

Program EVALPLOT  
(Version 2021-1)

by

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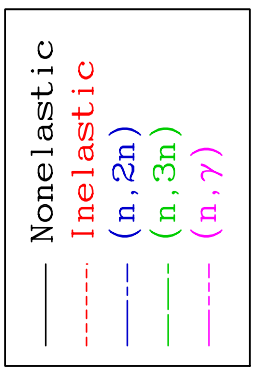
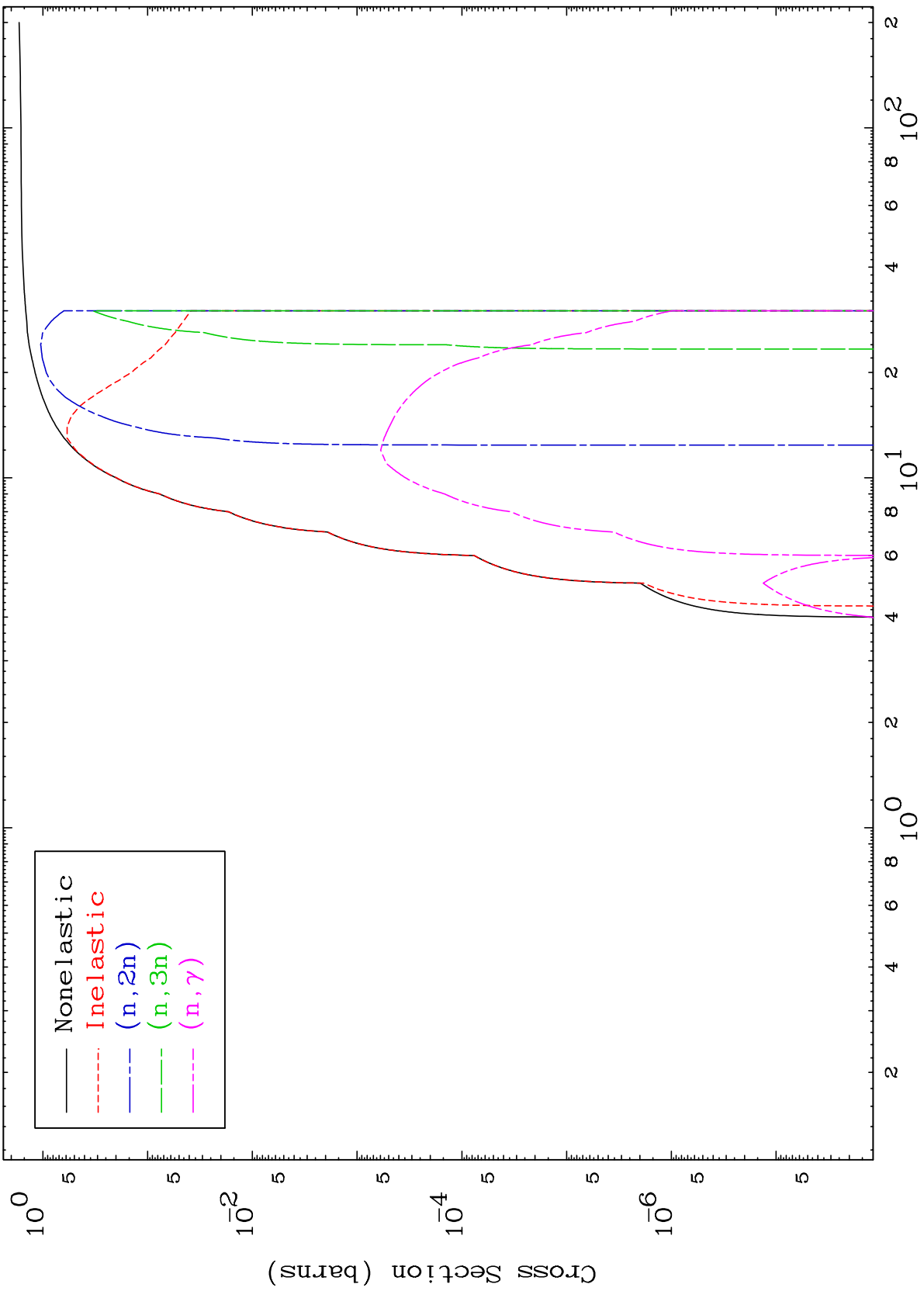
Press Mouse Button to Start

MAT 2937

$\alpha$  Major

29-Cu-67

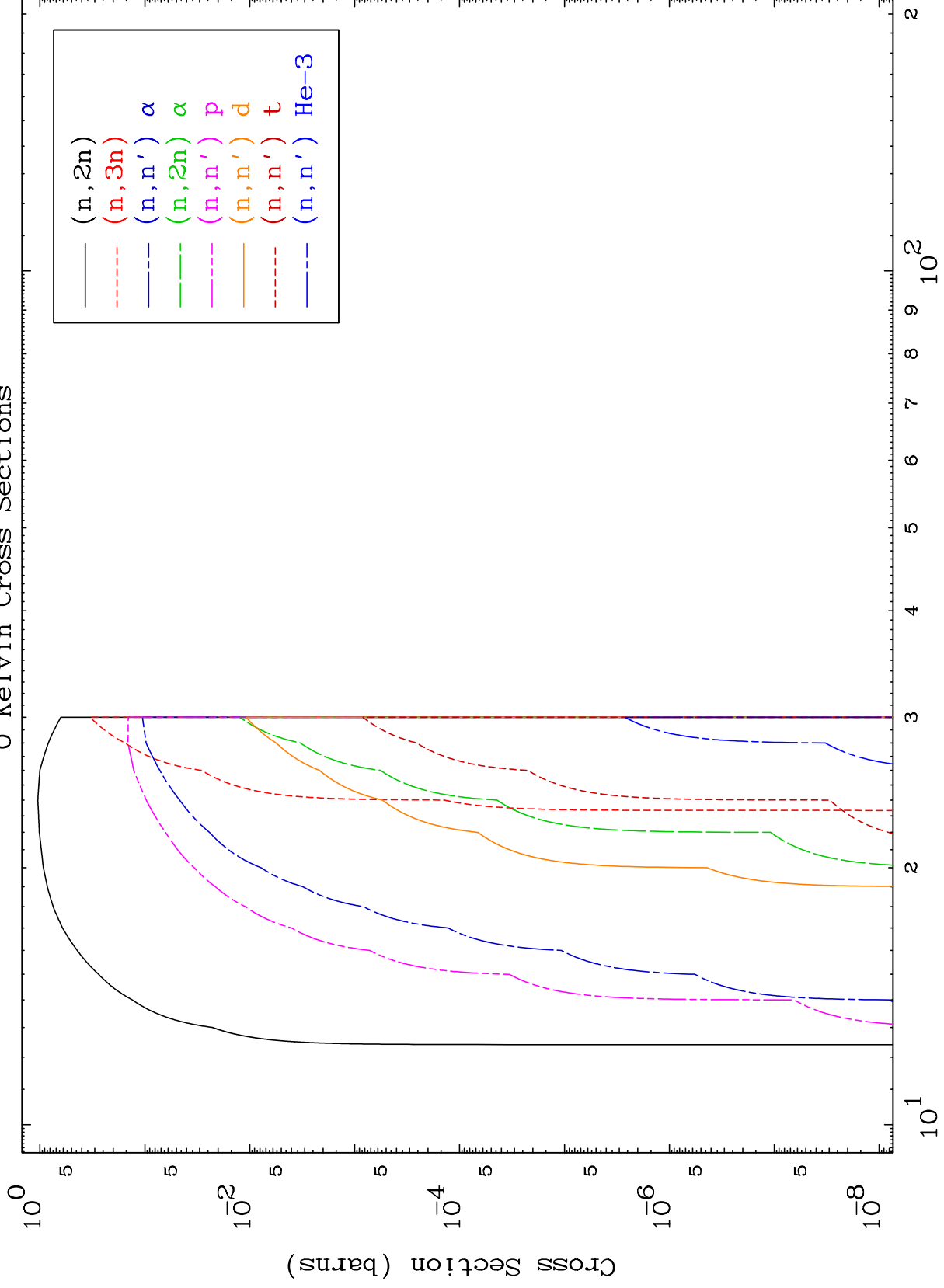
0 Kelvin Cross Sections



MAT 2937

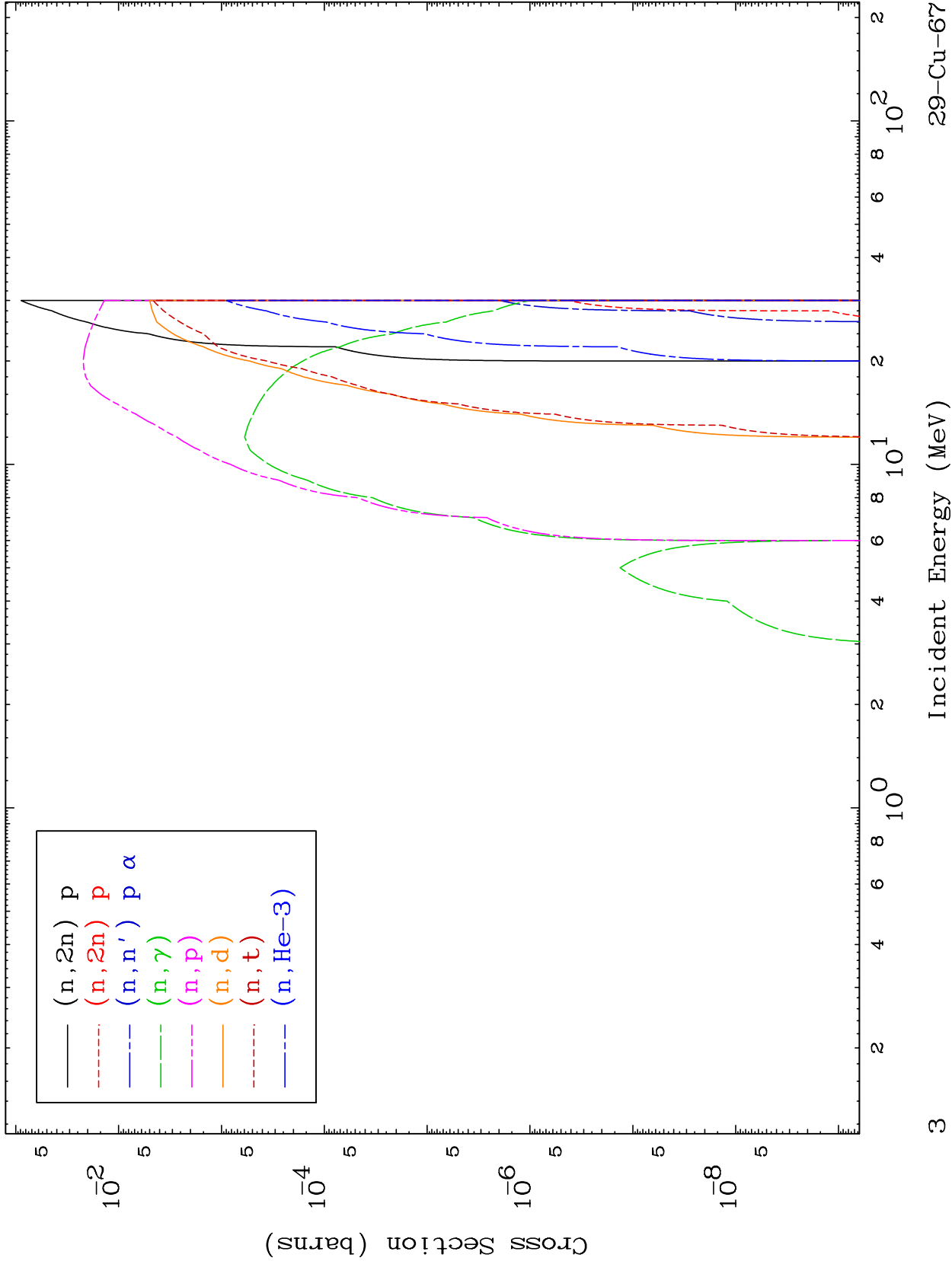
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

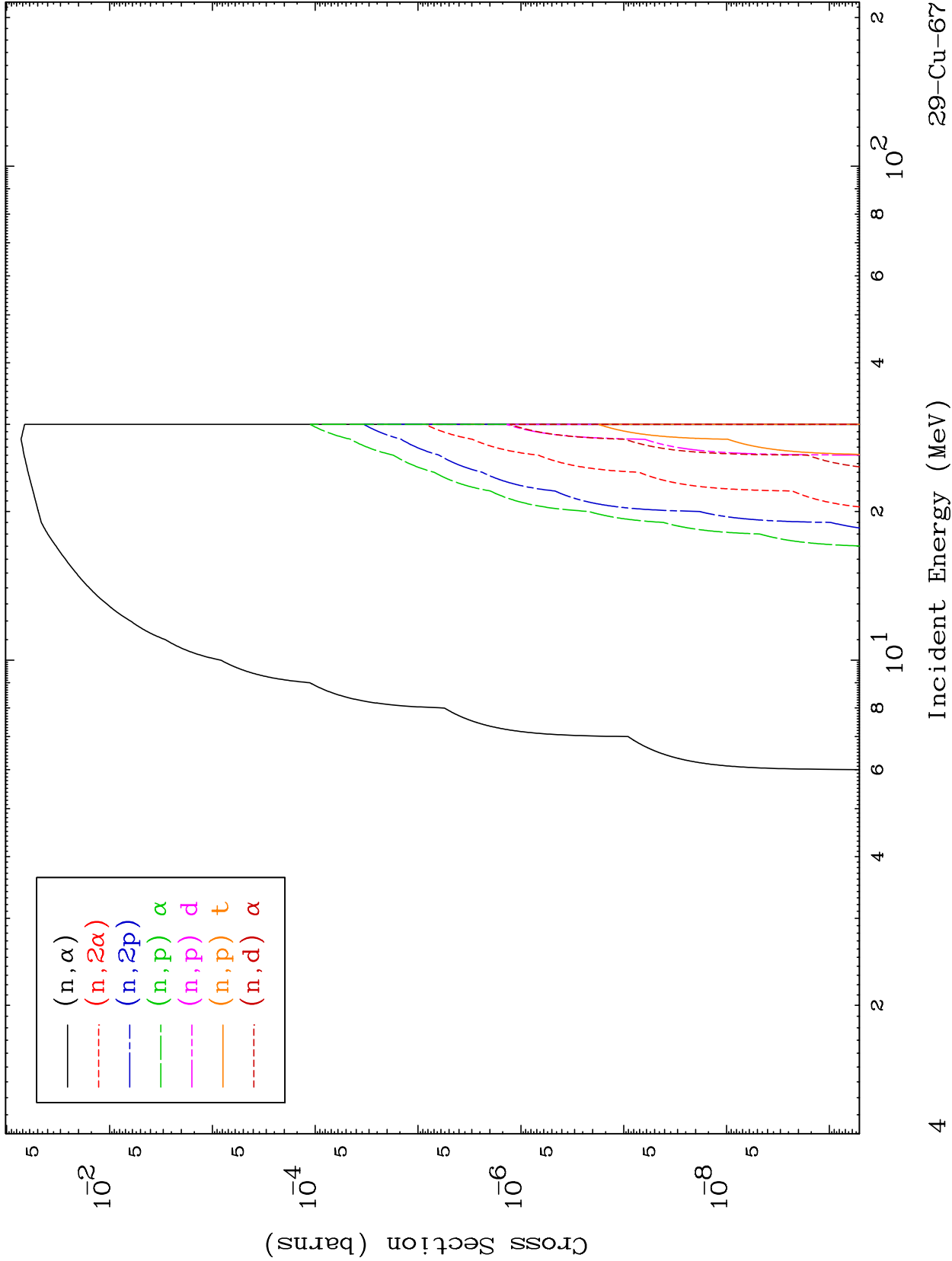
29-Cu-67

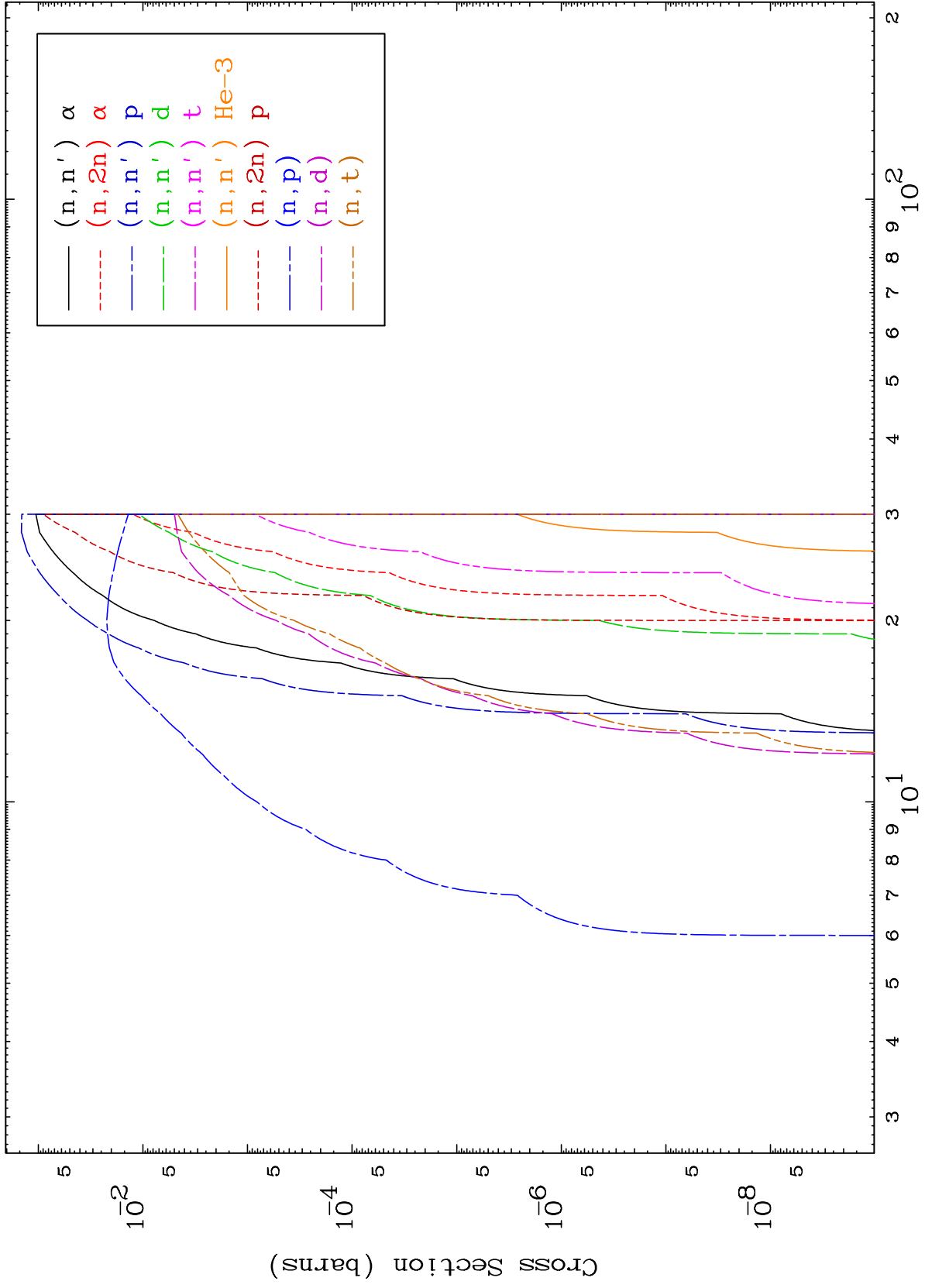


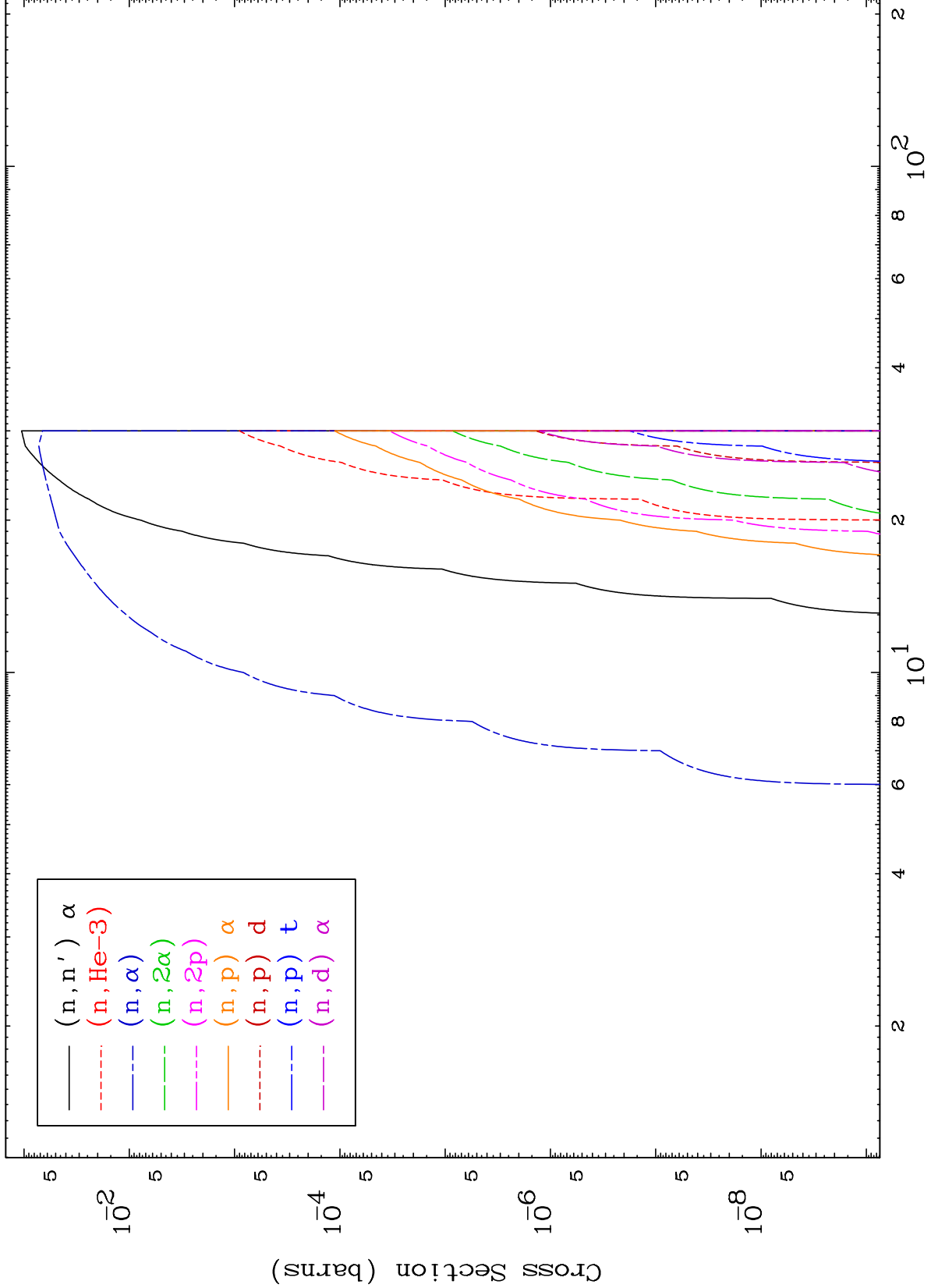
Incident Energy (MeV)

29-Cu-67

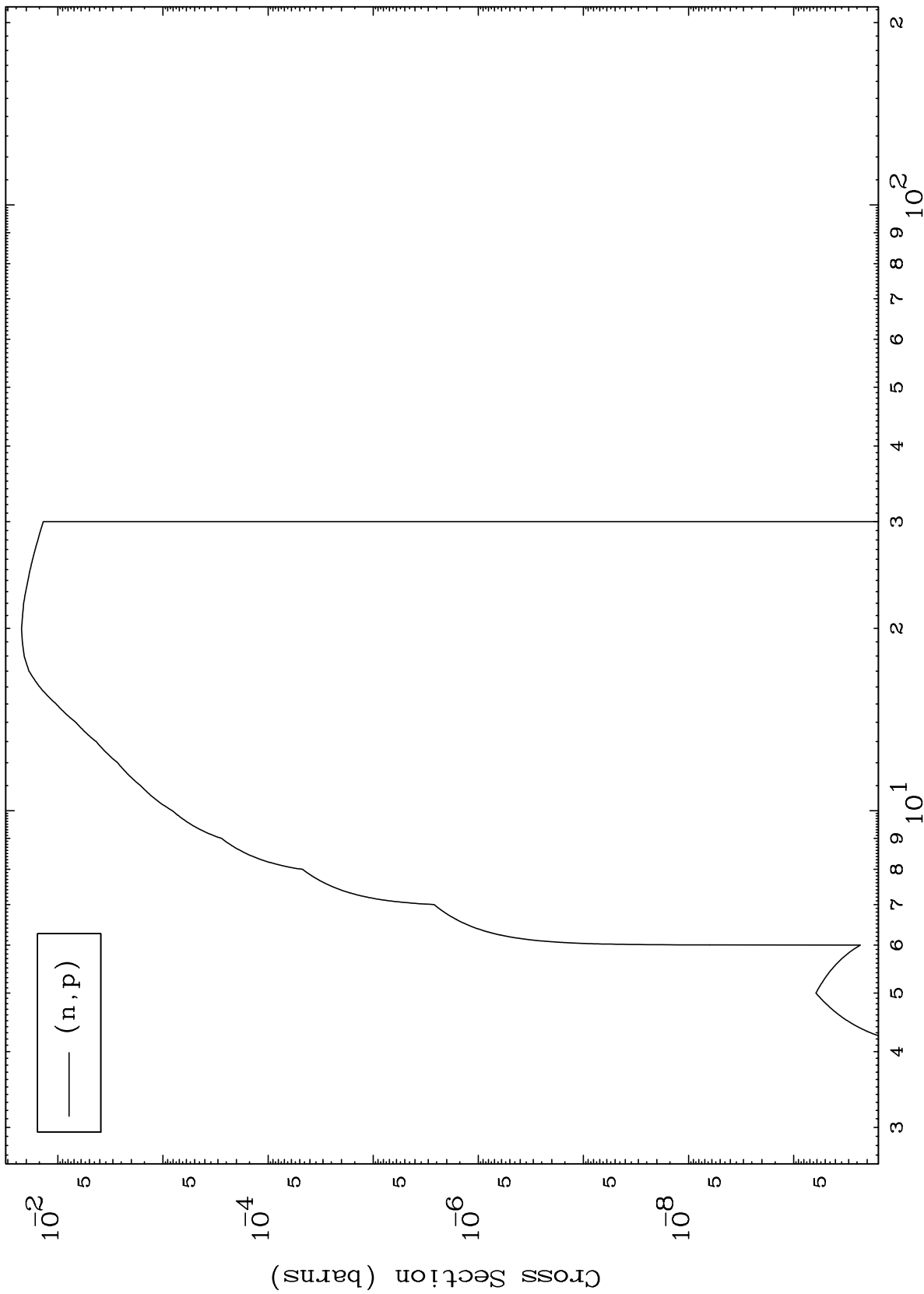








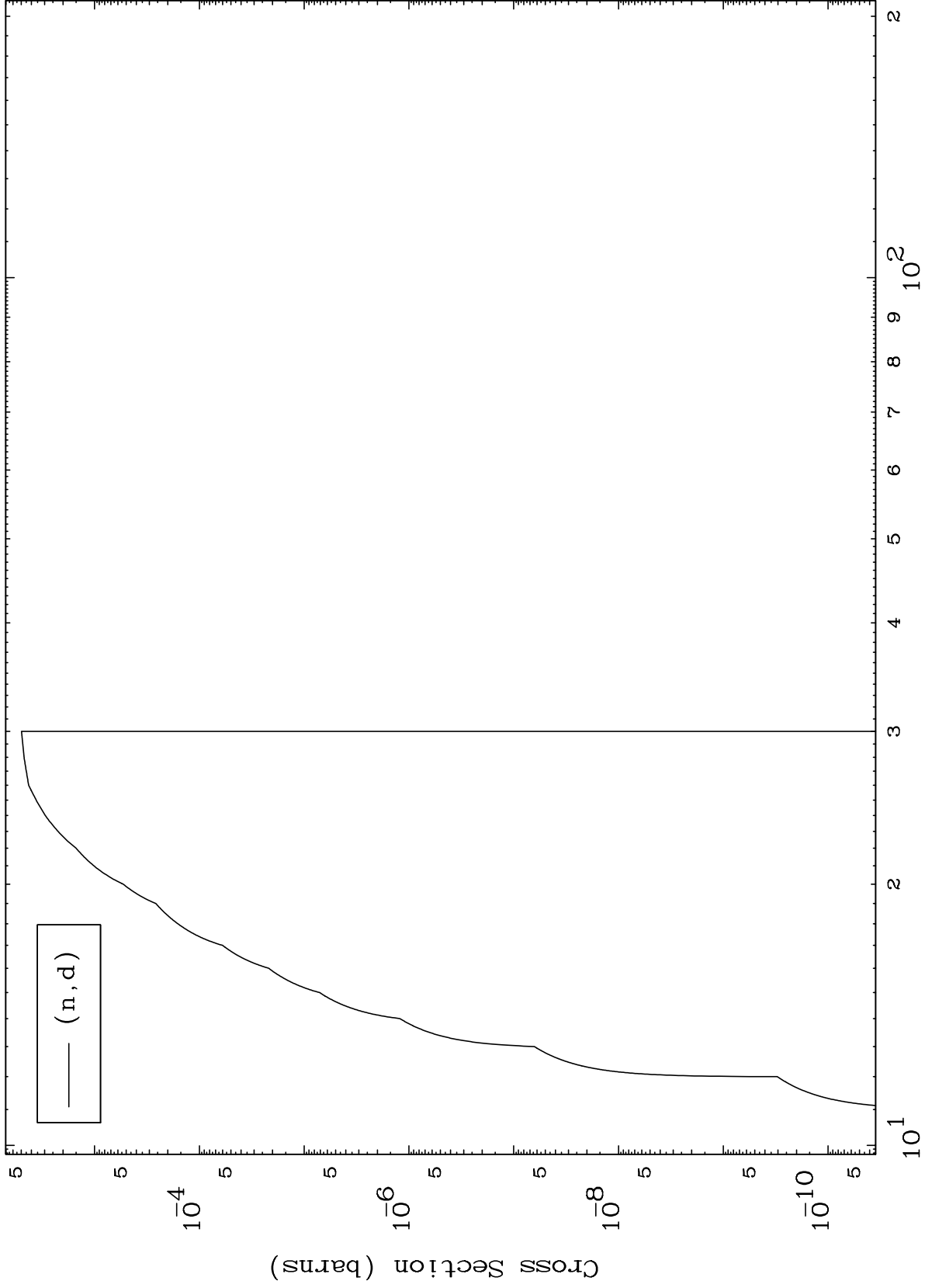
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



MAT 2937

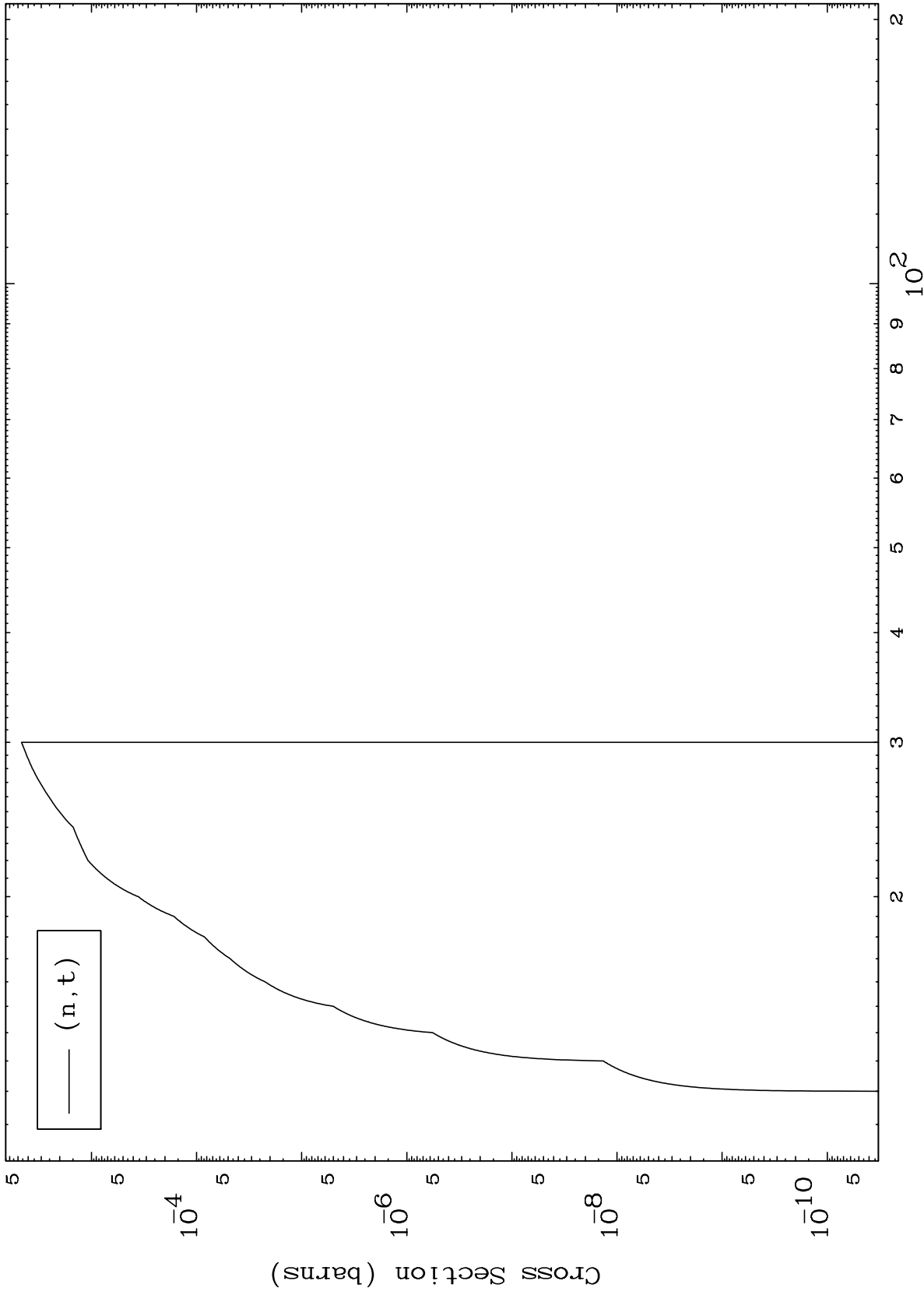
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

29-Cu-67



Incident Energy (MeV)

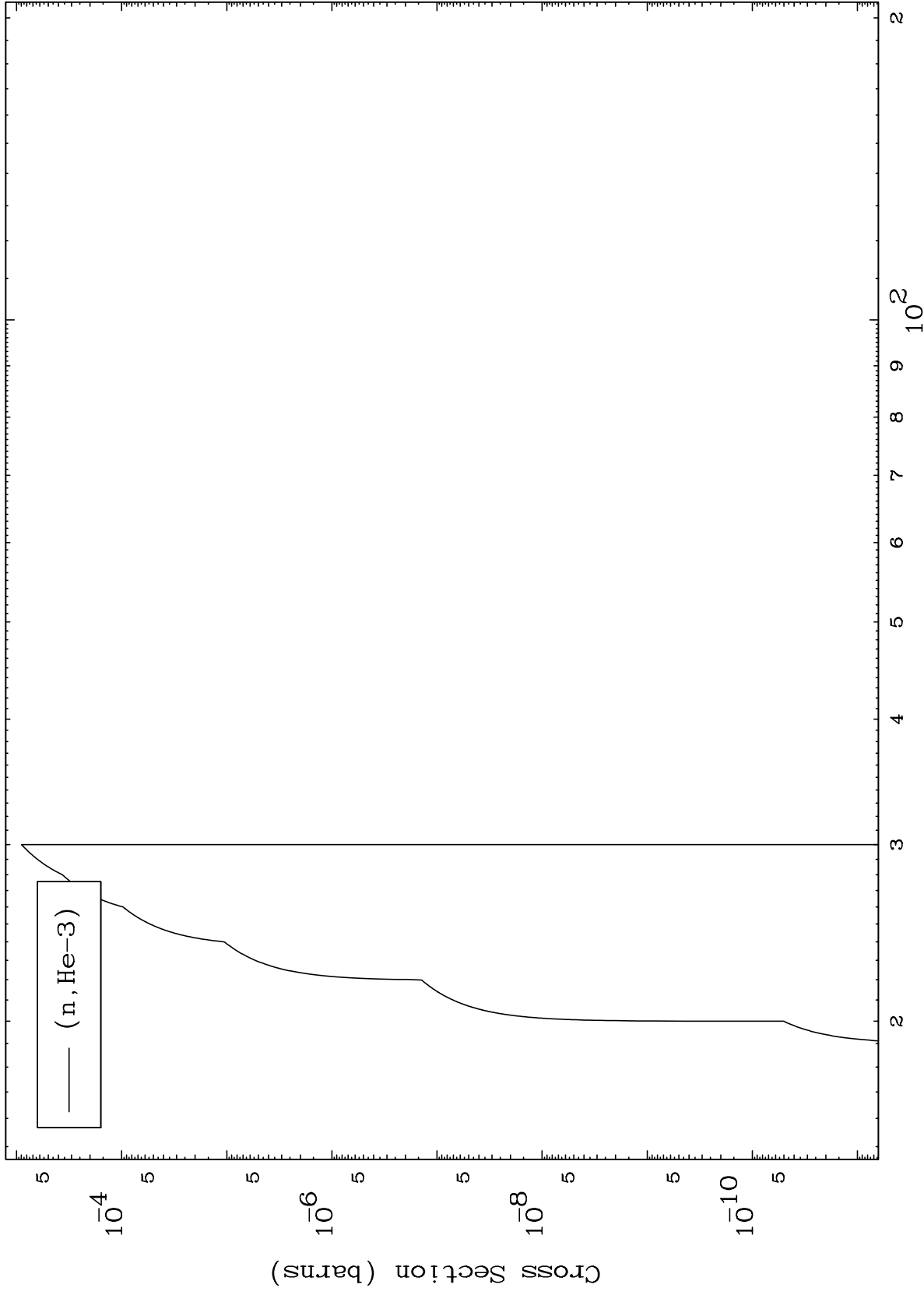
29-Cu-67



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( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

29-Cu-67



10

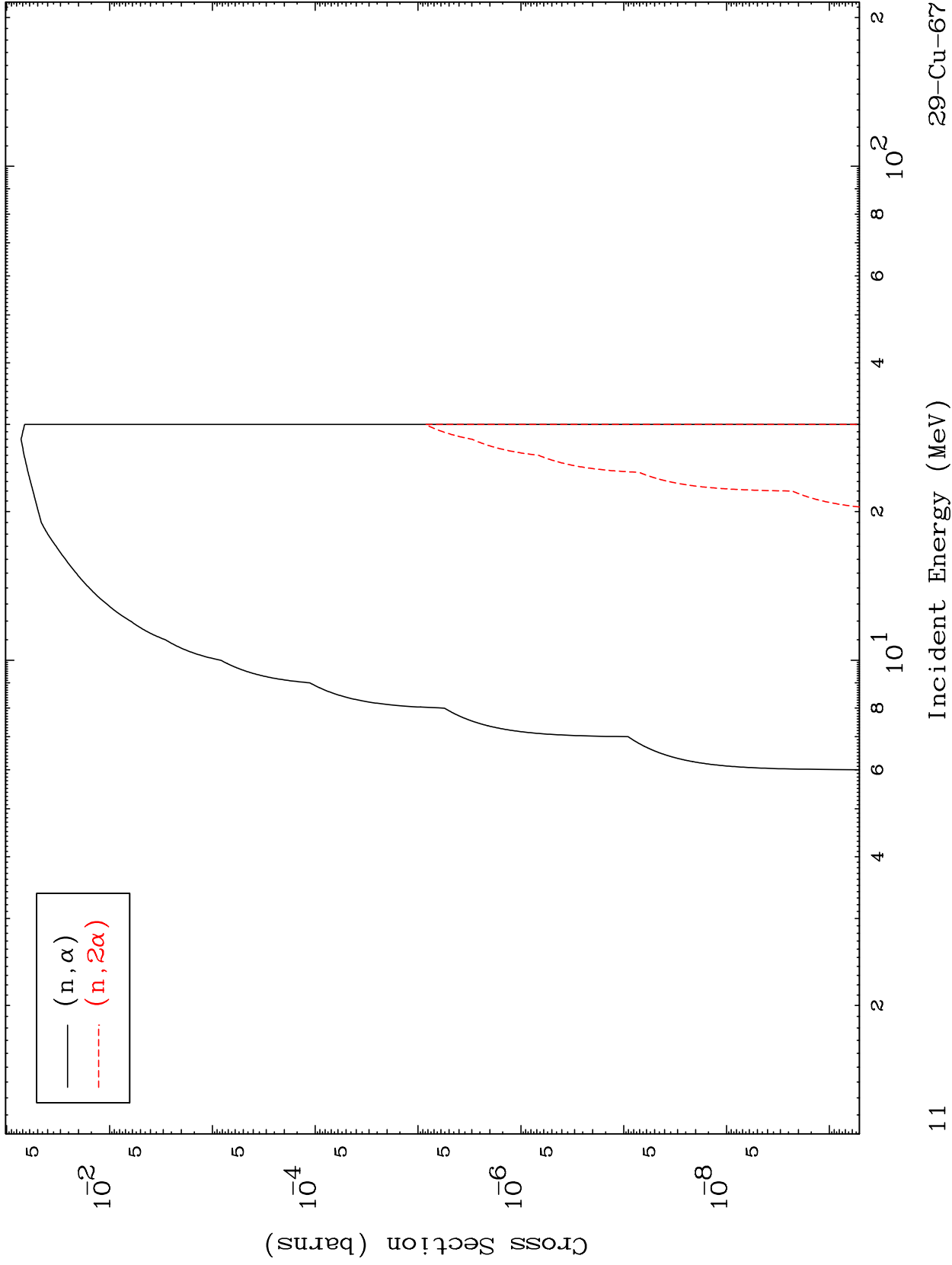
Incident Energy (MeV)

29-Cu-67

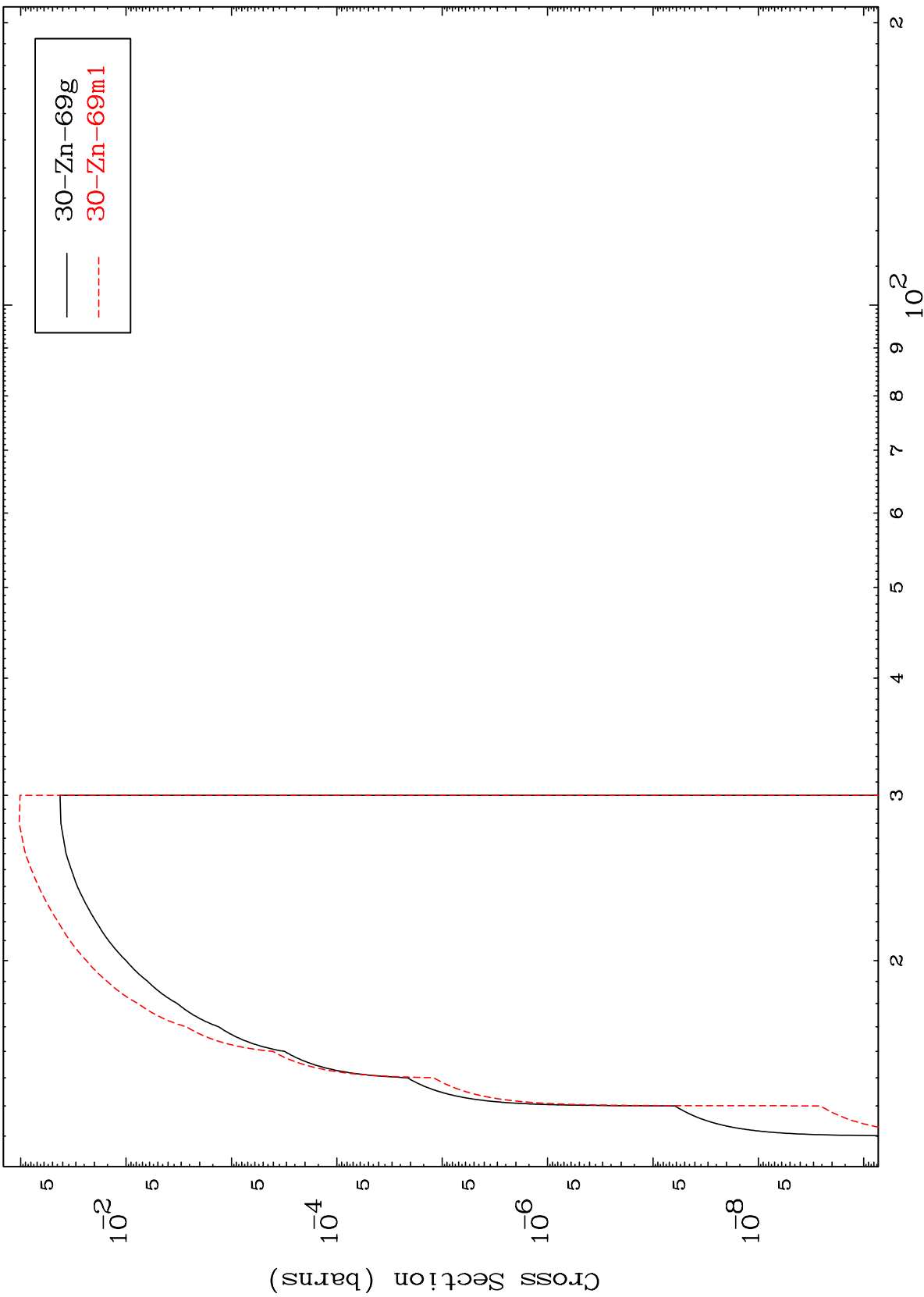
MAT 2937

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

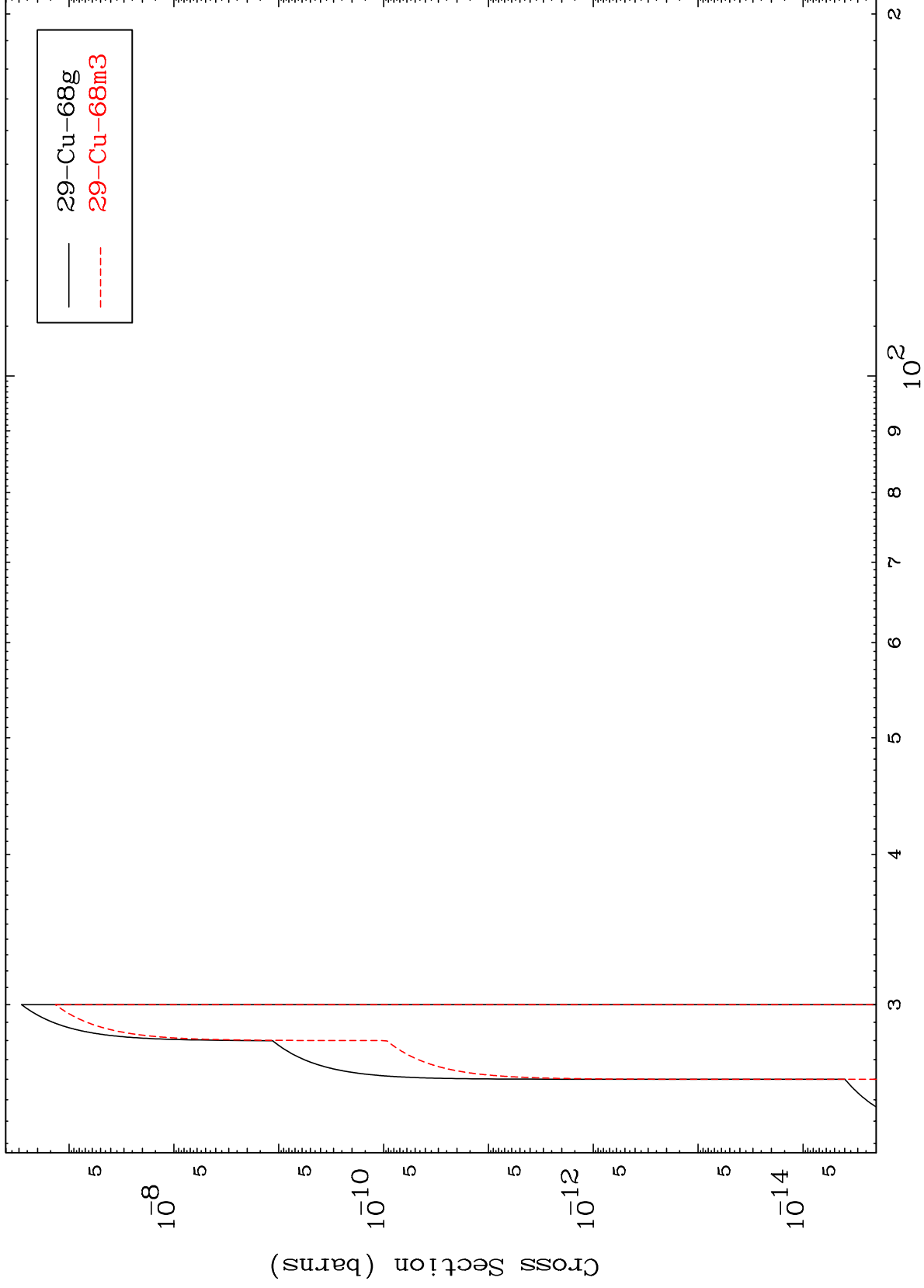
29-Cu-67



Radionuclide Production Cross Section



Radionuclide Production Cross Section

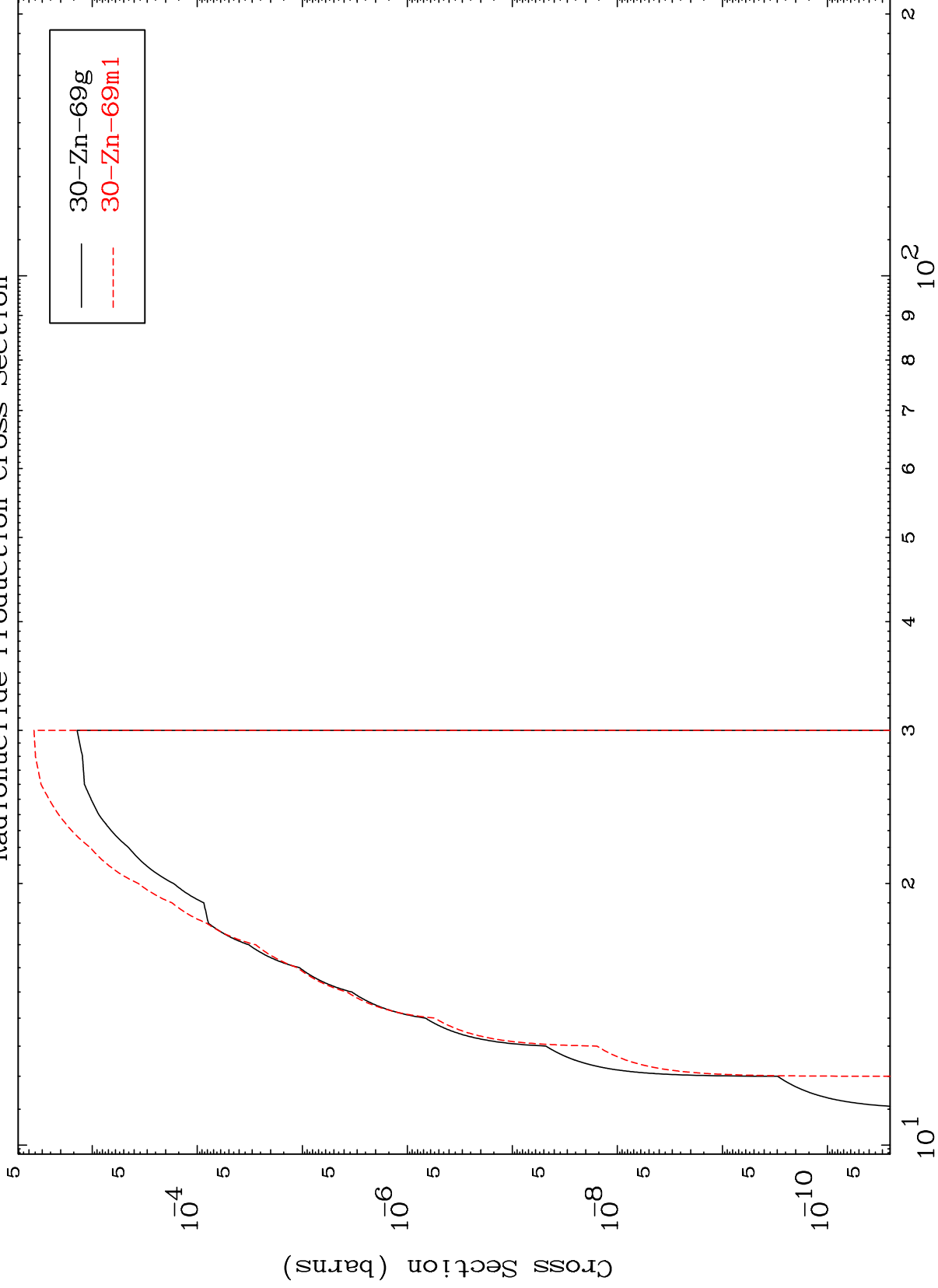


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(n,d)

29-Cu-67

Radionuclide Production Cross Section



Incident Energy (MeV)

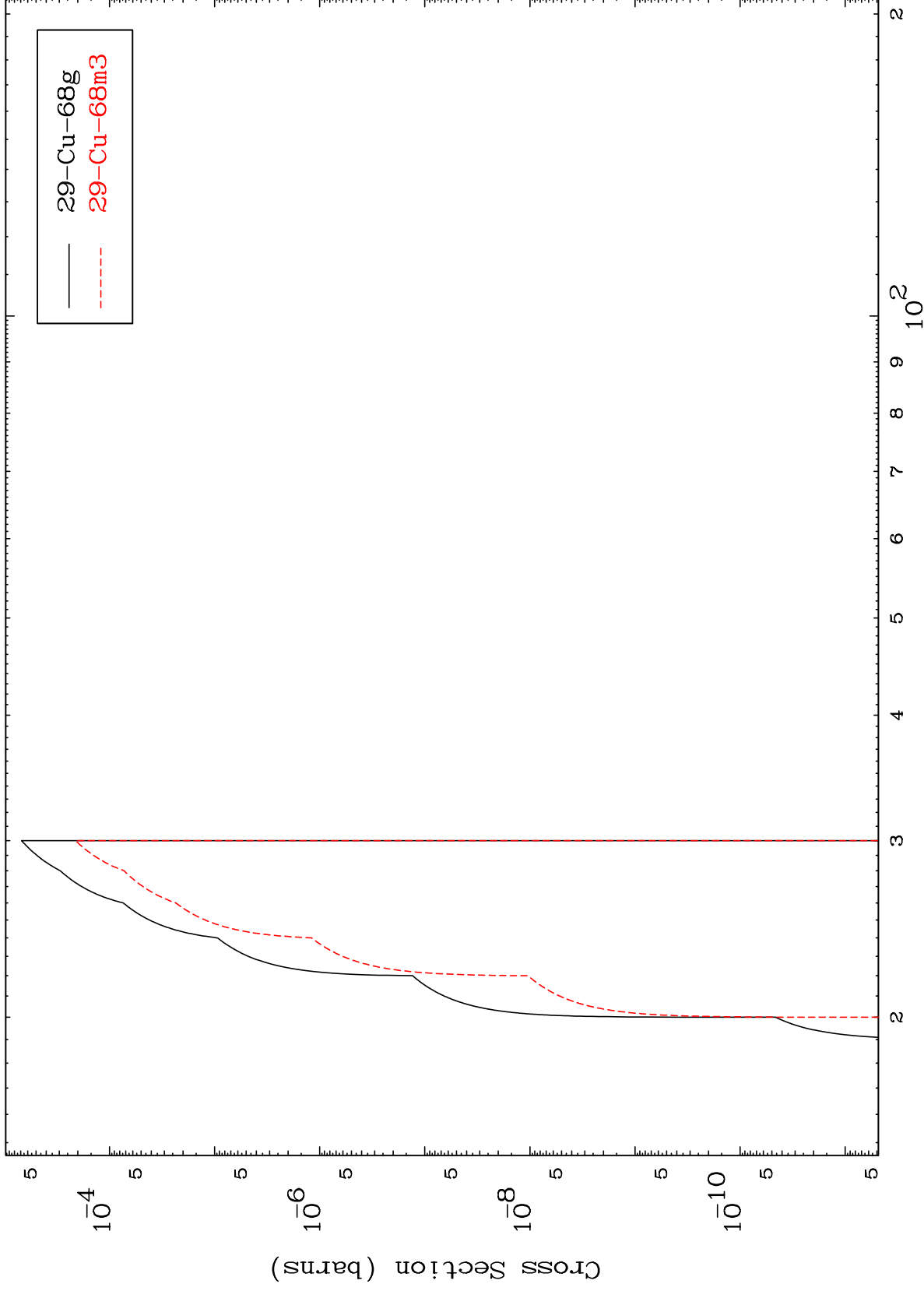
29-Cu-67

MAT 2937

(n,He-3)

29-Cu-67

Radionuclide Production Cross Section



15

Incident Energy (MeV)

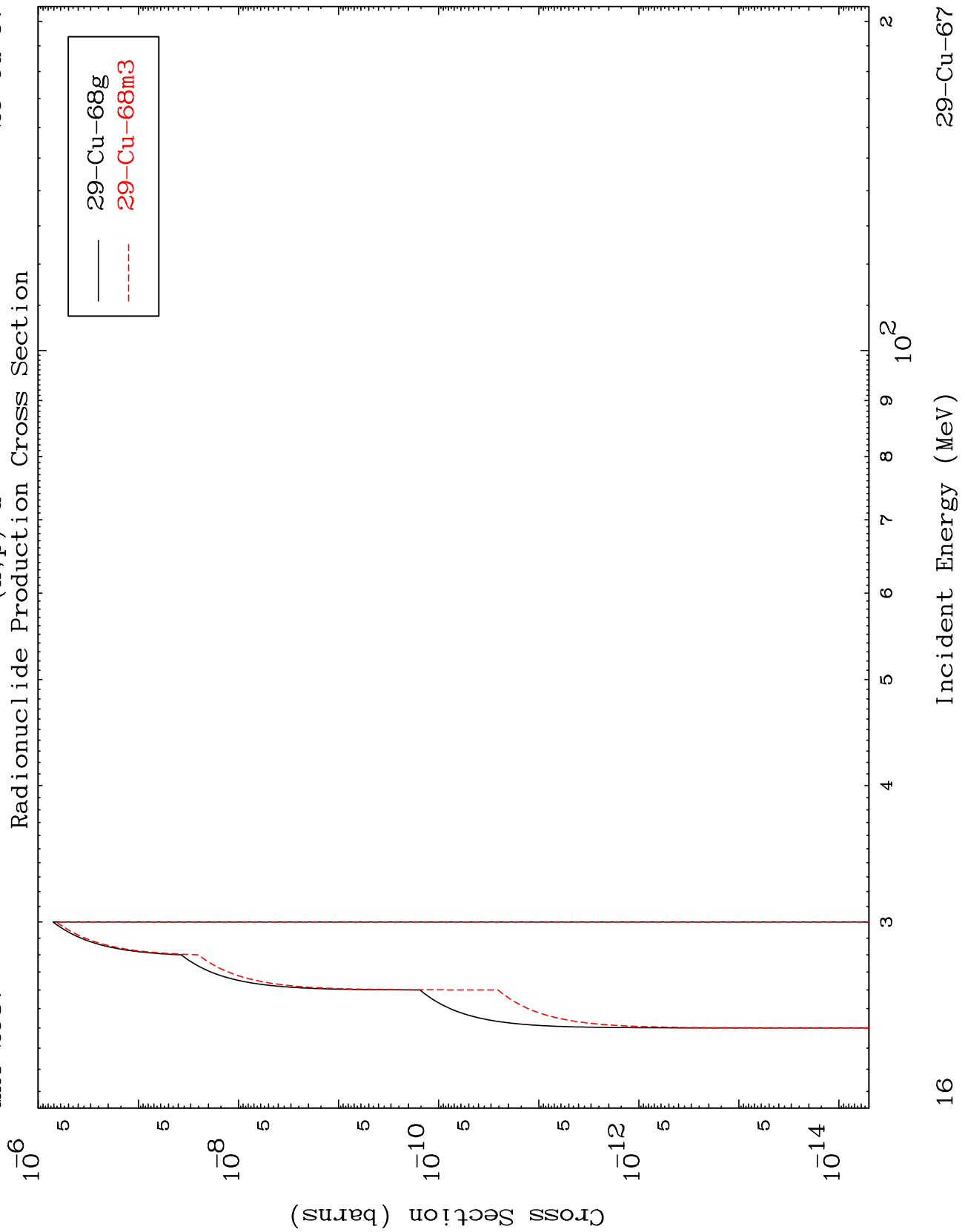
29-Cu-67

MAT 2937

(n,p) d

29-Cu-67

Radionuclide Production Cross Section



16

29-Cu-67