

Program EVALPLOT
(Version 2021-1)

by

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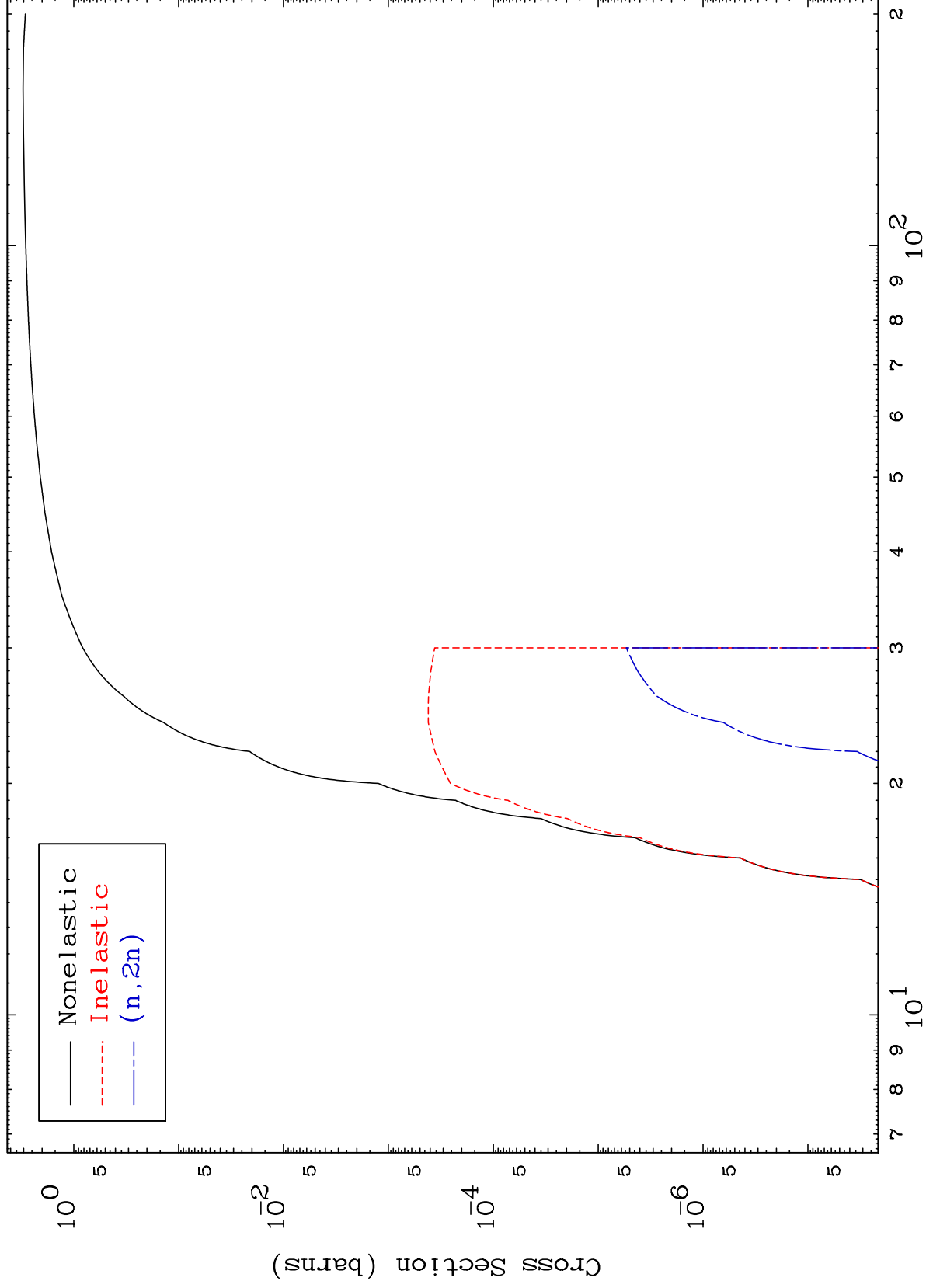
Press Mouse Button to Start

MAT 9743

α Major

97-Bk-246

0 Kelvin Cross Sections



1

Incident Energy (MeV)

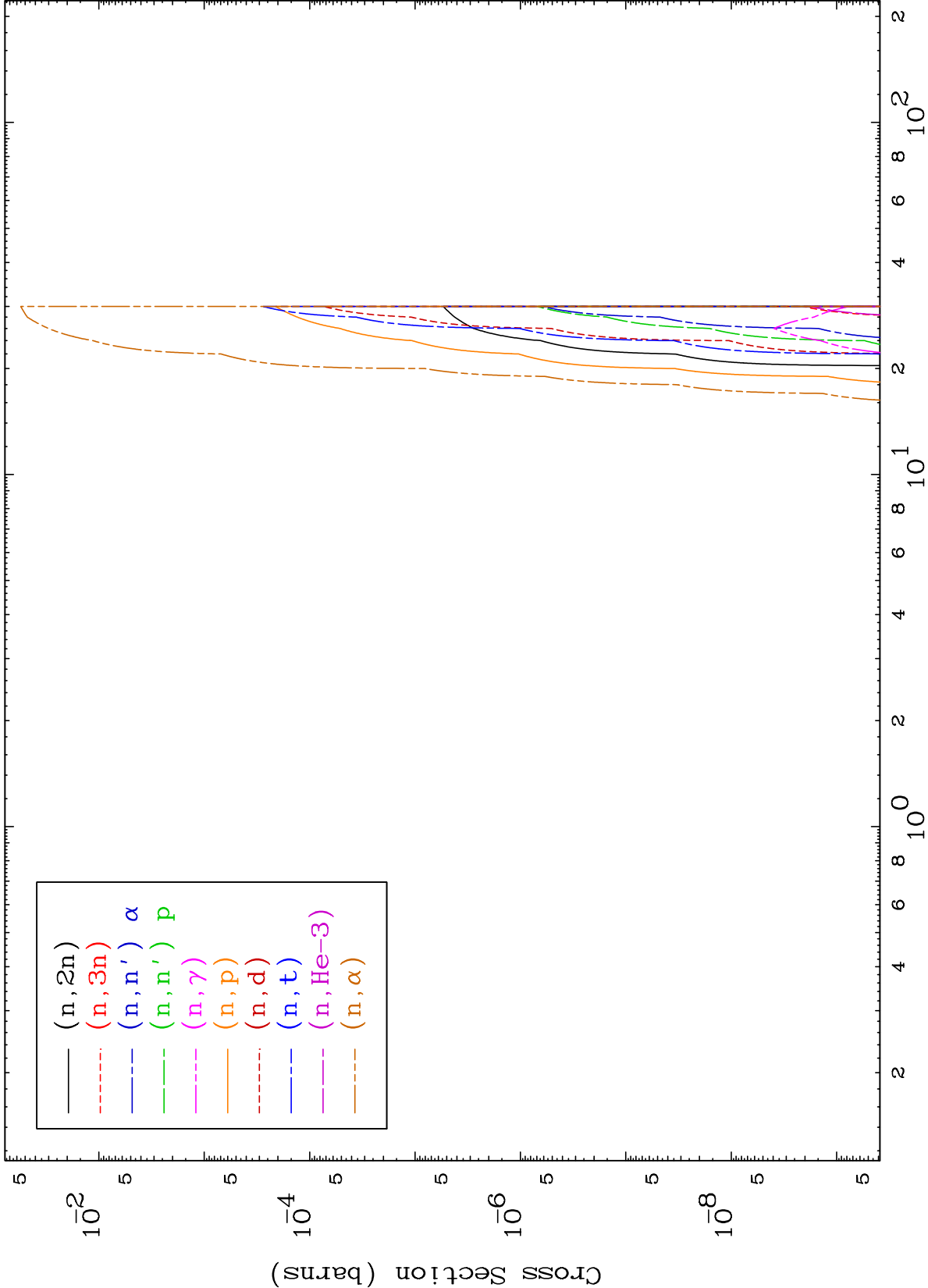
97-Bk-246

MAT 9743

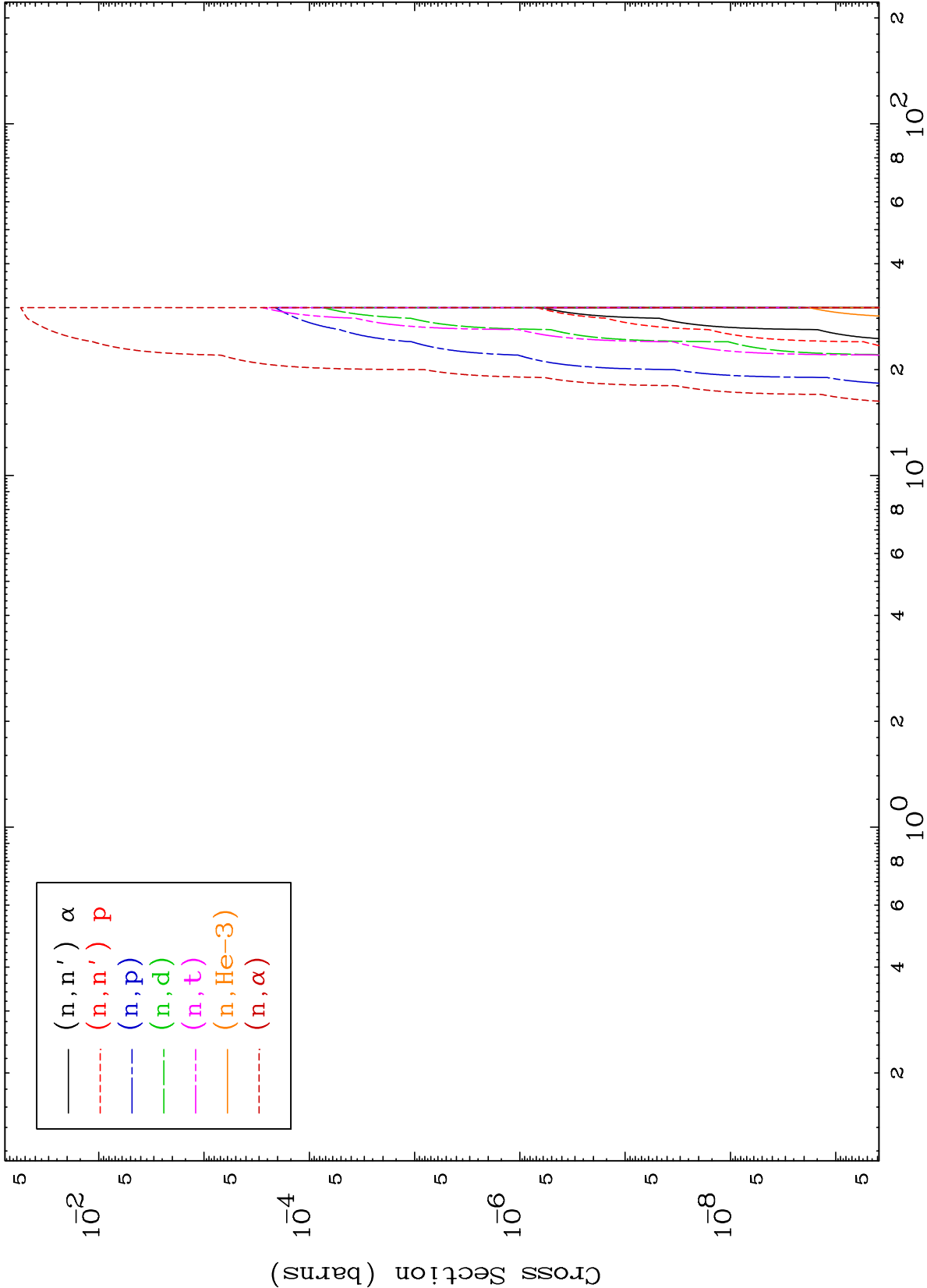
α Neutron Absorption

97-Bk-246

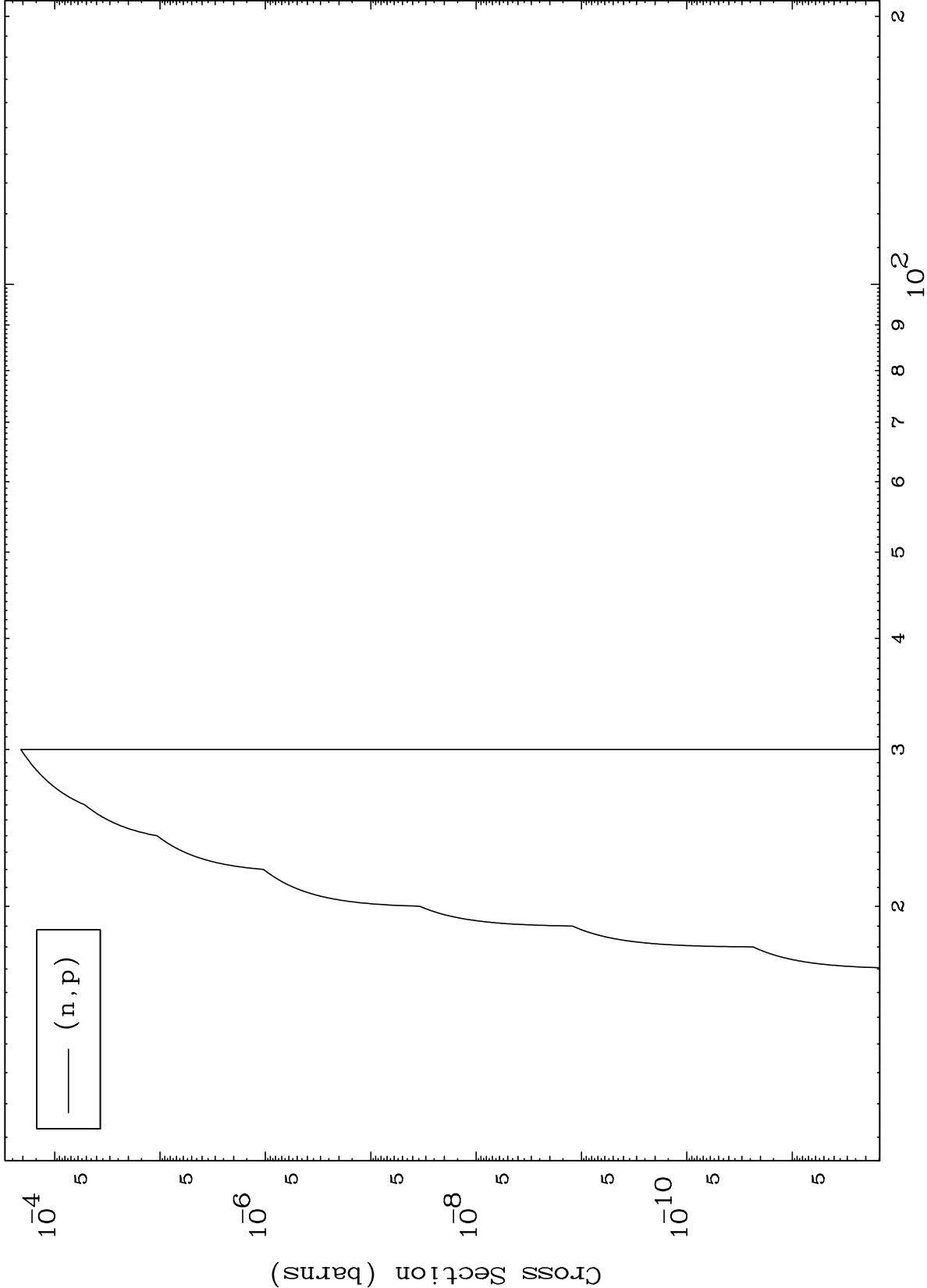
0 Kelvin Cross Sections

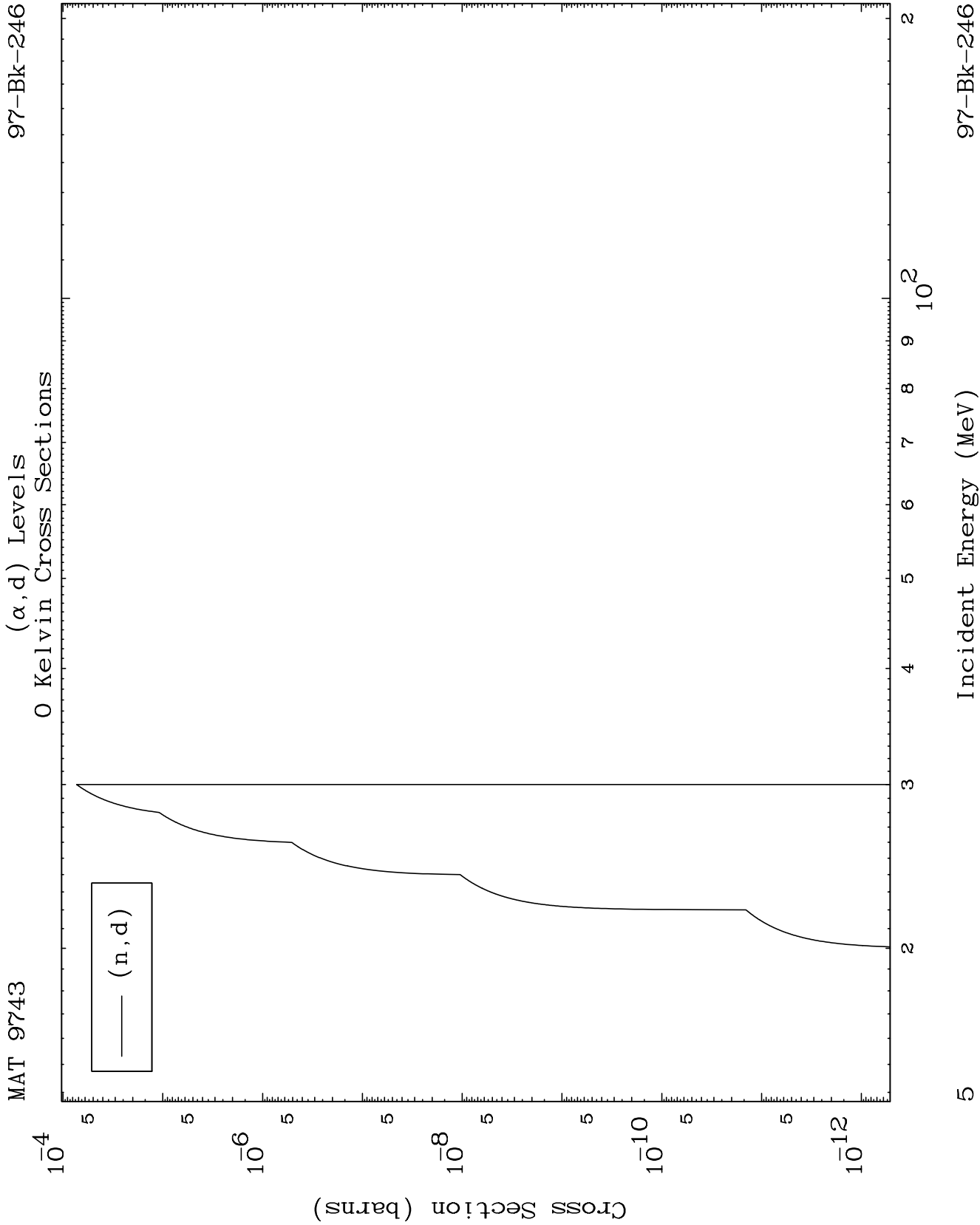


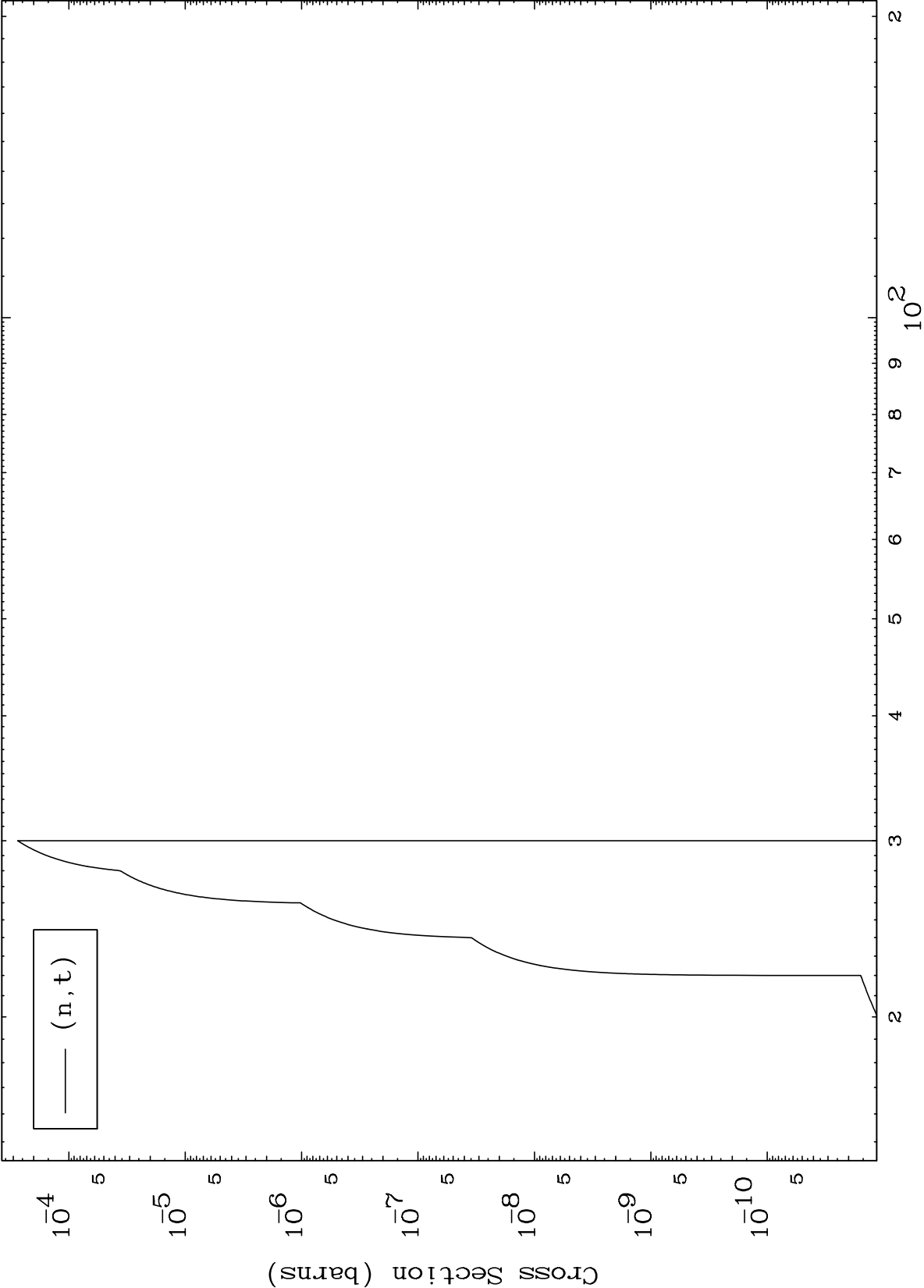
0 Kelvin Cross Sections



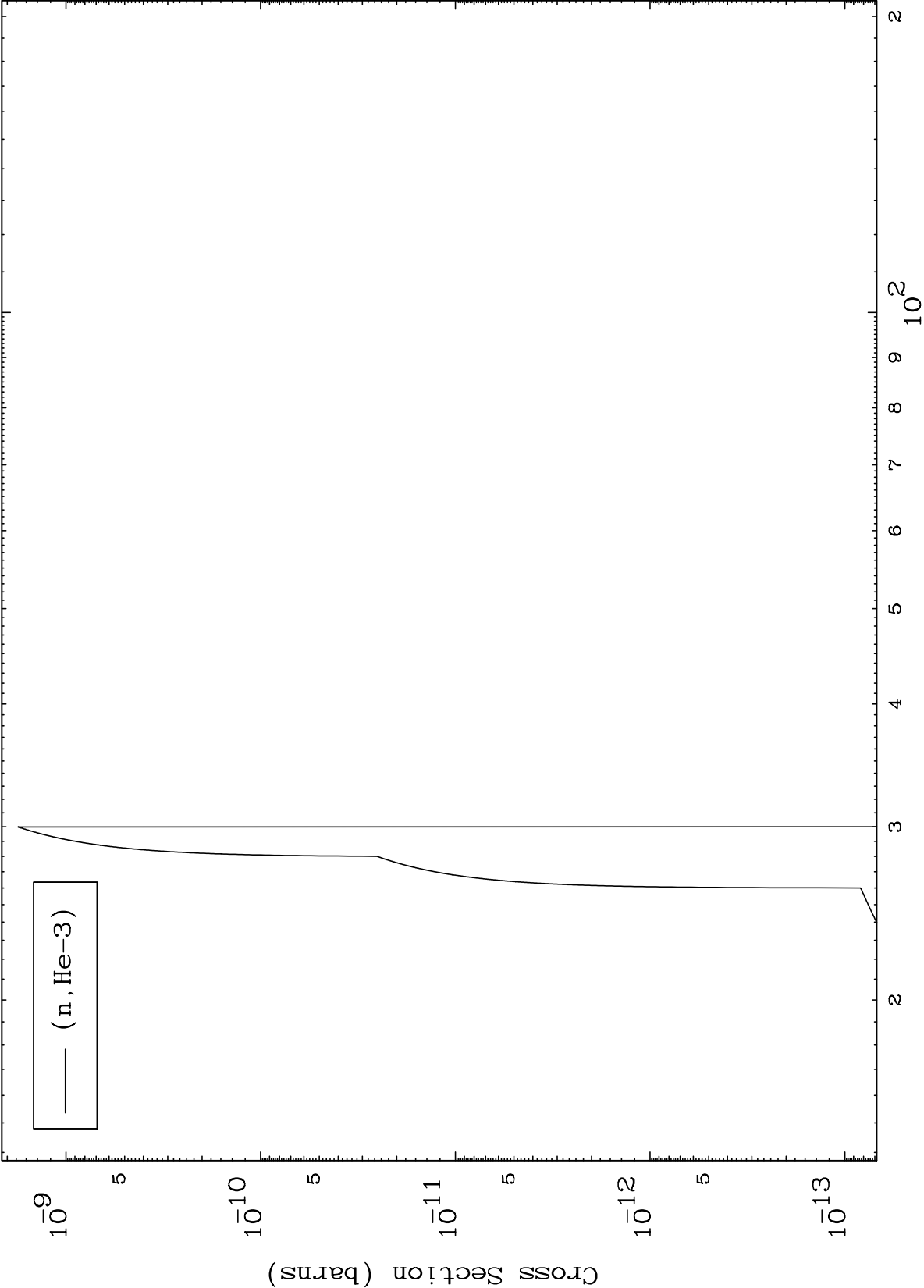
Incident Energy (MeV)







(α ,He3) Levels
0 Kelvin Cross Sections

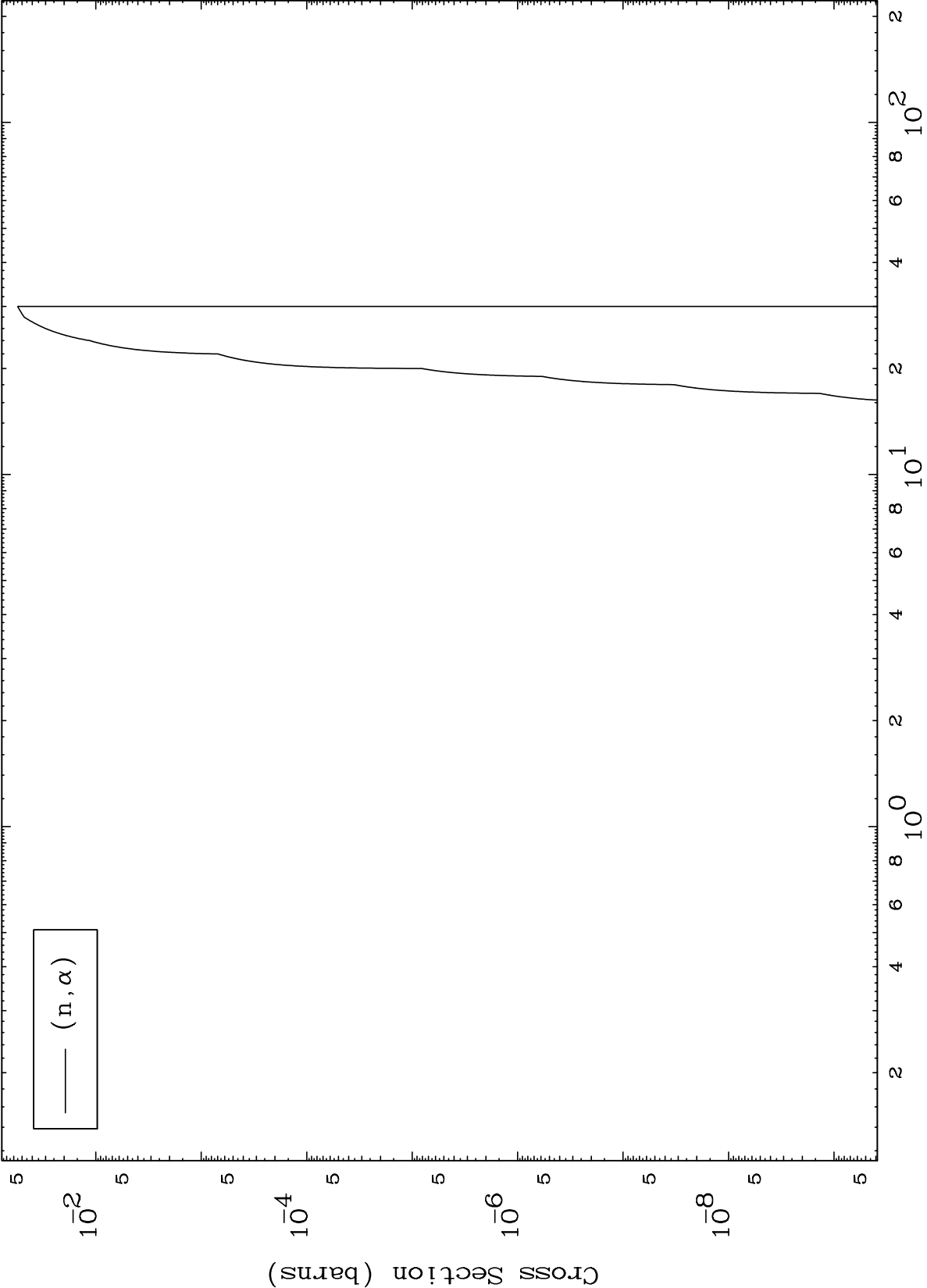


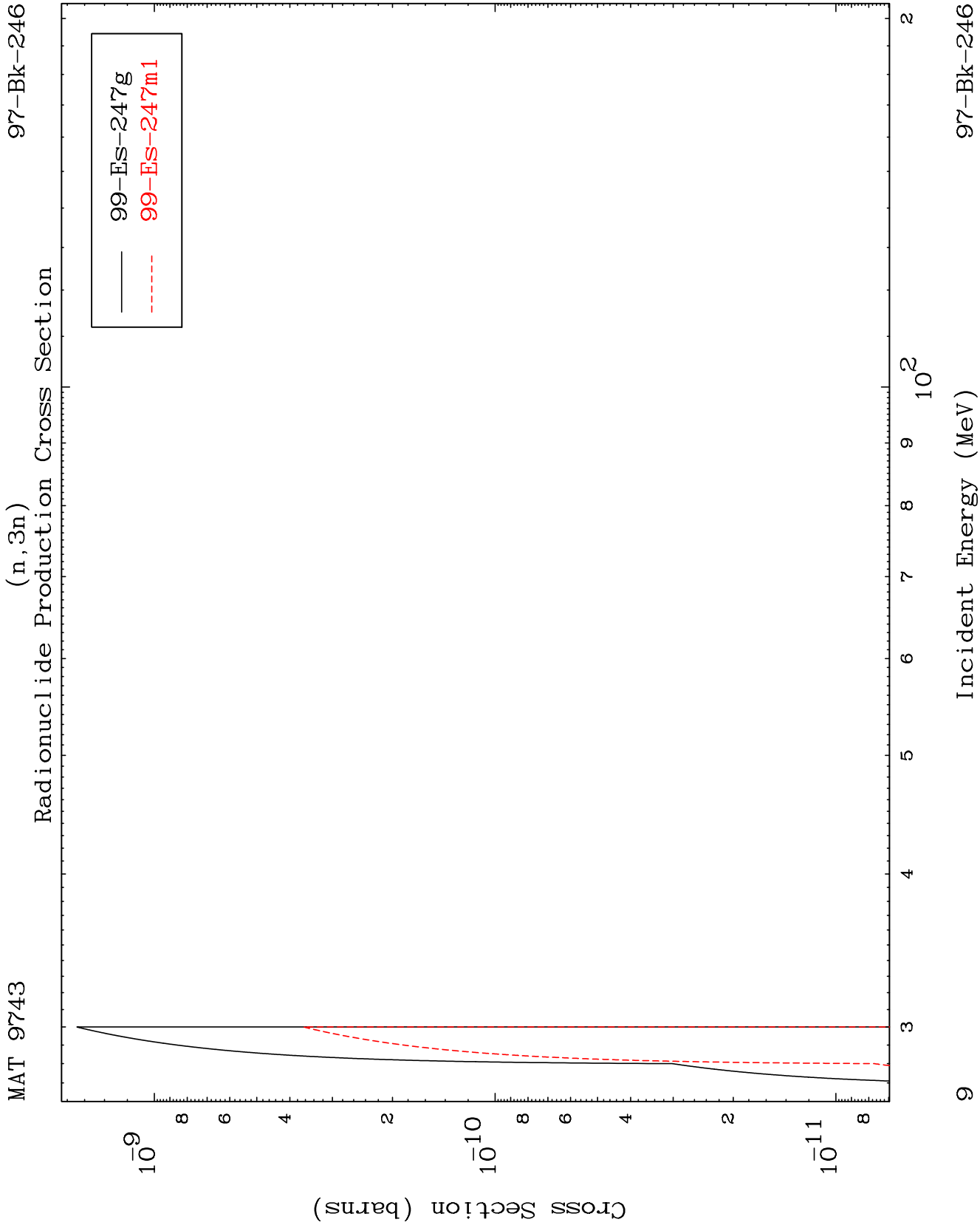
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(α, α) Levels

97-Bk-246

0 Kelvin Cross Sections



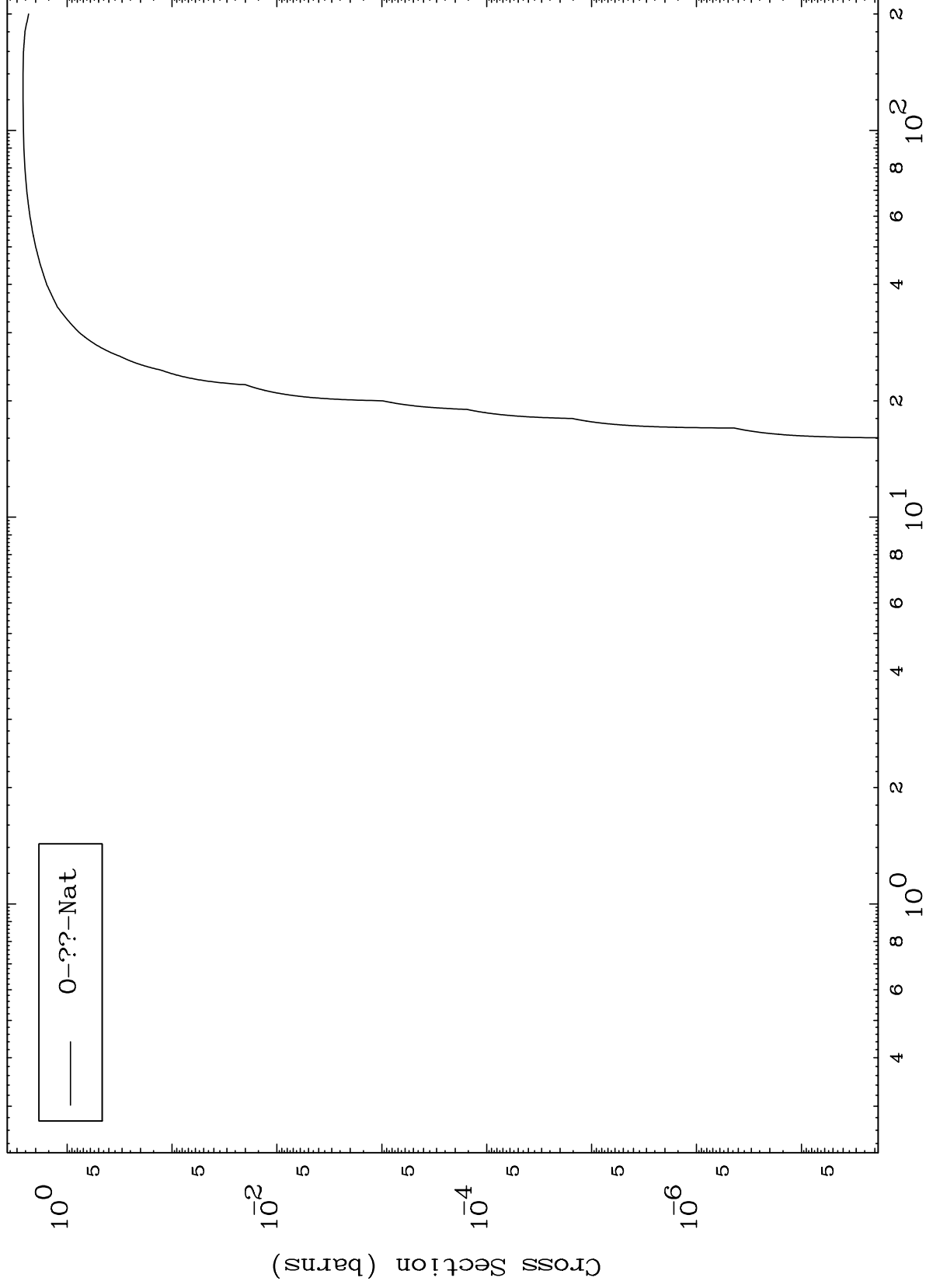


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Fission

97-Bk-246

Radionuclide Production Cross Section



10

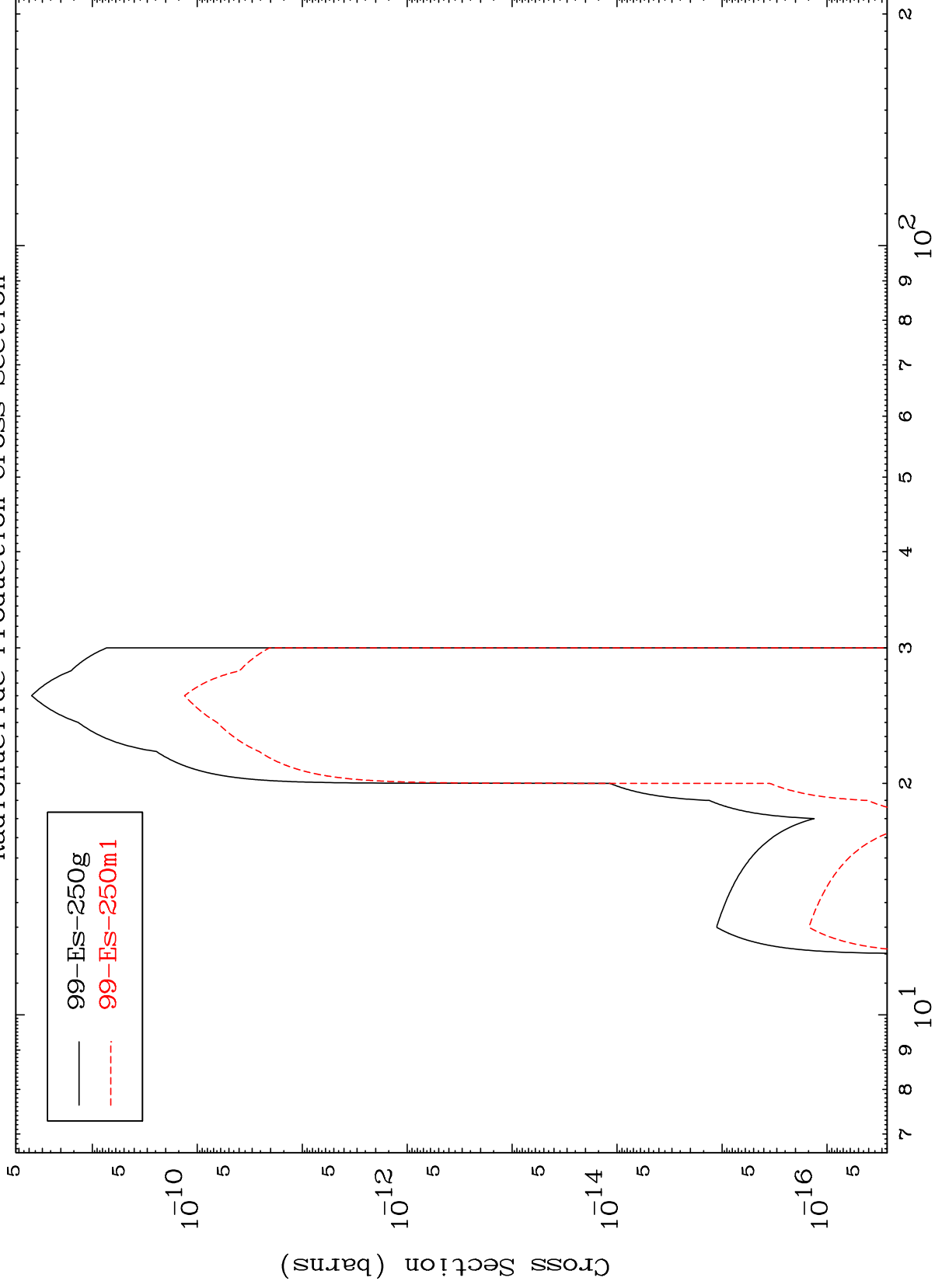
Incident Energy (MeV)

97-Bk-246

MAT 9743

97-Bk-246

Radionuclide Production Cross Section



11

97-Bk-246