

Program EVALPLOT  
(Version 2021-1)

by

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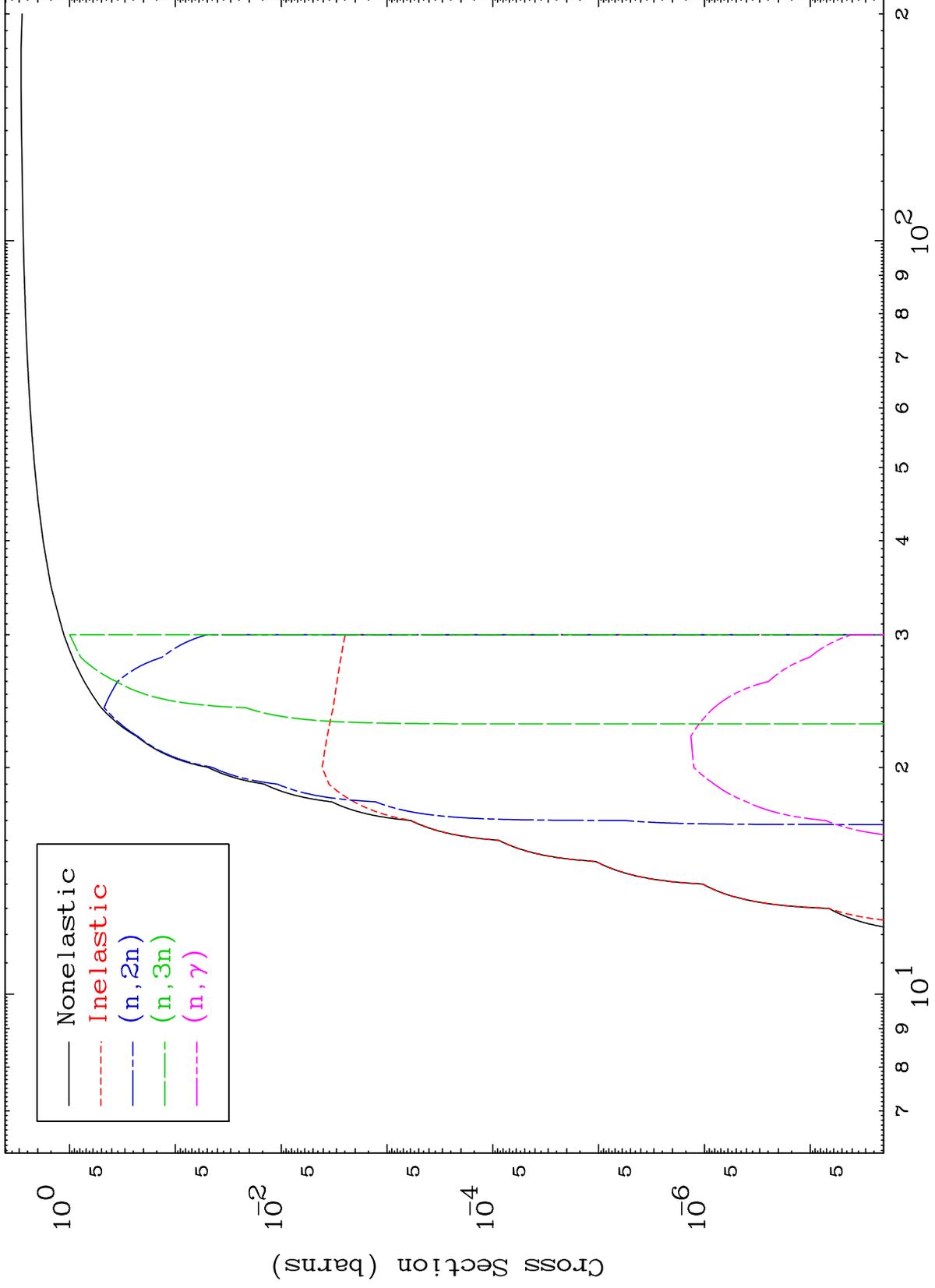
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Press Mouse Button to Start

MAT 8343

0 Kelvin  
 $\alpha$  Major Cross Sections

83-Bi-215



1

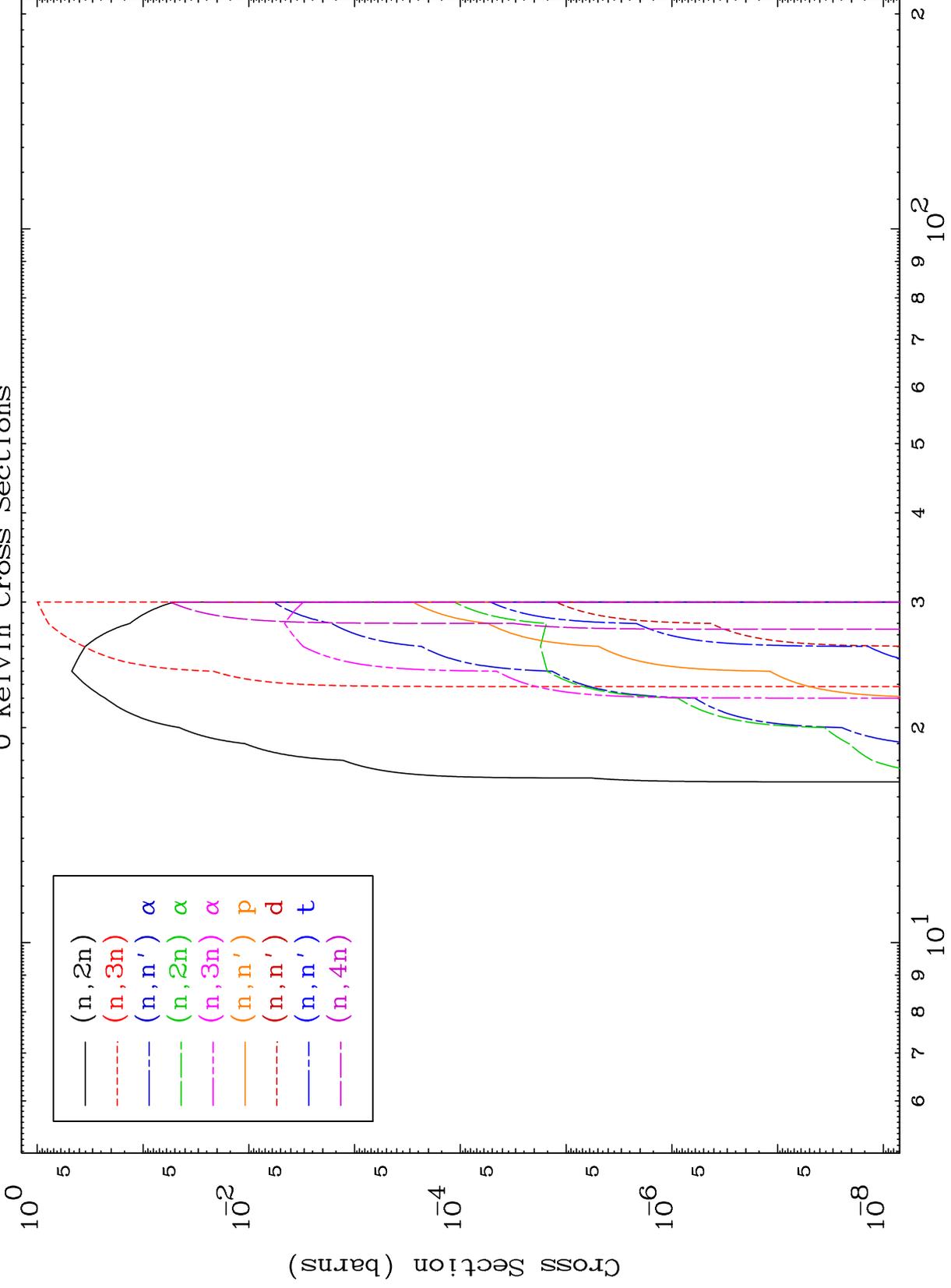
Incident Energy (MeV)

83-Bi-215

MAT 8343

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

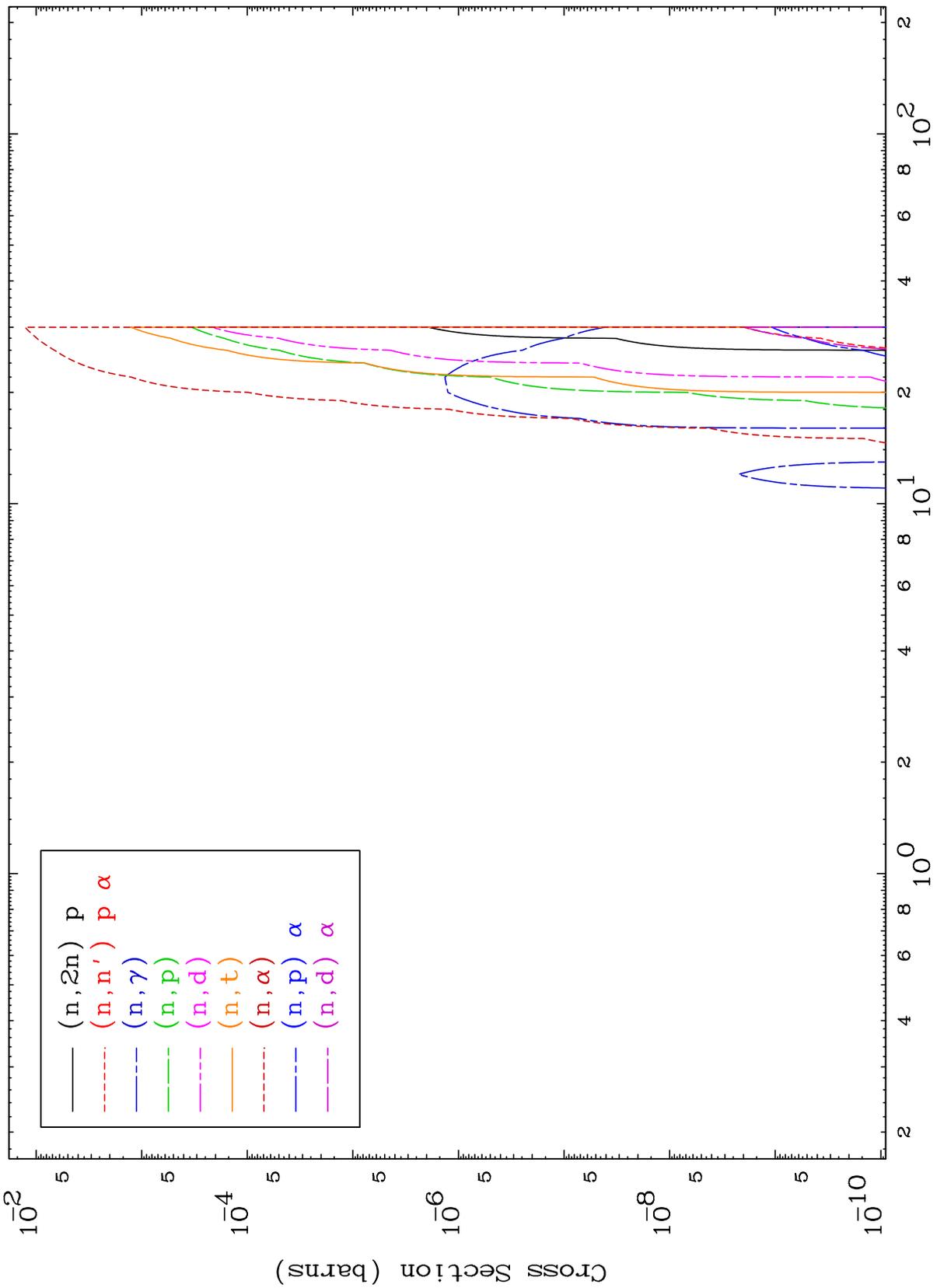
83-Bi-215



MAT 8343

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-215



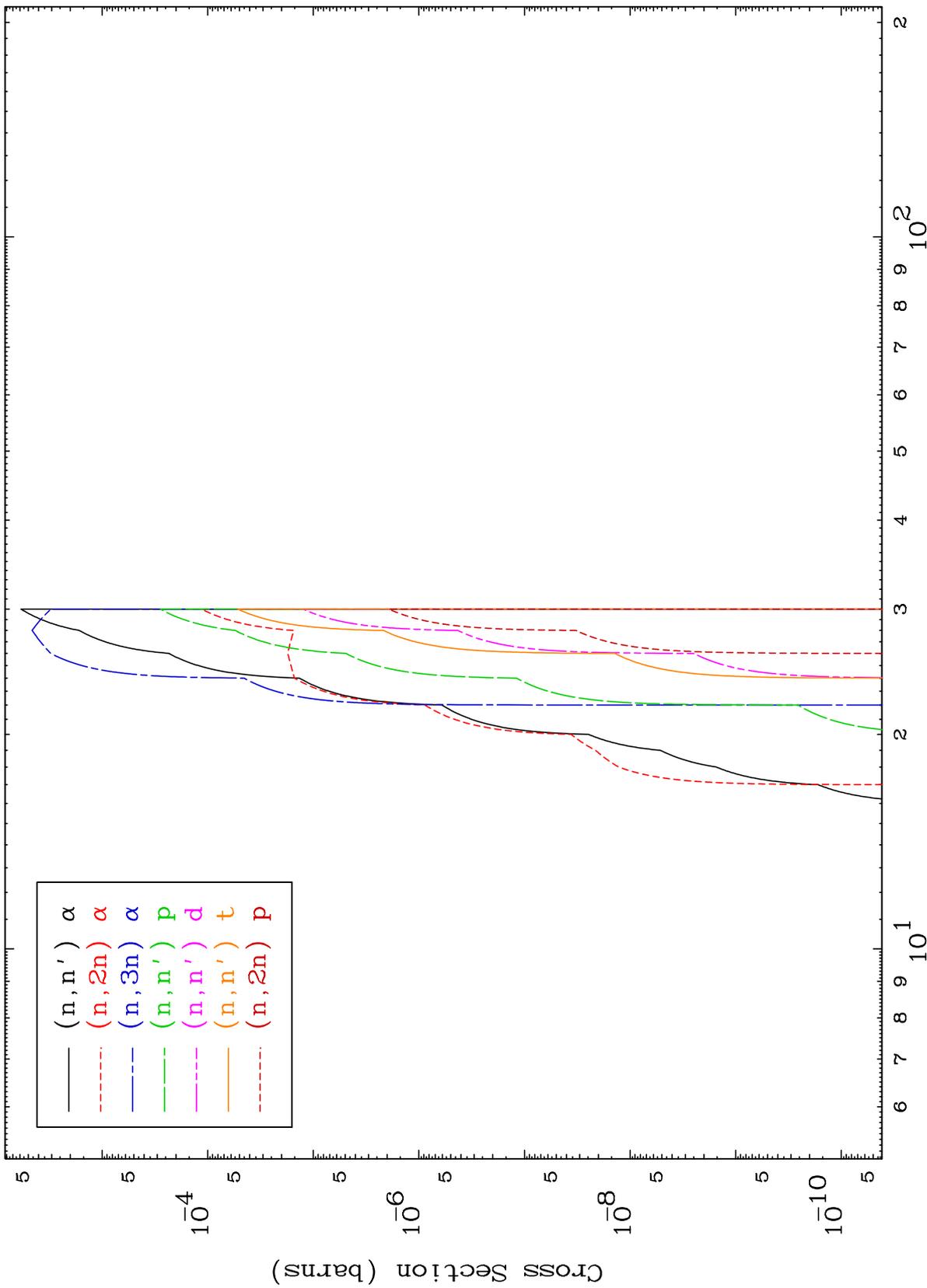
Incident Energy (MeV)

83-Bi-215

MAT 8343

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

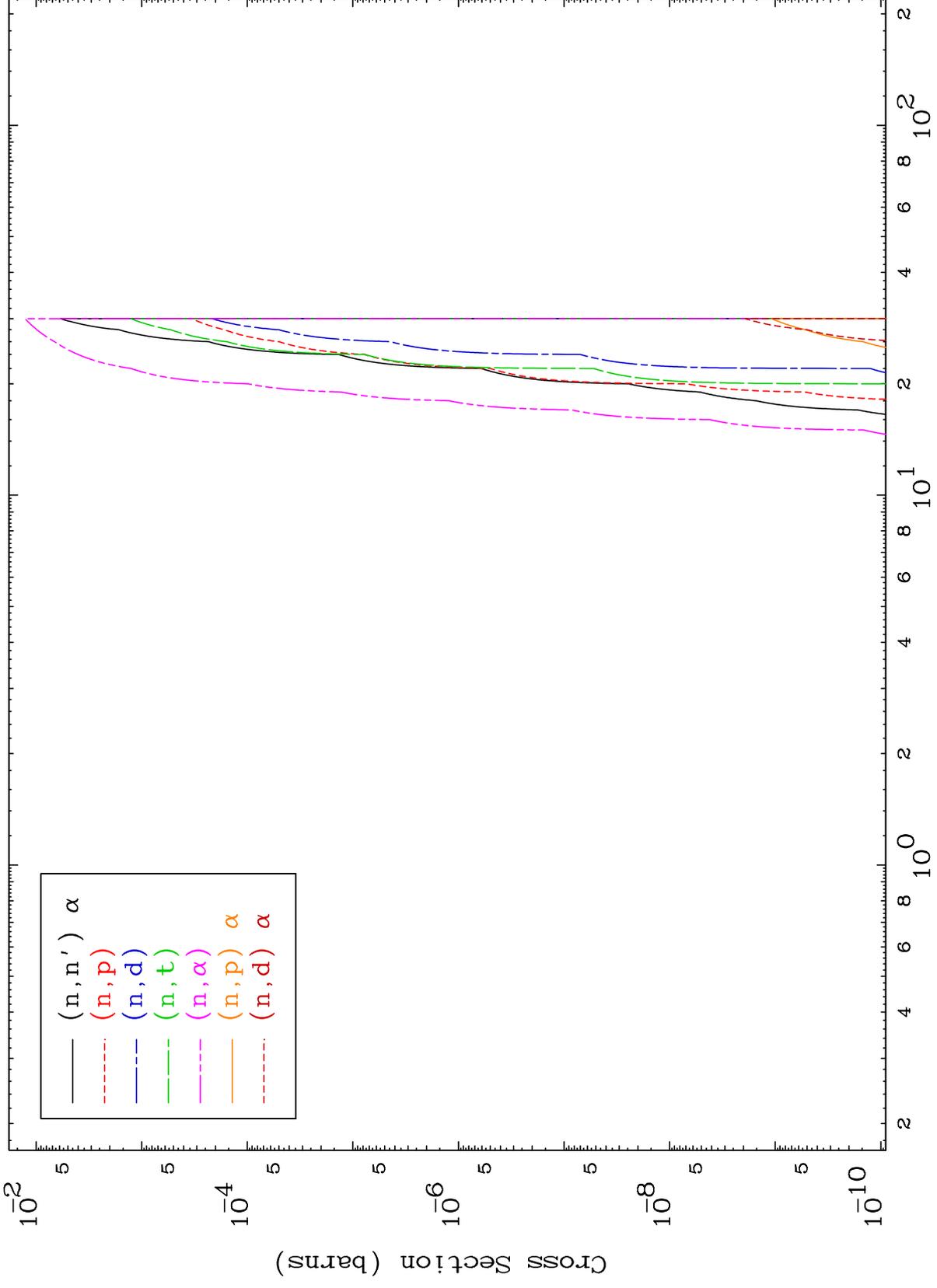
83-Bi-215



MAT 8343

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-215



5

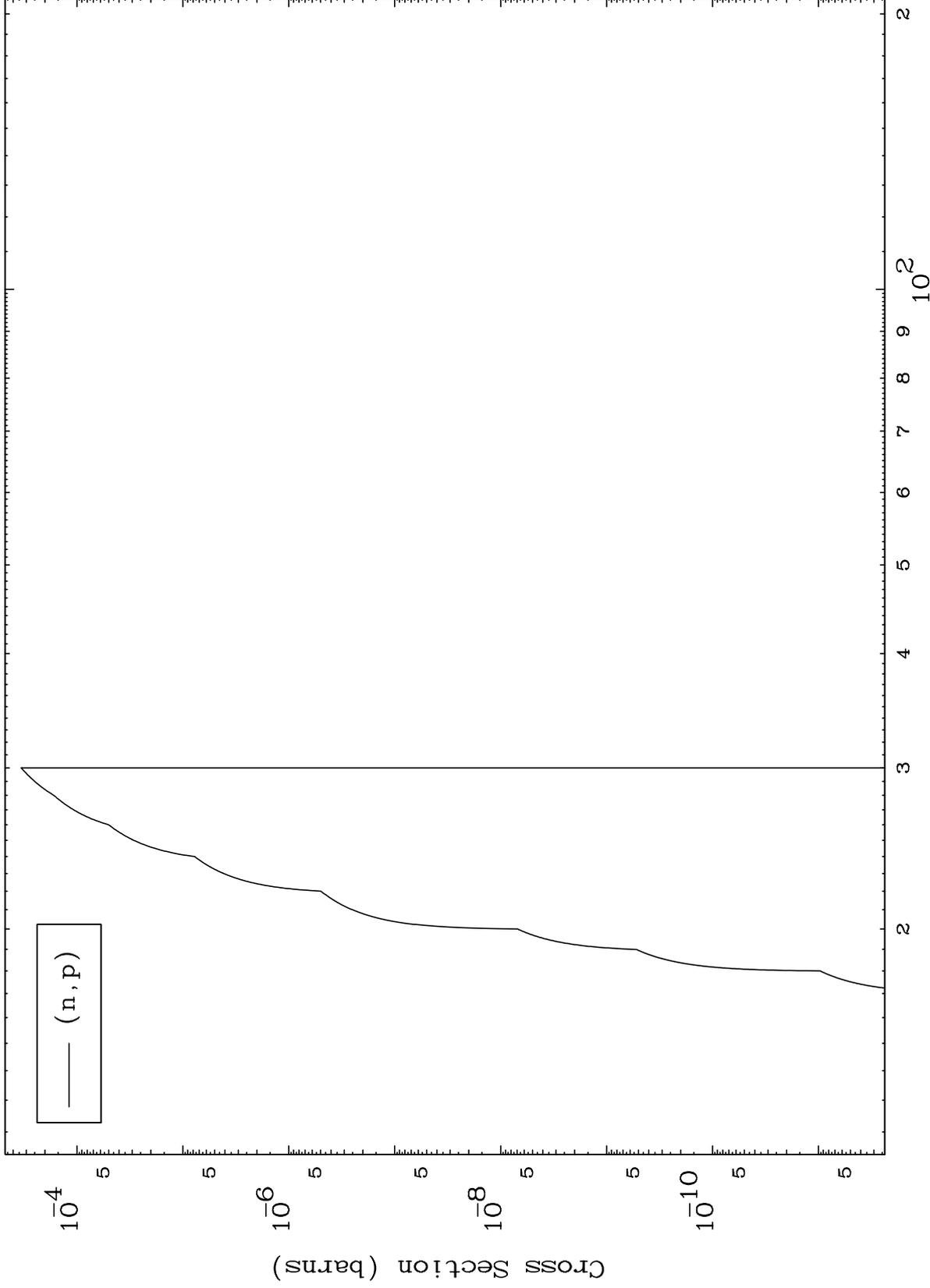
Incident Energy (MeV)

83-Bi-215

MAT 8343

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-215



6

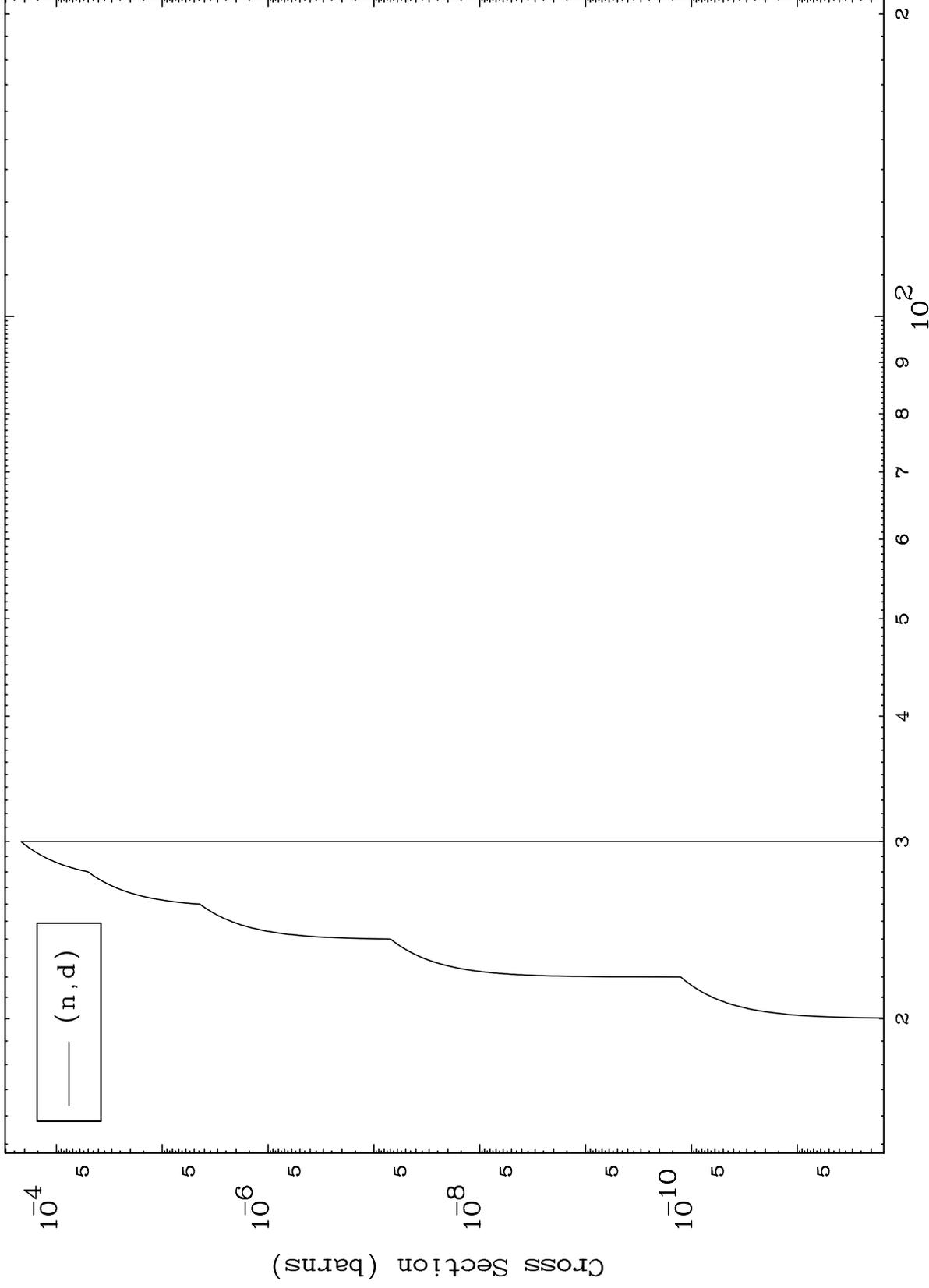
Incident Energy (MeV)

83-Bi-215

MAT 8343

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

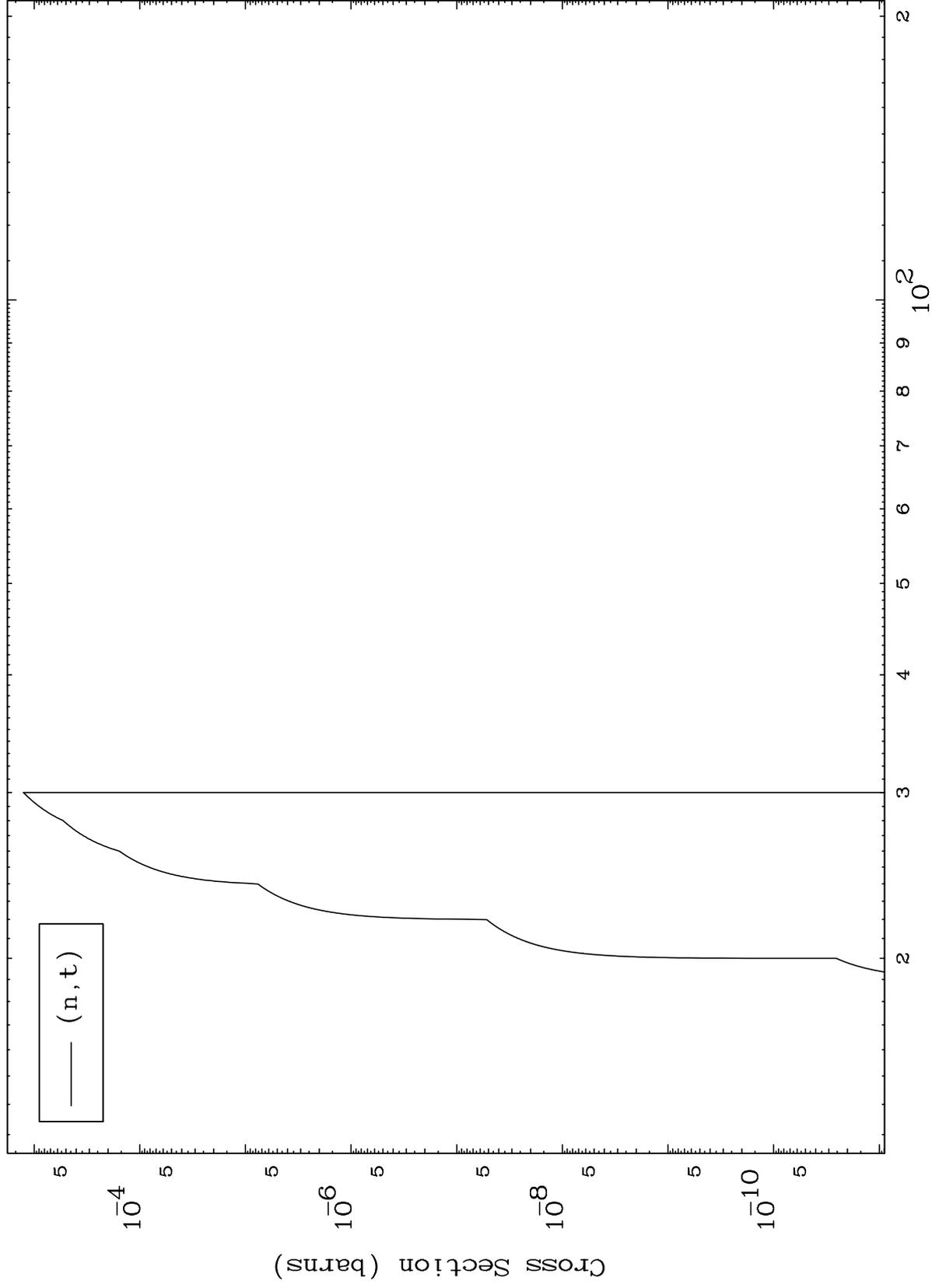
83-Bi-215



MAT 8343

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-215

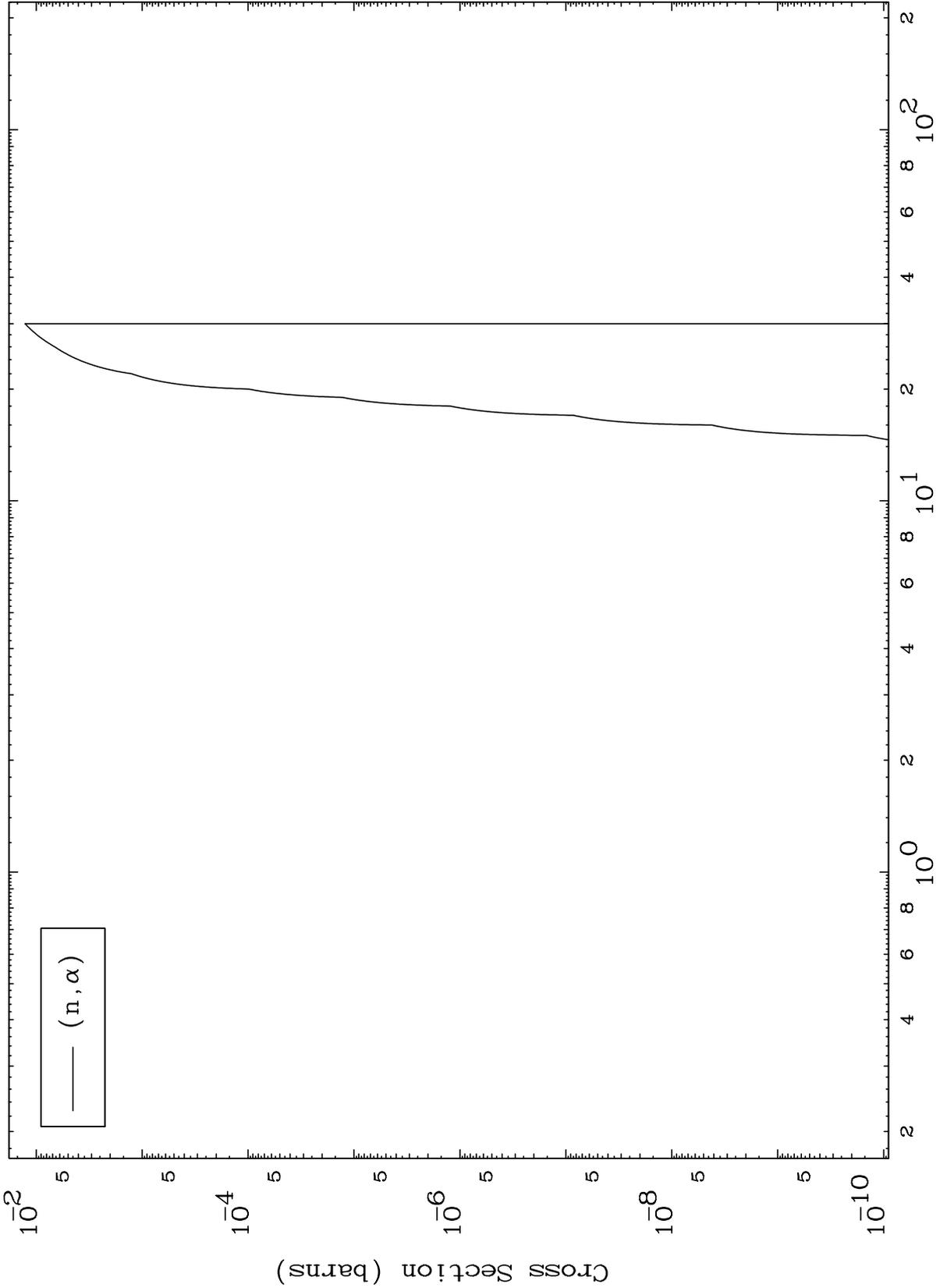


MAT 8343

( $\alpha, \alpha$ ) Levels

83-Bi-215

0 Kelvin Cross Sections

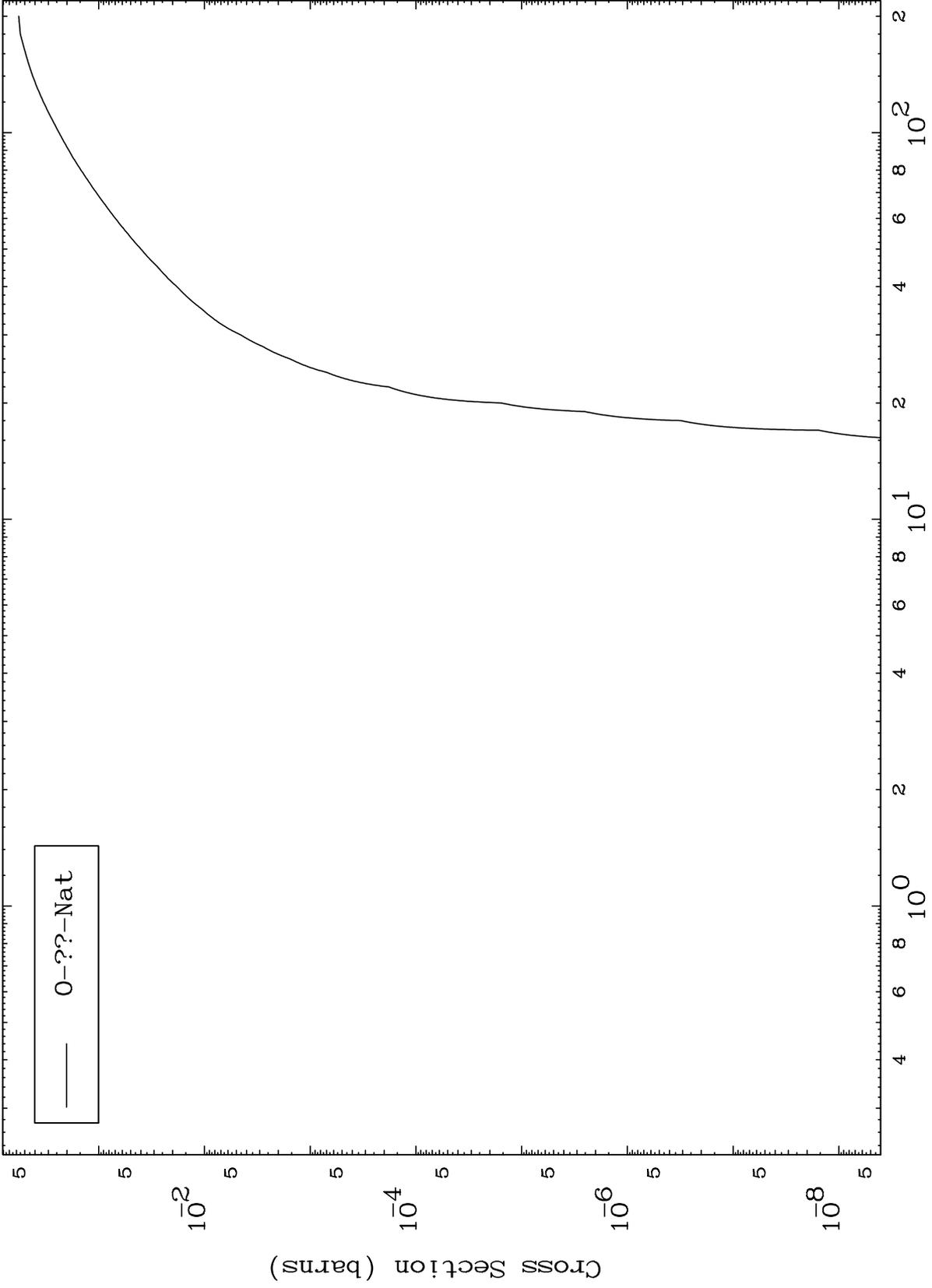


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Fission

83-Bi-215

Radionuclide Production Cross Section



10

Incident Energy (MeV)

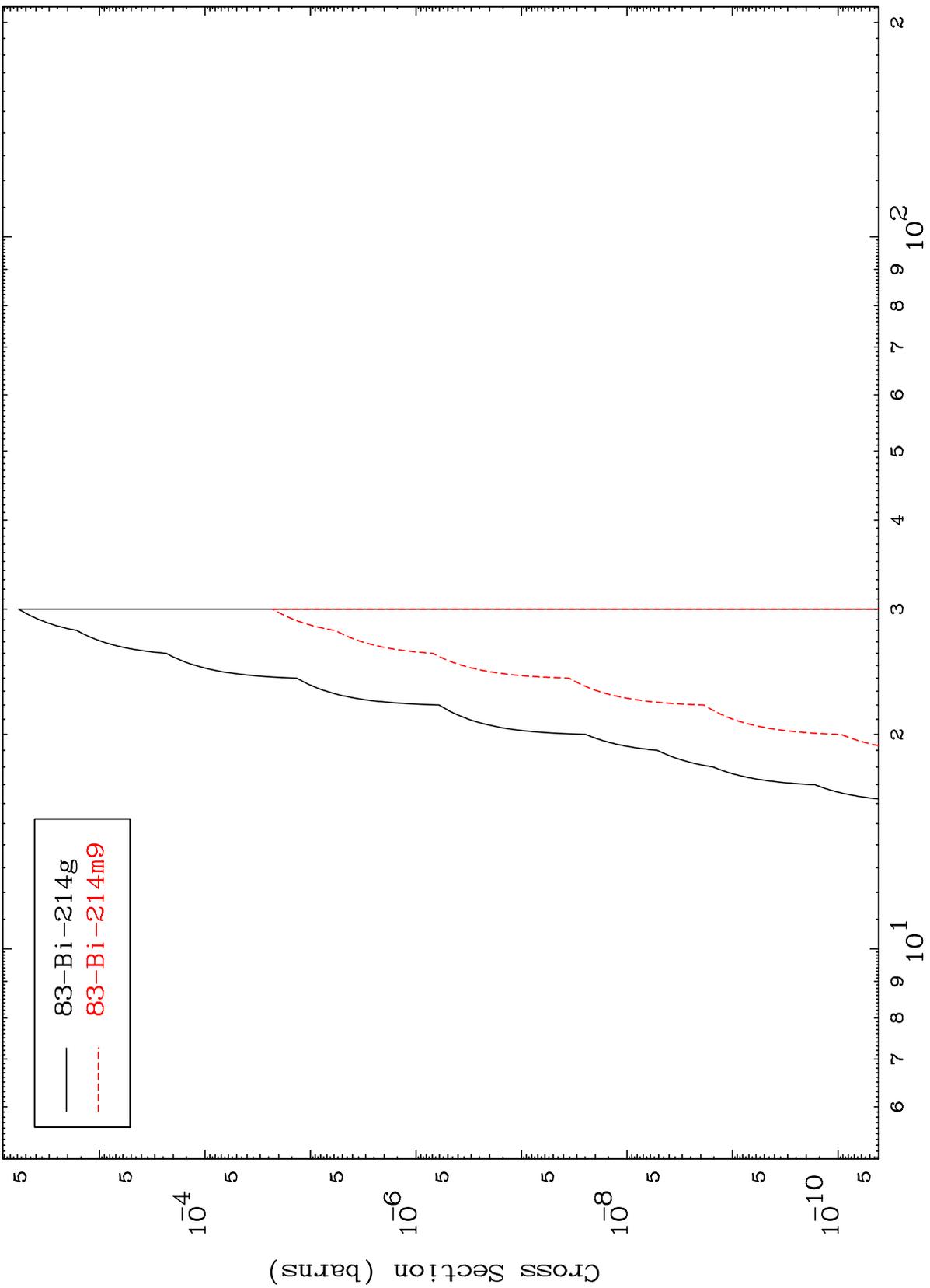
83-Bi-215

MAT 8343

83-Bi-215

(n,n')  $\alpha$

Radionuclide Production Cross Section



83-Bi-214g  
83-Bi-214m9

83-Bi-215

Incident Energy (MeV)

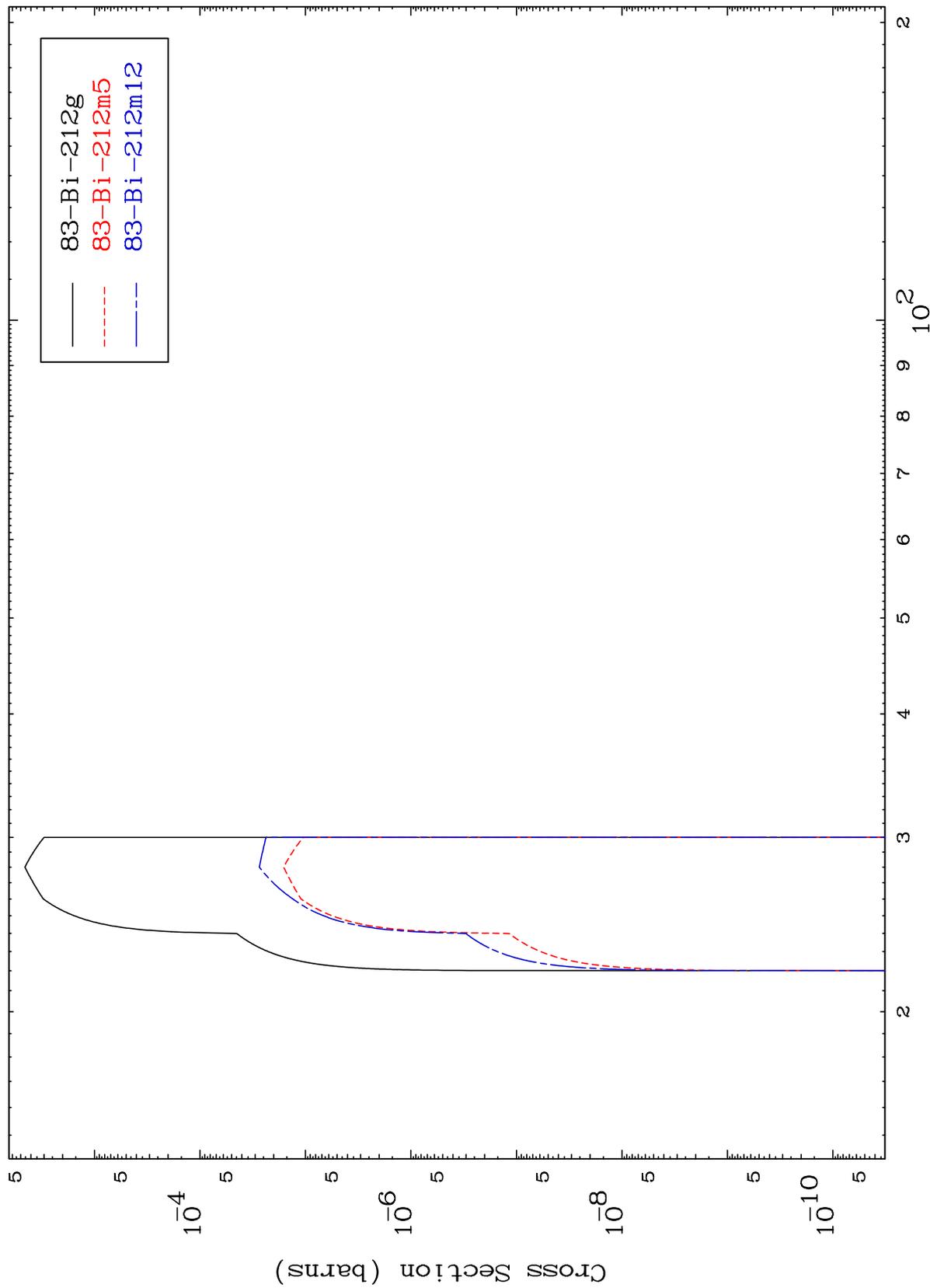
11

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$(n,3n) \alpha$

83-Bi-215

Radionuclide Production Cross Section



12

Incident Energy (MeV)

83-Bi-215

MAT 8343

83-Bi-215

(n,  $\alpha$ )

Radionuclide Production Cross Section

