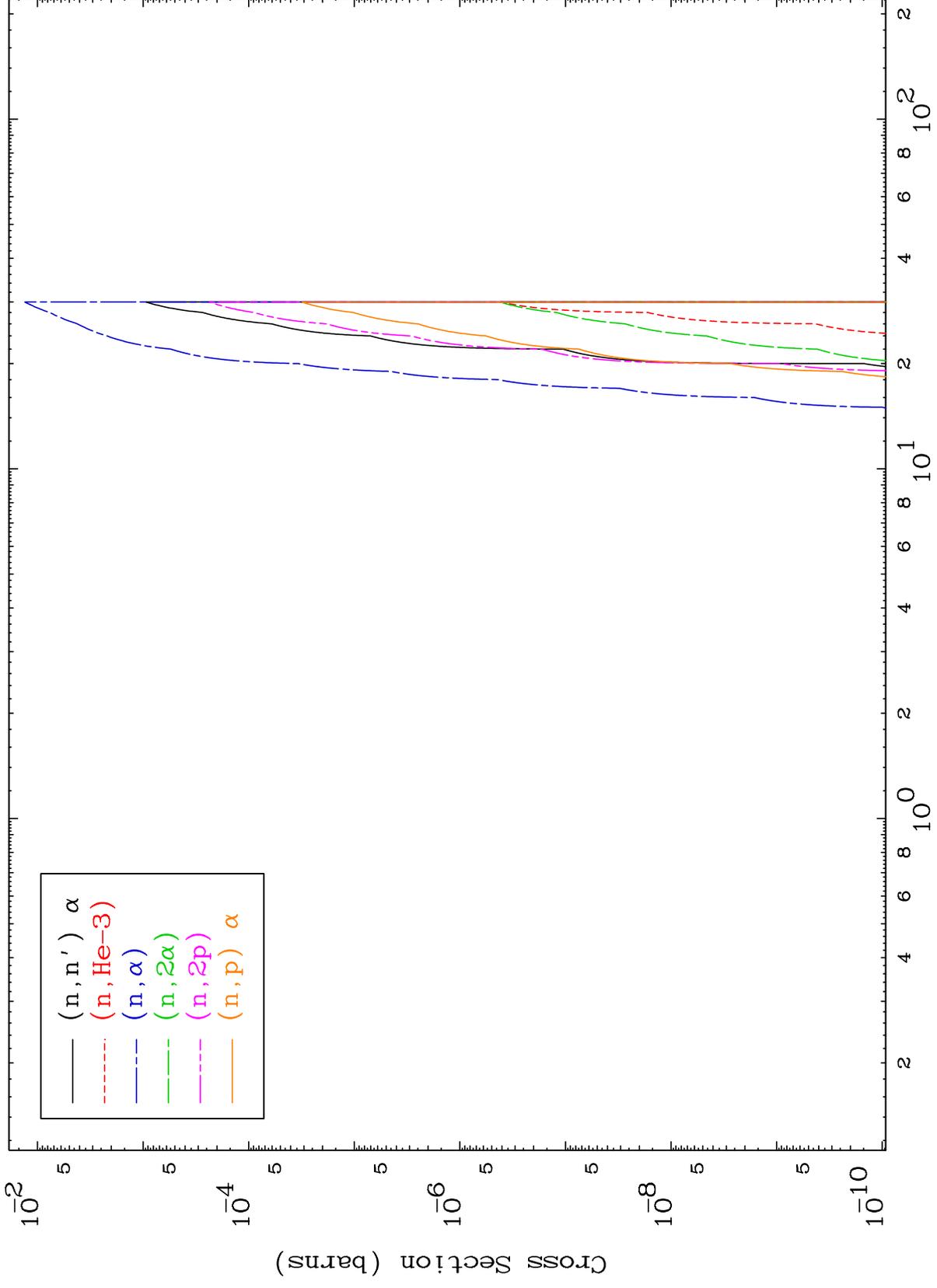
MAT 8289

α Neutron Absorption
0 Kelvin Cross Sections

83-Bi-197



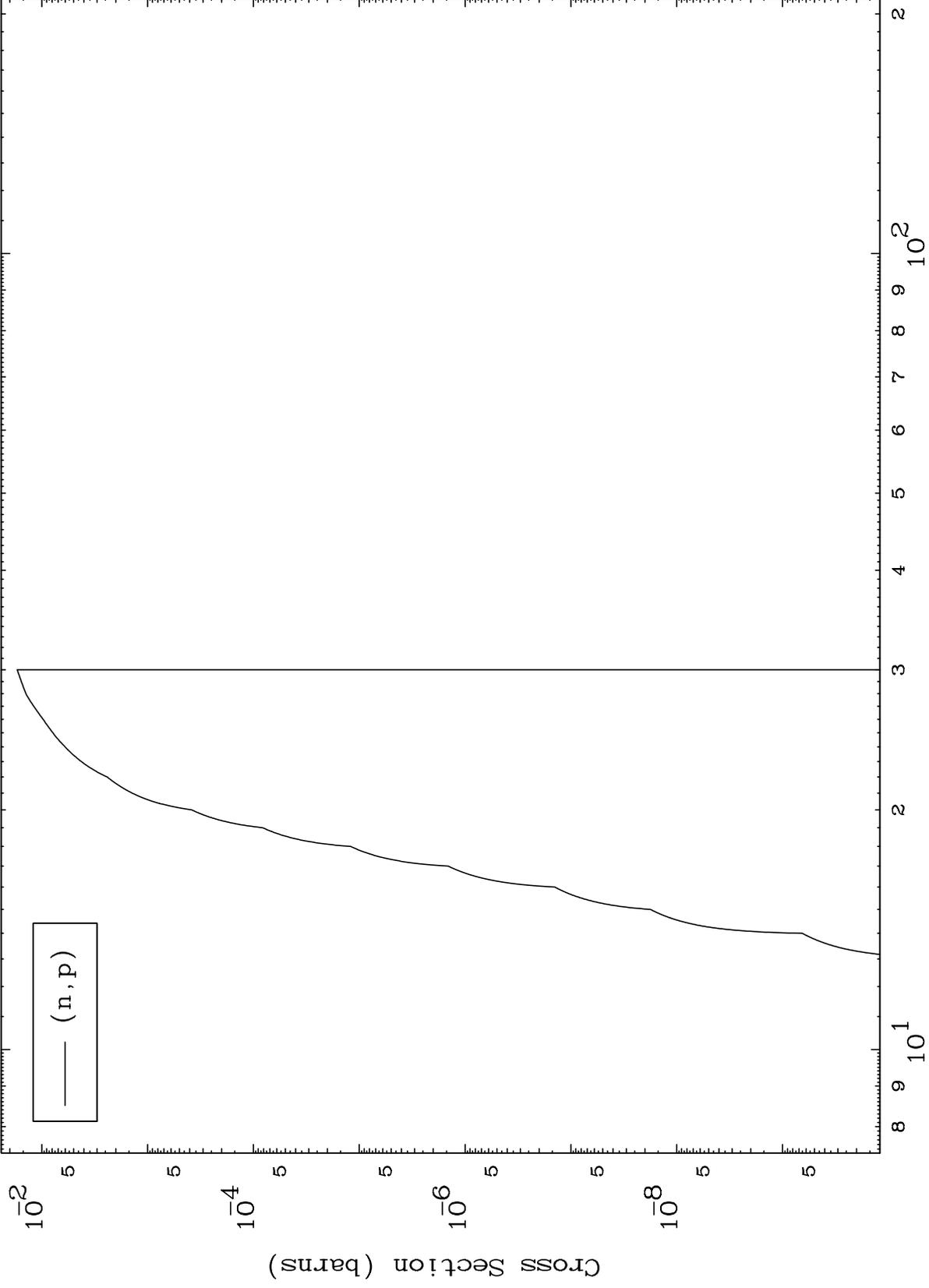




MAT 8289

(α, p) Levels
0 Kelvin Cross Sections

83-Bi-197



6

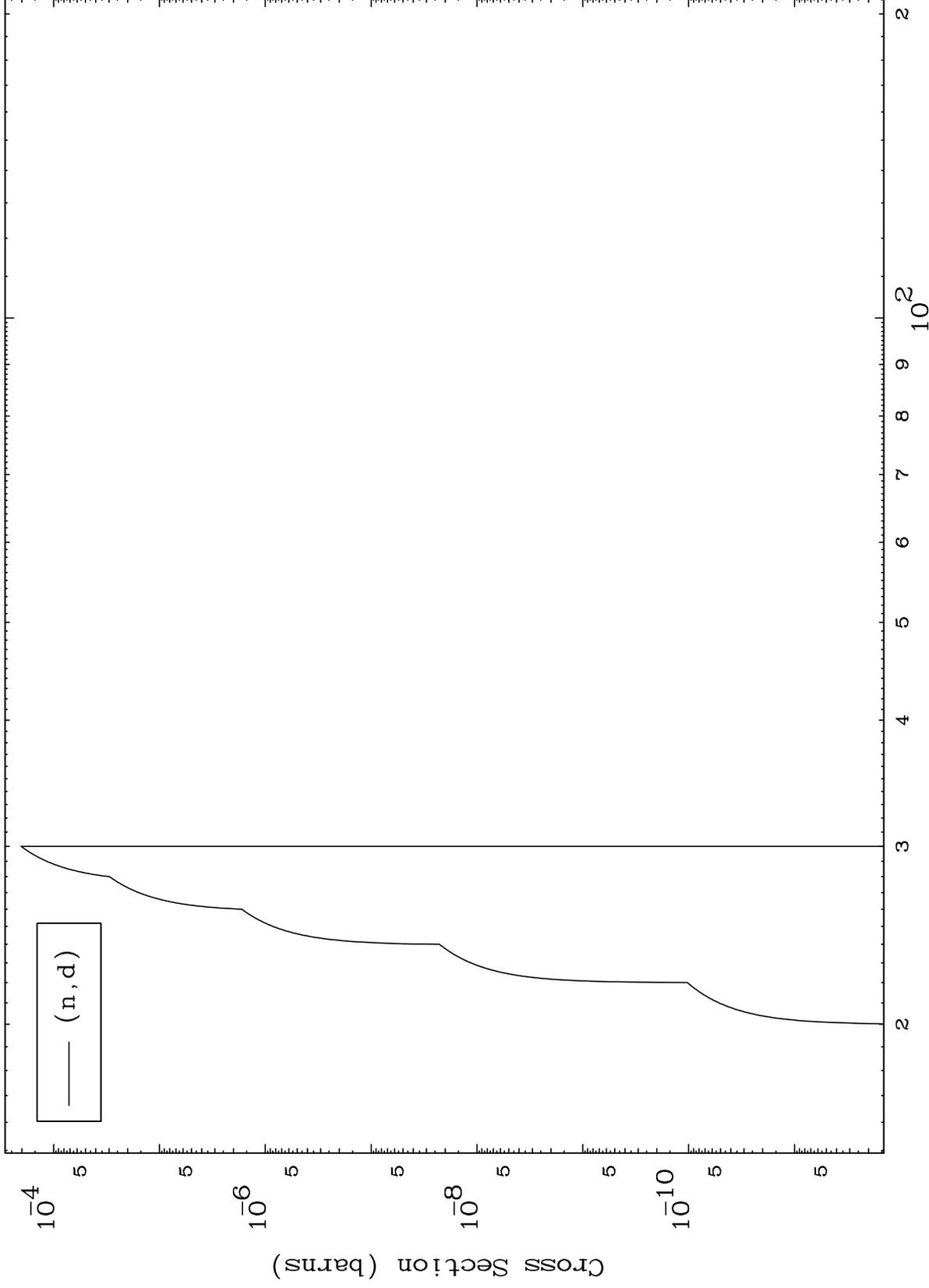
Incident Energy (MeV)

83-Bi-197

MAT 8289

(α, d) Levels
0 Kelvin Cross Sections

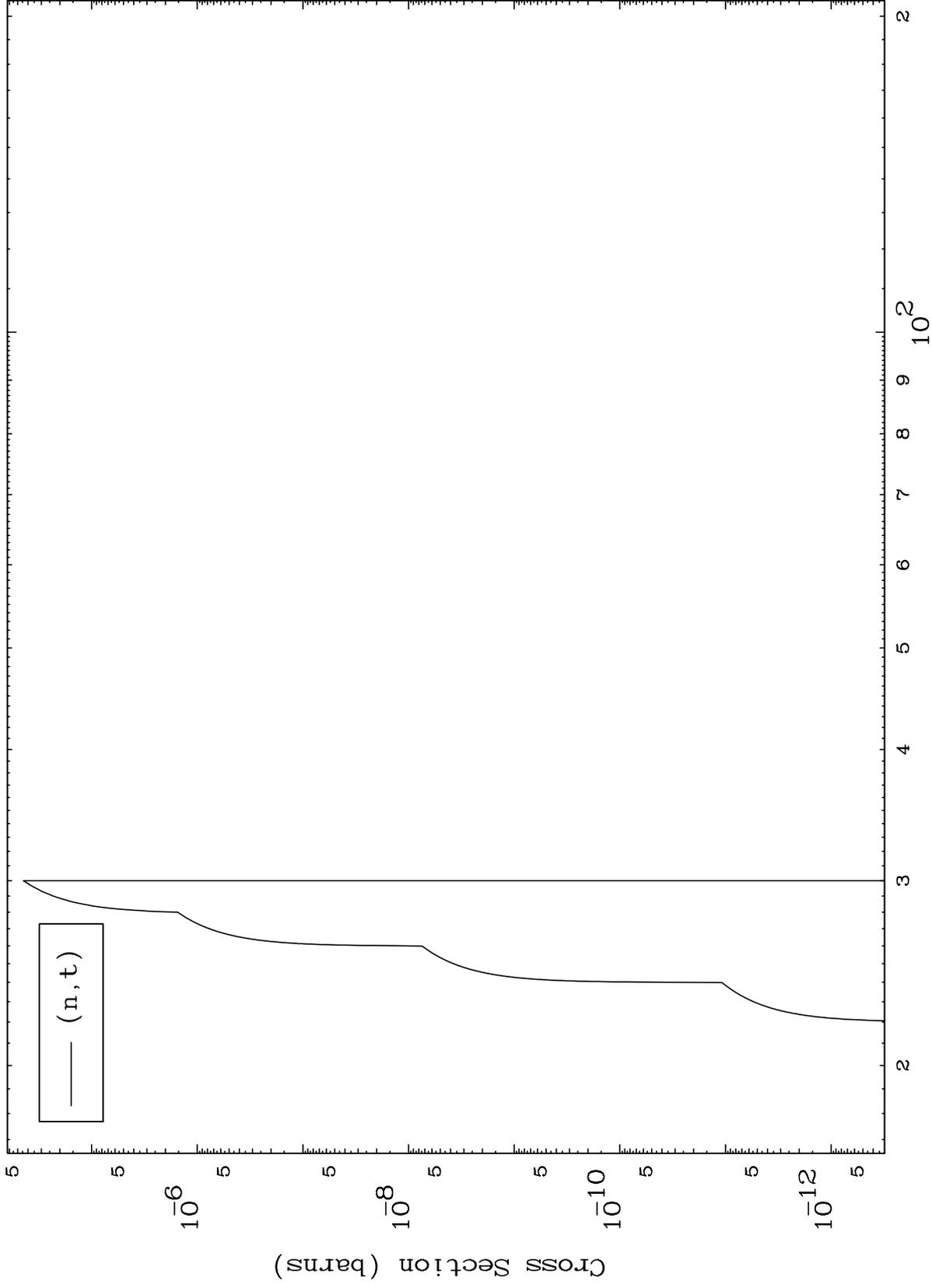
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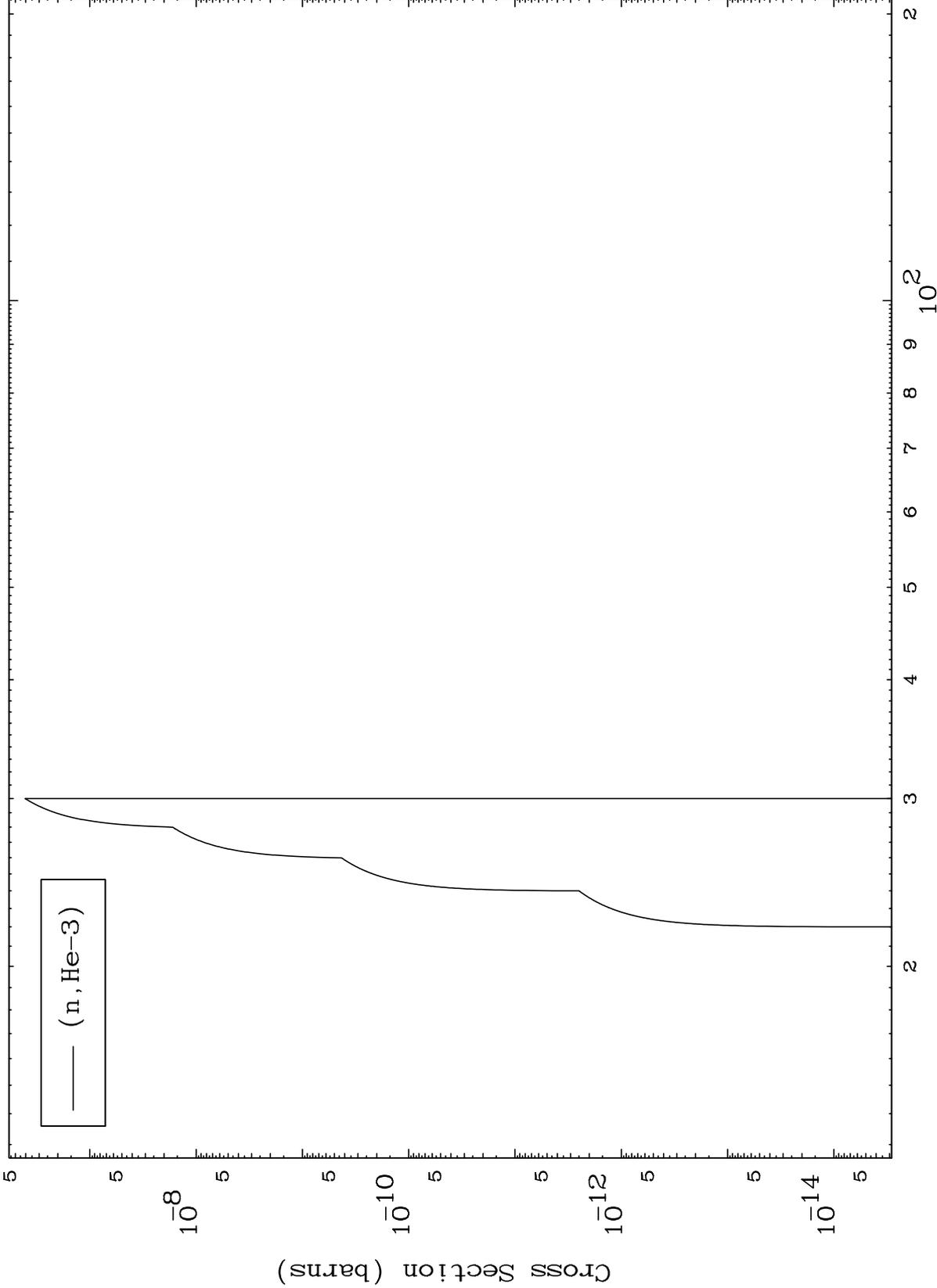


7

Incident Energy (MeV)

83-Bi-197



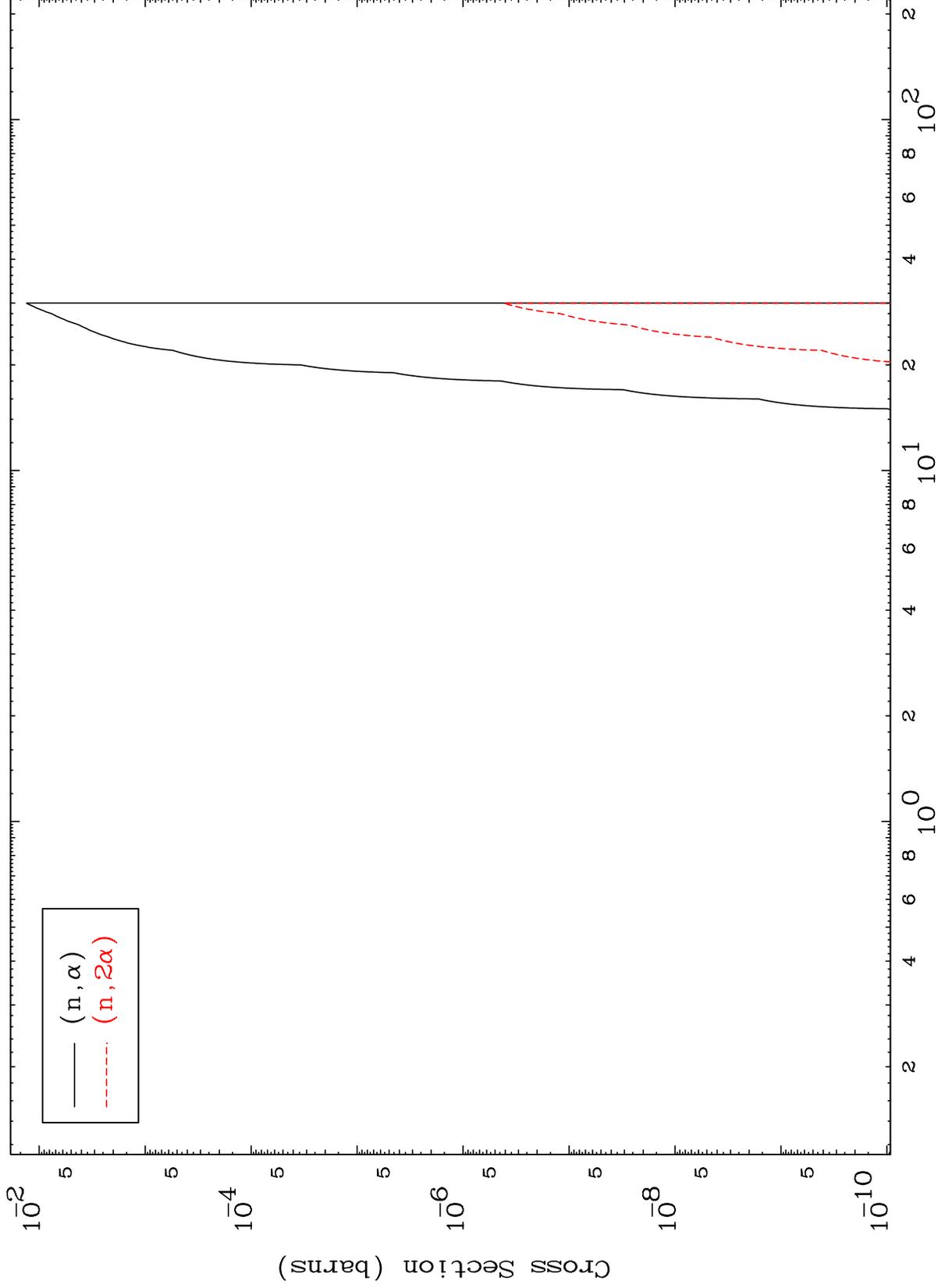


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(α, α) Levels

83-Bi-197

0 Kelvin Cross Sections

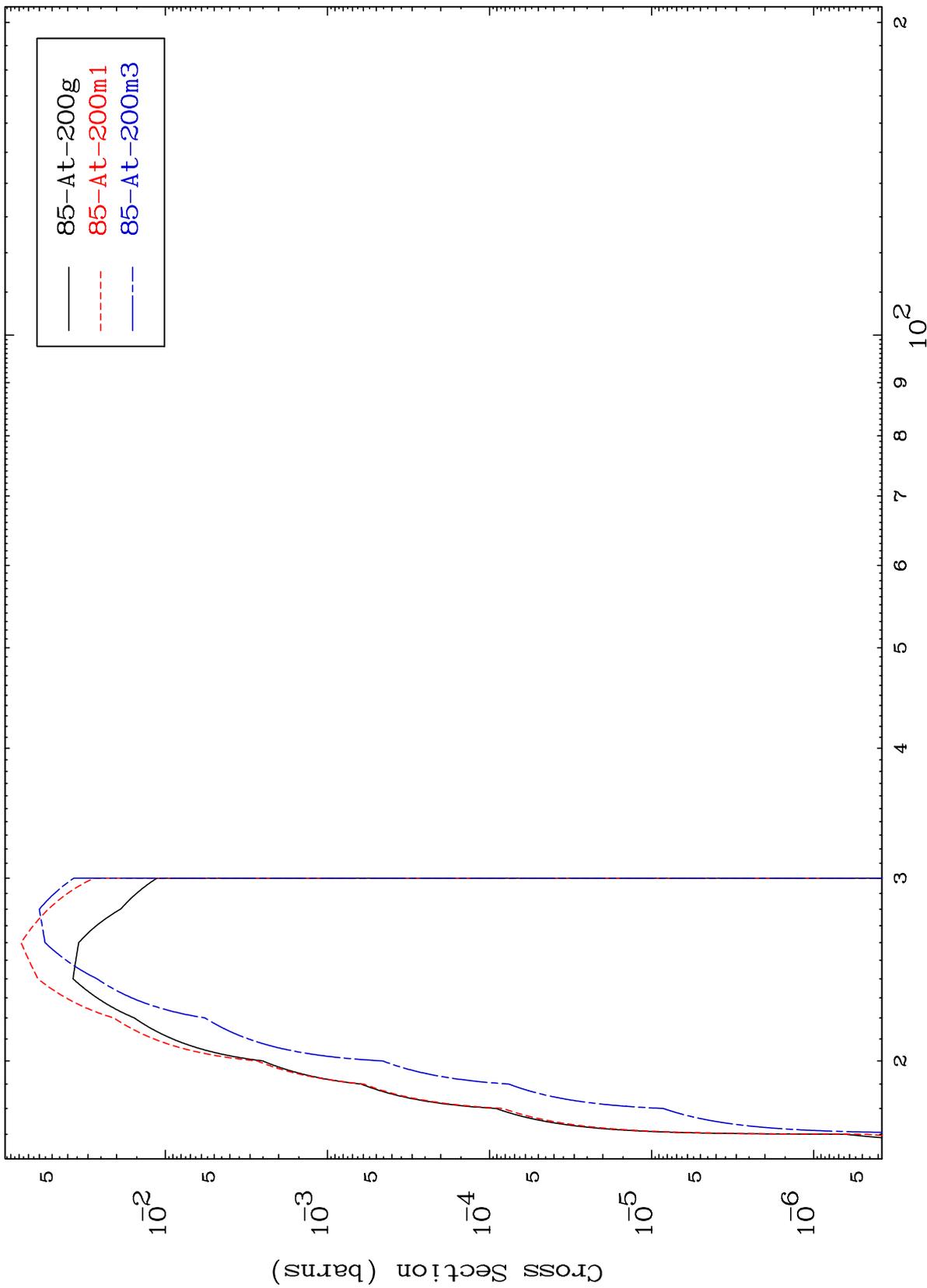


10

Incident Energy (MeV)

83-Bi-197

Inelastic
Radionuclide Production Cross Section

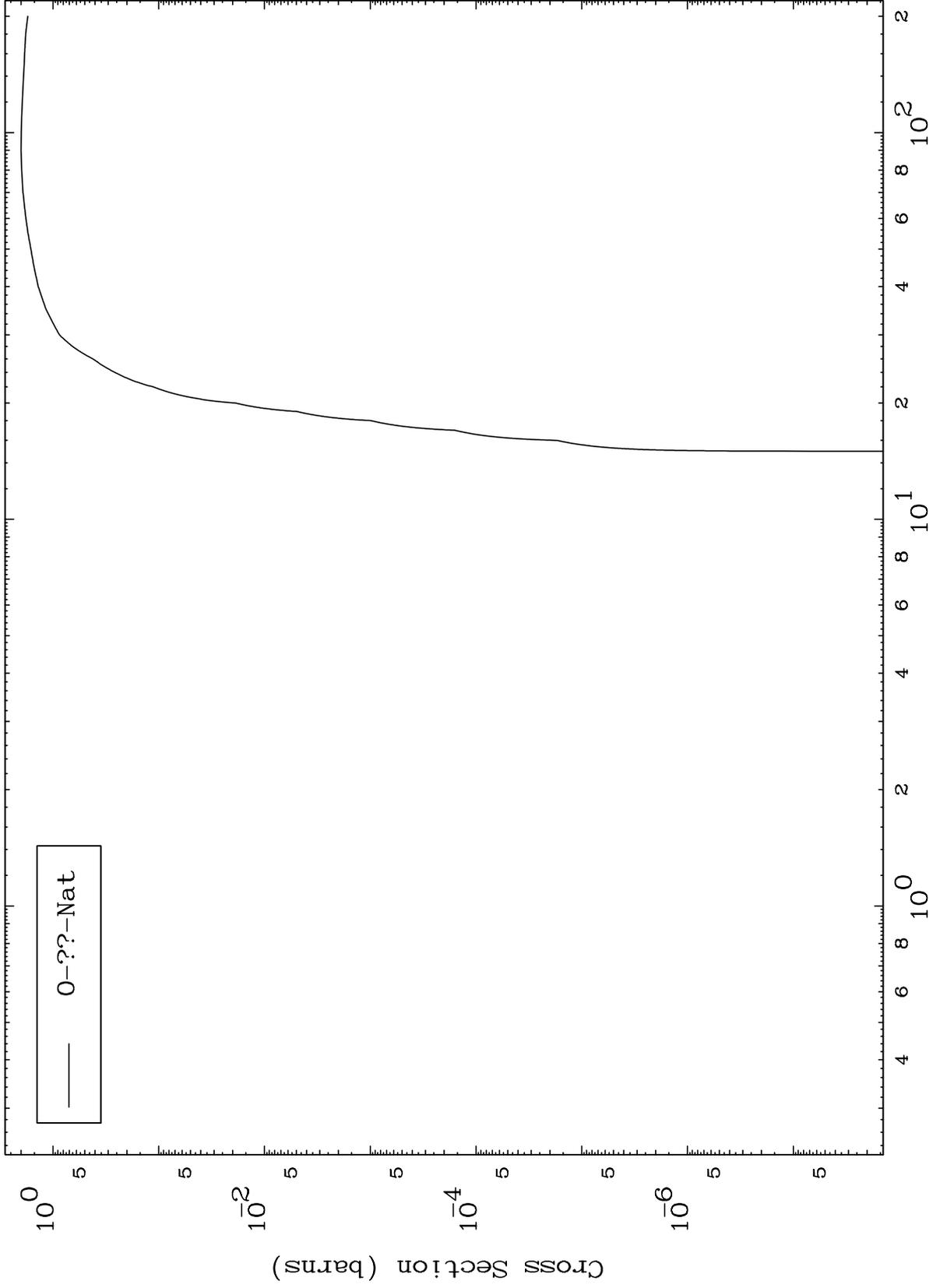


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Fission

83-Bi-197

Radionuclide Production Cross Section



12

Incident Energy (MeV)

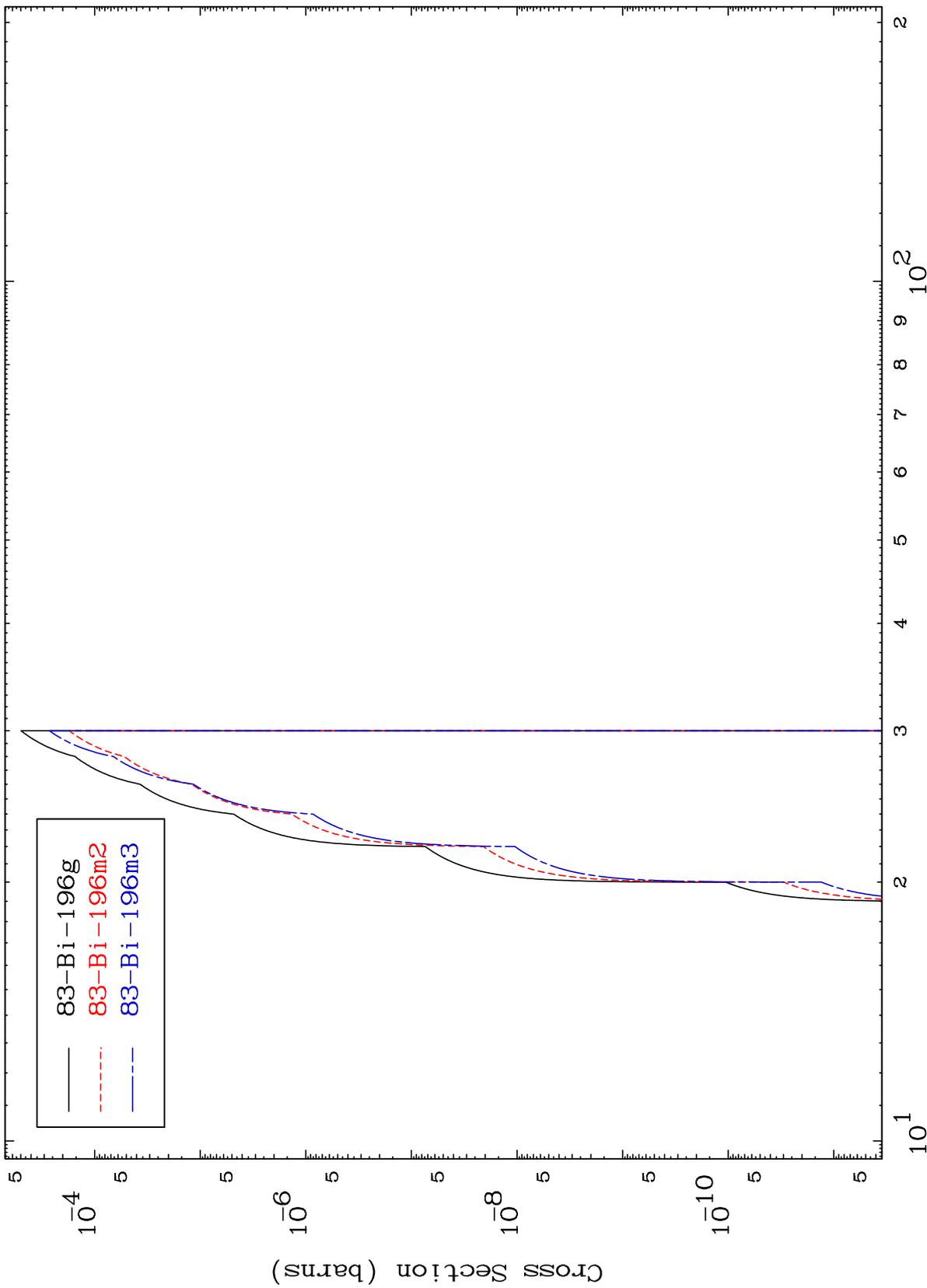
83-Bi-197

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$(n, n') \alpha$

83-Bi-197

Radionuclide Production Cross Section



83-Bi-196g
83-Bi-196m2
83-Bi-196m3

Incident Energy (MeV)

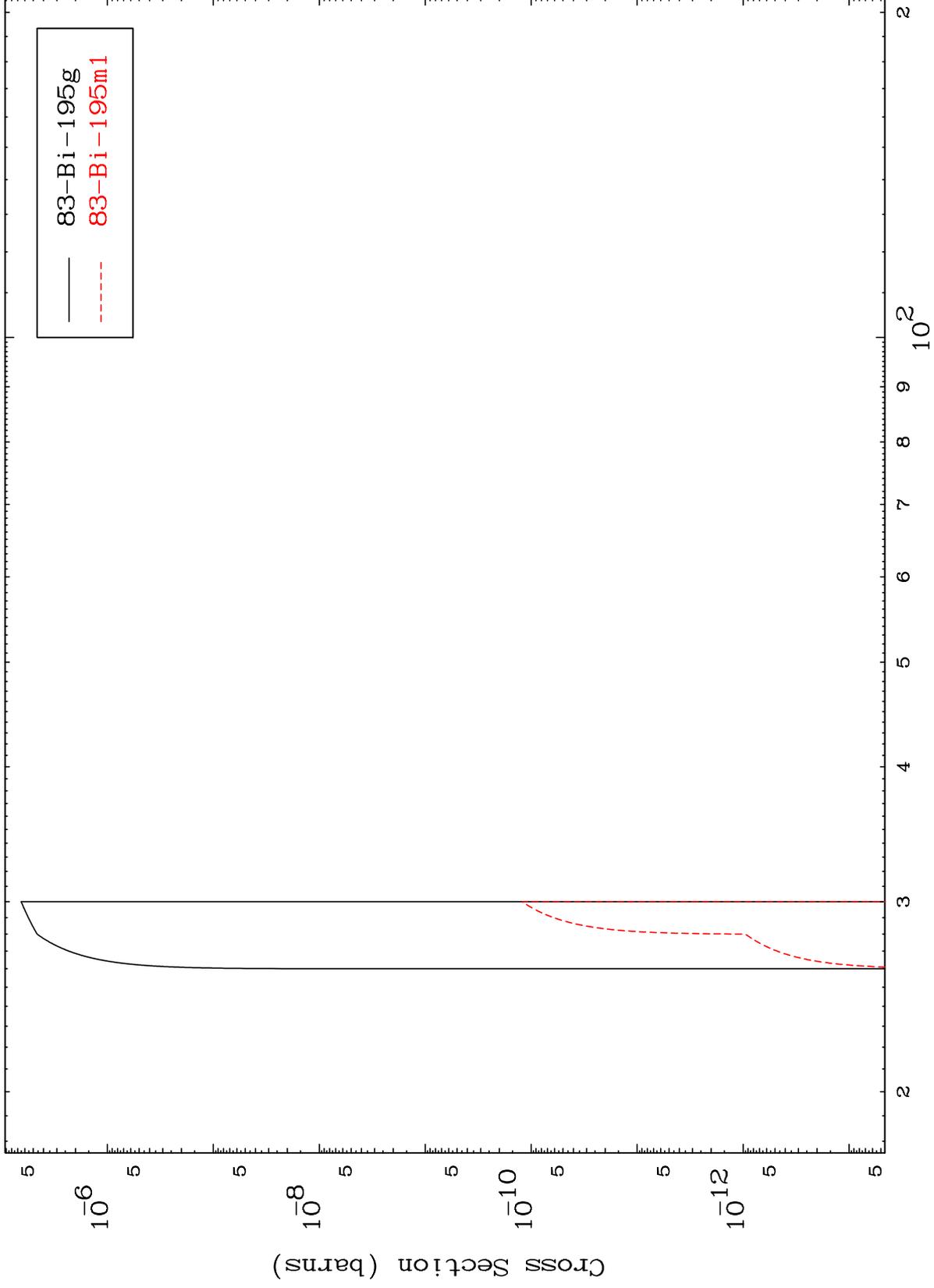
83-Bi-197

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$(n,2n) \alpha$

83-Bi-197

Radionuclide Production Cross Section



14

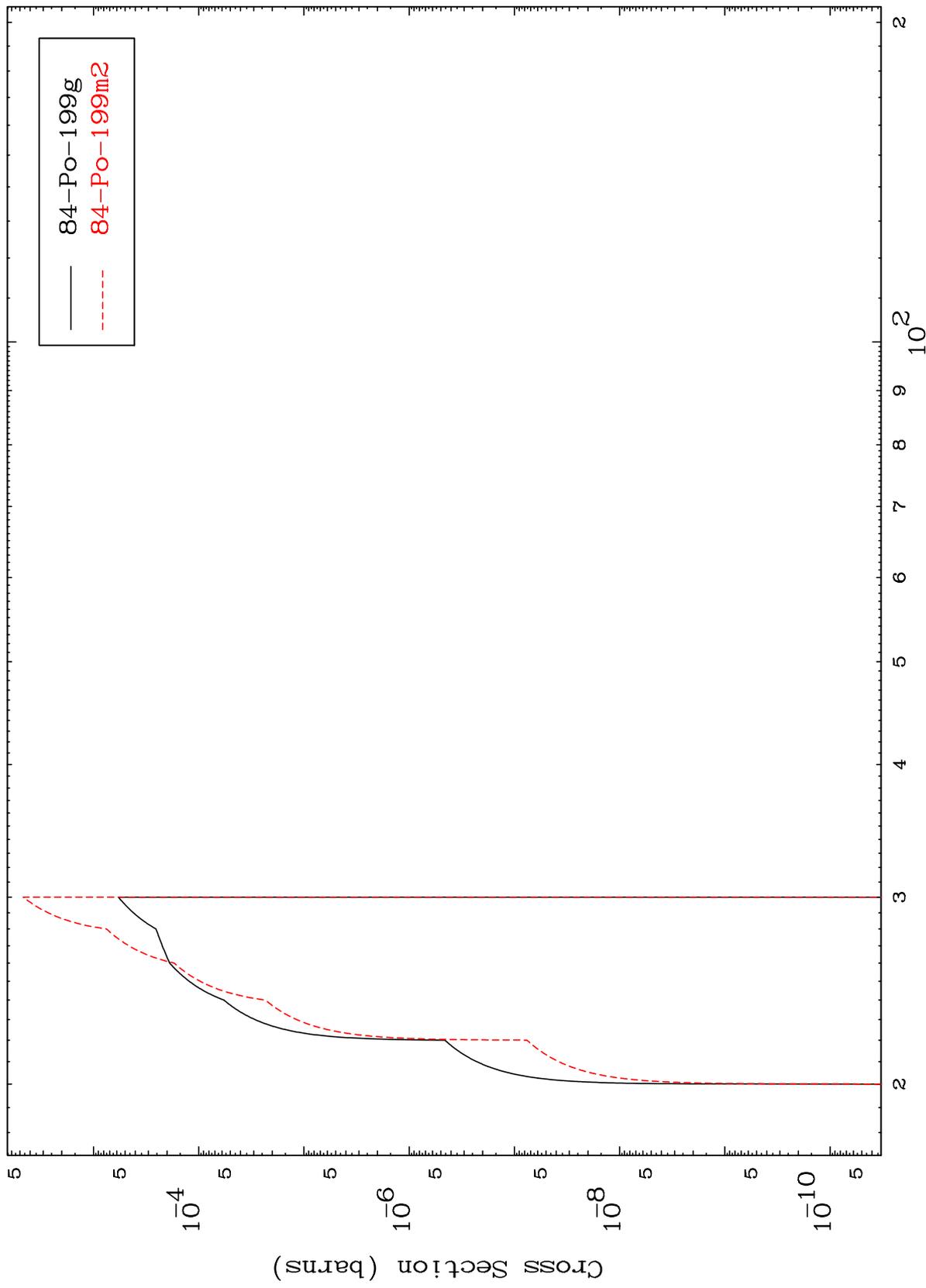
Incident Energy (MeV)

83-Bi-197

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83-Bi-197

(n,n') p
Radionuclide Production Cross Section



15

Incident Energy (MeV)

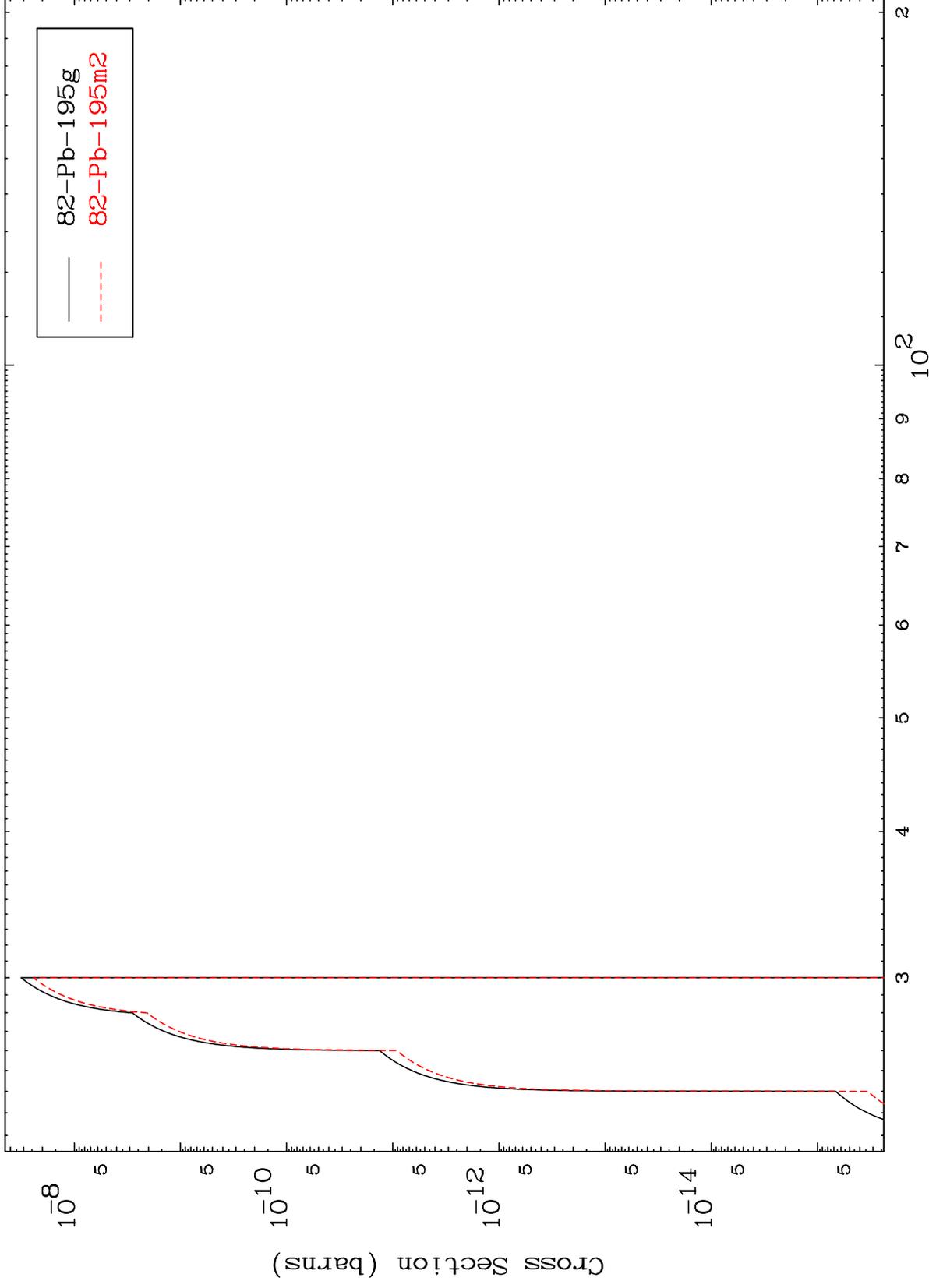
83-Bi-197

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(n,n') p α

83-Bi-197

Radionuclide Production Cross Section

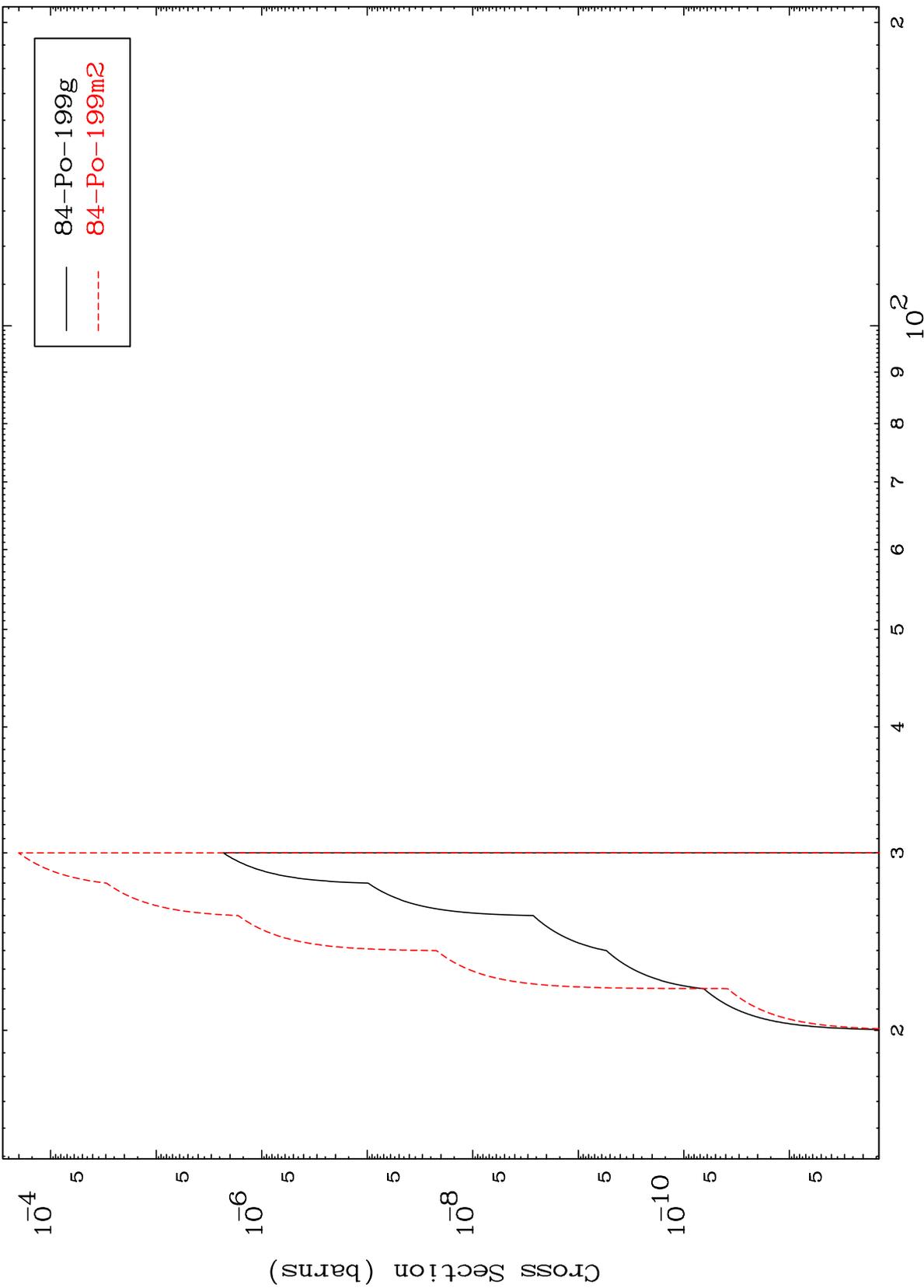


16

Incident Energy (MeV)

83-Bi-197

(n,d)
Radionuclide Production Cross Section

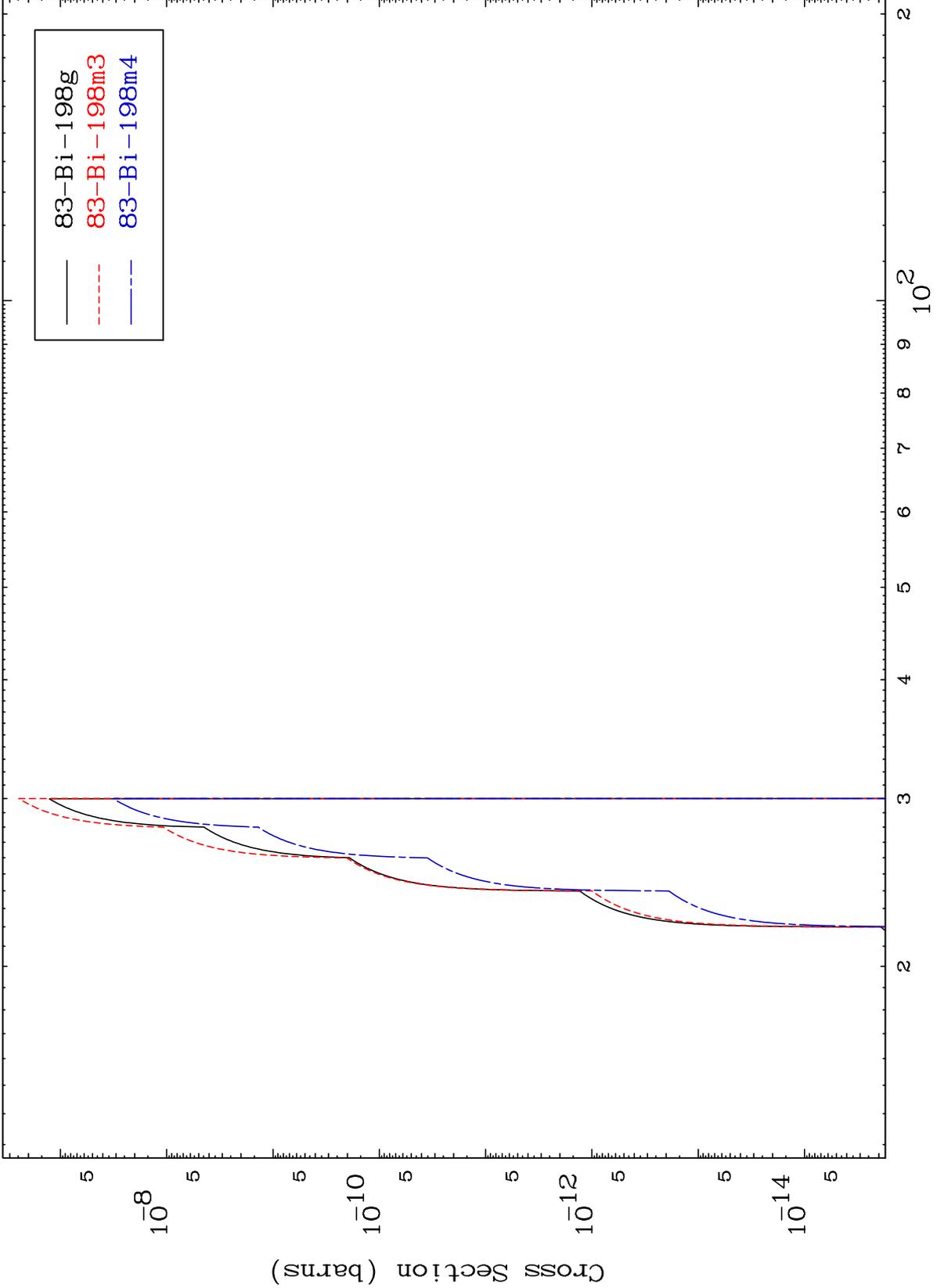


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(n,He-3)

83-Bi-197

Radionuclide Production Cross Section



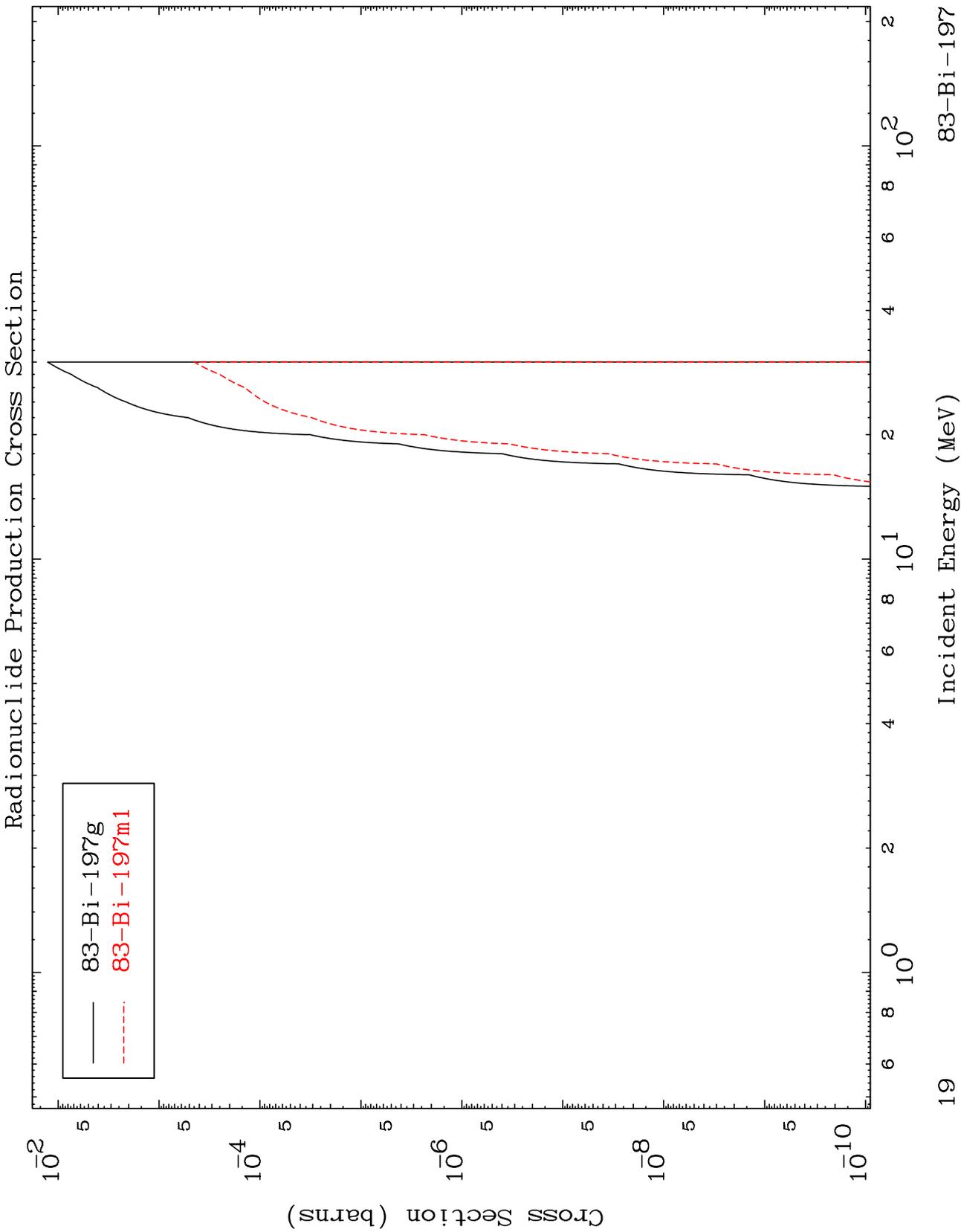
18

Incident Energy (MeV)

83-Bi-197

MAT 8289

83-Bi-197



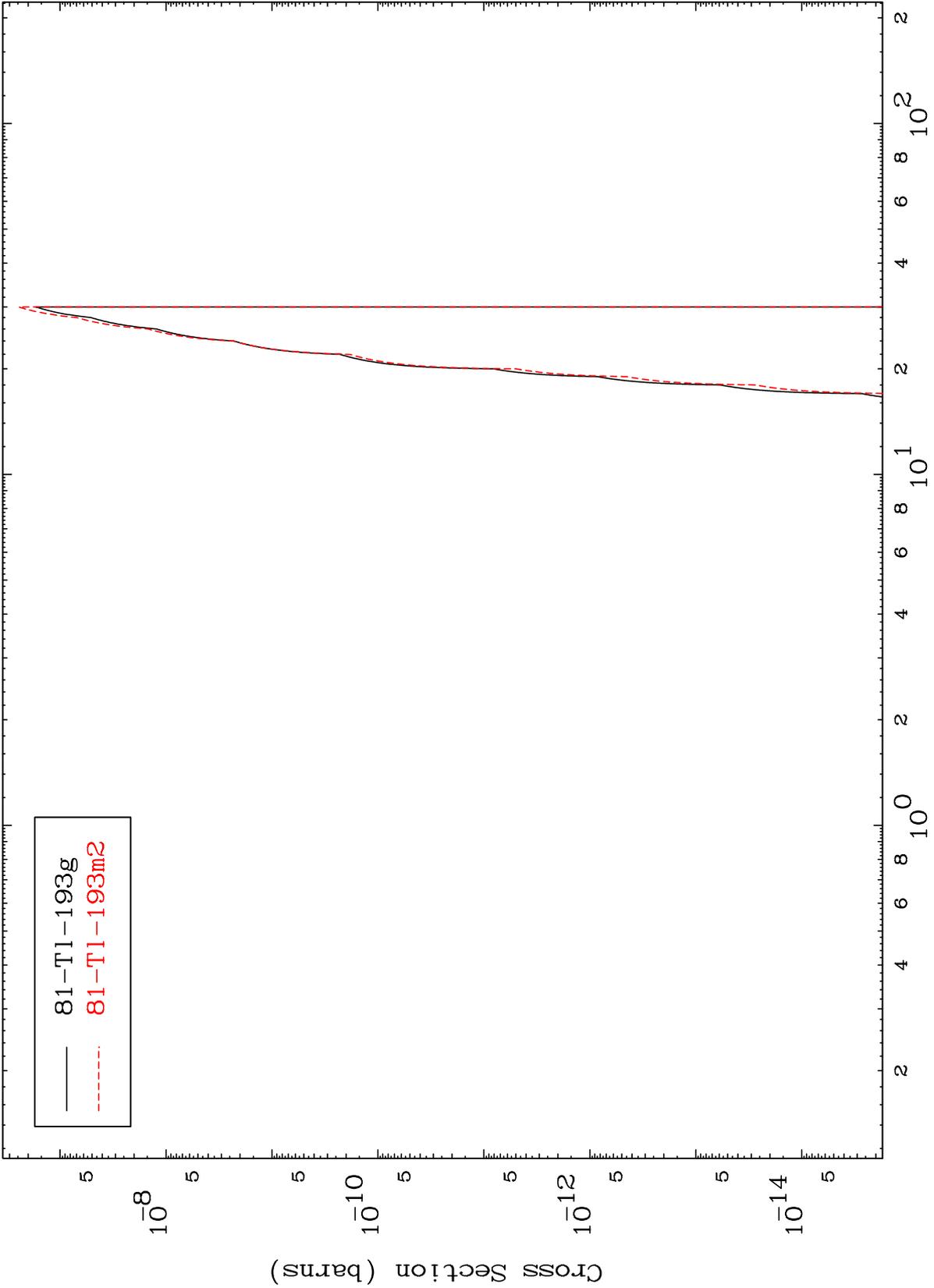
19

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(n,2α)

83-Bi-197

Radionuclide Production Cross Section

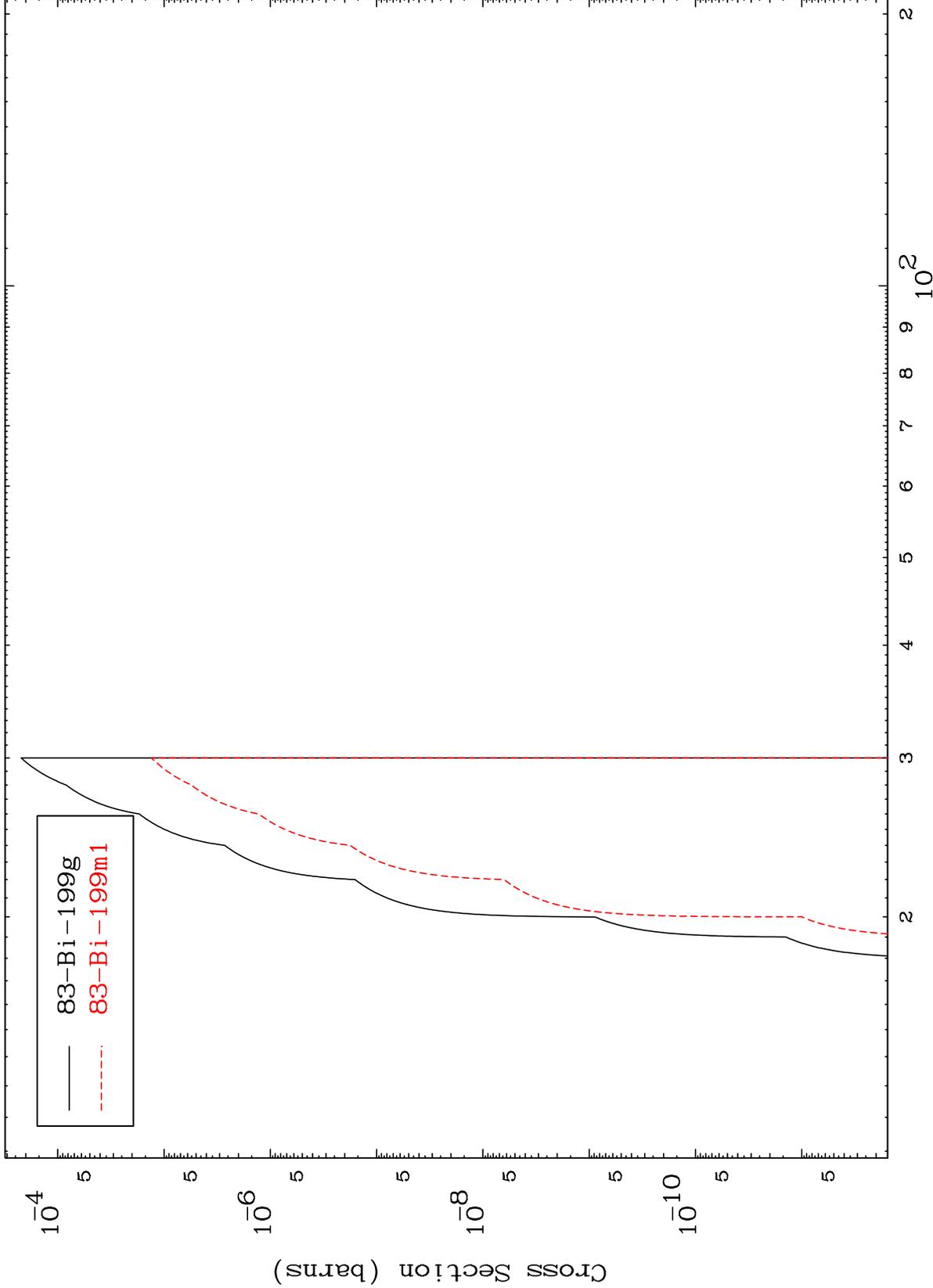


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(n,2p)

83-Bi-197

Radionuclide Production Cross Section



Incident Energy (MeV)

83-Bi-197