

Program EVALPLOT  
(Version 2021-1)

by

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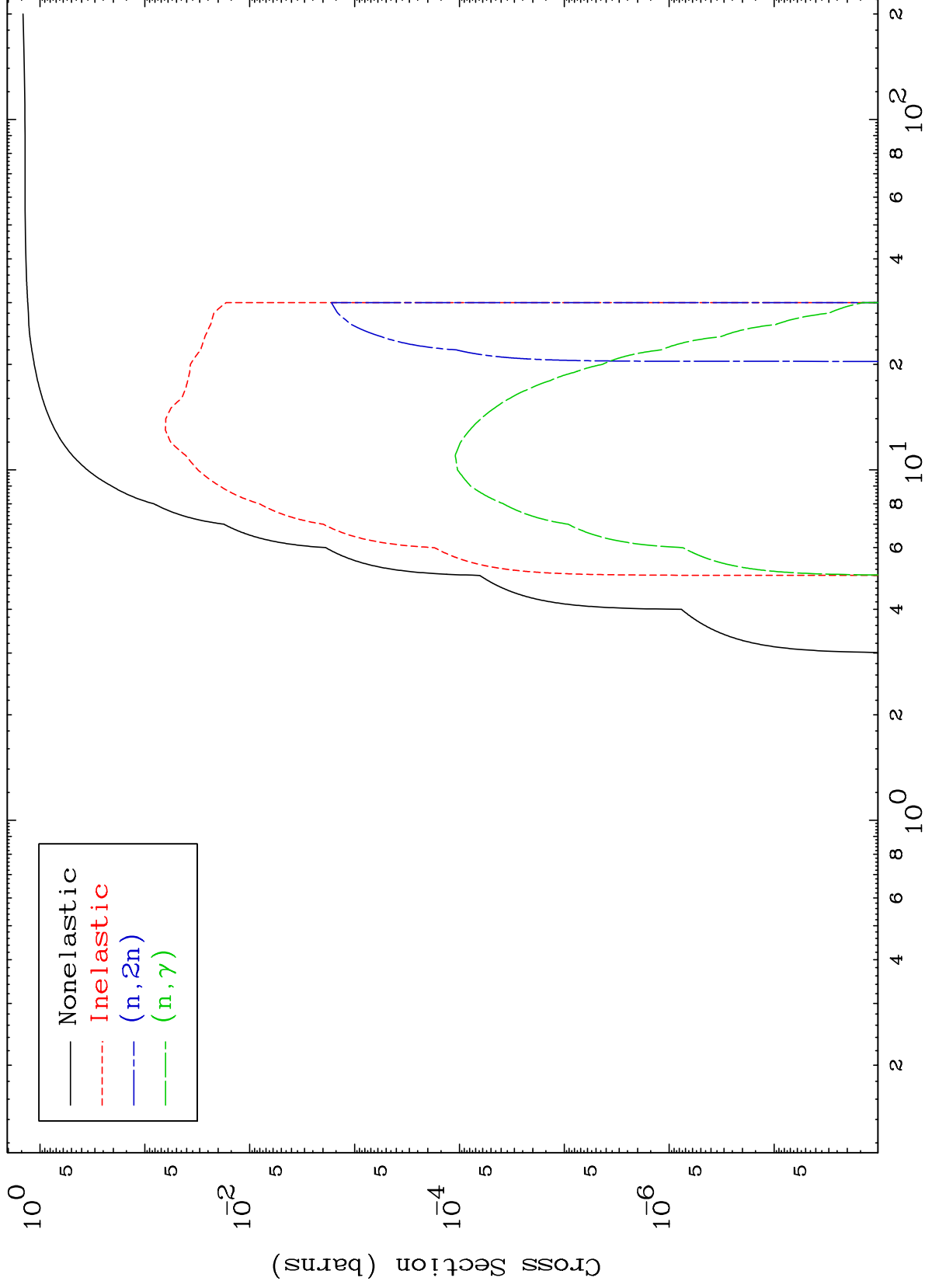
Press Mouse Button to Start

MAT 2422

$\alpha$  Major

24-Cr-49

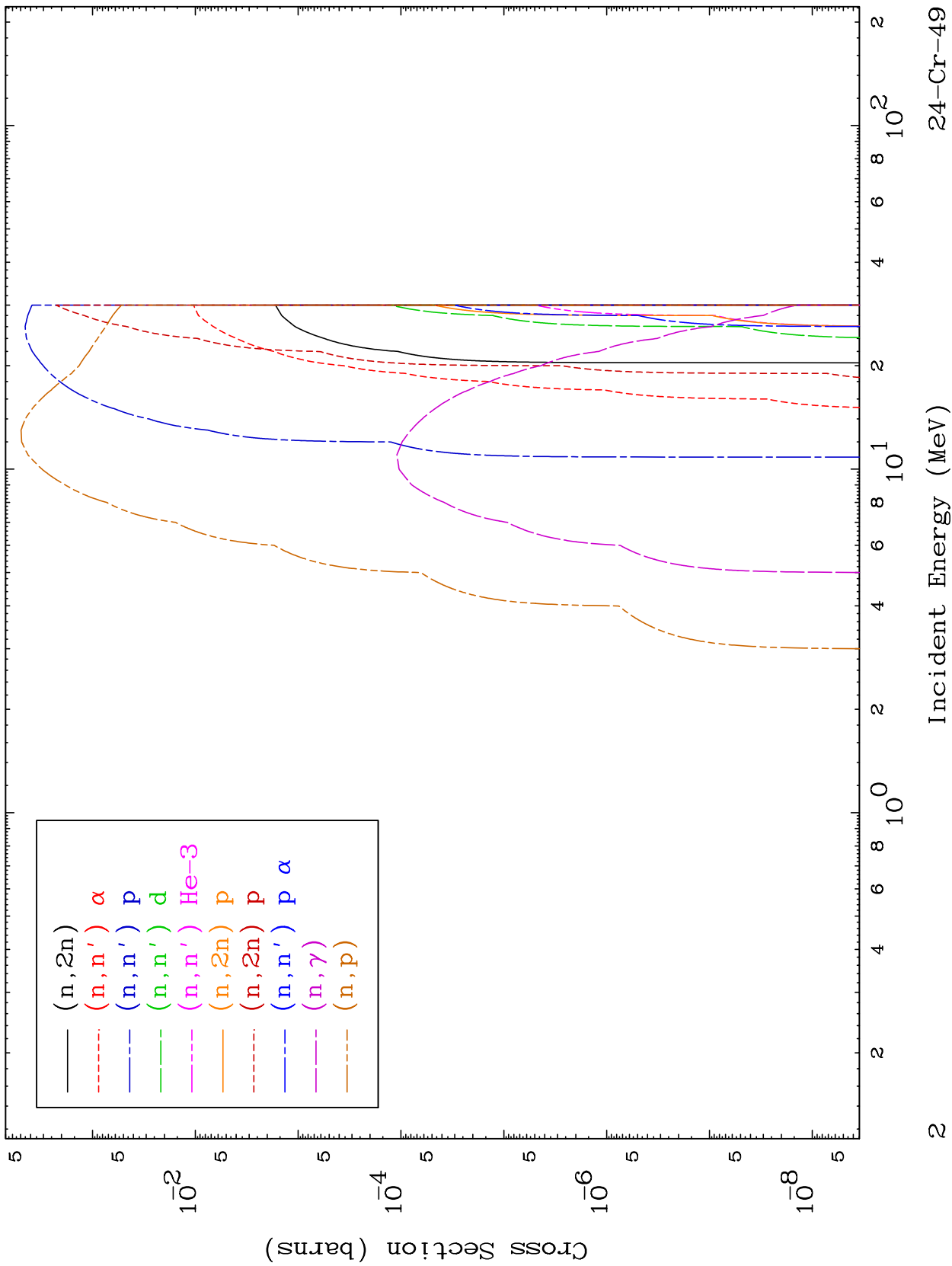
0 Kelvin Cross Sections



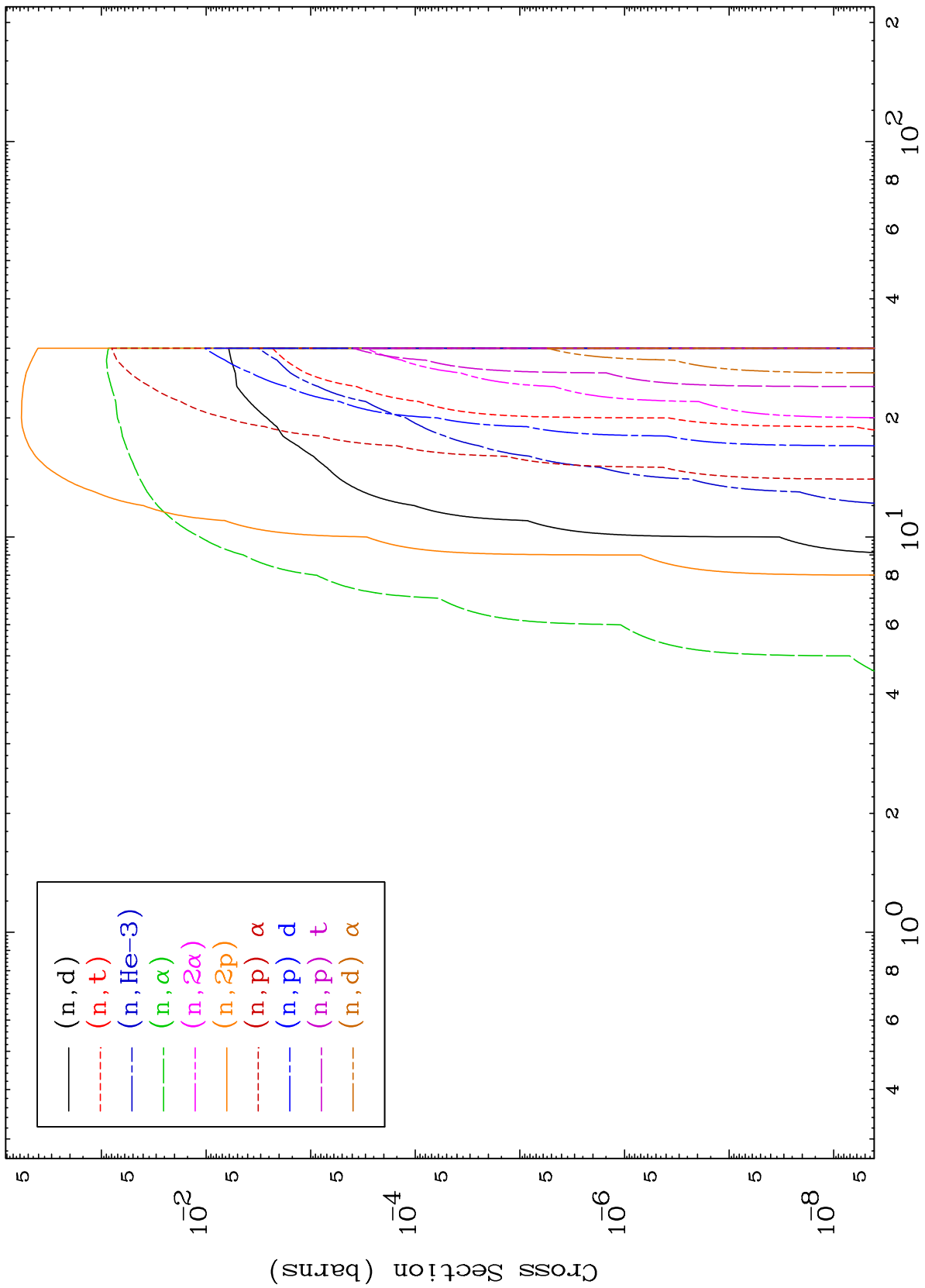
1

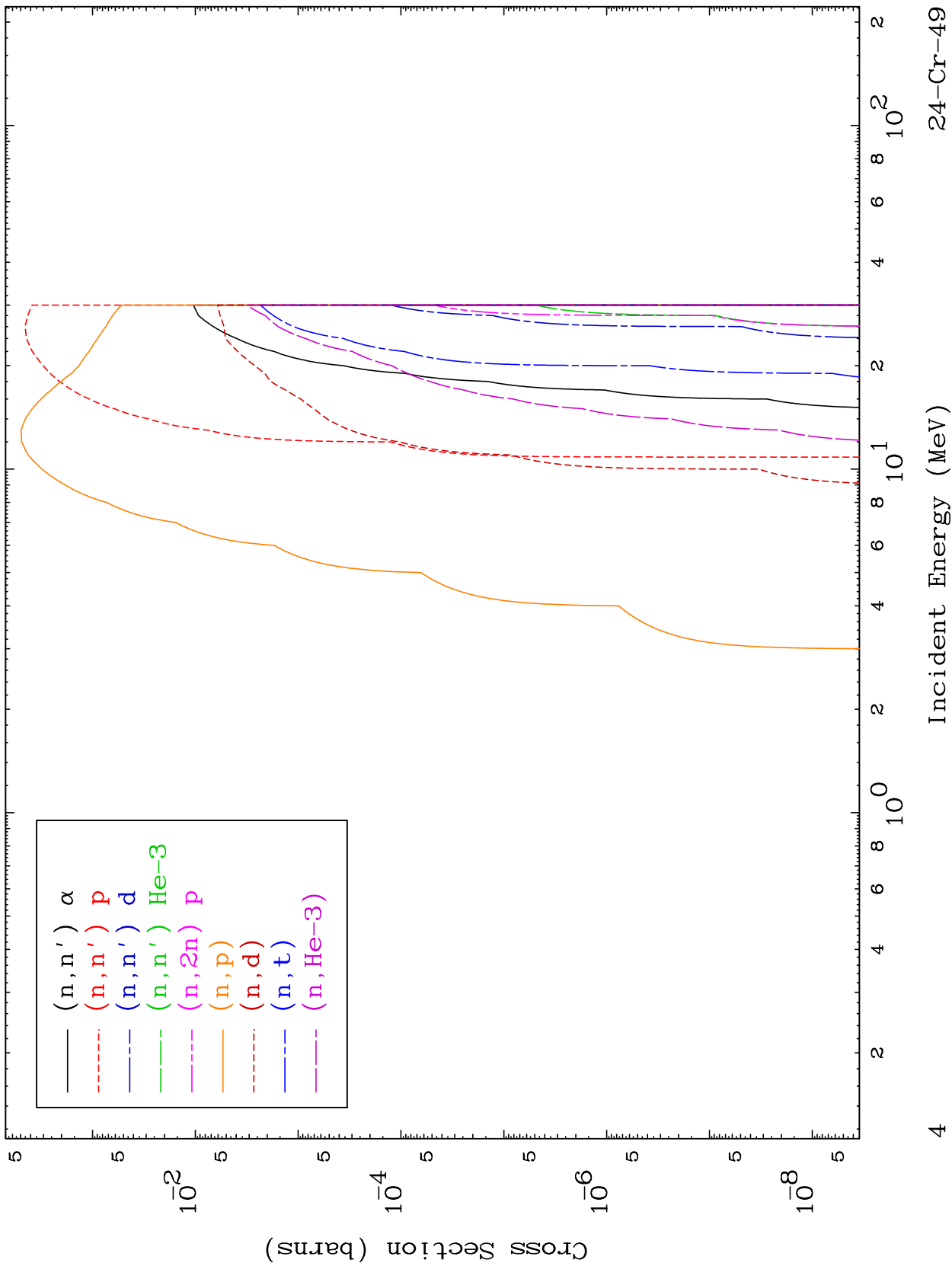
Incident Energy (MeV)

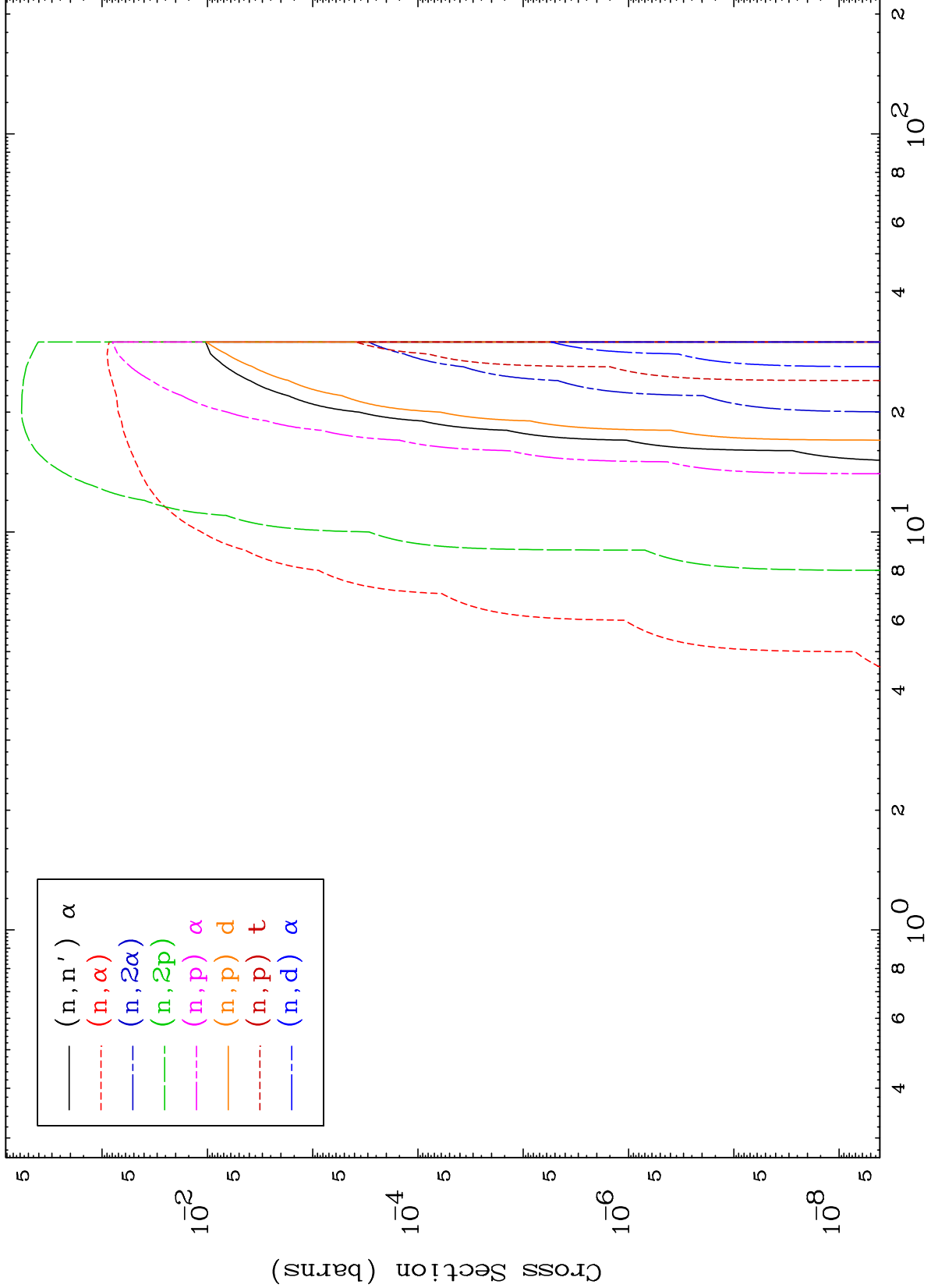
24-Cr-49



$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections





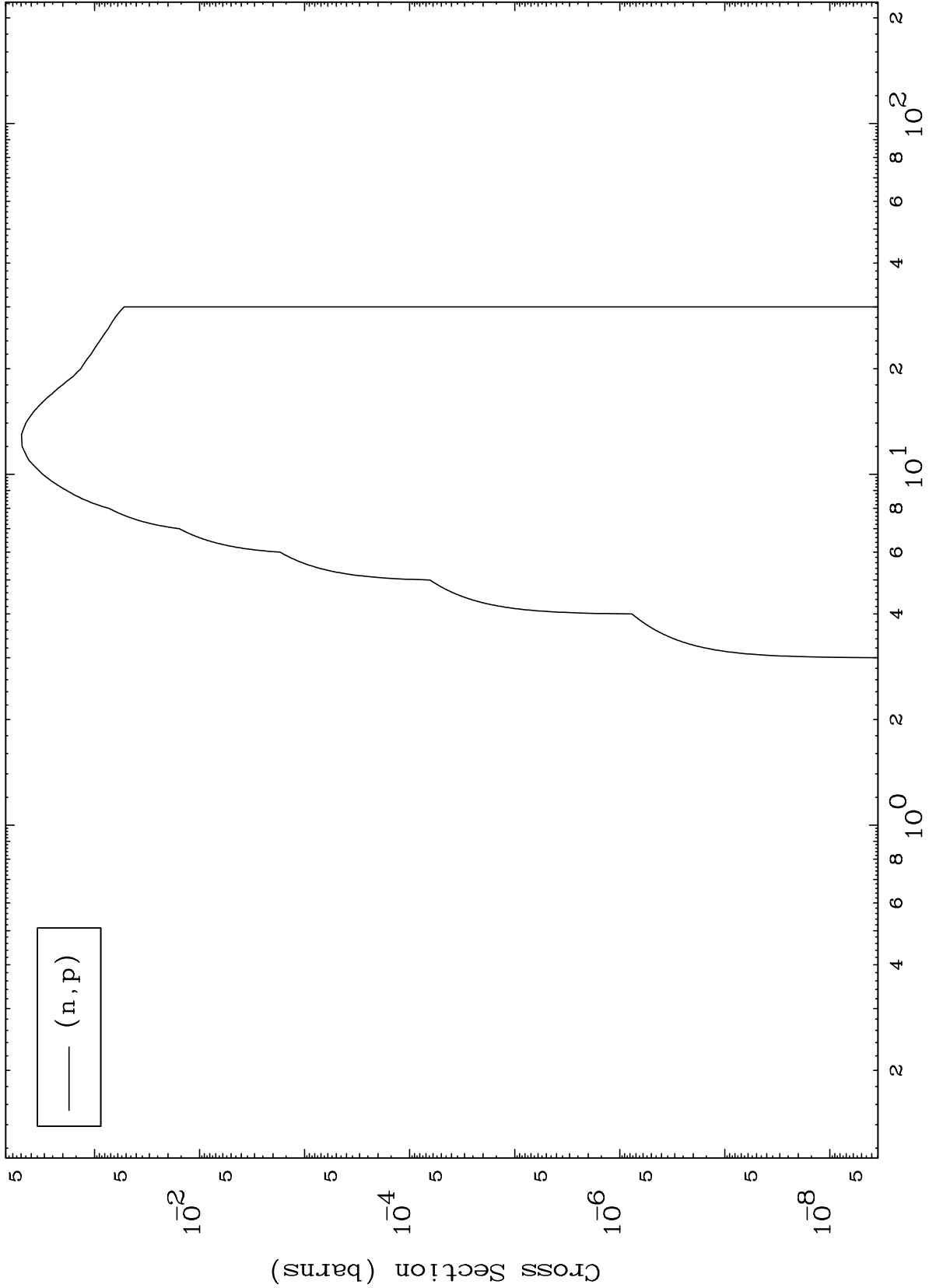


MAT 2422

( $\alpha, p$ ) Levels

24-Cr-49

0 Kelvin Cross Sections

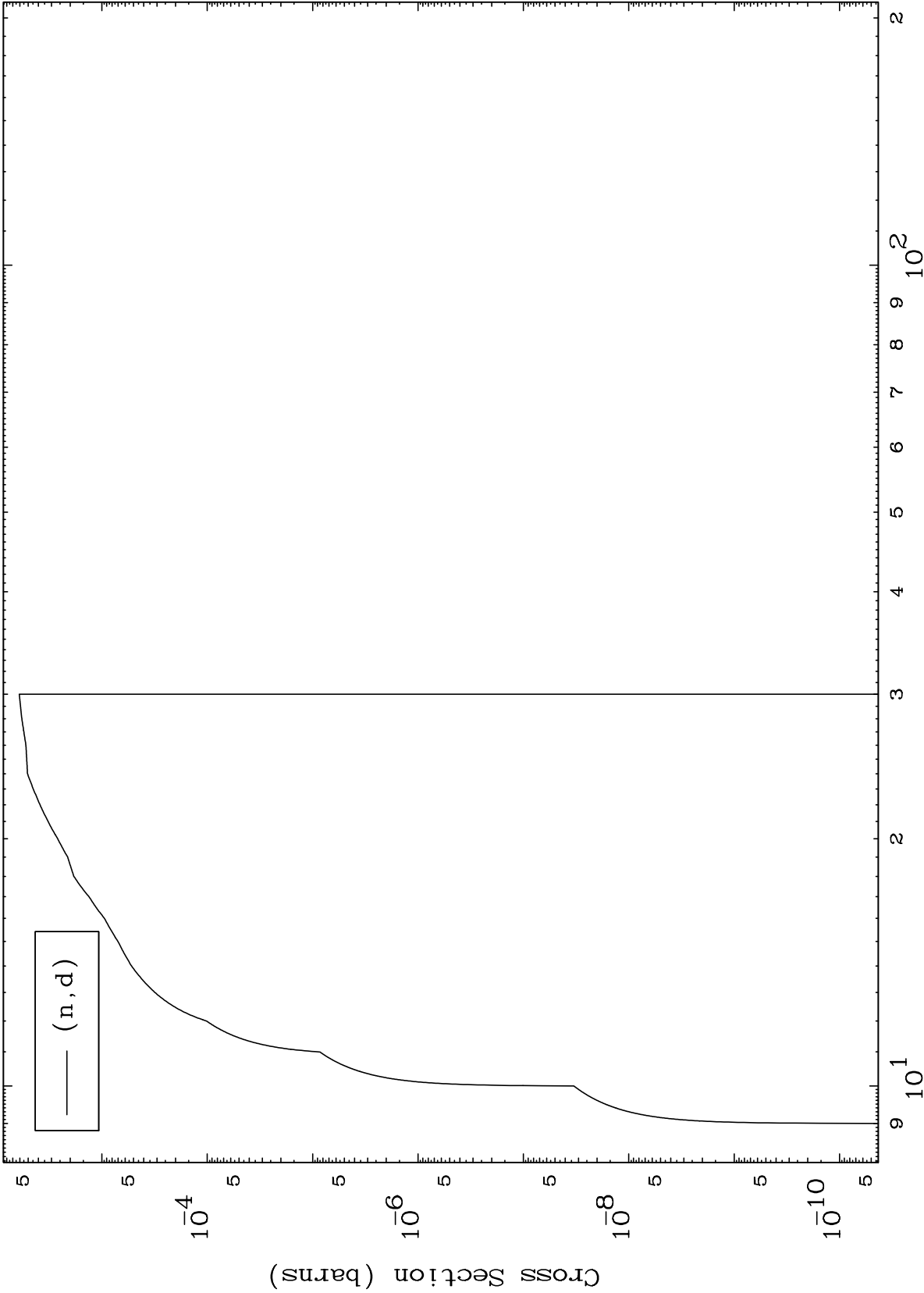


MAT 2422

( $\alpha, d$ ) Levels

24-Cr-49

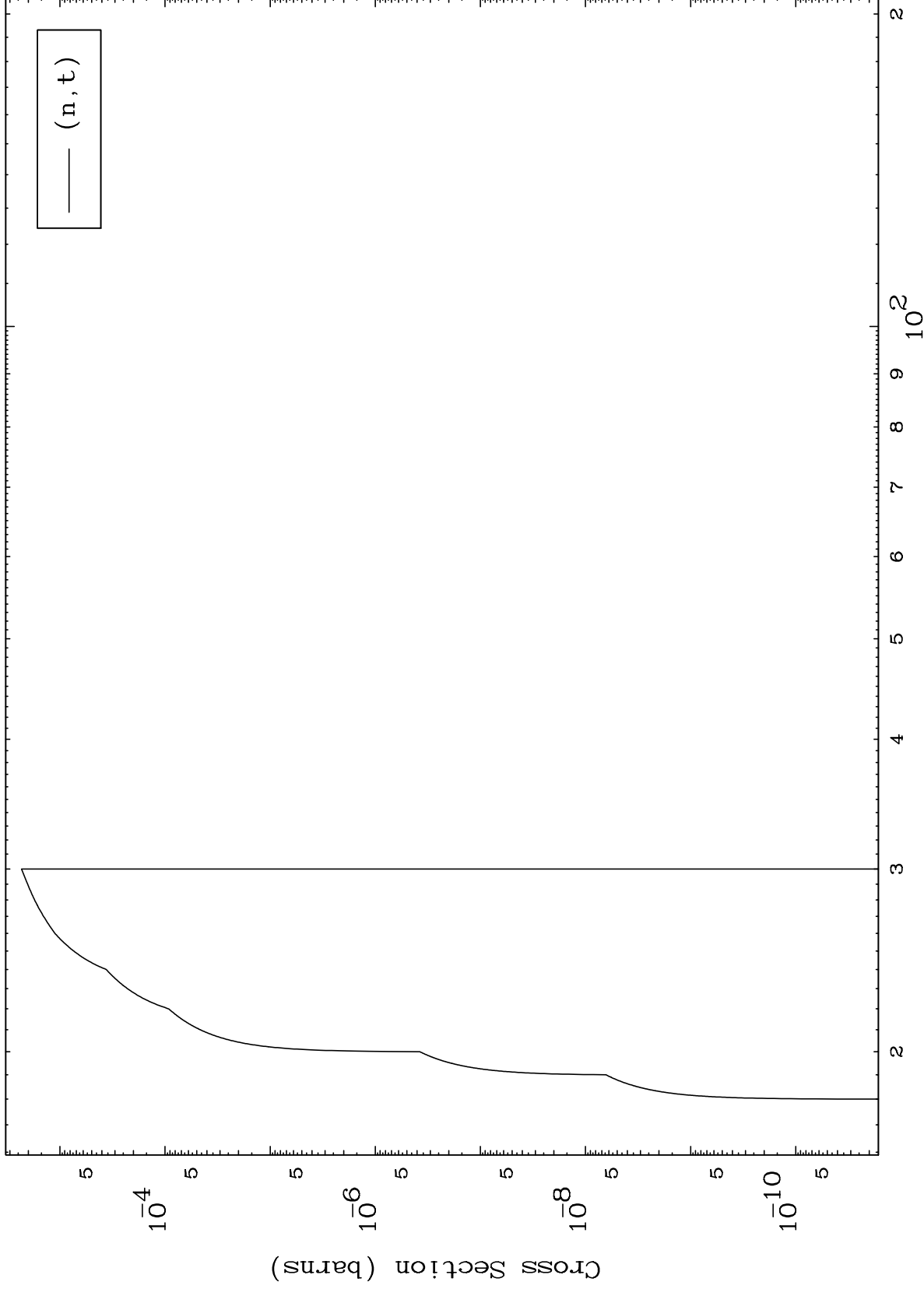
0 Kelvin Cross Sections



Incident Energy (MeV)

24-Cr-49

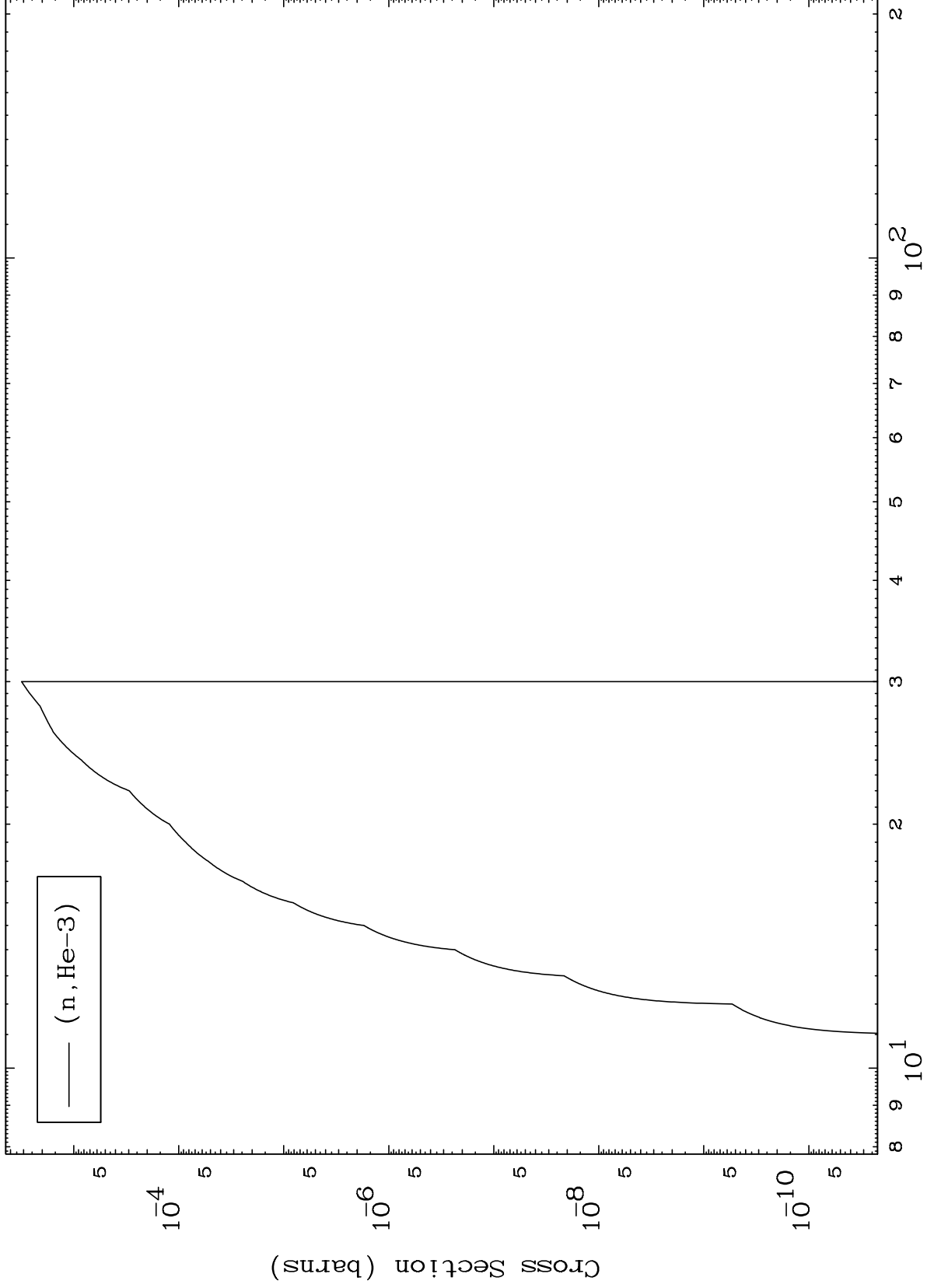
7



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( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

$^{24}\text{Cr-49}$



9

Incident Energy (MeV)

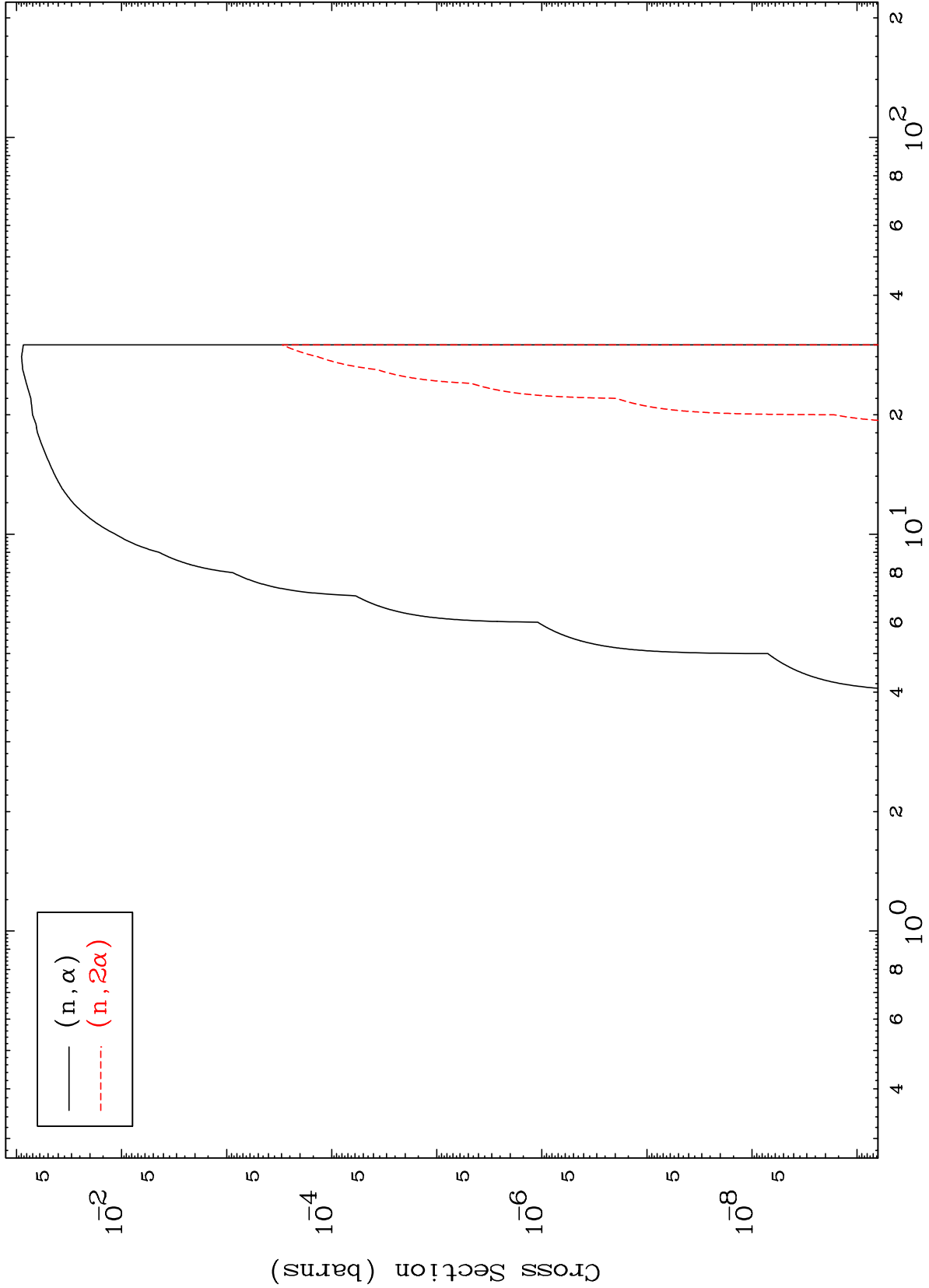
$^{24}\text{Cr-49}$

MAT 2422

( $\alpha, \alpha$ ) Levels

<sup>24</sup>Cr-49

0 Kelvin Cross Sections



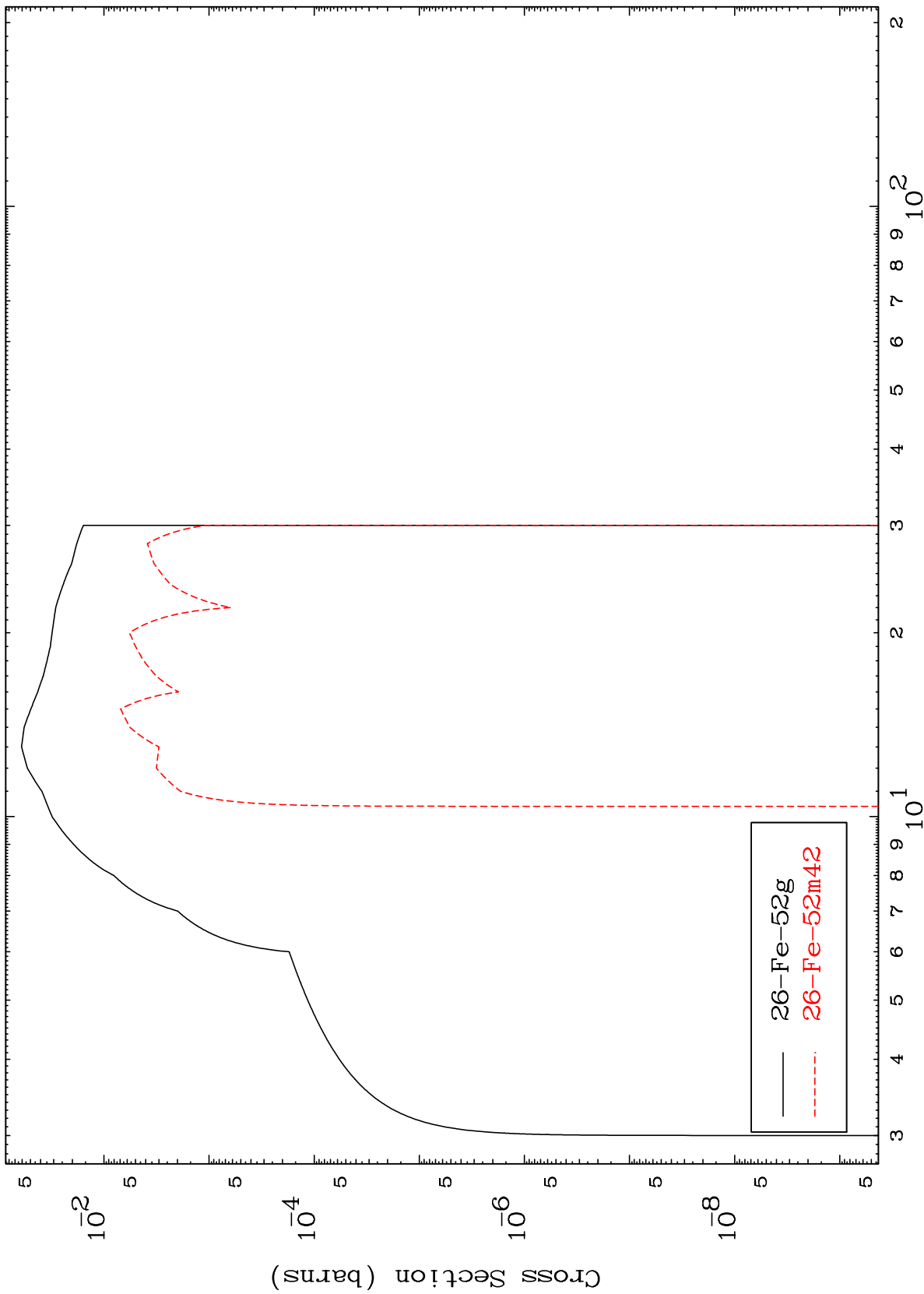
— (n,  $\alpha$ )  
- - - (n, 2 $\alpha$ )

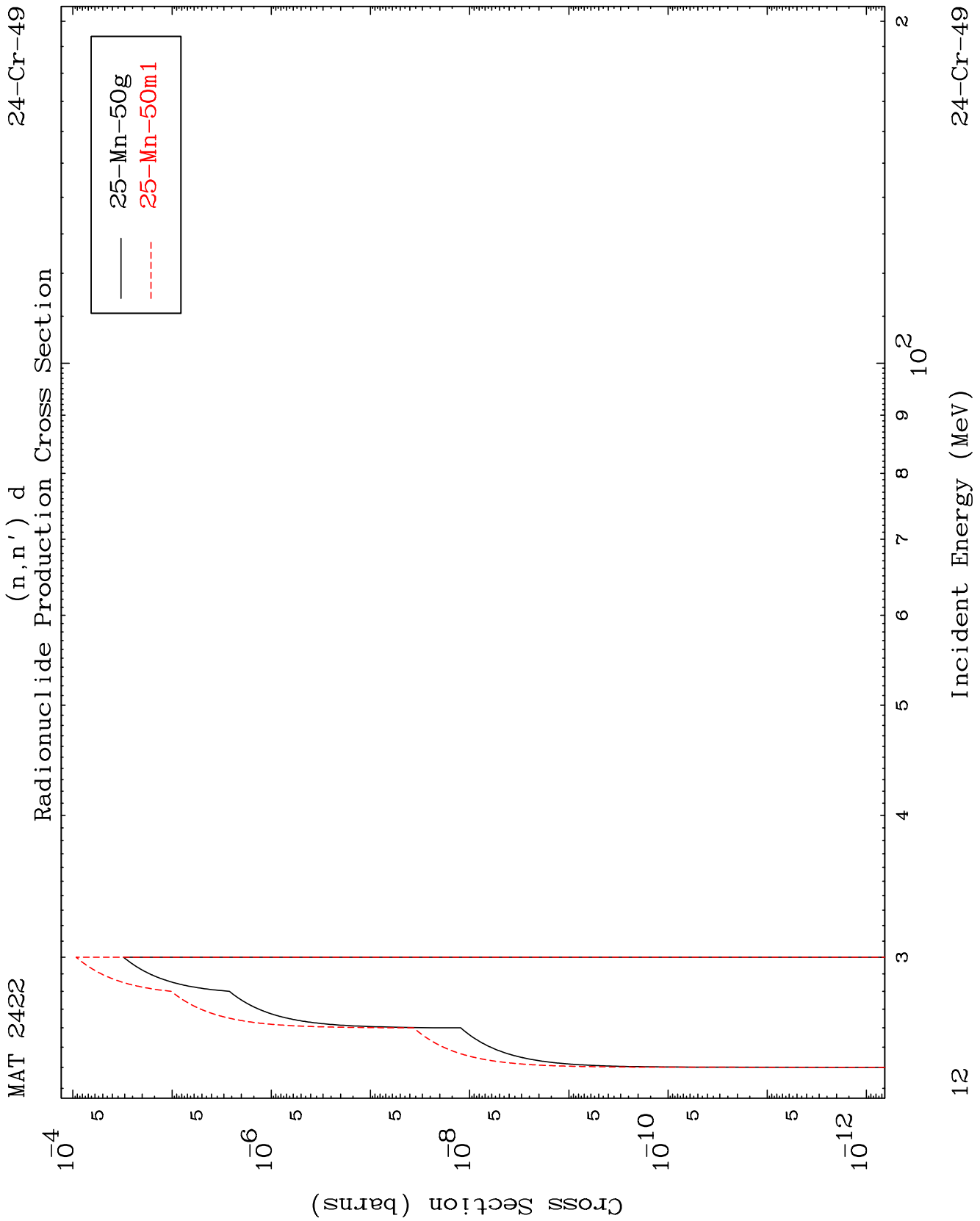
10

Incident Energy (MeV)

<sup>24</sup>Cr-49

Inelastic  
Radionuclide Production Cross Section

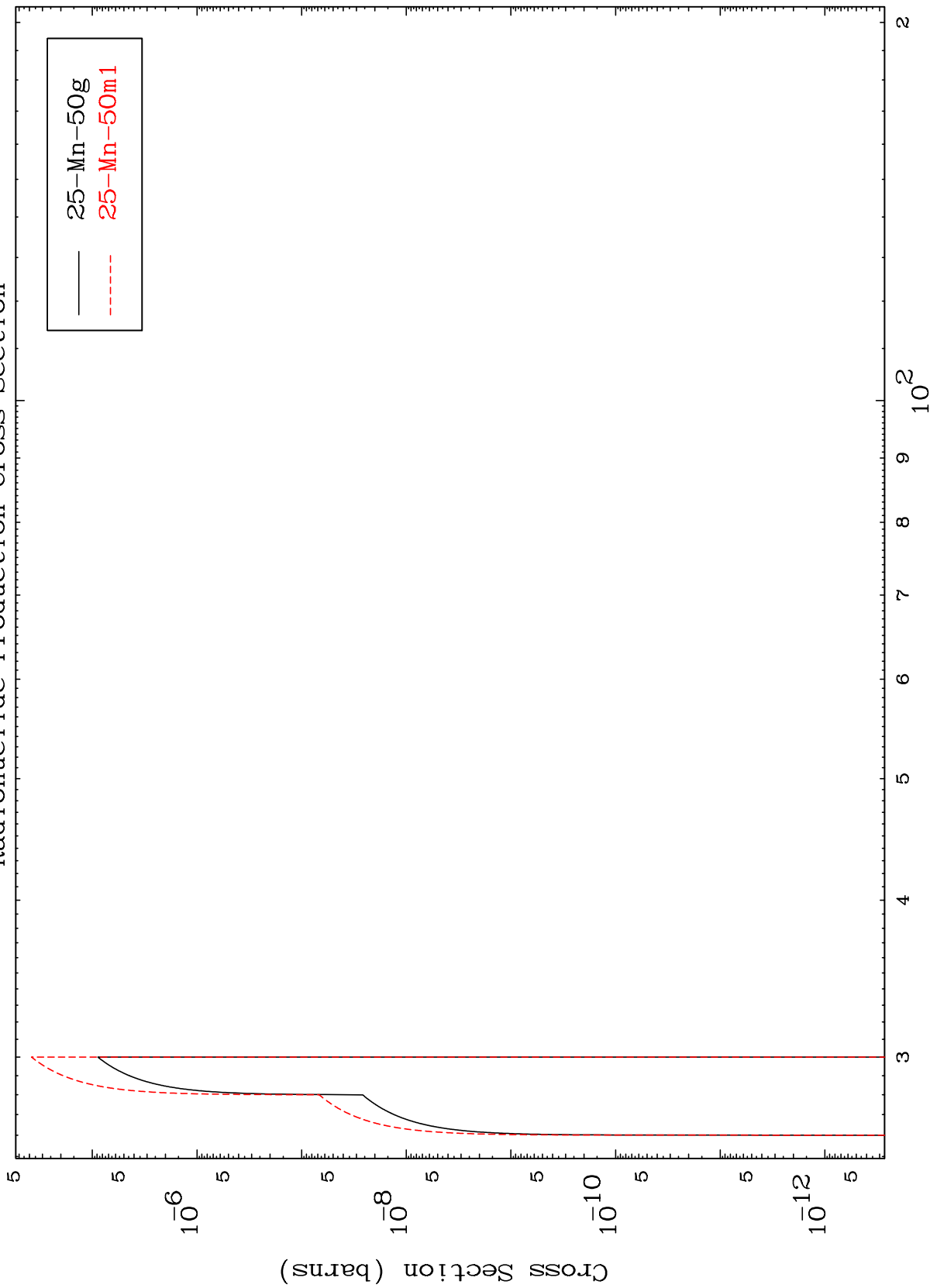




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24-Cr-49

(n,2n) p  
Radionuclide Production Cross Section



— 25-Mn-50g  
- - - 25-Mn-50ml

24-Cr-49

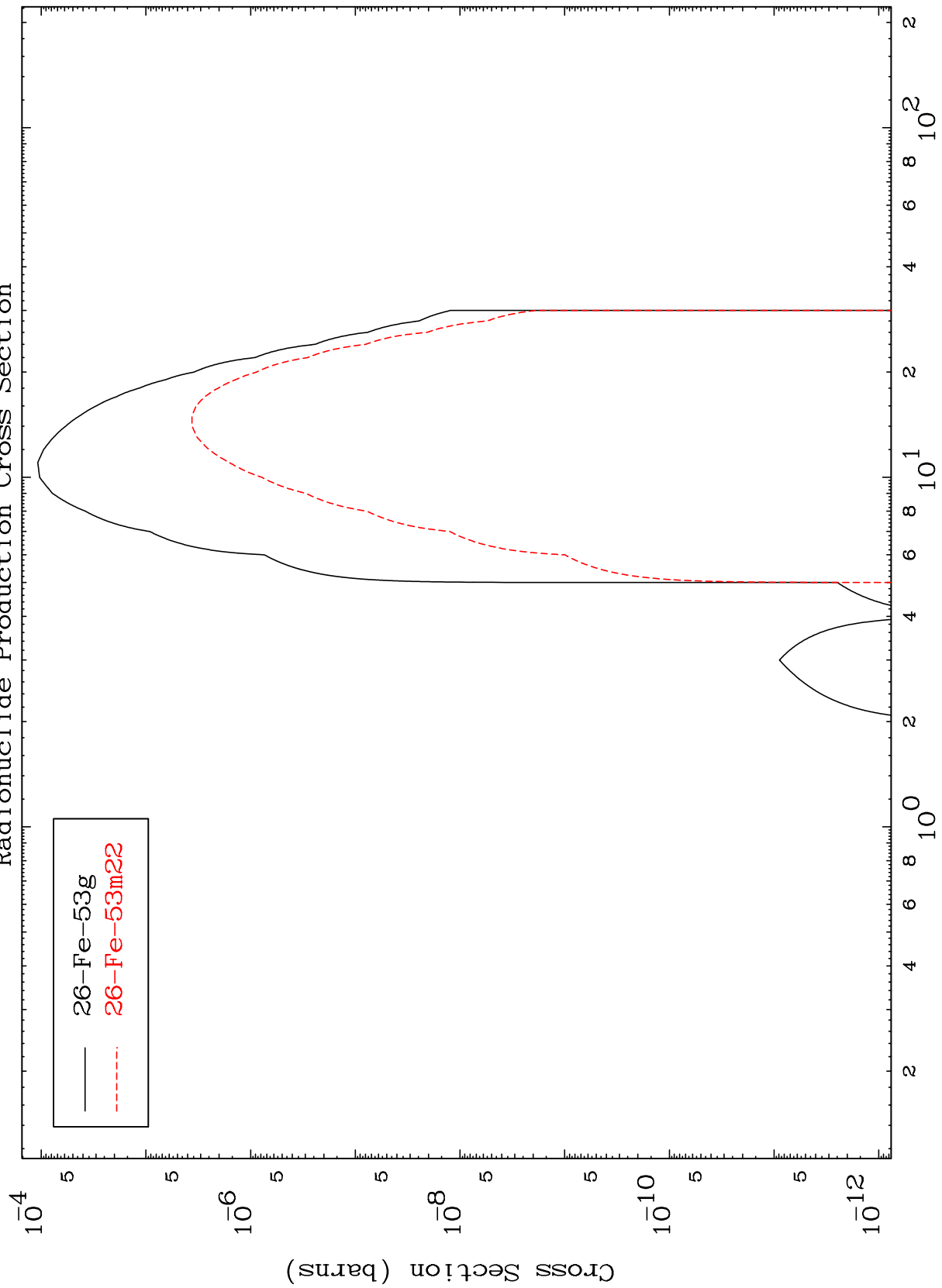
Incident Energy (MeV)

13

MAT 2422

24-Cr-49

(n,  $\gamma$ )  
Radionuclide Production Cross Section



24-Cr-49

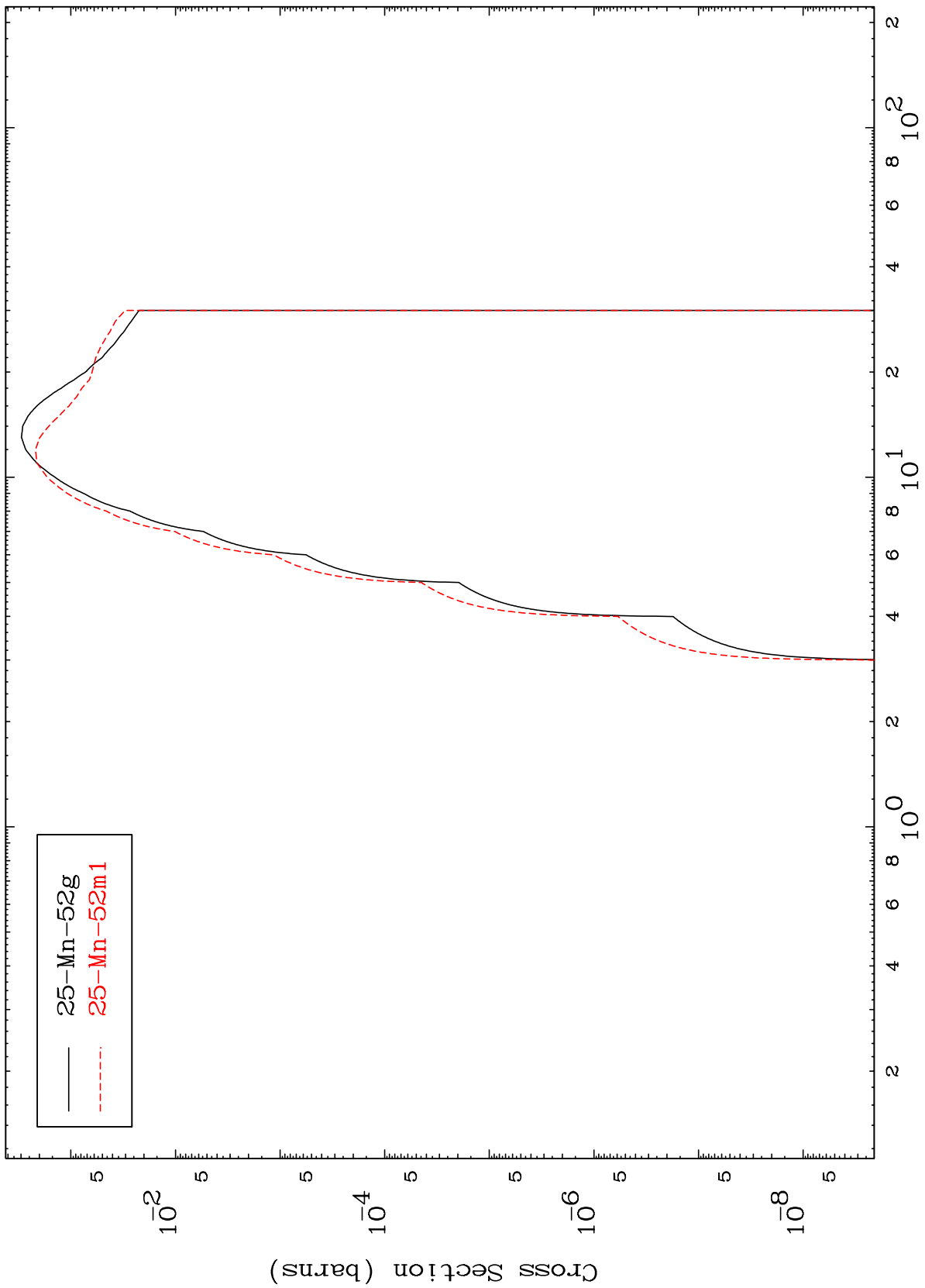
Incident Energy (MeV)

14

MAT 2422

24-Cr-49

(n,p)  
Radionuclide Production Cross Section



— 25-Mn-52g  
- - - 25-Mn-52m1

24-Cr-49

Incident Energy (MeV)

15

(n,t)  
Radionuclide Production Cross Section

