

Program EVALPLOT  
(Version 2021-1)

by

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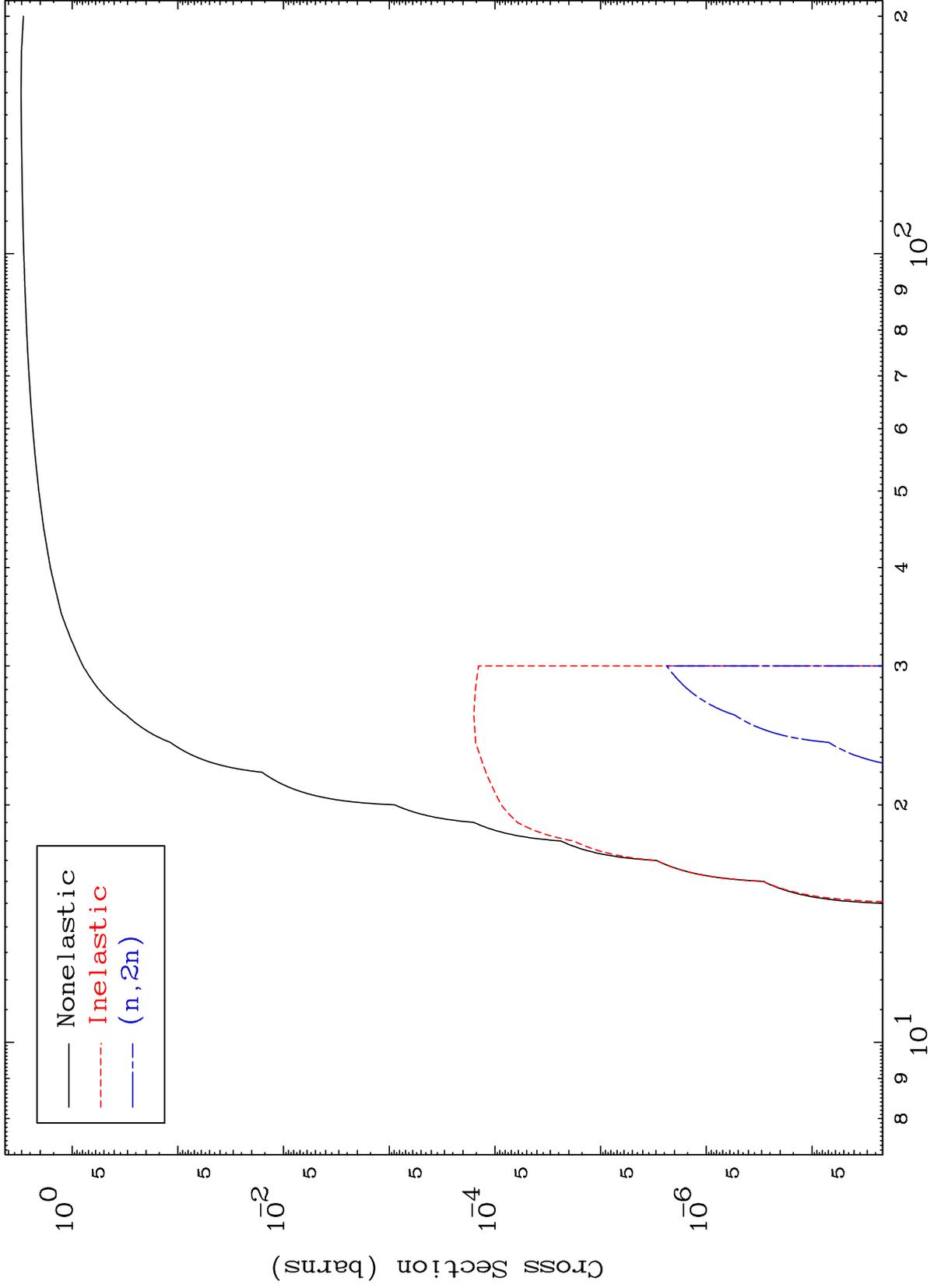
Press Mouse Button to Start

MAT 9846

$\alpha$  Major

98-Cf-247

0 Kelvin Cross Sections



1

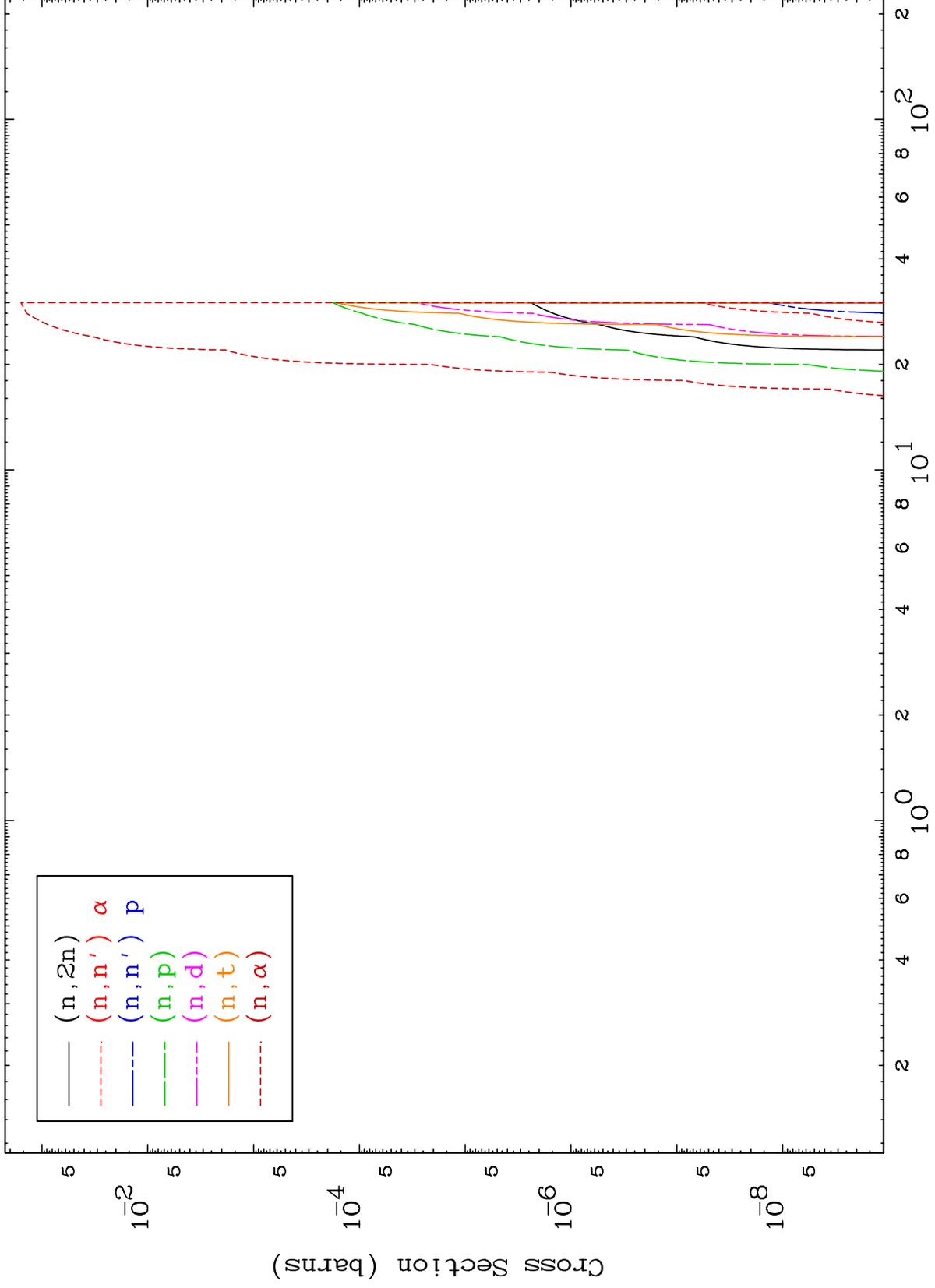
Incident Energy (MeV)

98-Cf-247

MAT 9846

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

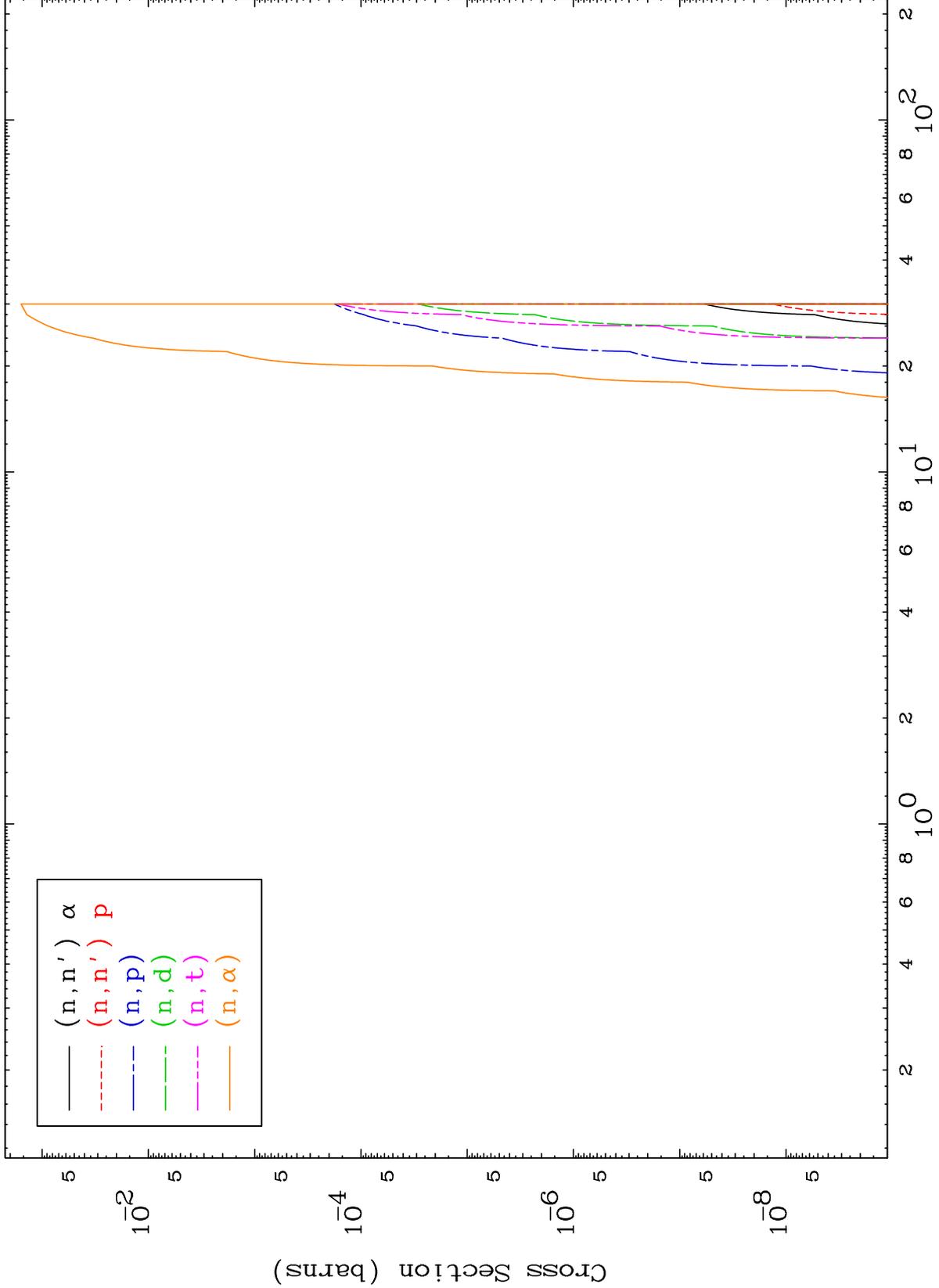
98-Cf-247

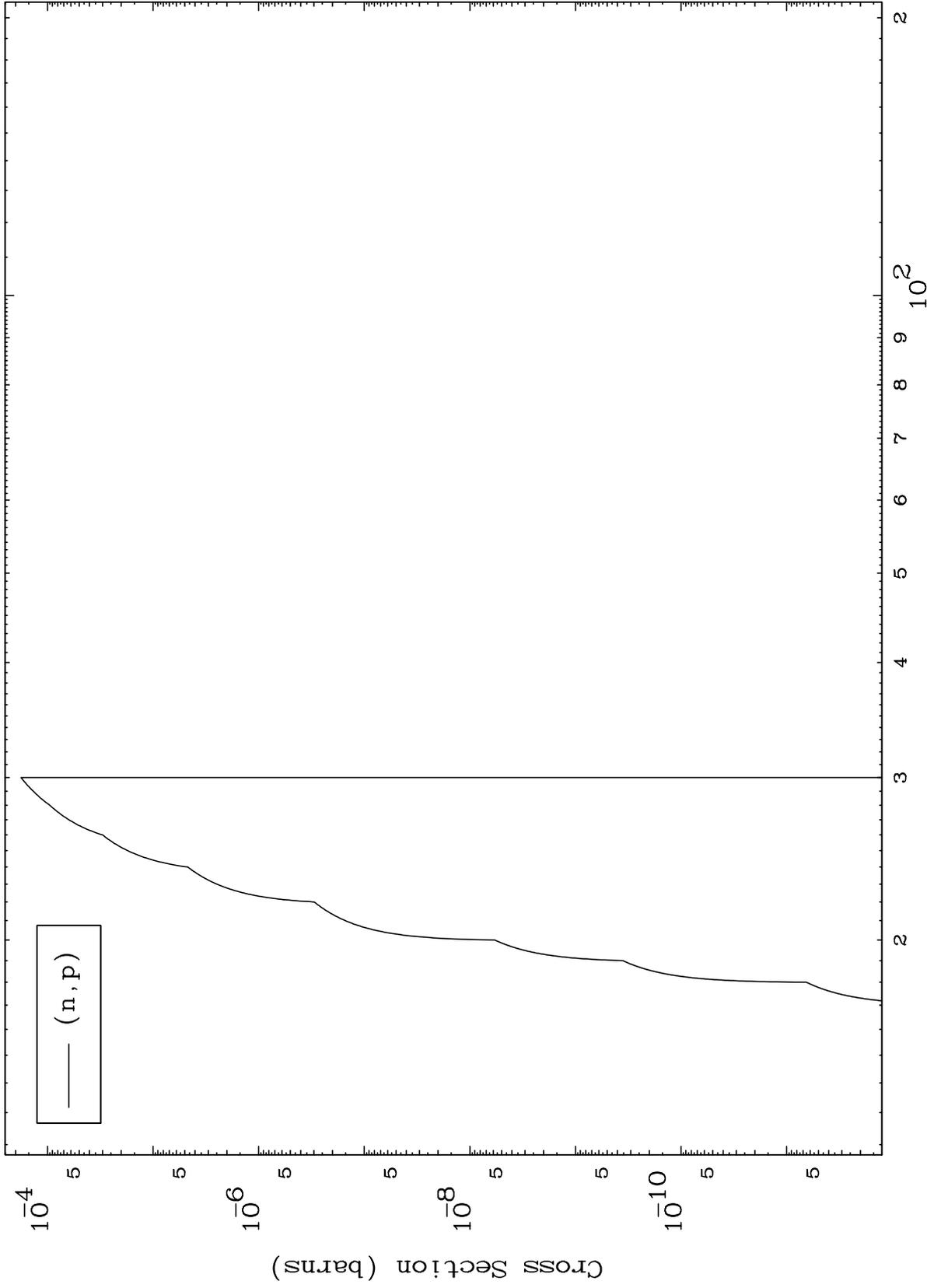


98-Cf-247

Incident Energy (MeV)

2

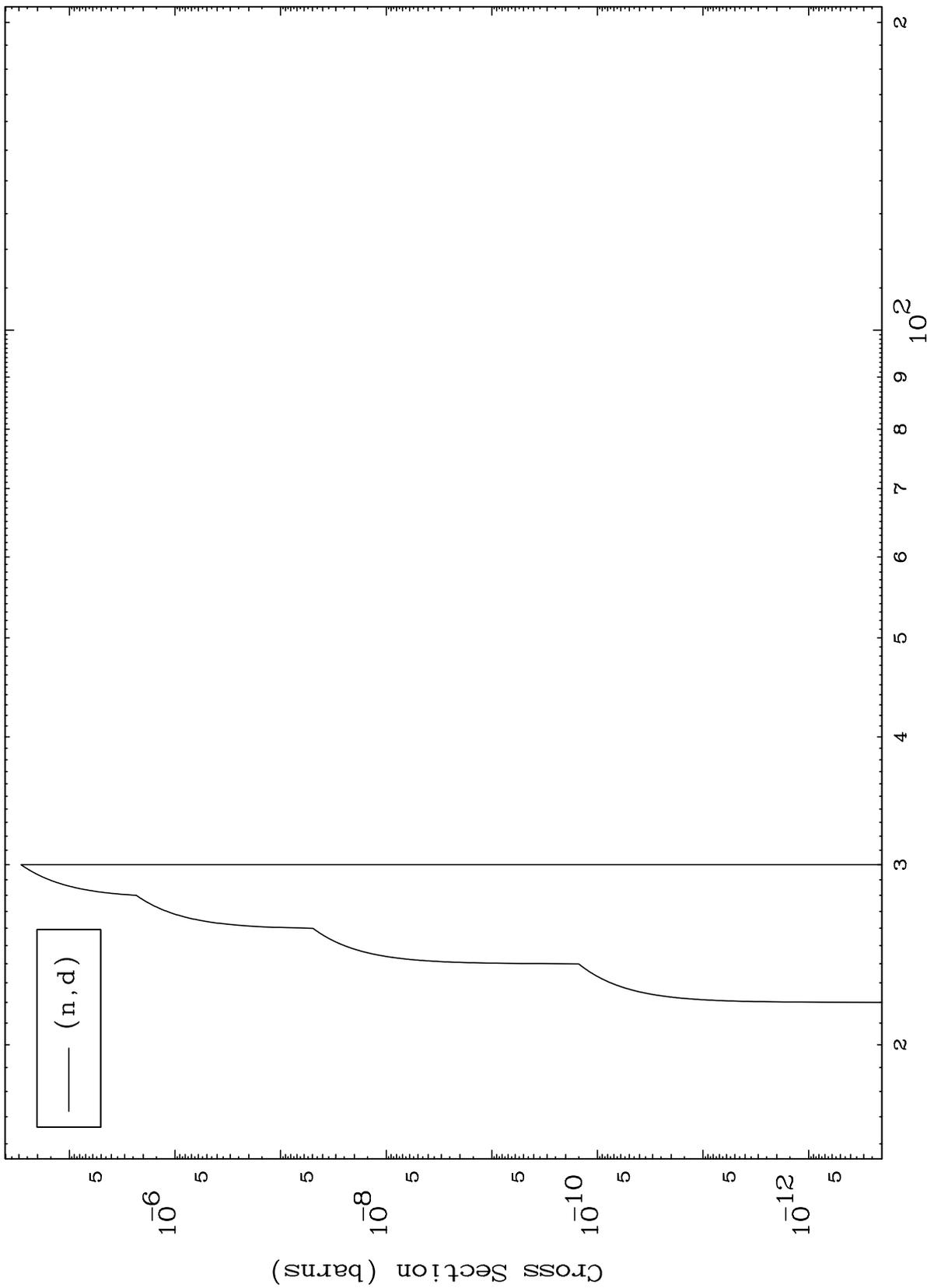




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98-Cf-247

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



98-Cf-247

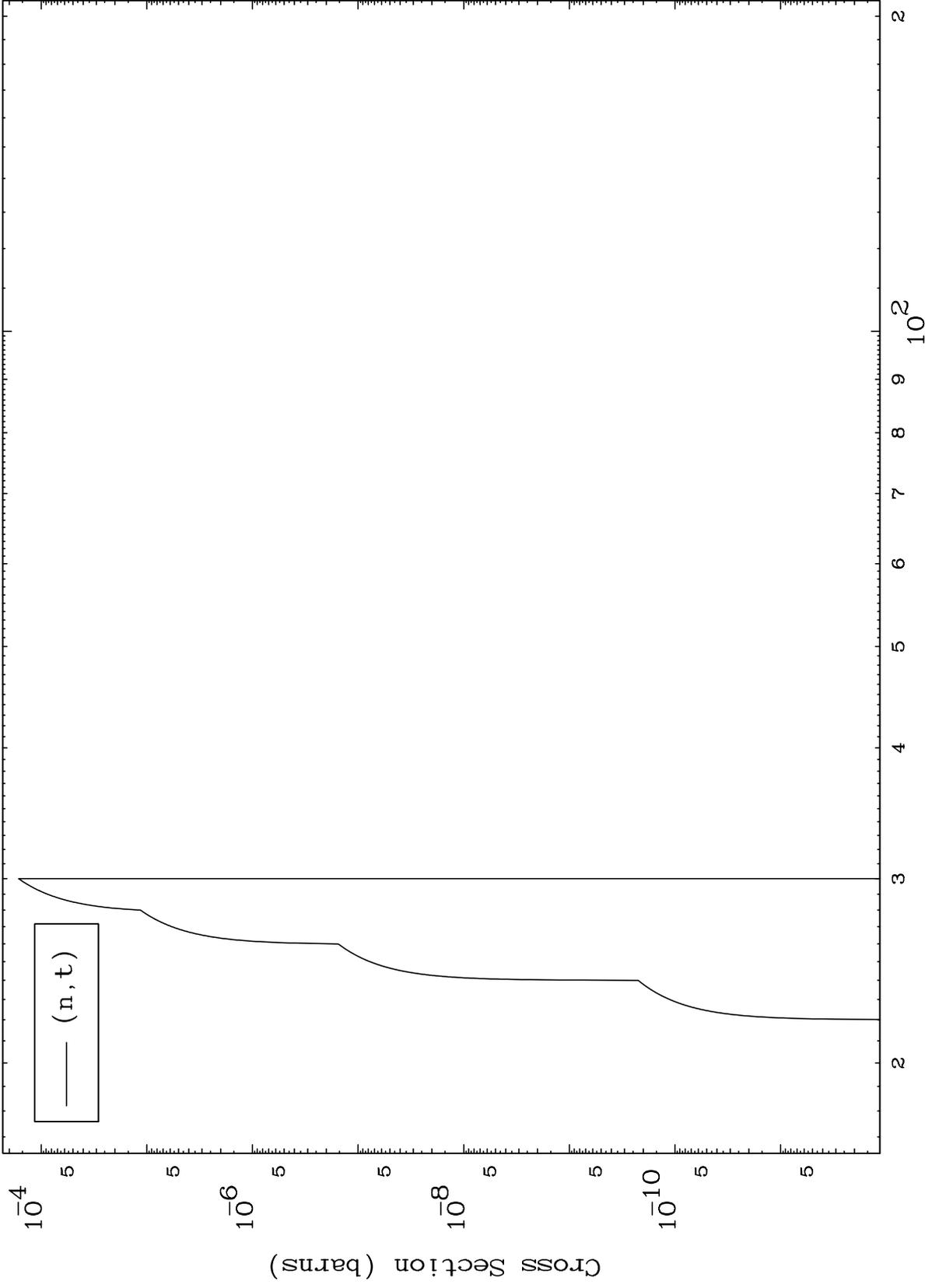
Incident Energy (MeV)

5

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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

98-Cf-247



6

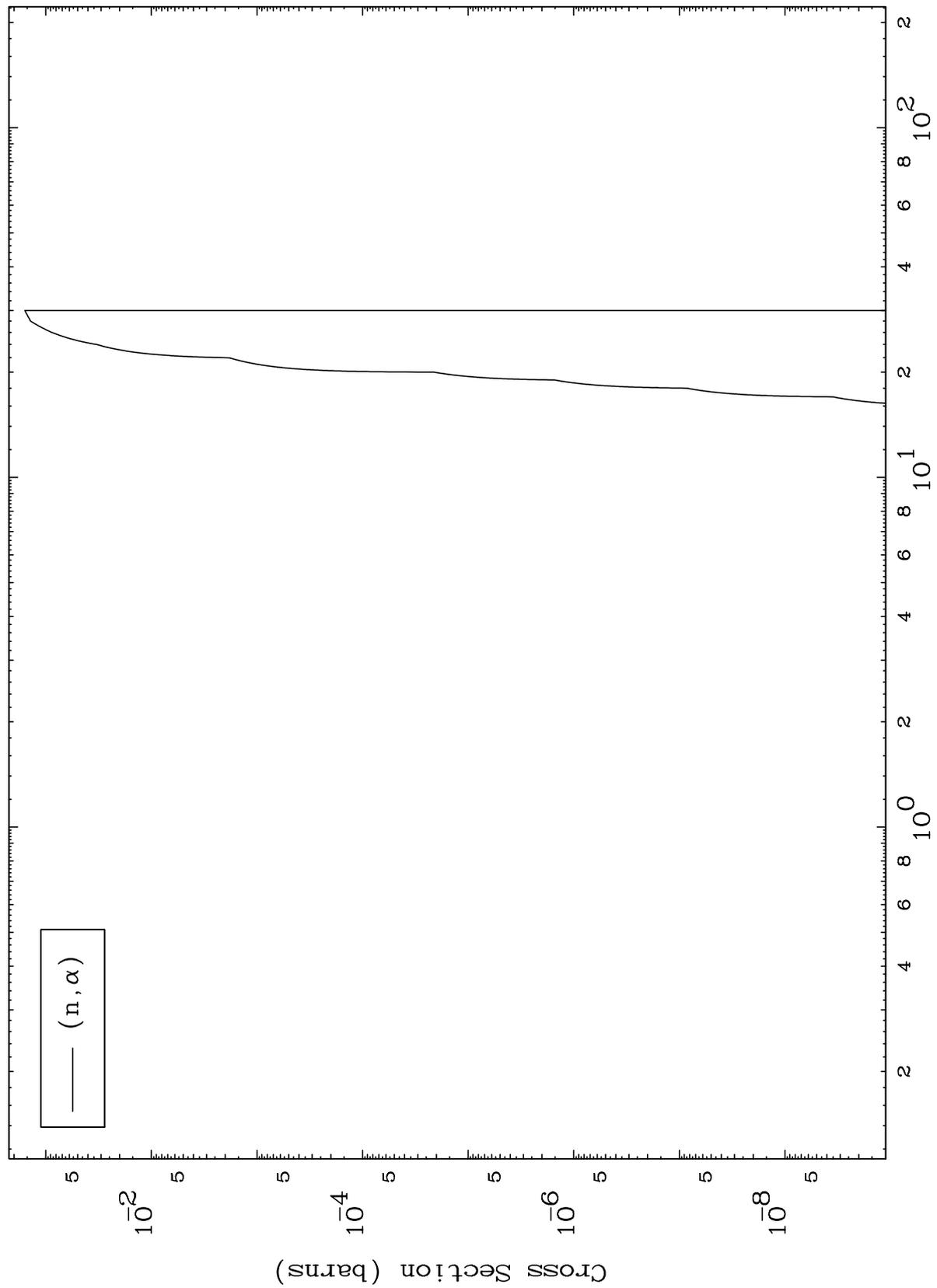
Incident Energy (MeV)

98-Cf-247

MAT 9846

98-Cf-247

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



98-Cf-247

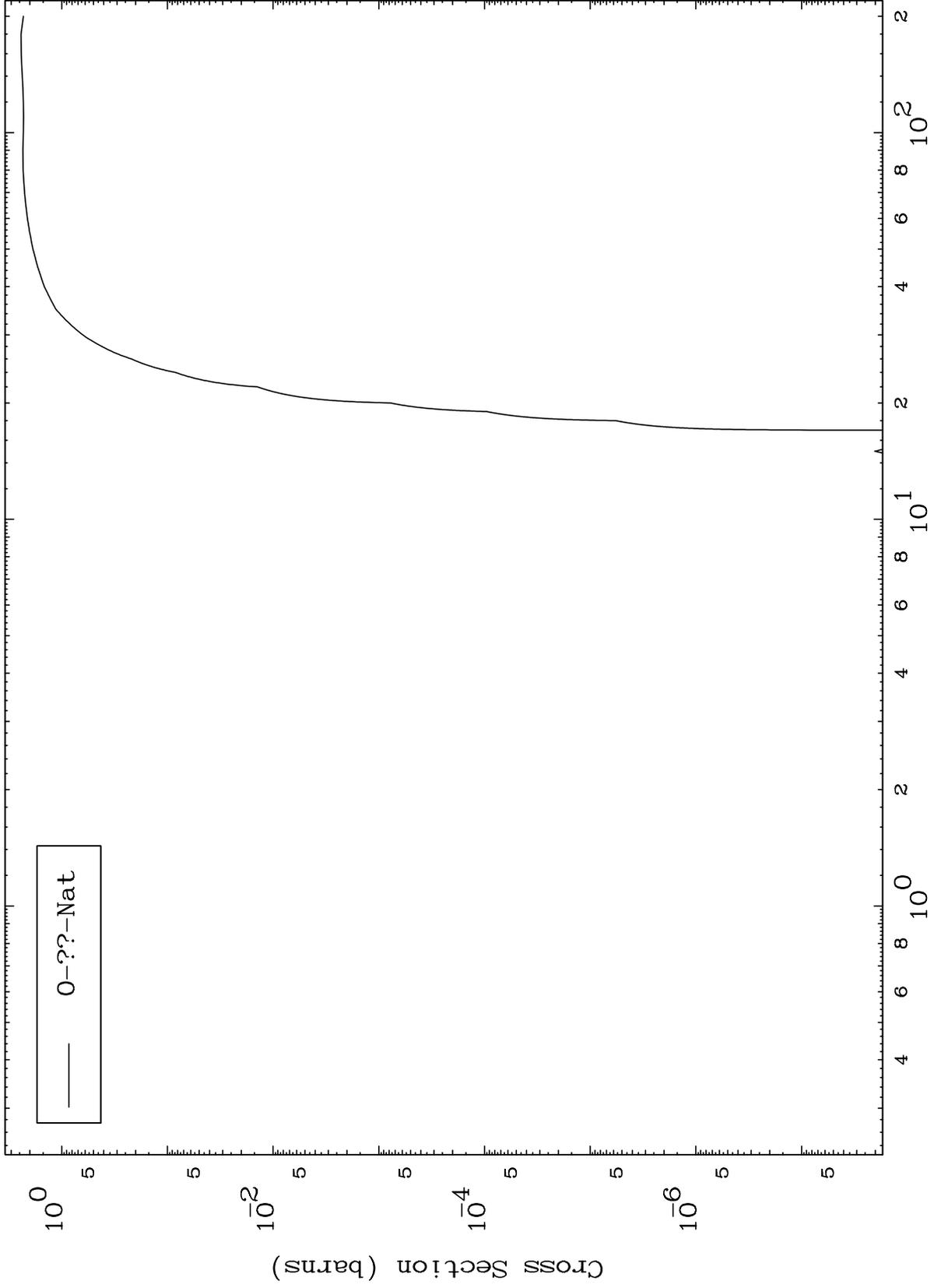
Incident Energy (MeV)

MAT 9846

Fission

98-Cf-247

Radionuclide Production Cross Section

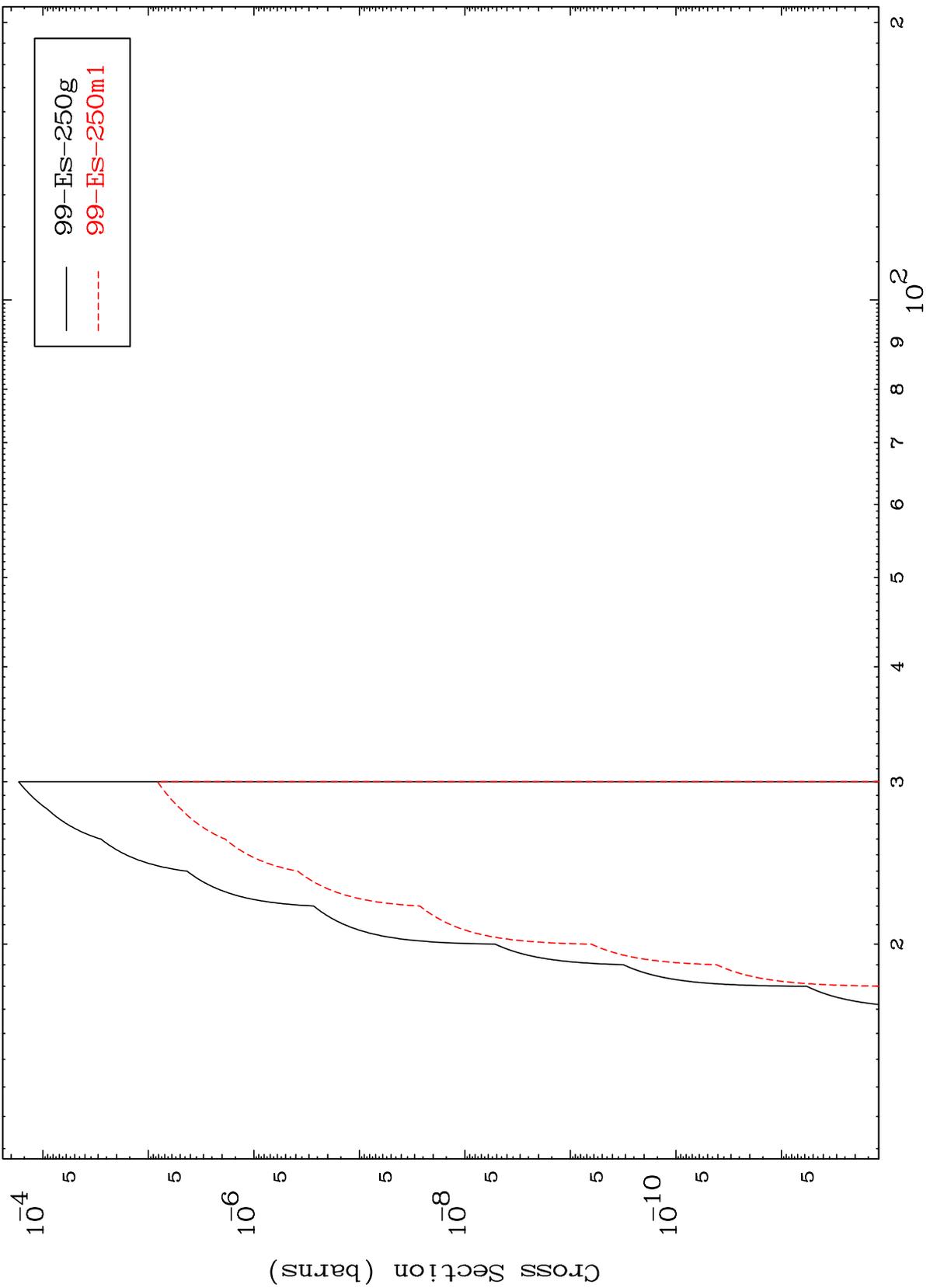


0-??-Nat

MAT 9846

98-Cf-247

(n,p)  
Radionuclide Production Cross Section



9

Incident Energy (MeV)

98-Cf-247