

Program EVALPLOT
(Version 2021-1)

by

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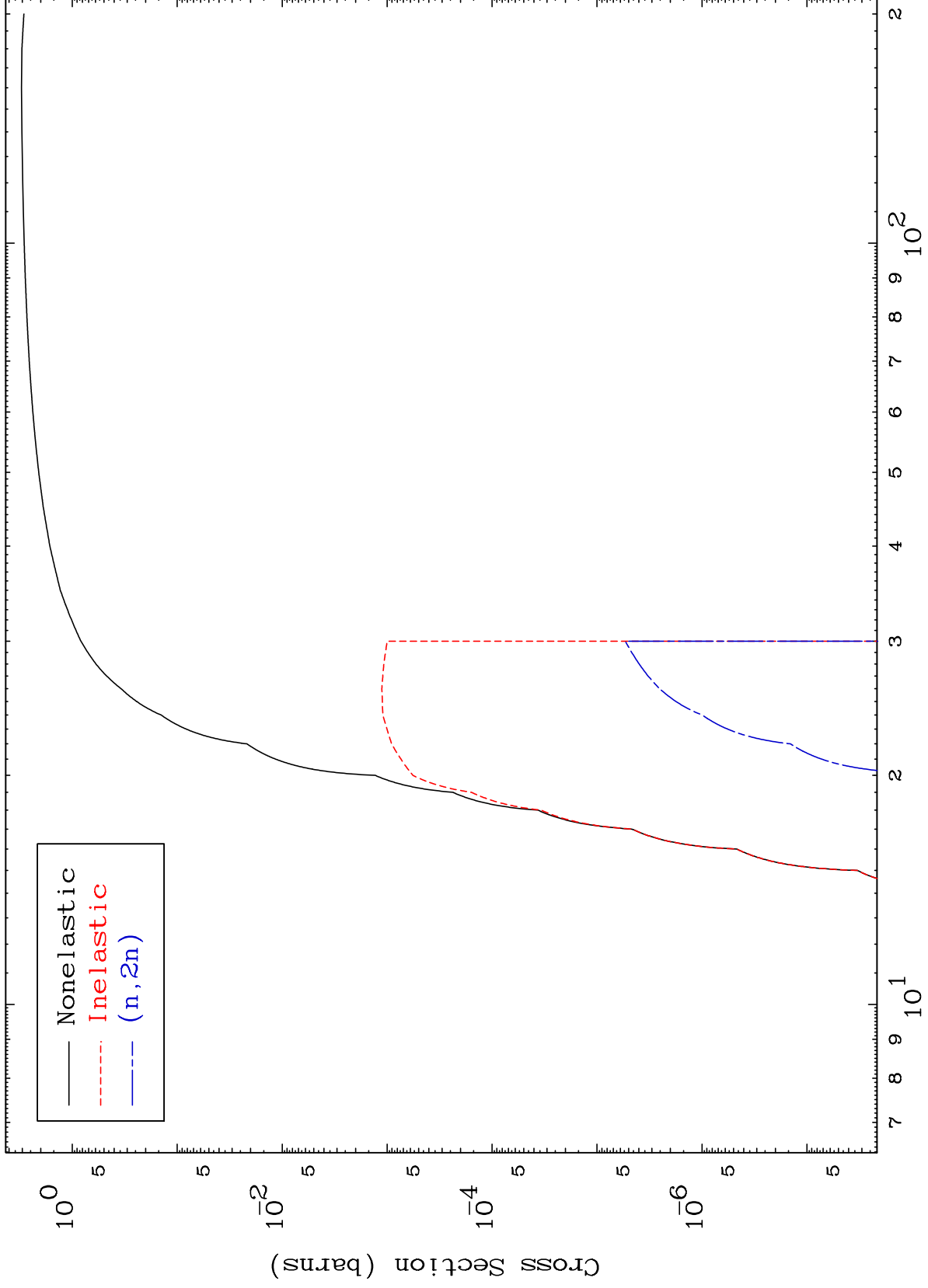
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9746

0 Kelvin Cross Sections

97-Bk-247



1

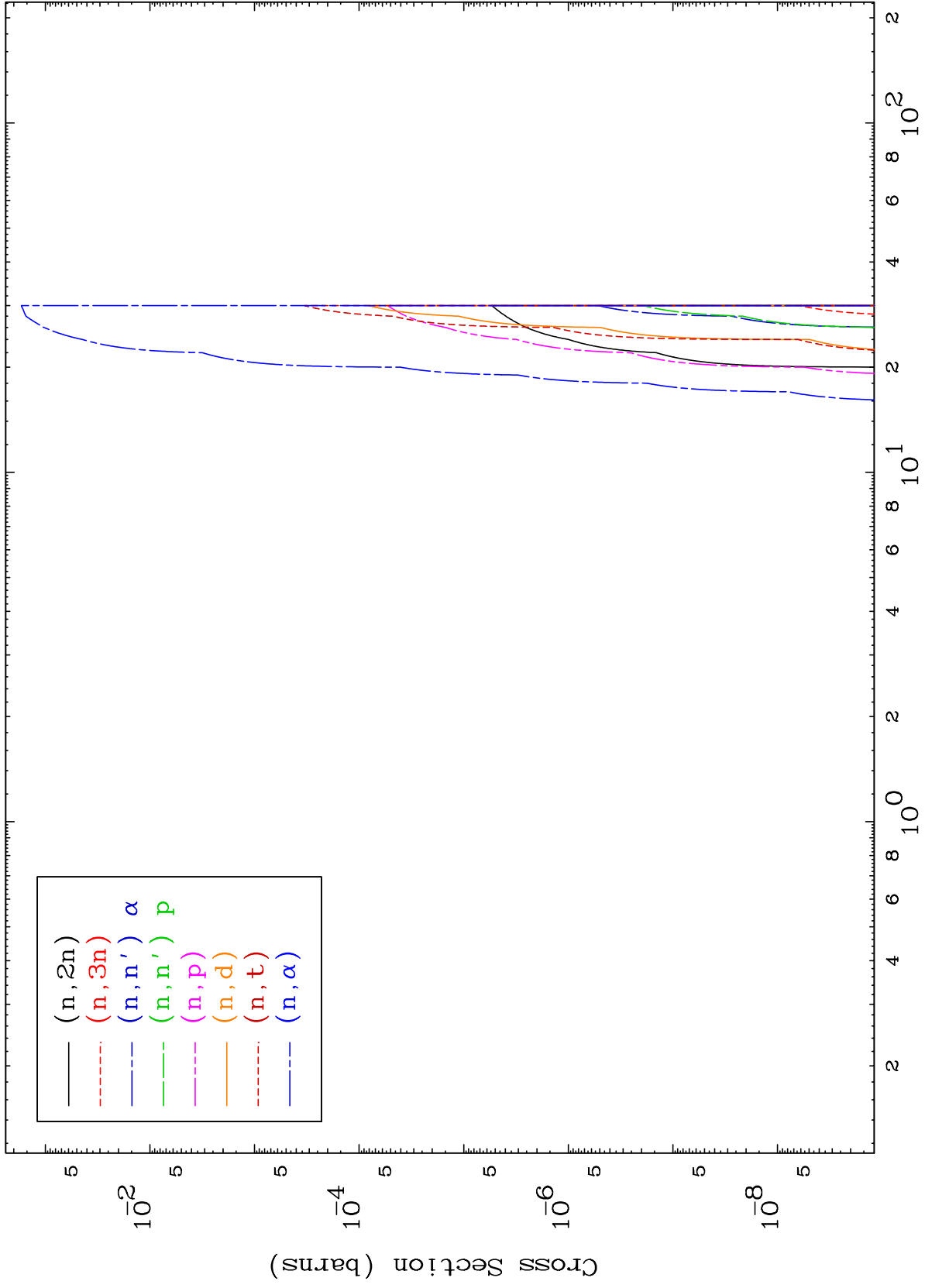
Incident Energy (MeV)

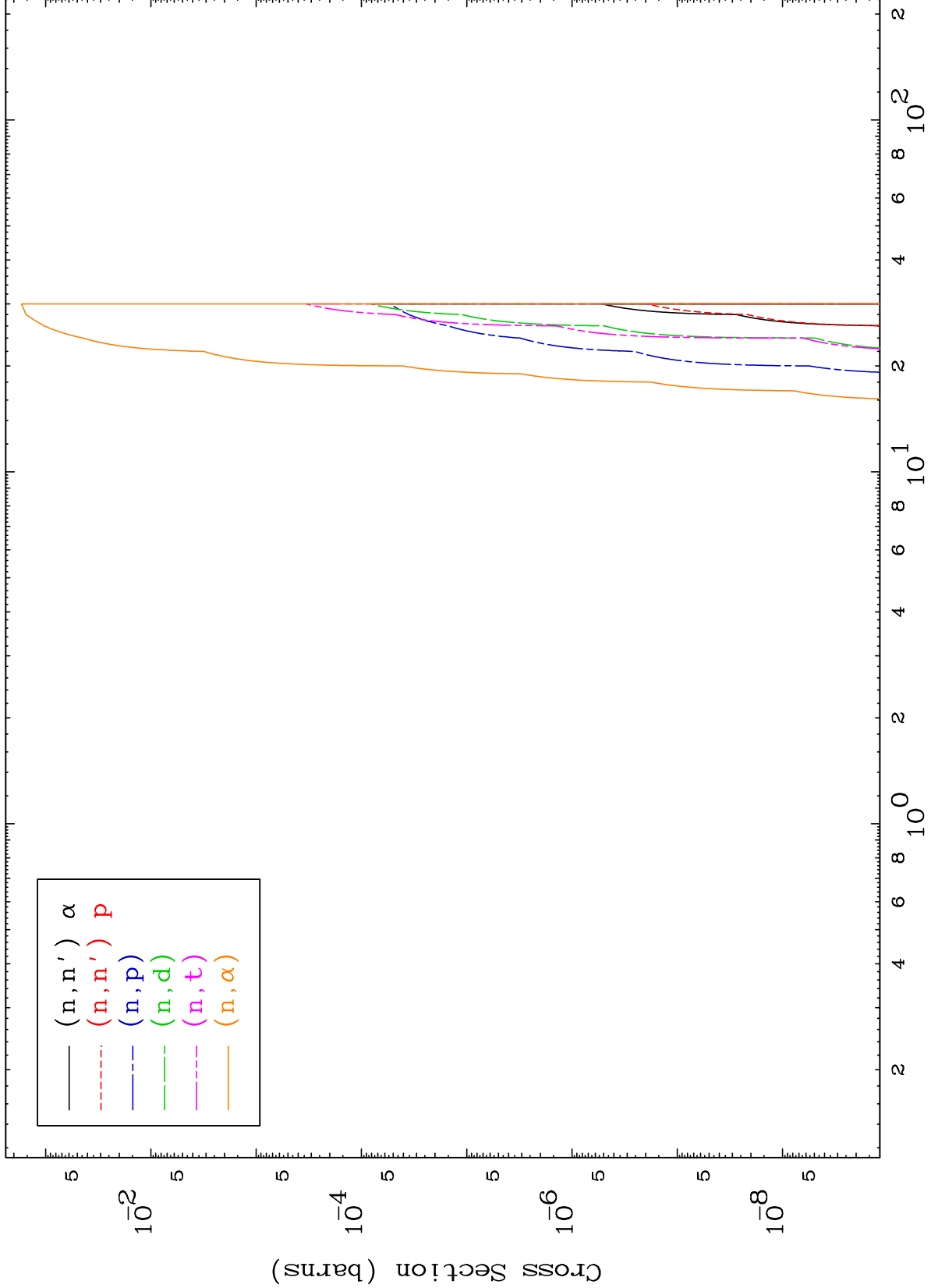
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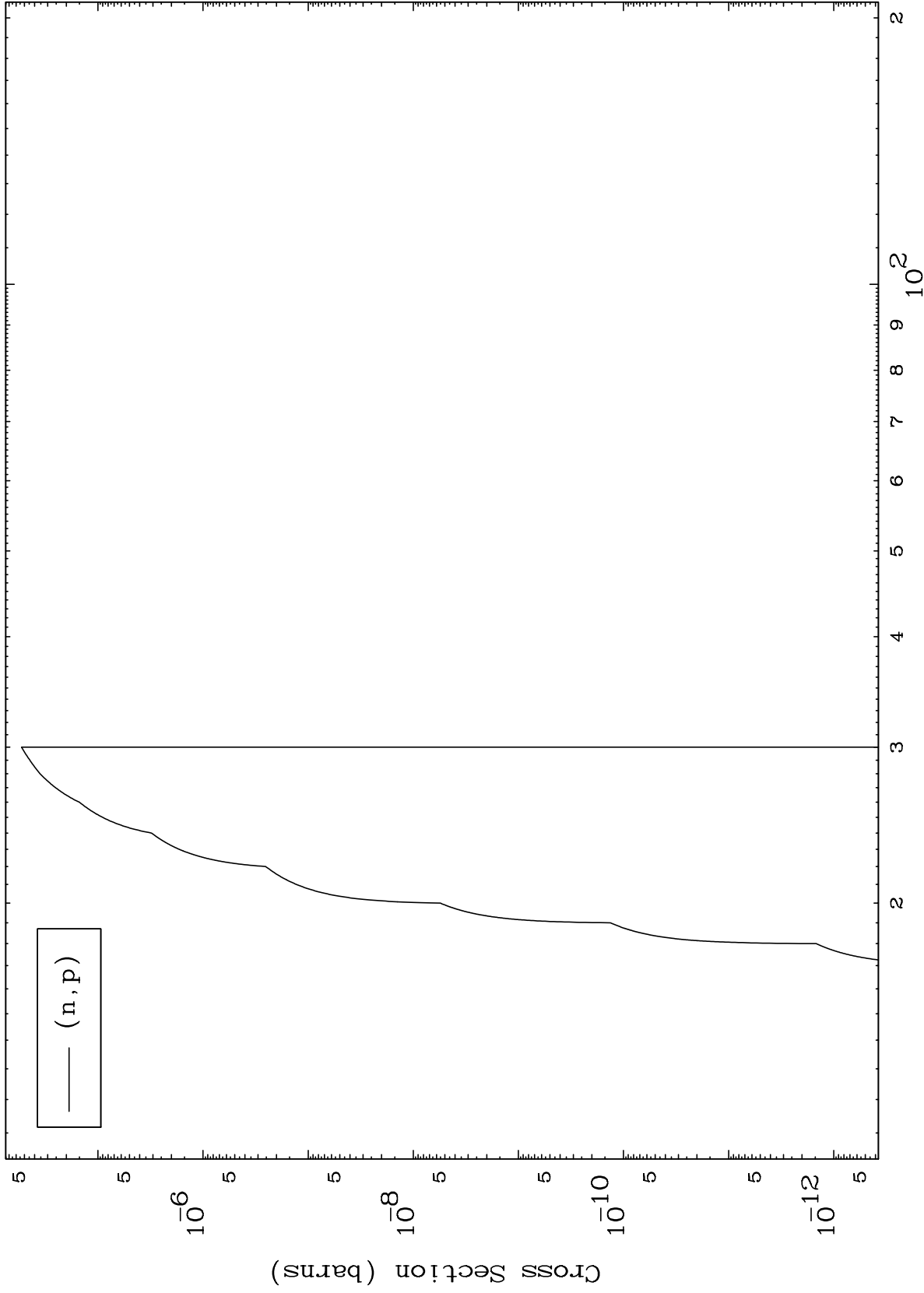
MAT 9746

α Neutron Absorption
0 Kelvin Cross Sections

97-Bk-247



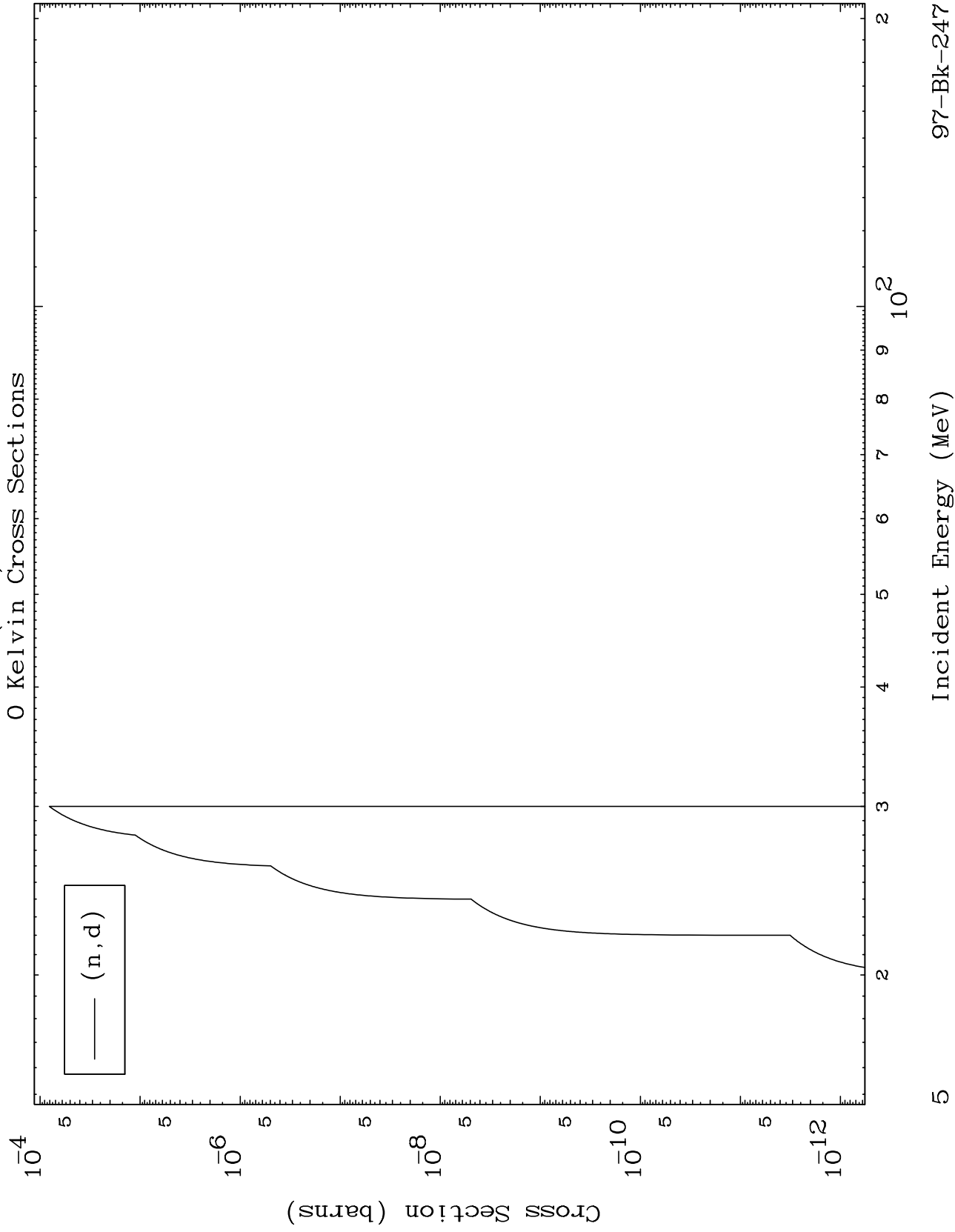




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(α, d) Levels

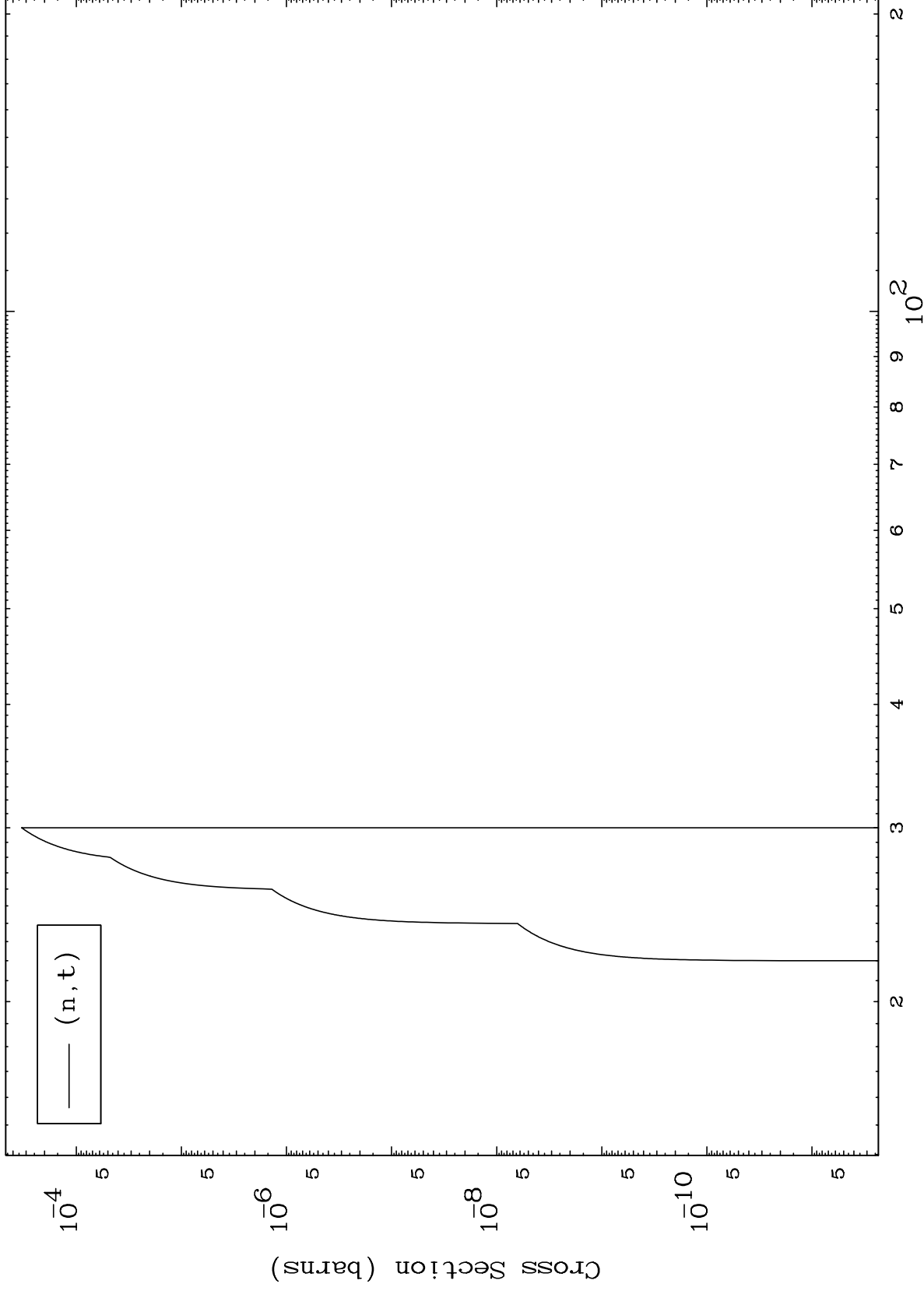
97-Bk-247



MAT 9746

(α, t) Levels
0 Kelvin Cross Sections

97-Bk-247



6

Incident Energy (MeV)

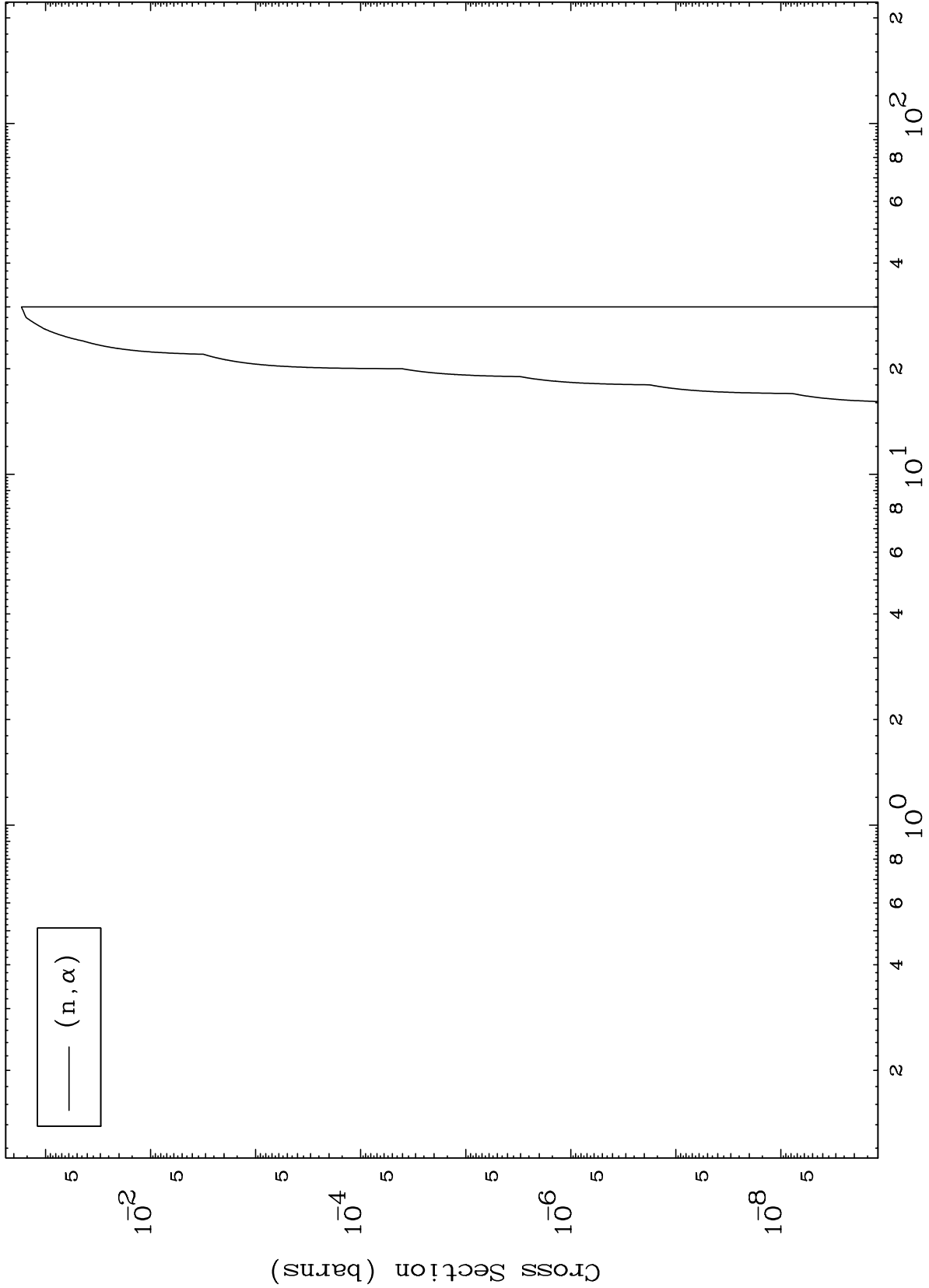
97-Bk-247

MAT 9746

(α, α) Levels

97-Bk-247

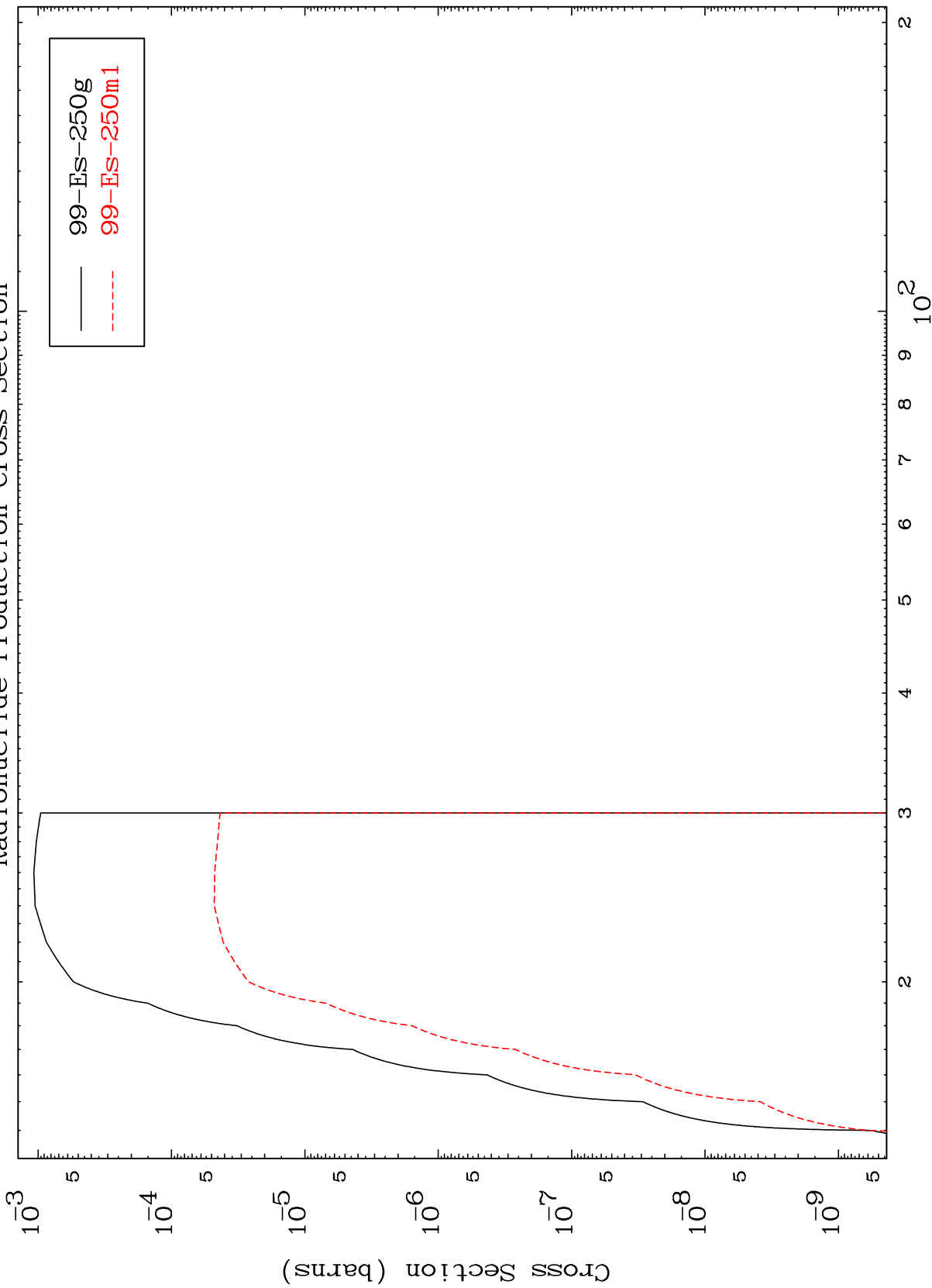
0 Kelvin Cross Sections



MAT 9746

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Inelastic
Radionuclide Production Cross Section



97-Bk-247

Incident Energy (MeV)

8

MAT 9746

Fission

97-Bk-247

Radionuclide Production Cross Section

