

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

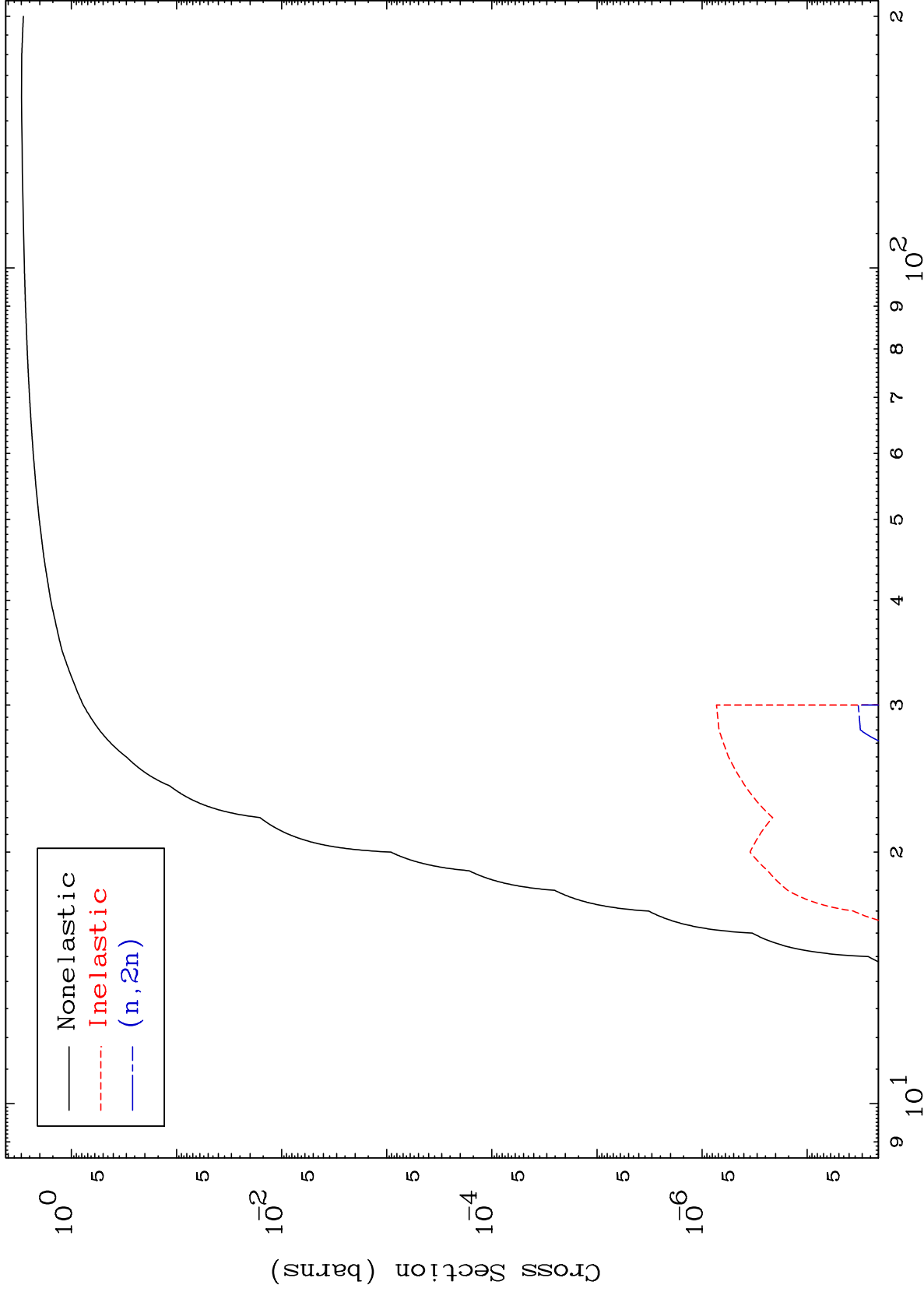
Press Mouse Button to Start

MAT 9719

$\alpha$  Major

97-Bk-238

0 Kelvin Cross Sections



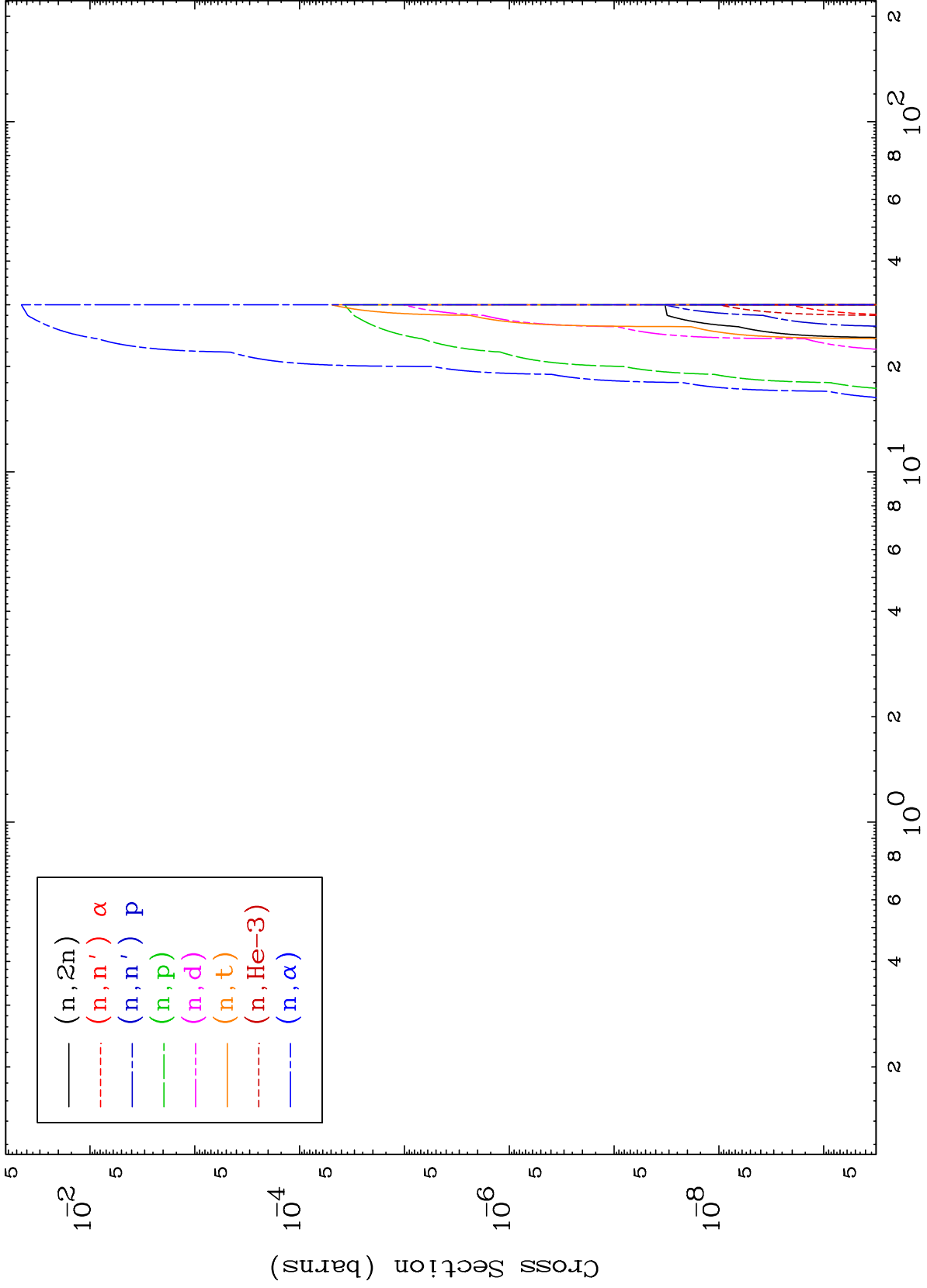
Incident Energy (MeV)

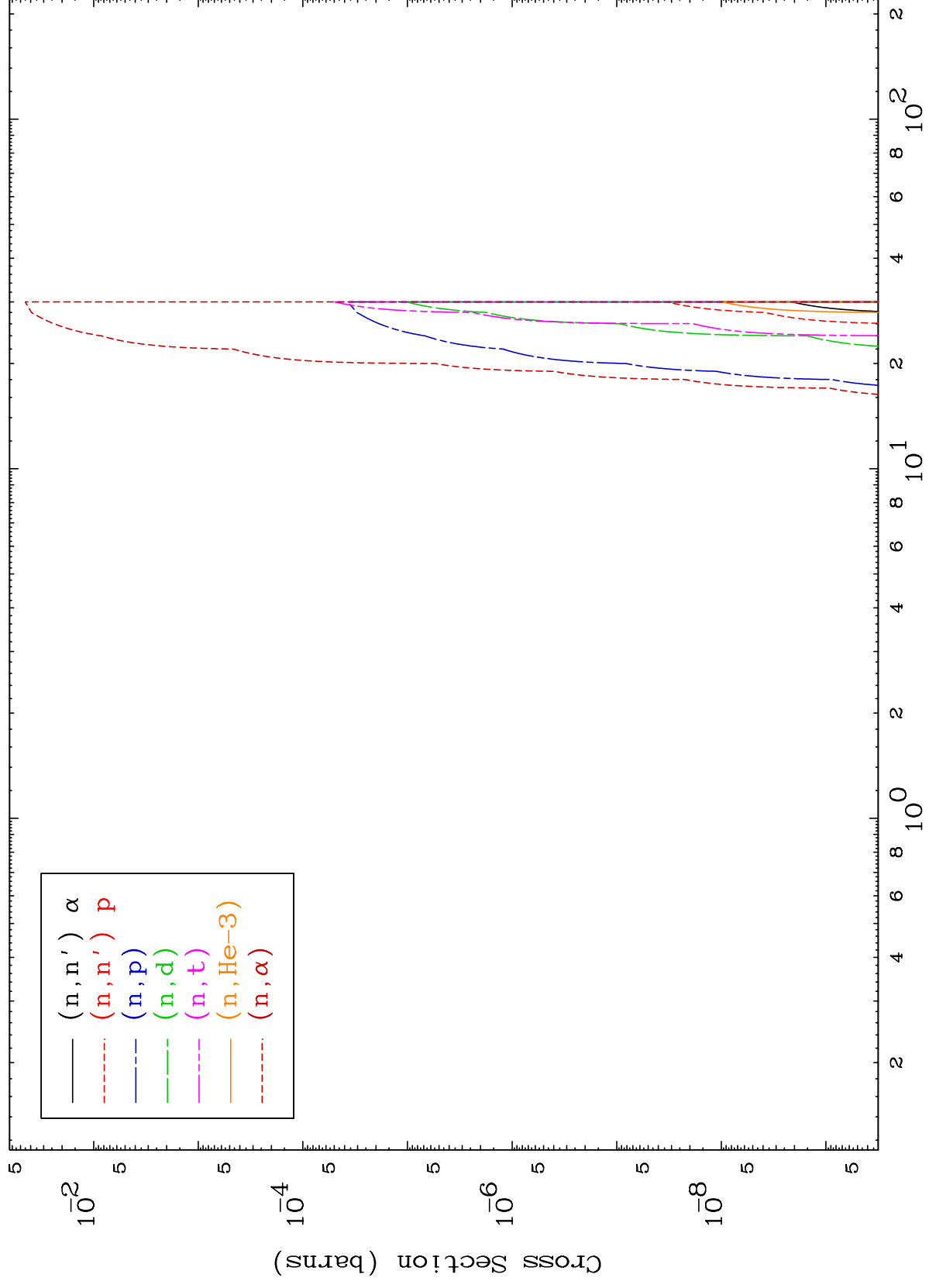
97-Bk-238

MAT 9719

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

97-Bk-238

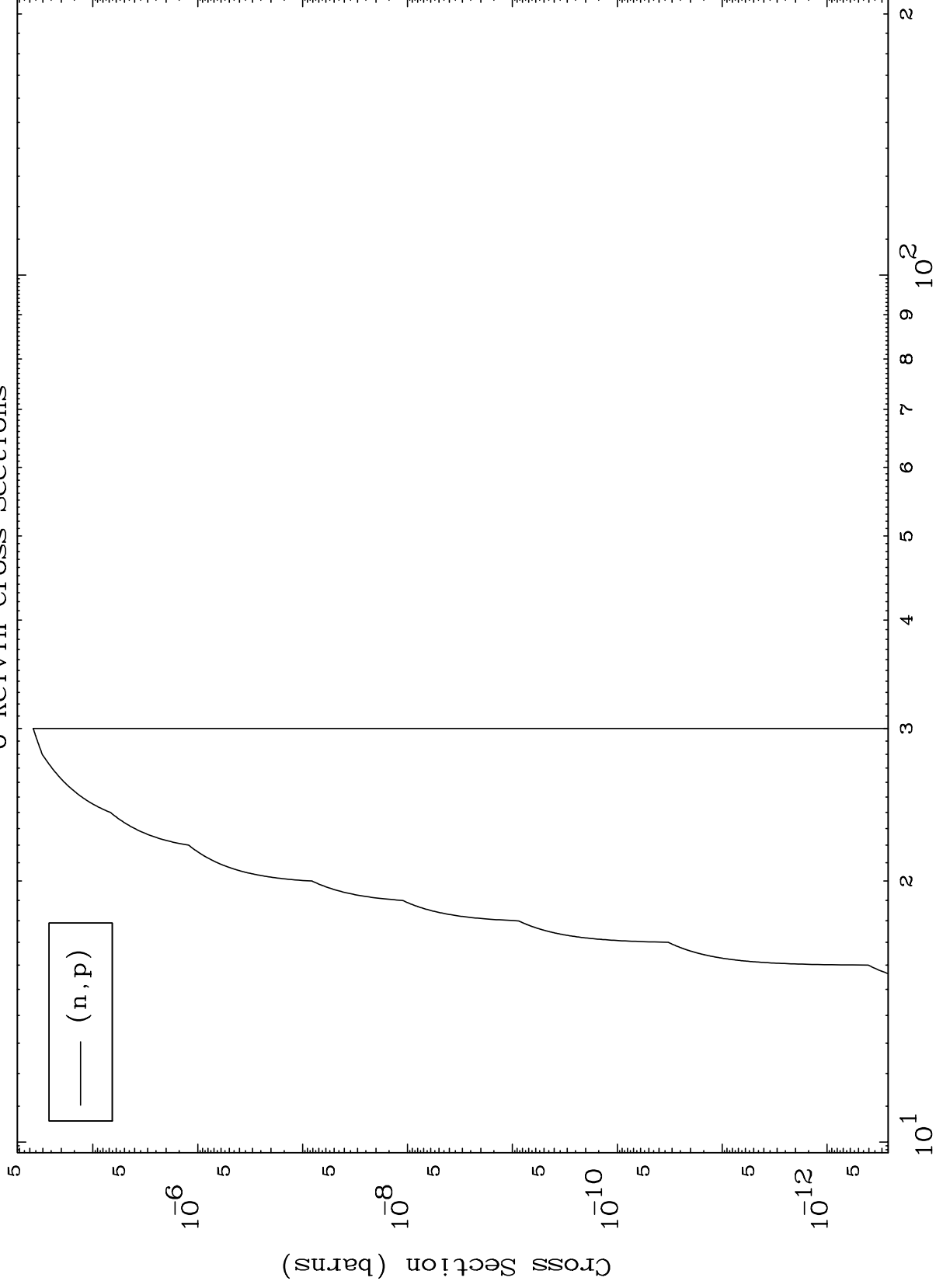




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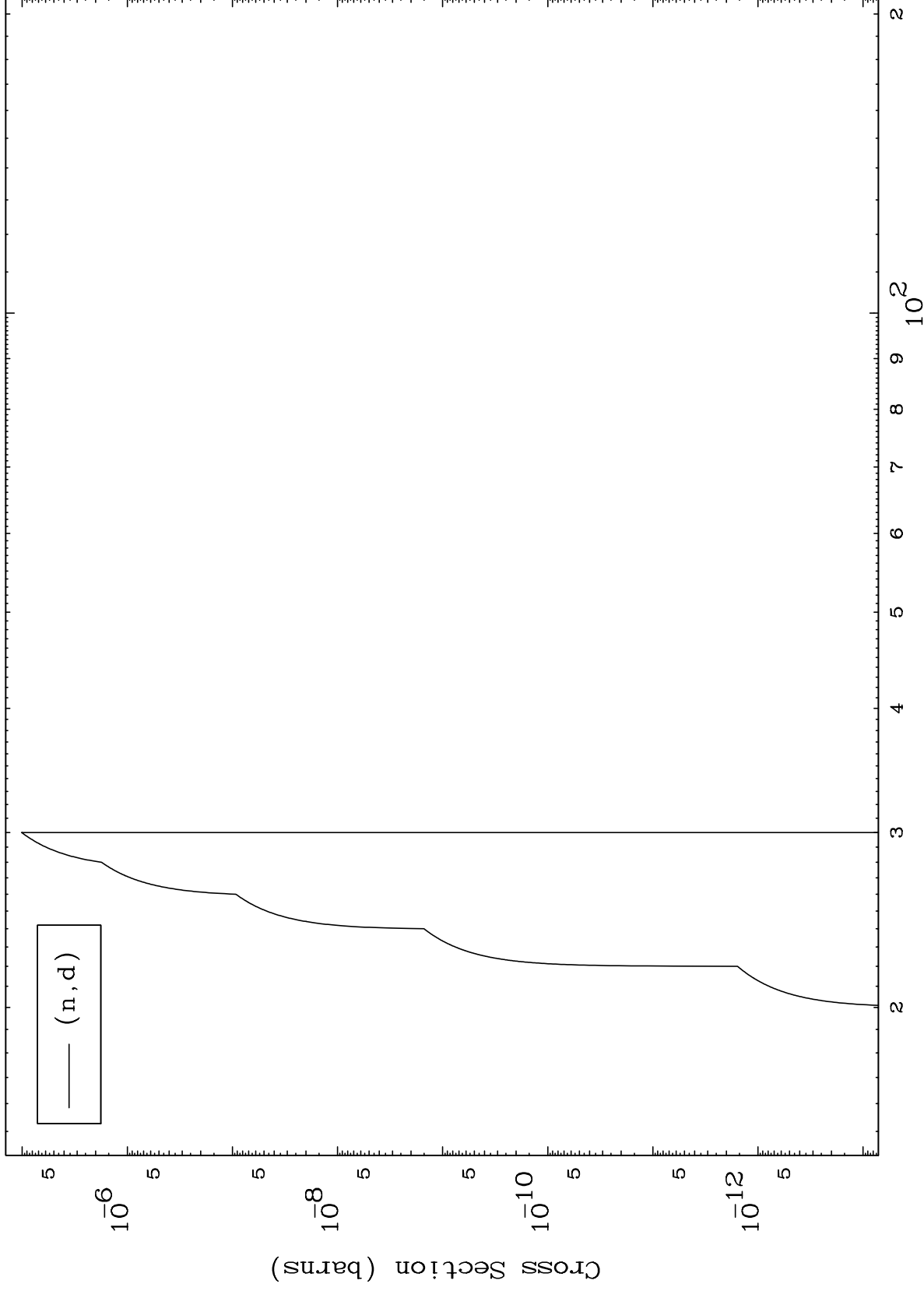
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

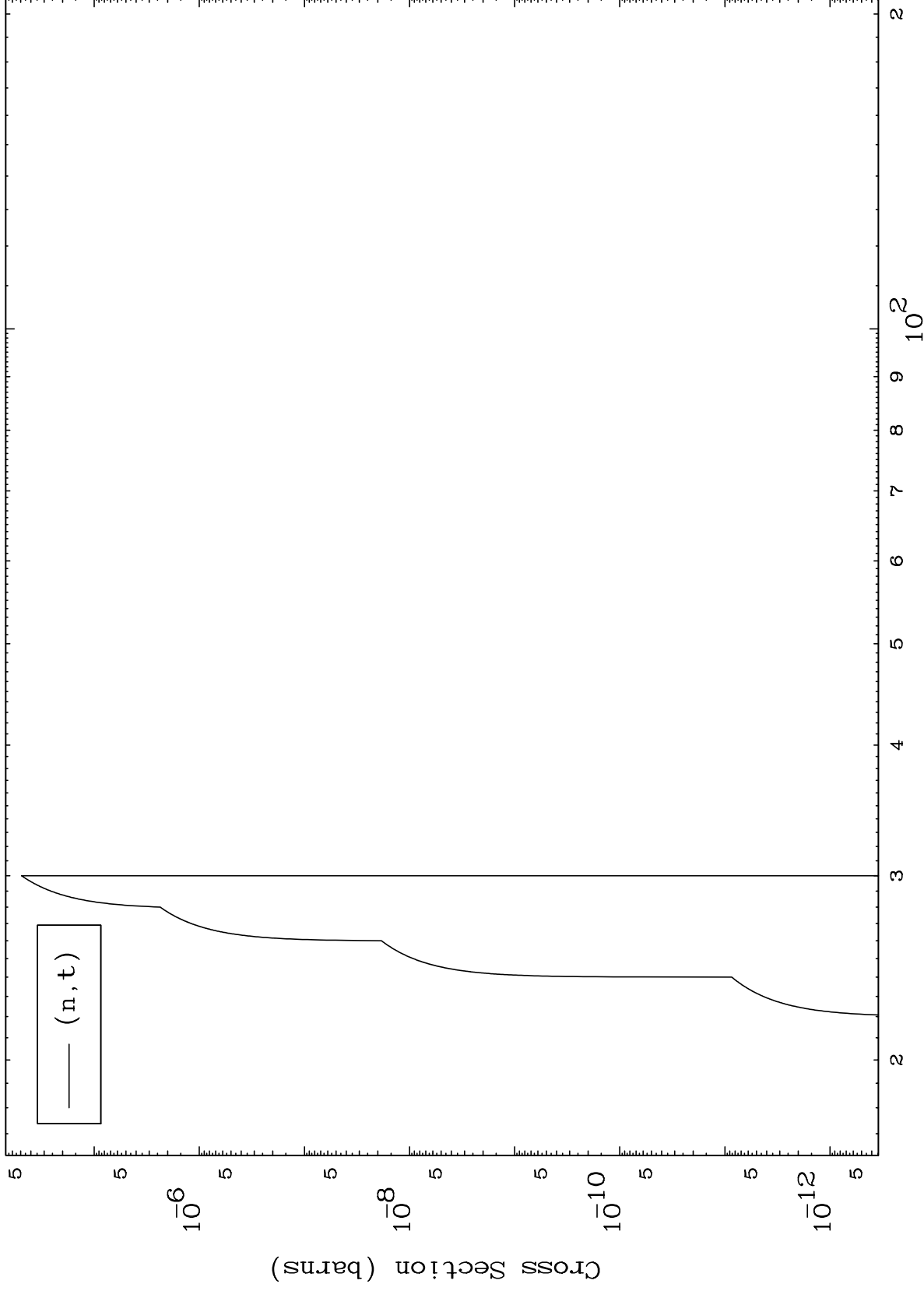
97-Bk-238

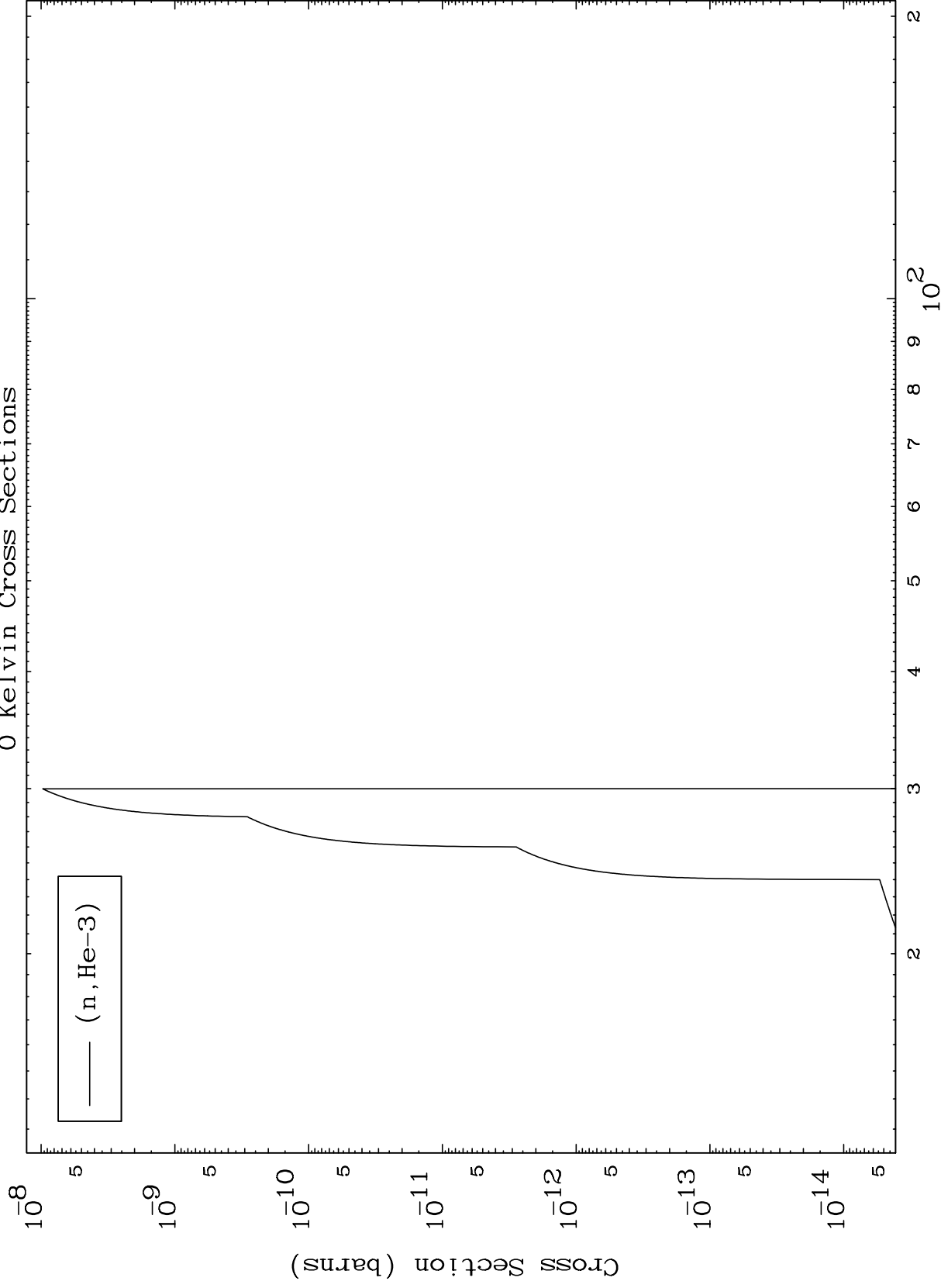


Incident Energy (MeV)

97-Bk-238





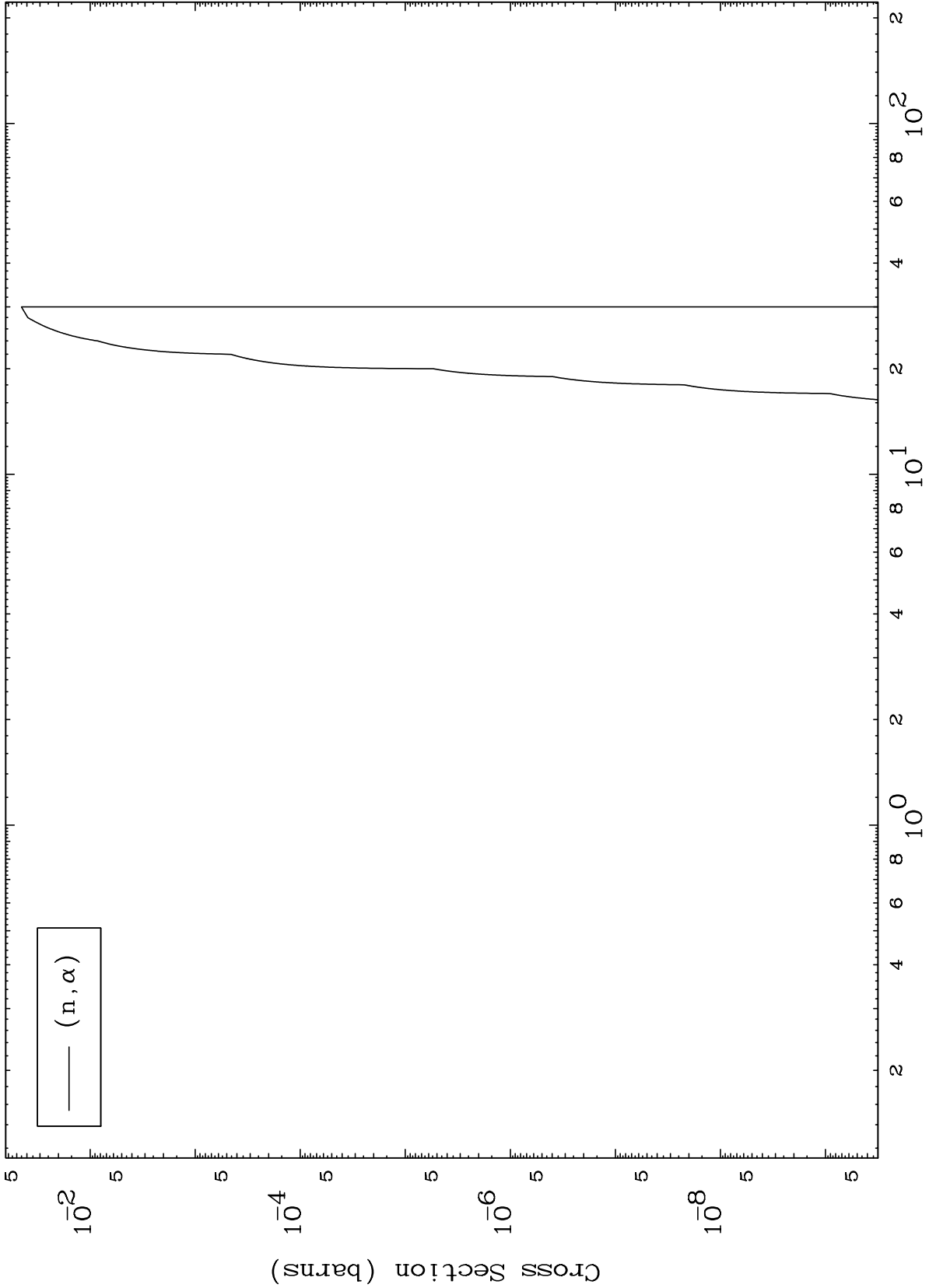


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( $\alpha, \alpha$ ) Levels

97-Bk-238

0 Kelvin Cross Sections



MAT 9719

Fission

97-Bk-238

Radionuclide Production Cross Section

