

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

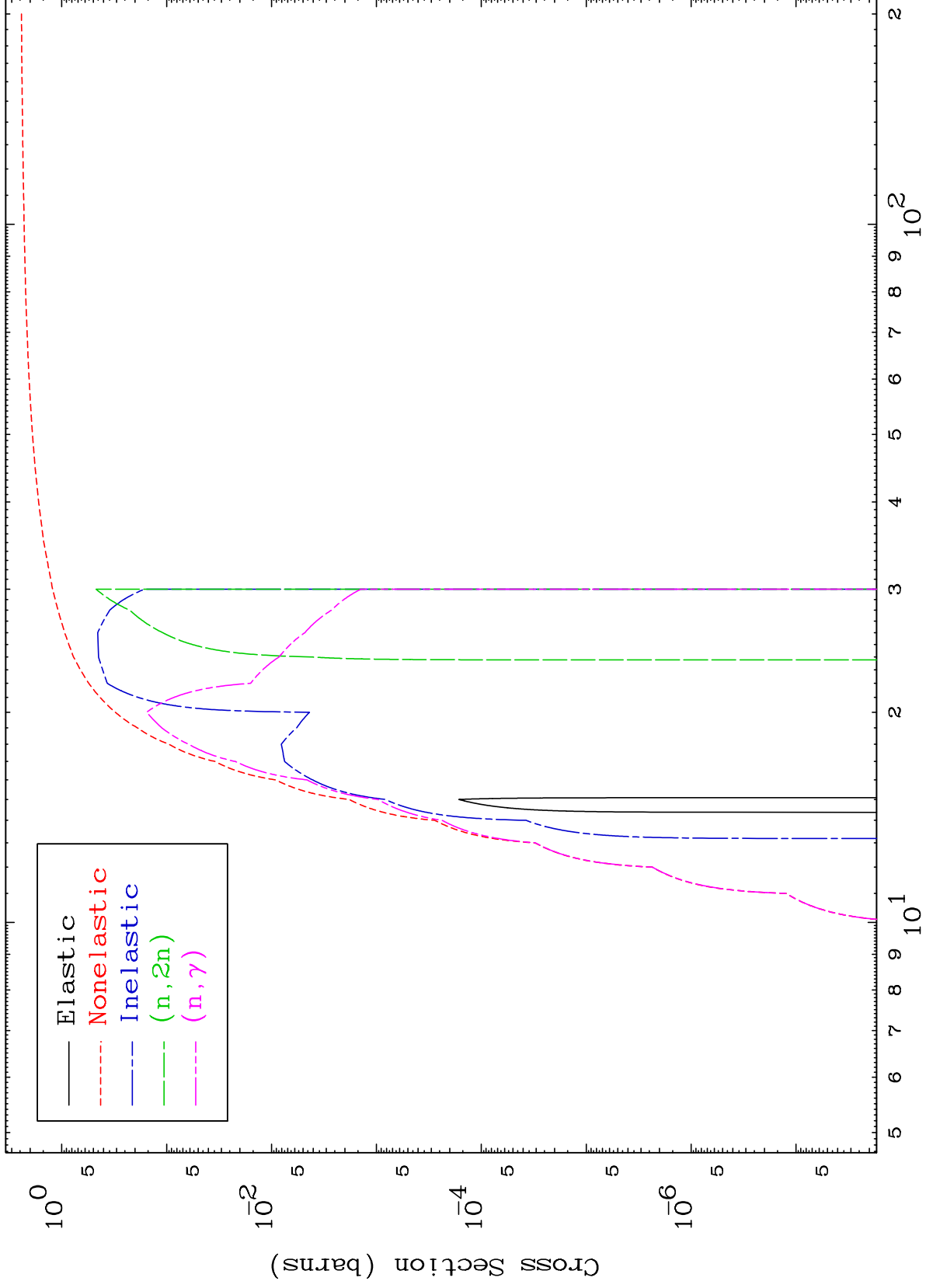
Press Mouse Button to Start

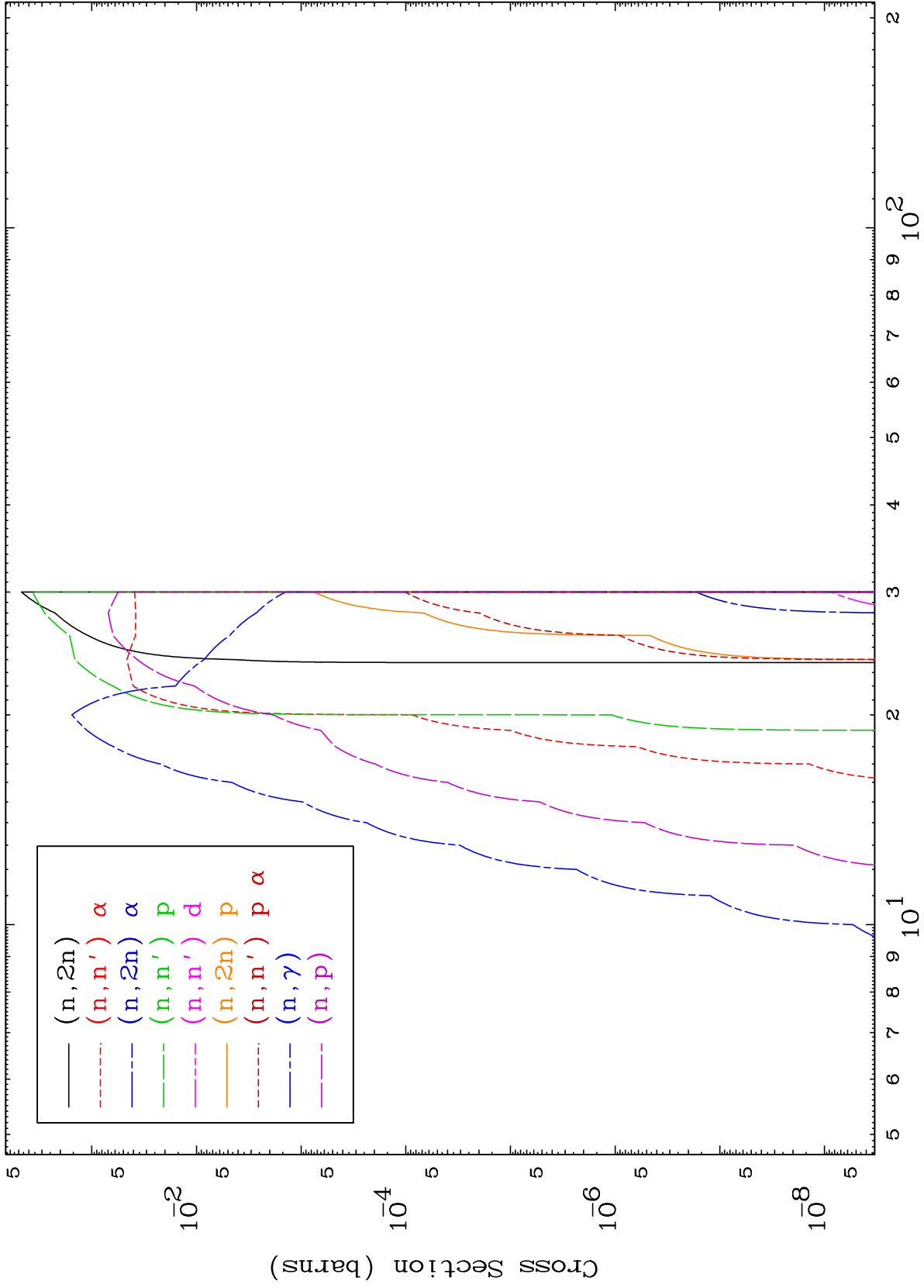
MAT 6604

0 Kelvin Cross Sections

$\alpha$  Major

66-Dy-149

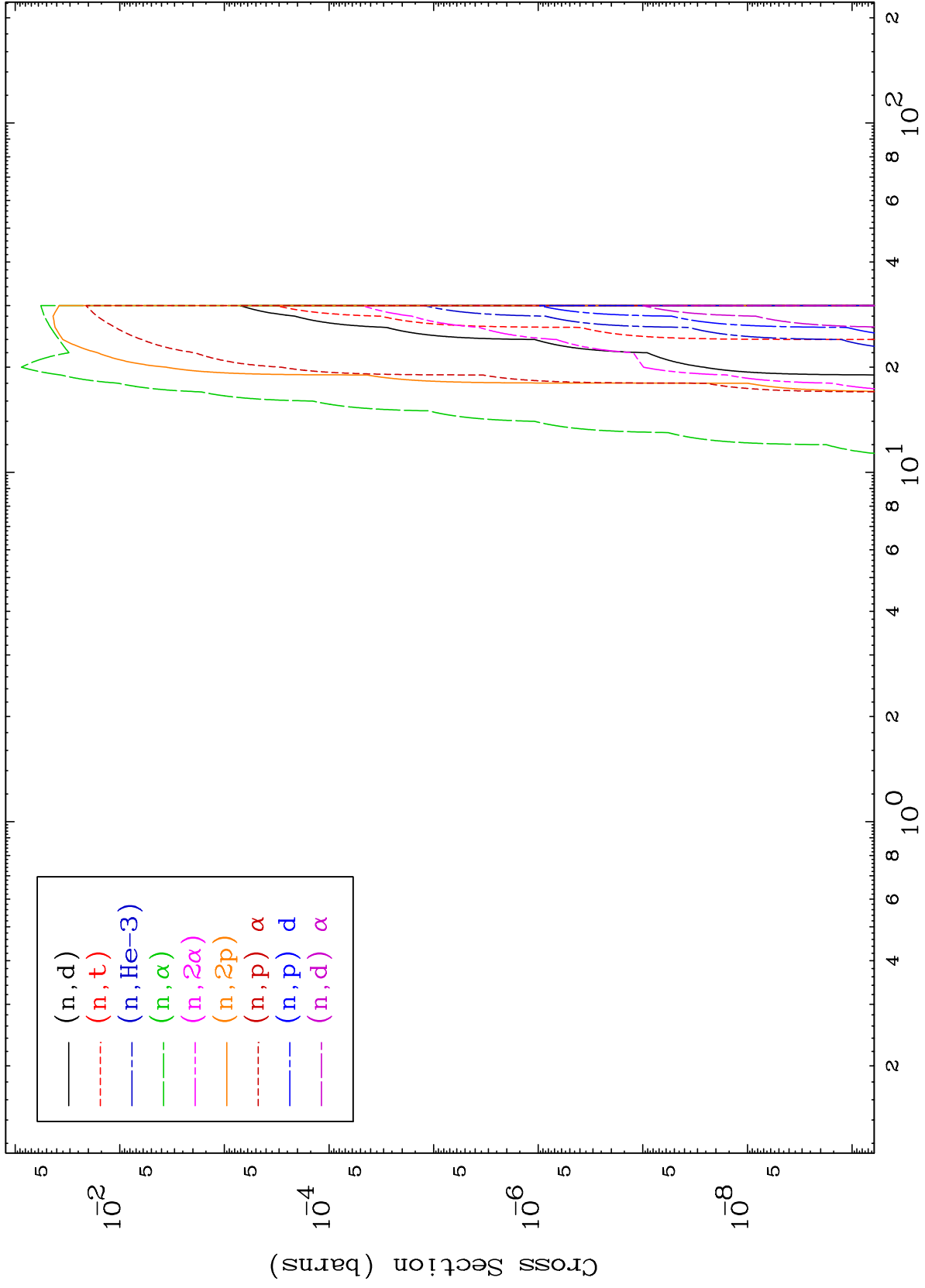


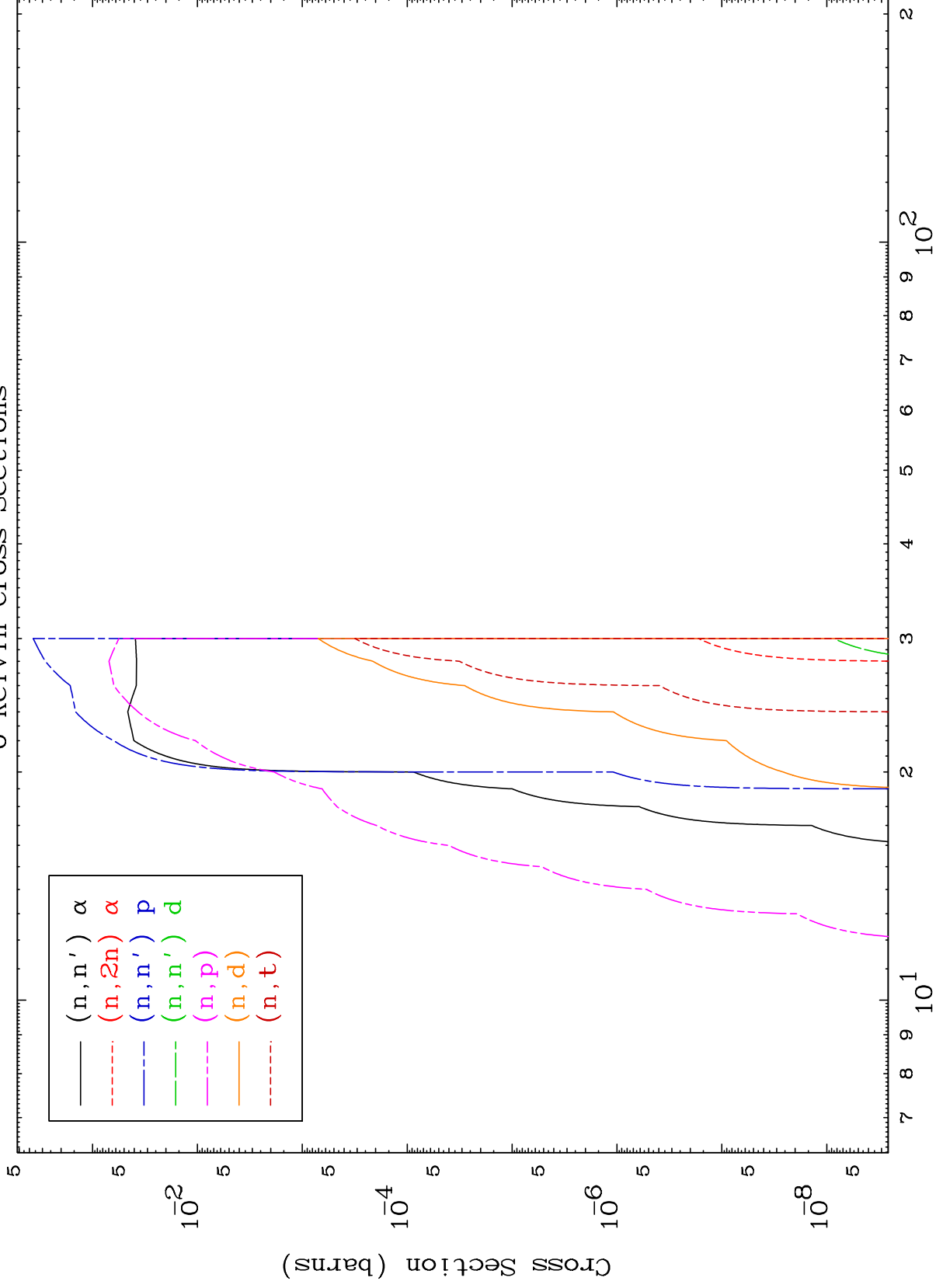


MAT 6604

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

66-Dy-149



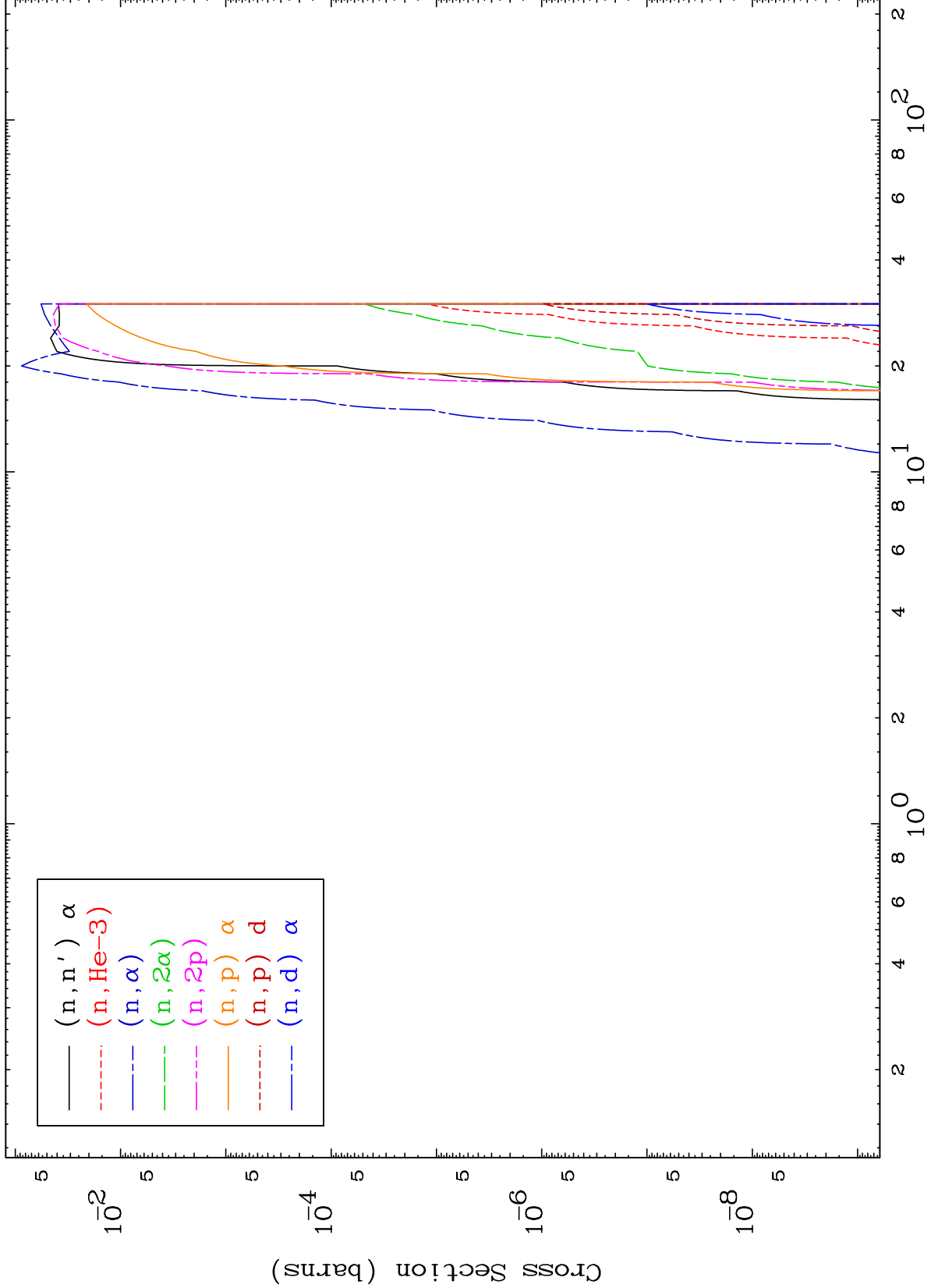


MAT 6604

$\alpha$  Charged Particle

66-Dy-149

0 Kelvin Cross Sections

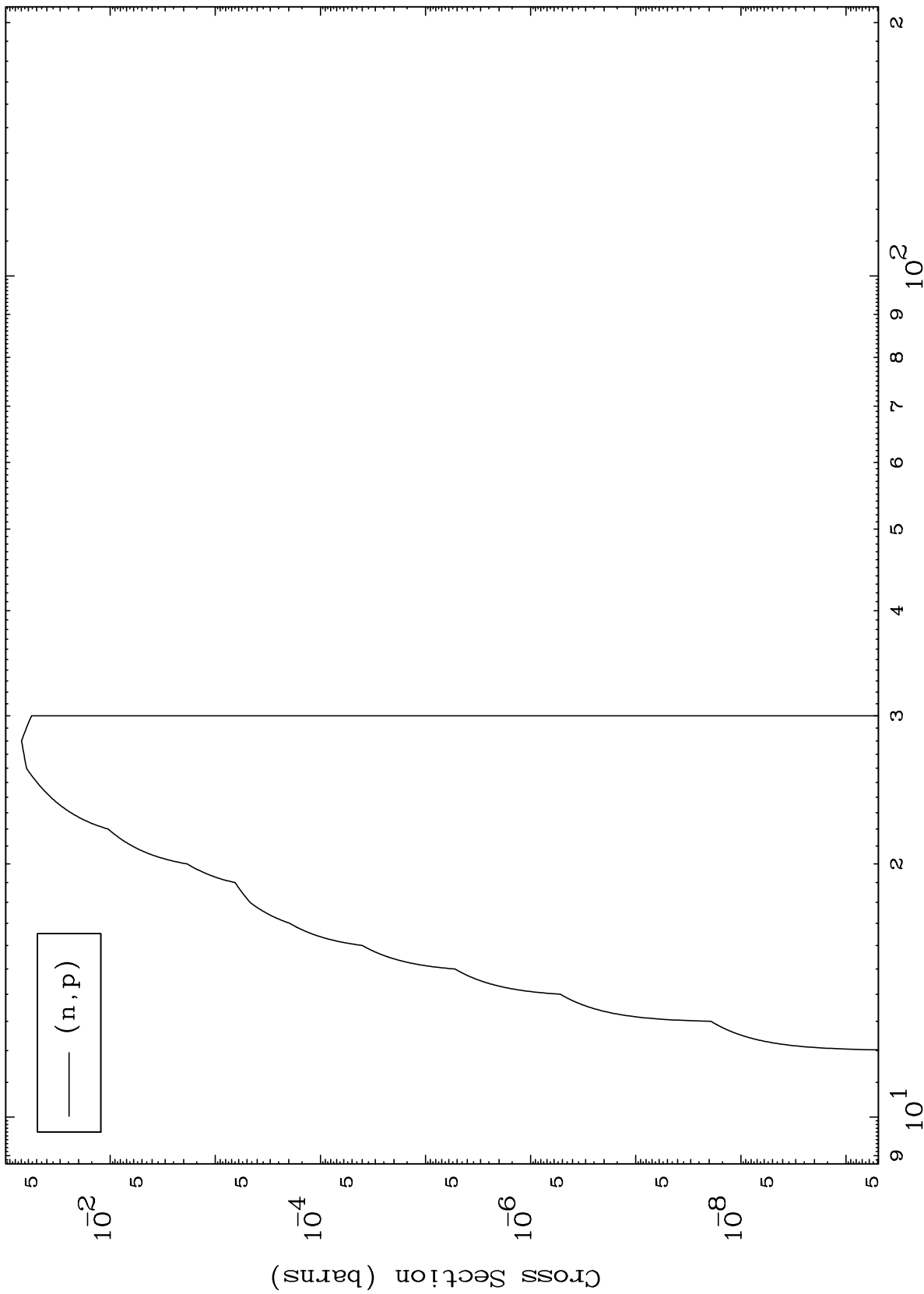


MAT 6604

( $\alpha, p$ ) Levels

66-Dy-149

0 Kelvin Cross Sections



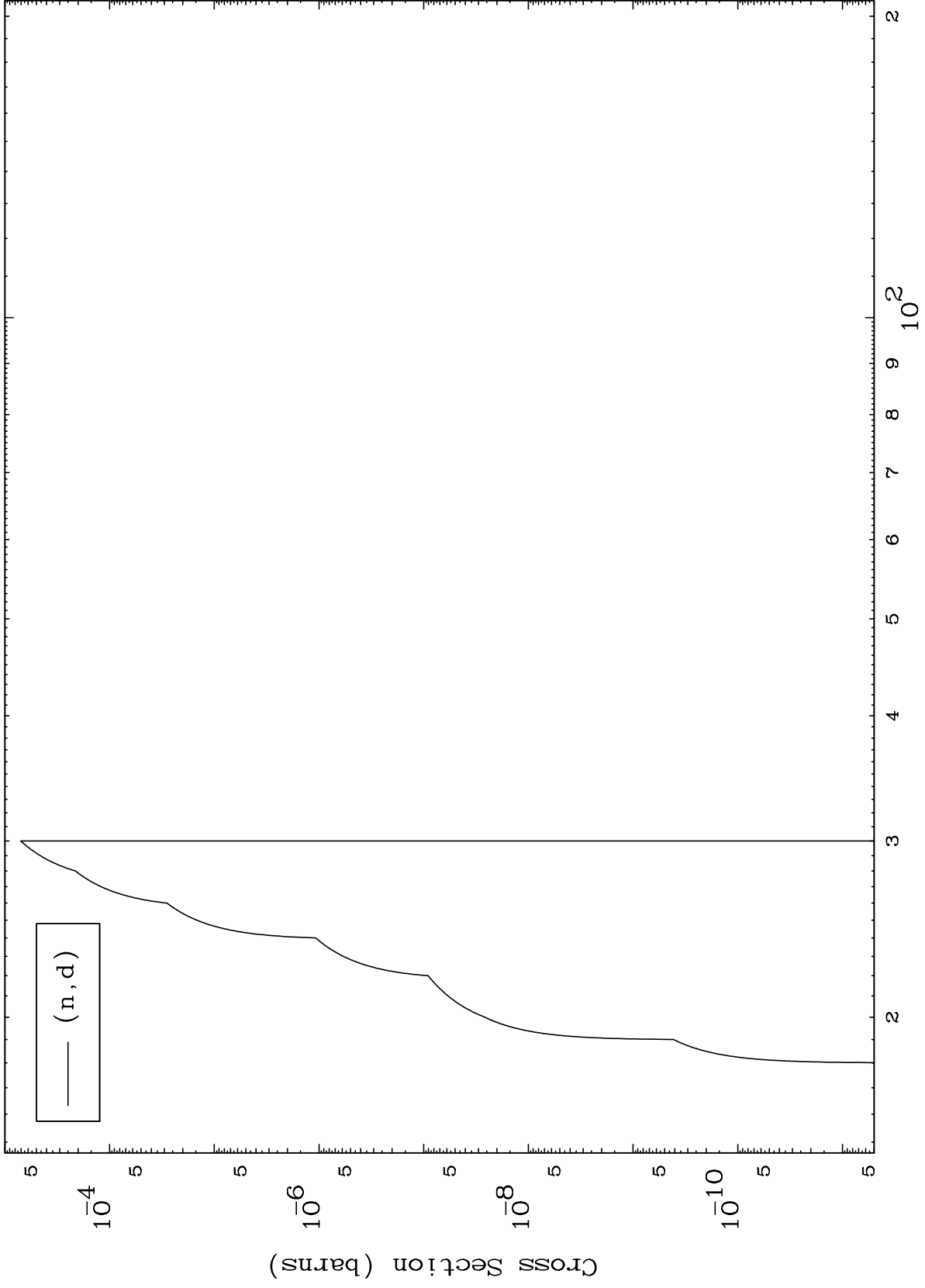
Incident Energy (MeV)

66-Dy-149

MAT 6604

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

66-Dy-149



7

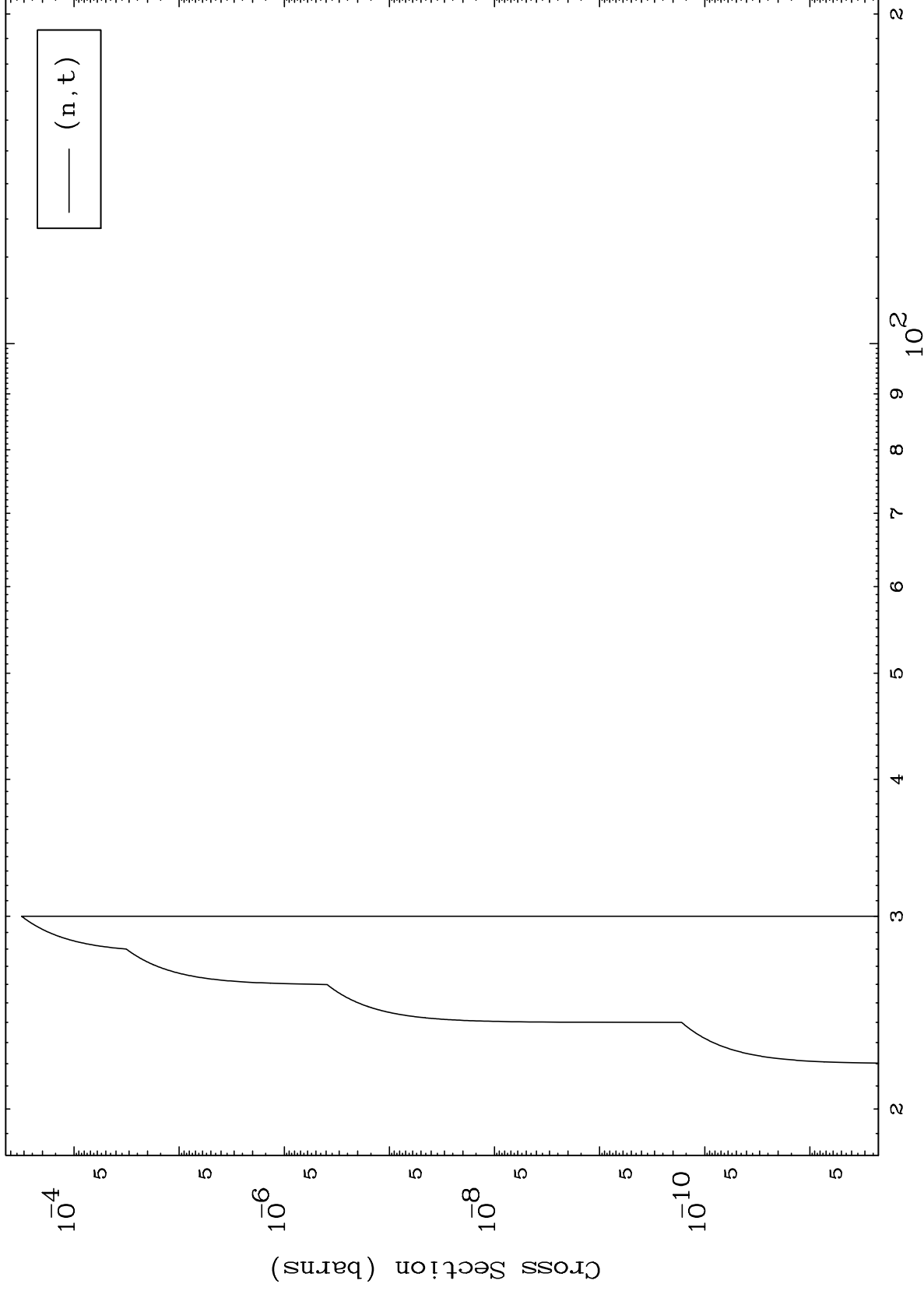
Incident Energy (MeV)

66-Dy-149

MAT 6604

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

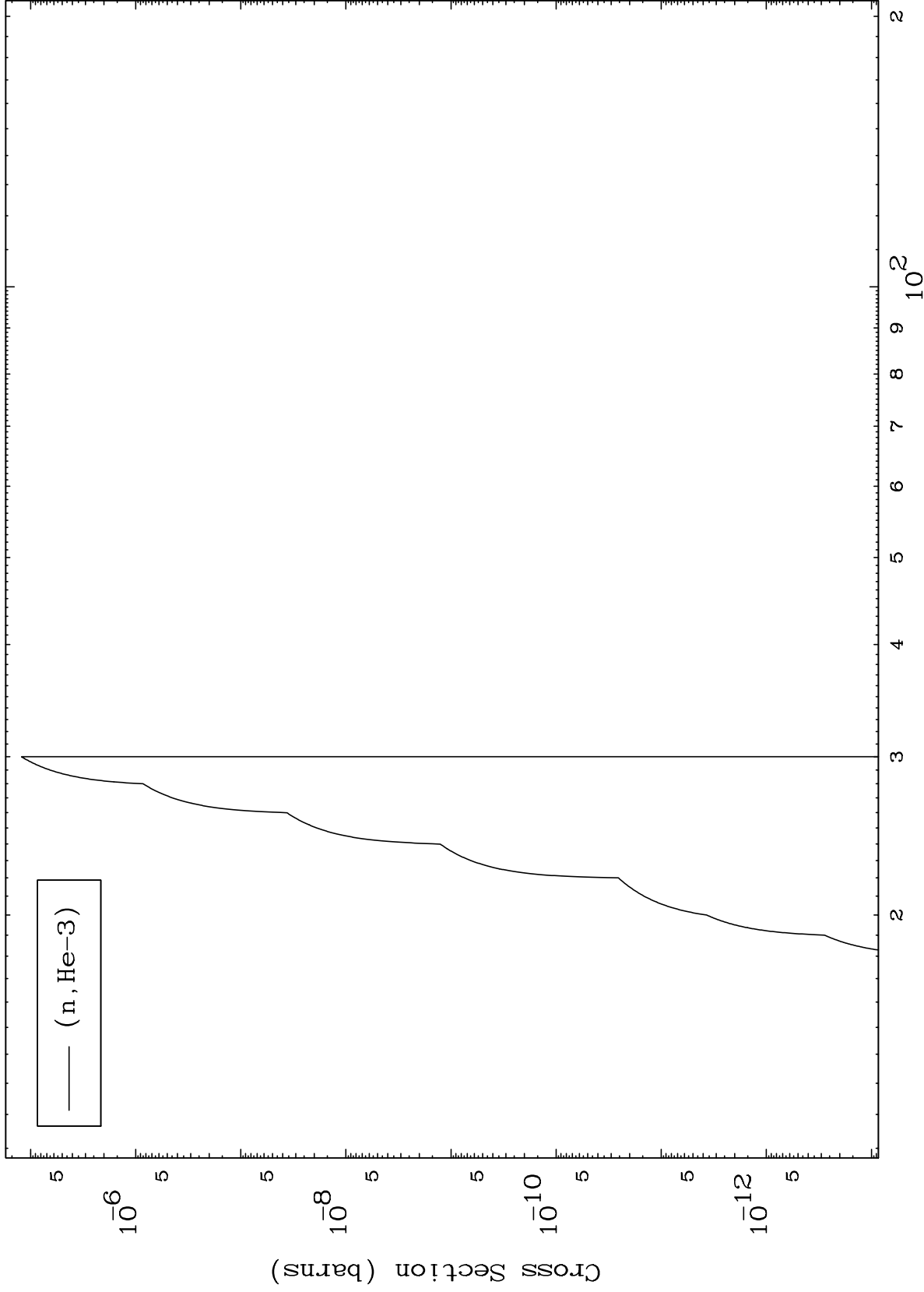
66-Dy-149



8

Incident Energy (MeV)

66-Dy-149

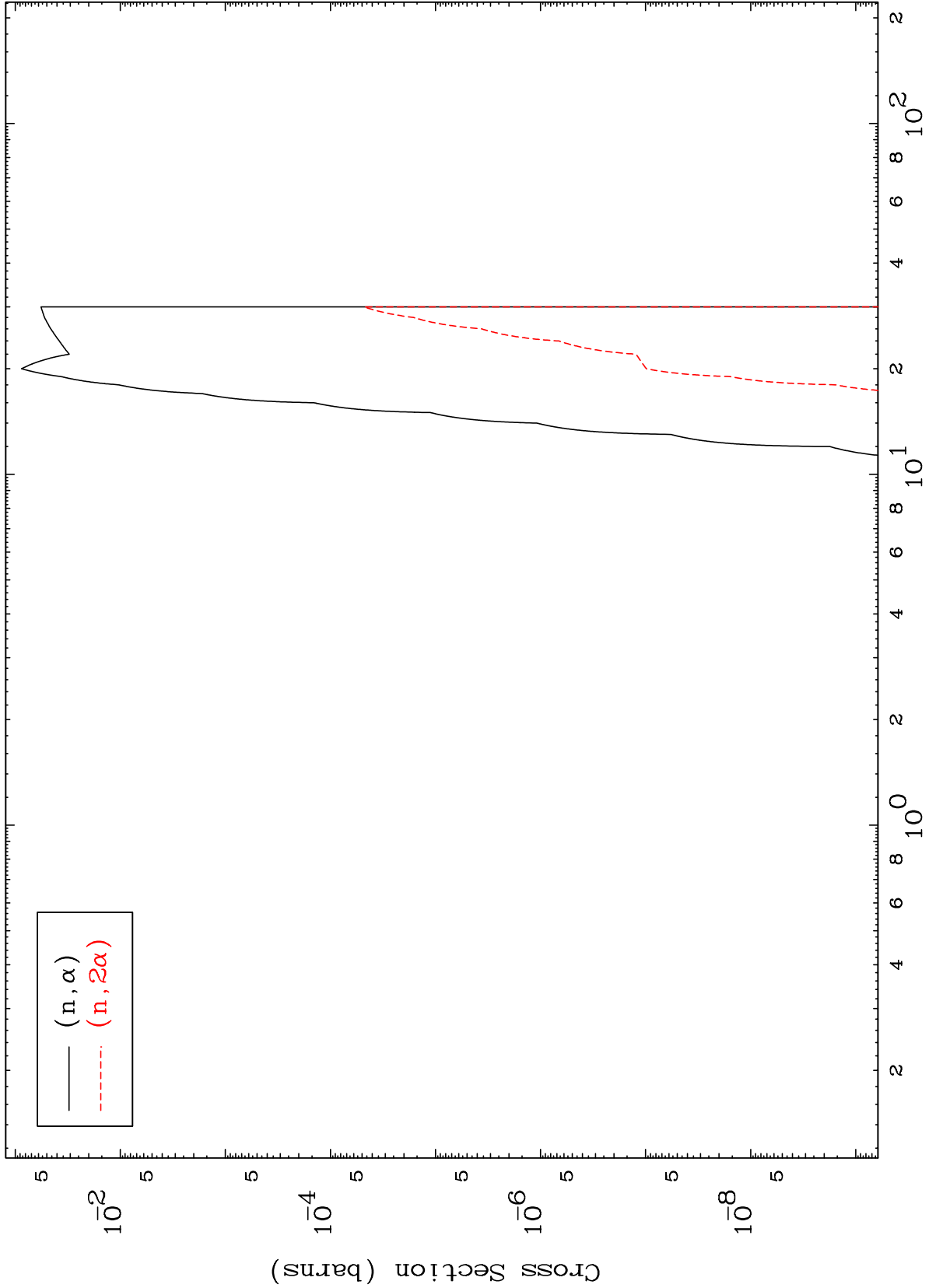


MAT 6604

( $\alpha, \alpha$ ) Levels

66-Dy-149

0 Kelvin Cross Sections



10

Incident Energy (MeV)

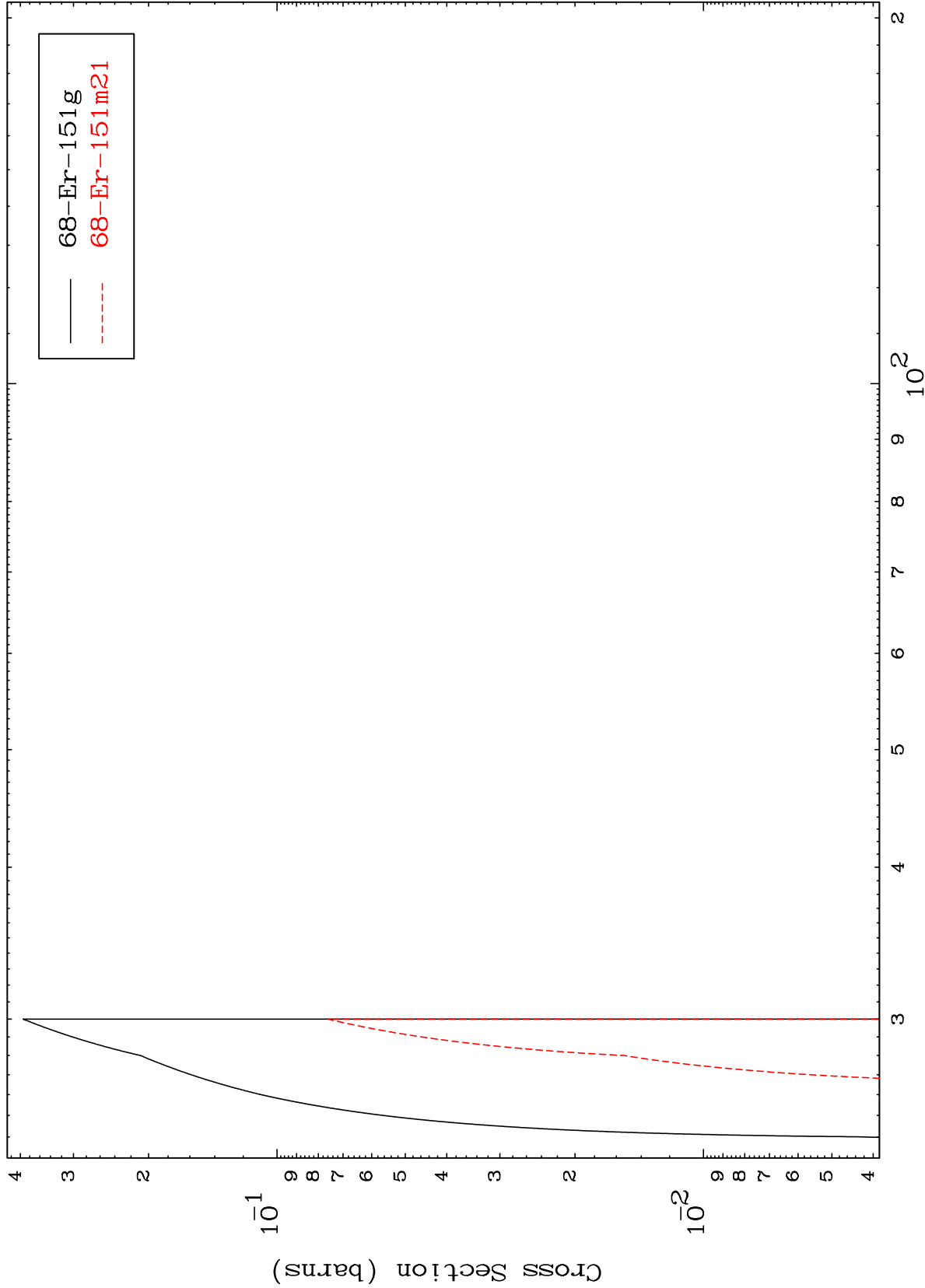
66-Dy-149

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(n,2n)

66-Dy-149

Radionuclide Production Cross Section

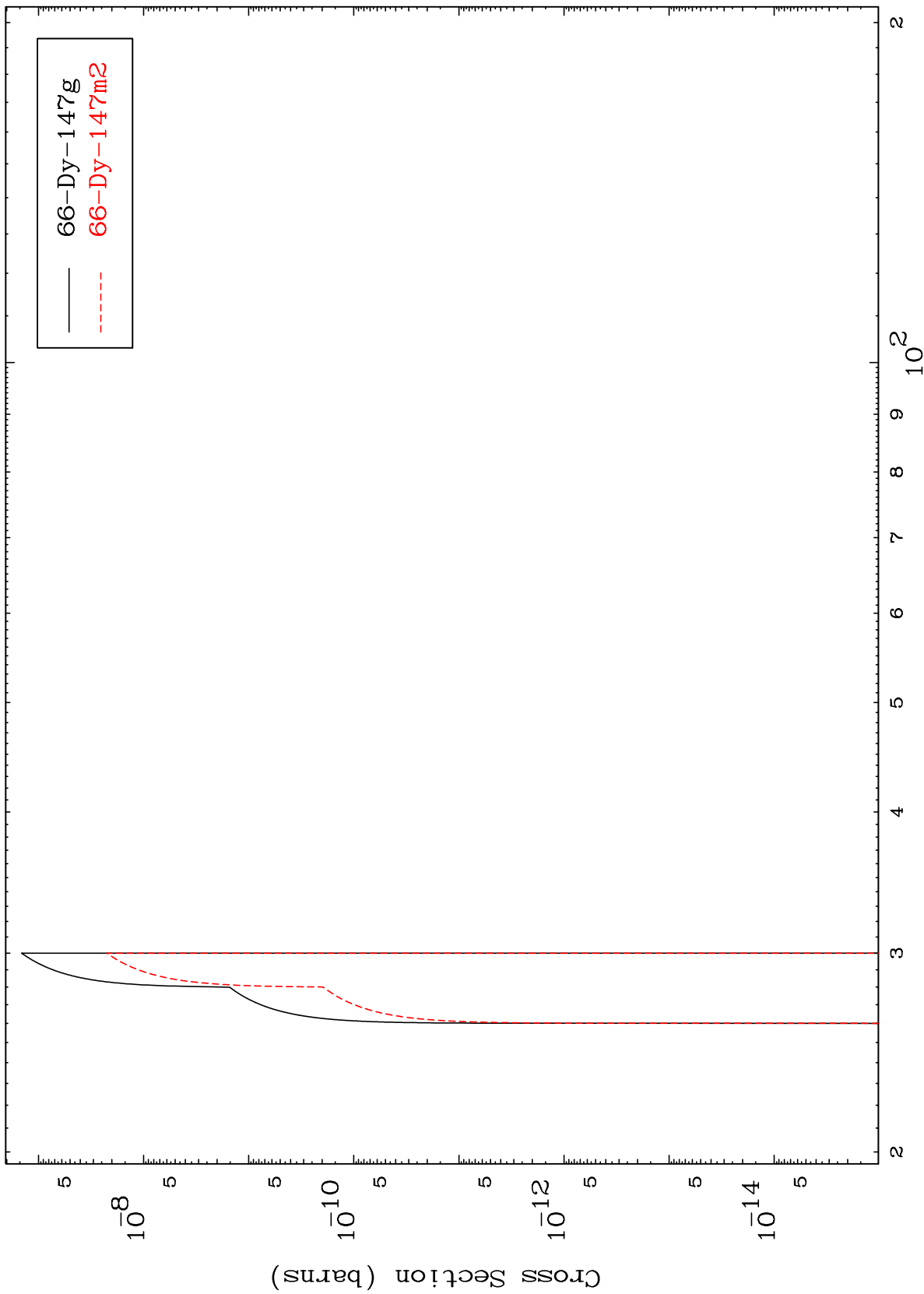


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(n,2n)  $\alpha$

66-Dy-149

Radionuclide Production Cross Section



12

Incident Energy (MeV)

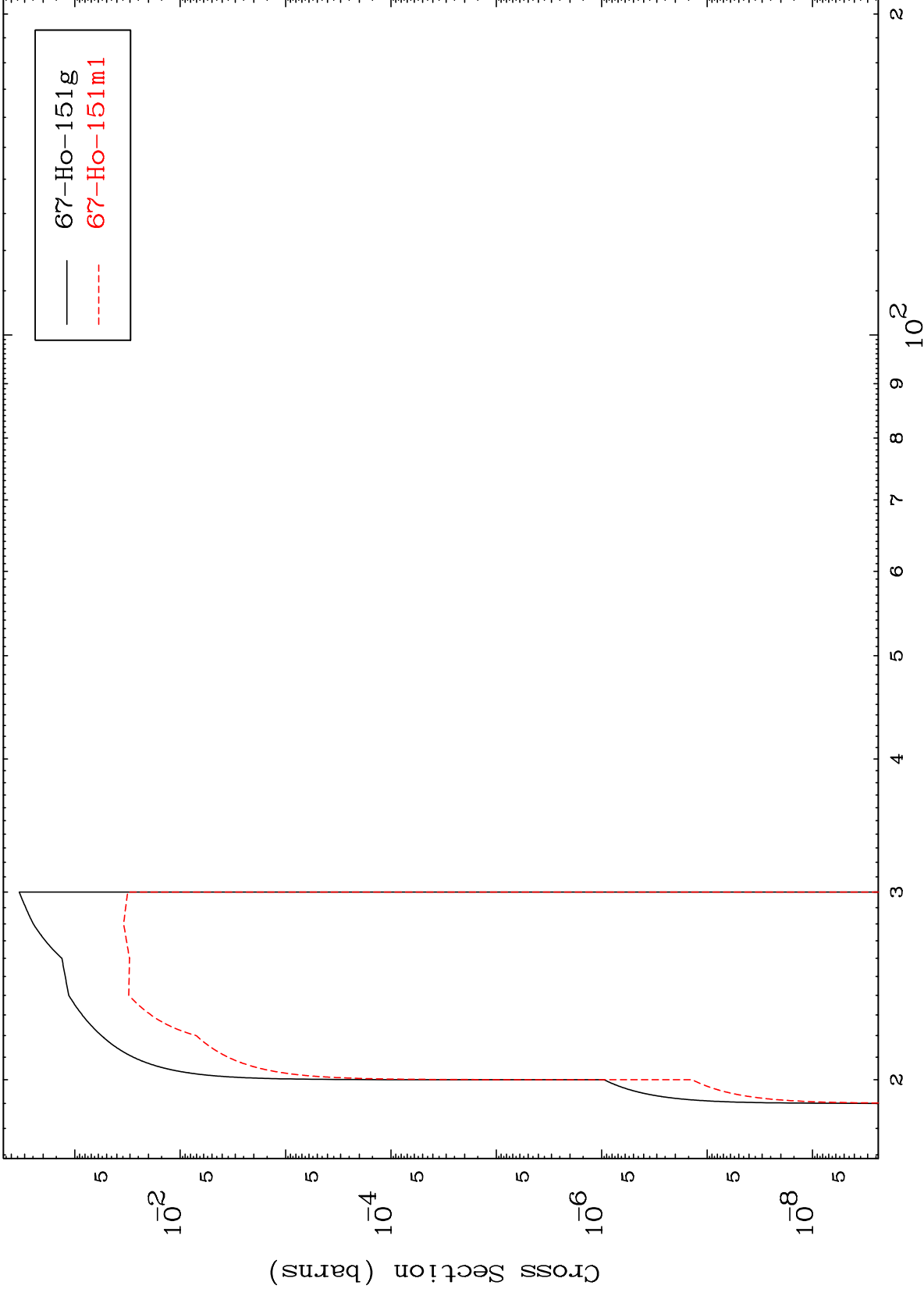
66-Dy-149

MAT 6604

(n,n') p

66-Dy-149

Radionuclide Production Cross Section



13

Incident Energy (MeV)

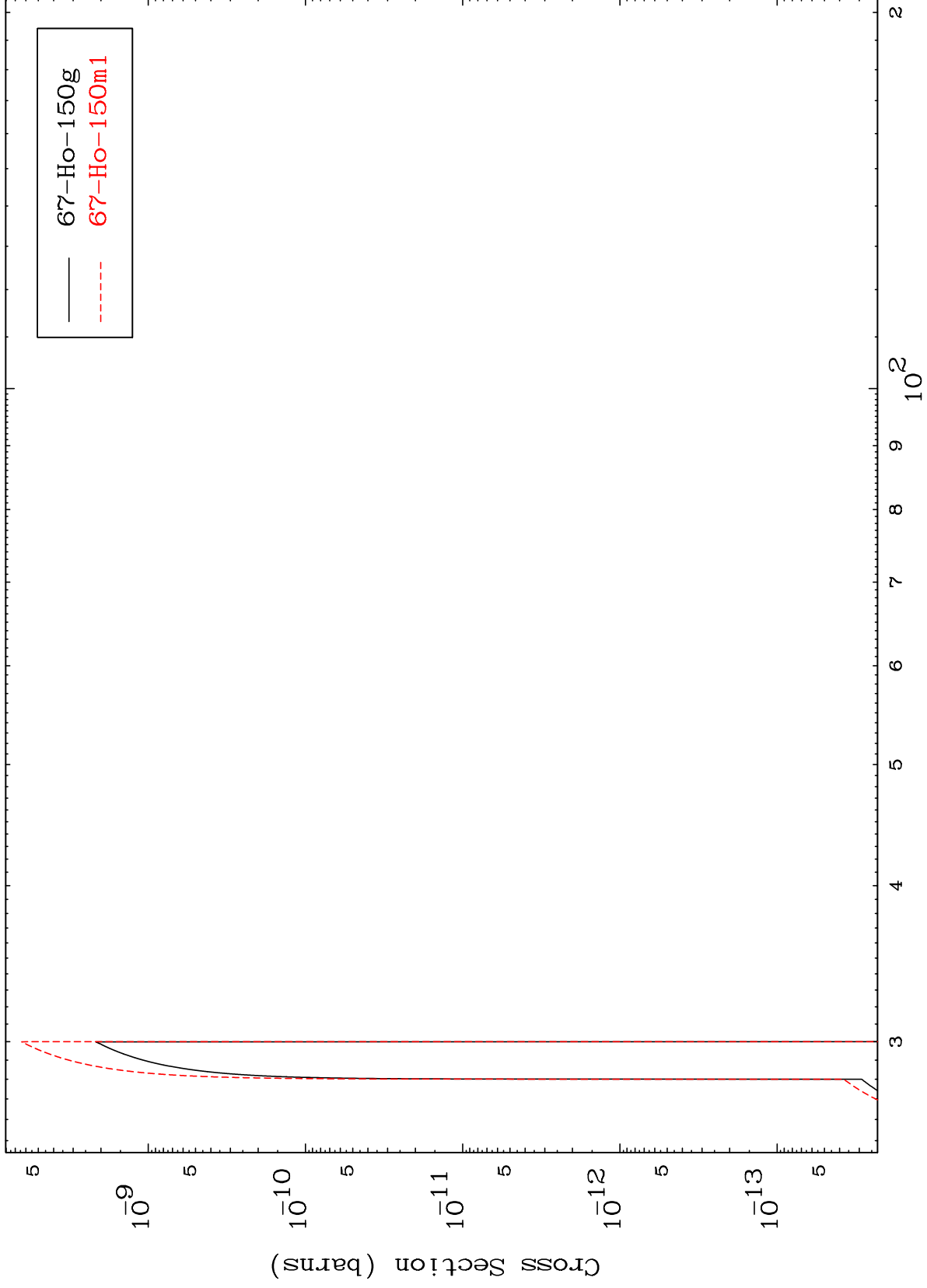
66-Dy-149

MAT 6604

(n,n') d

66-Dy-149

Radionuclide Production Cross Section



14

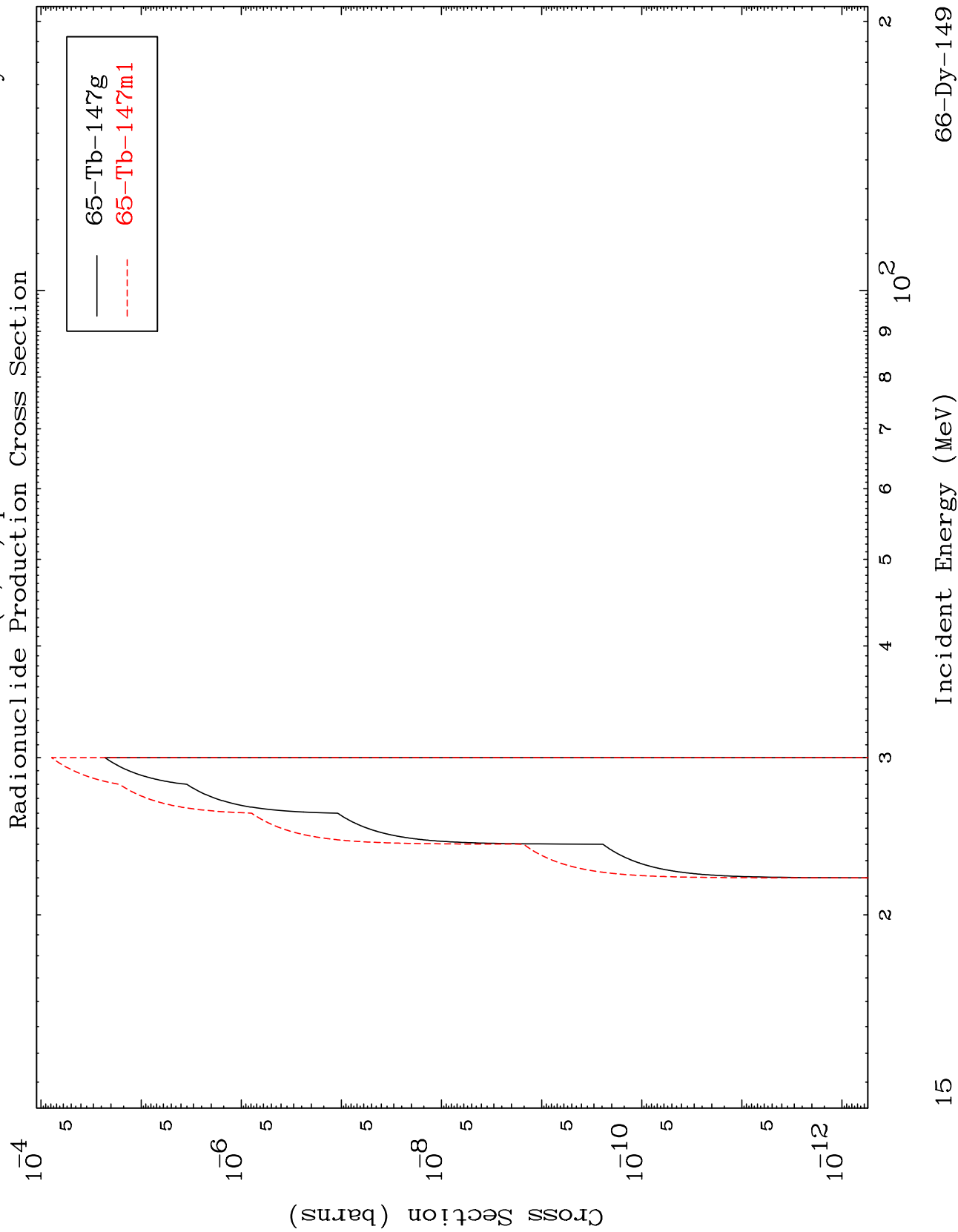
Incident Energy (MeV)

66-Dy-149

MAT 6604

(n,n') p  $\alpha$

66-Dy-149



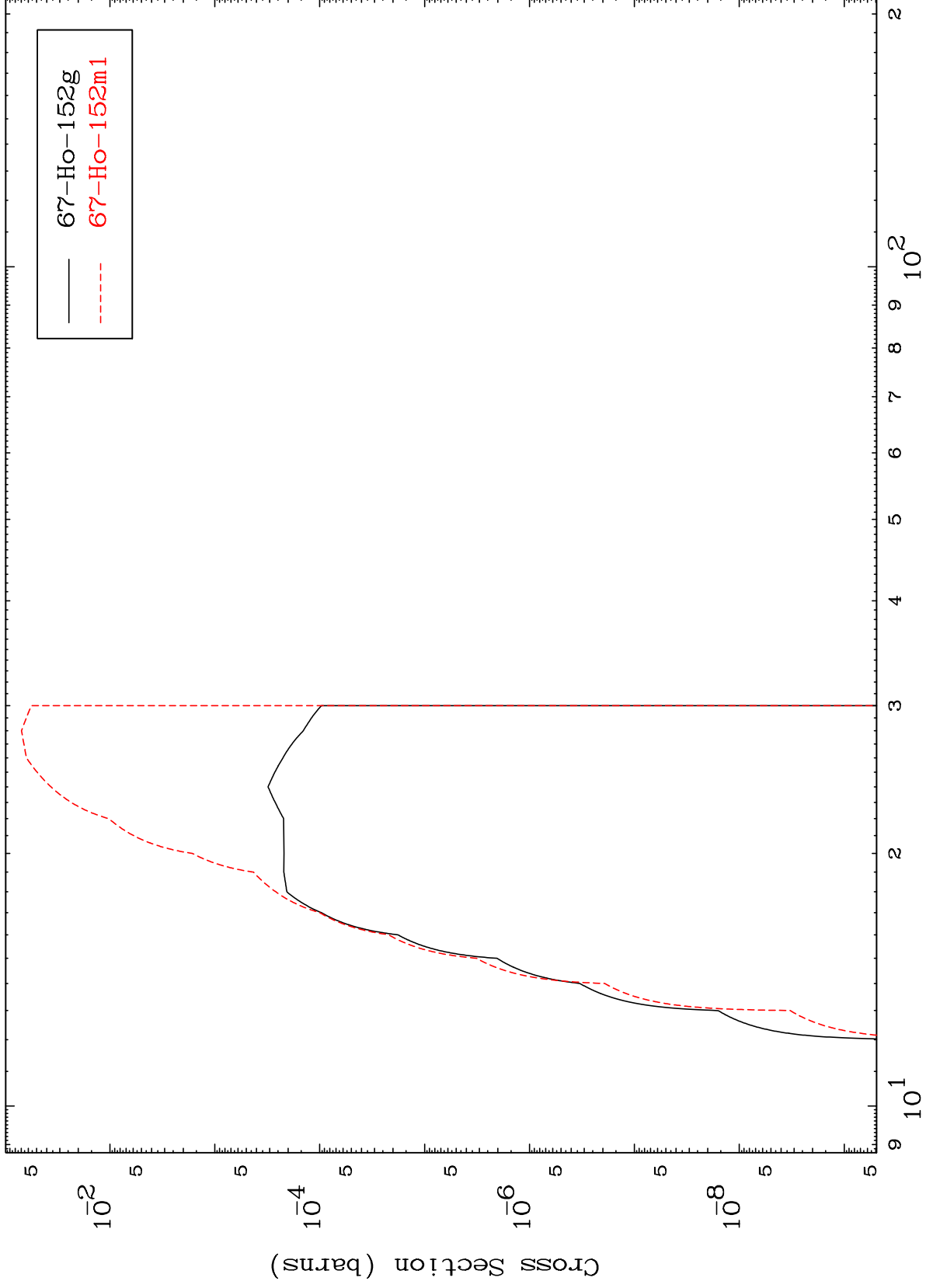
15

66-Dy-149

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66-Dy-149

(n,p)  
Radionuclide Production Cross Section



66-Dy-149

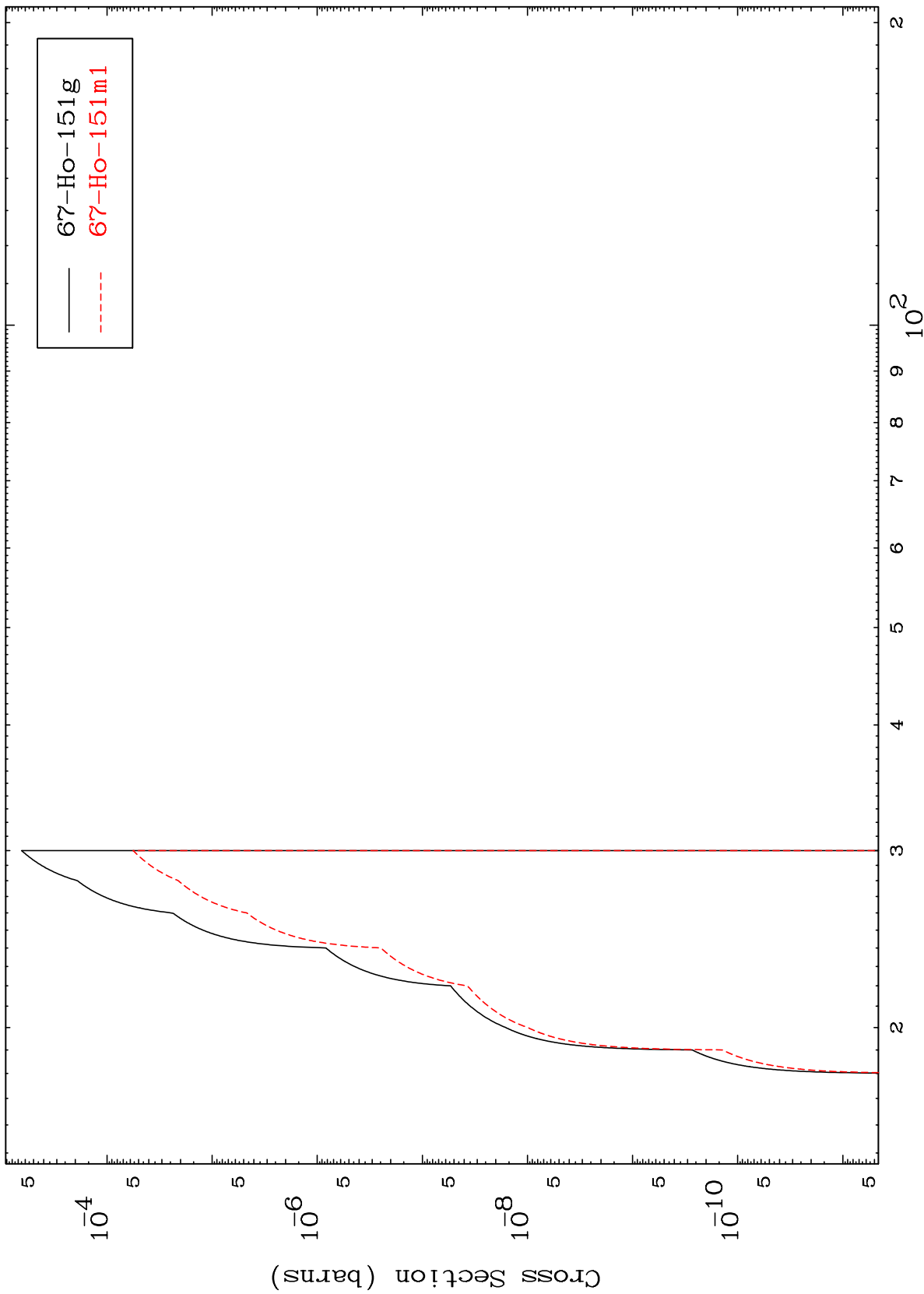
Incident Energy (MeV)

16

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66-Dy-149

(n,d)  
Radionuclide Production Cross Section

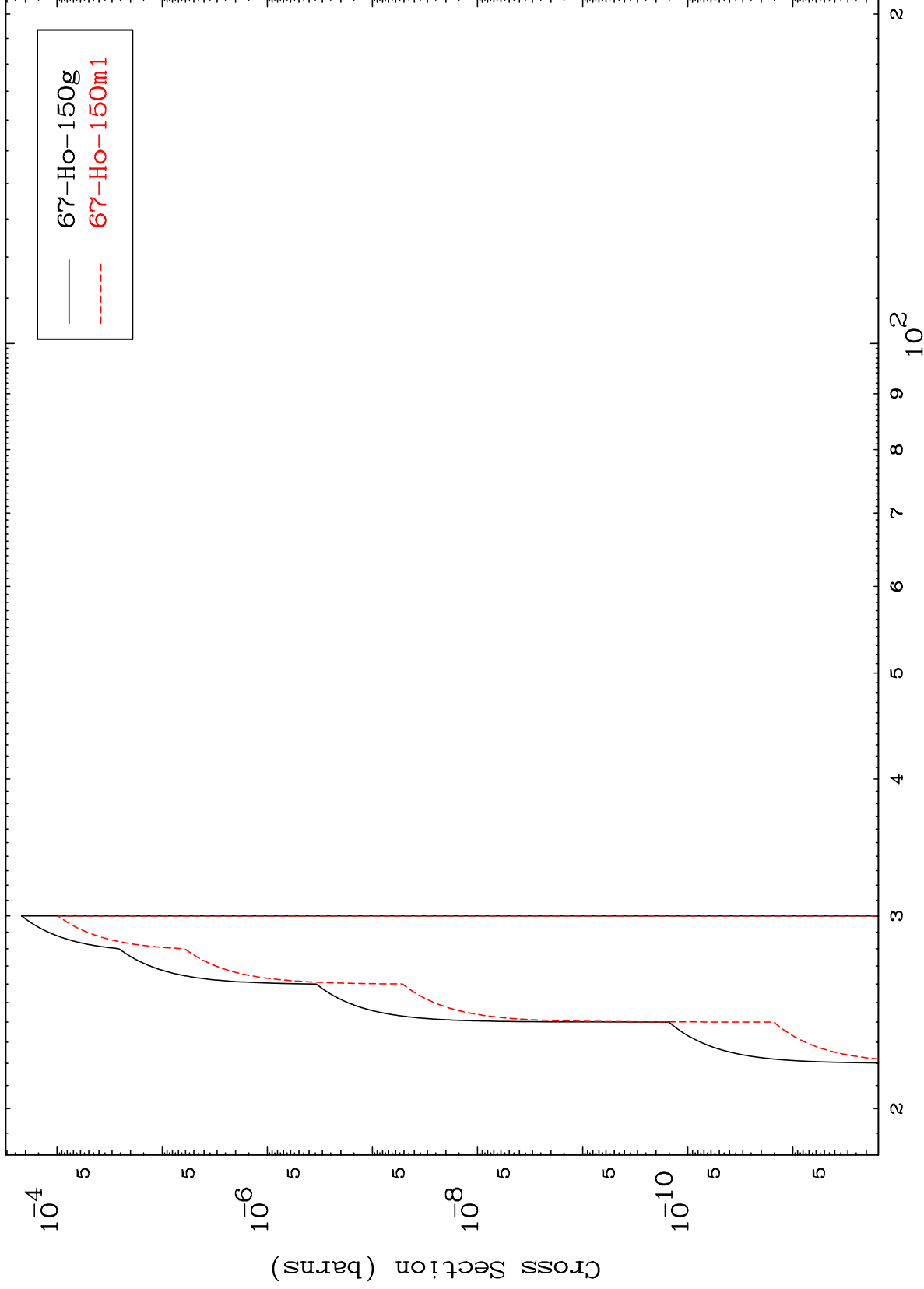


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(n, t)

66-Dy-149

Radionuclide Production Cross Section



18

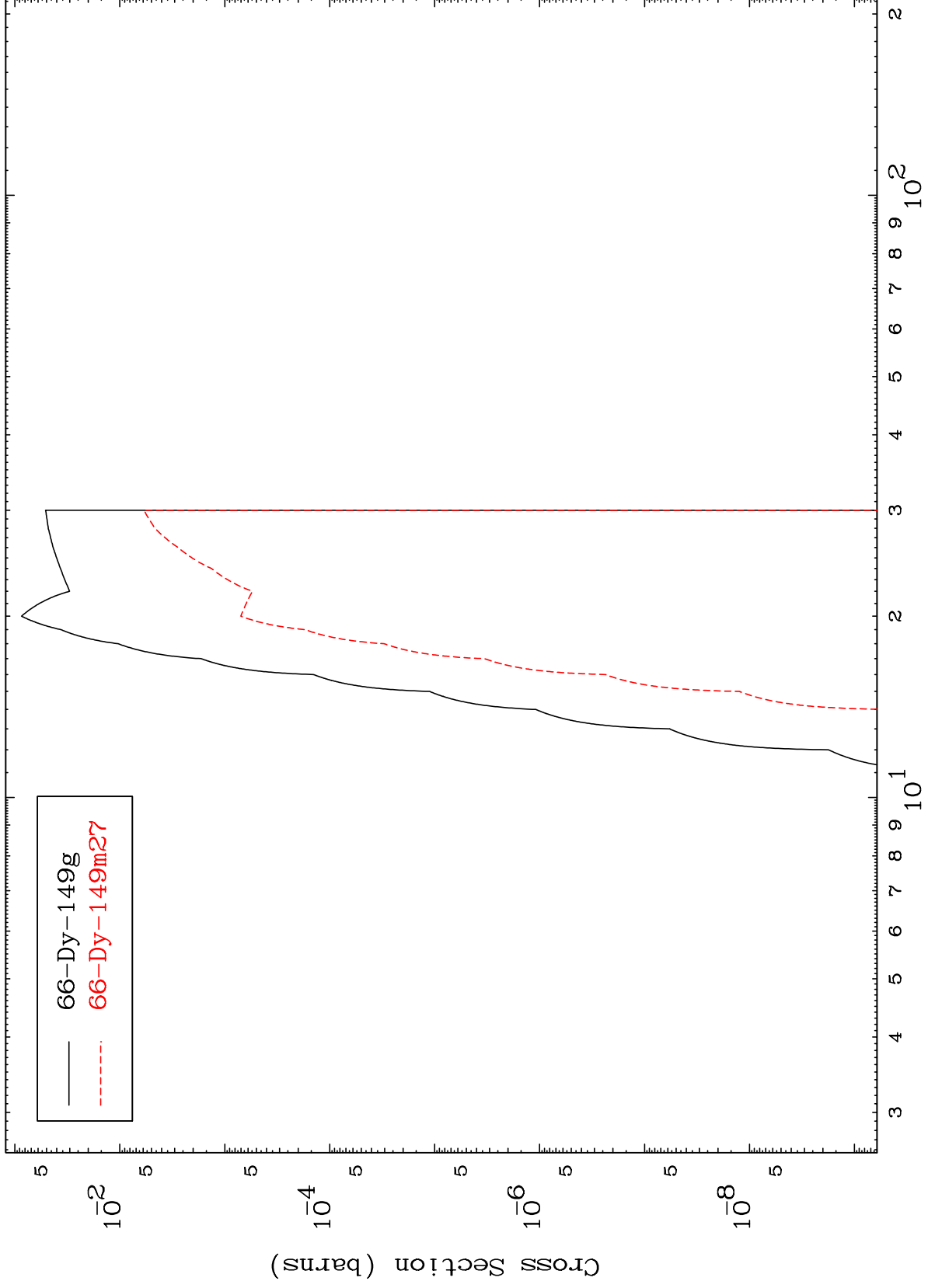
Incident Energy (MeV)

66-Dy-149

MAT 6604

66-Dy-149

(n,α)  
Radionuclide Production Cross Section



66-Dy-149g  
66-Dy-149m27

19

Incident Energy (MeV)

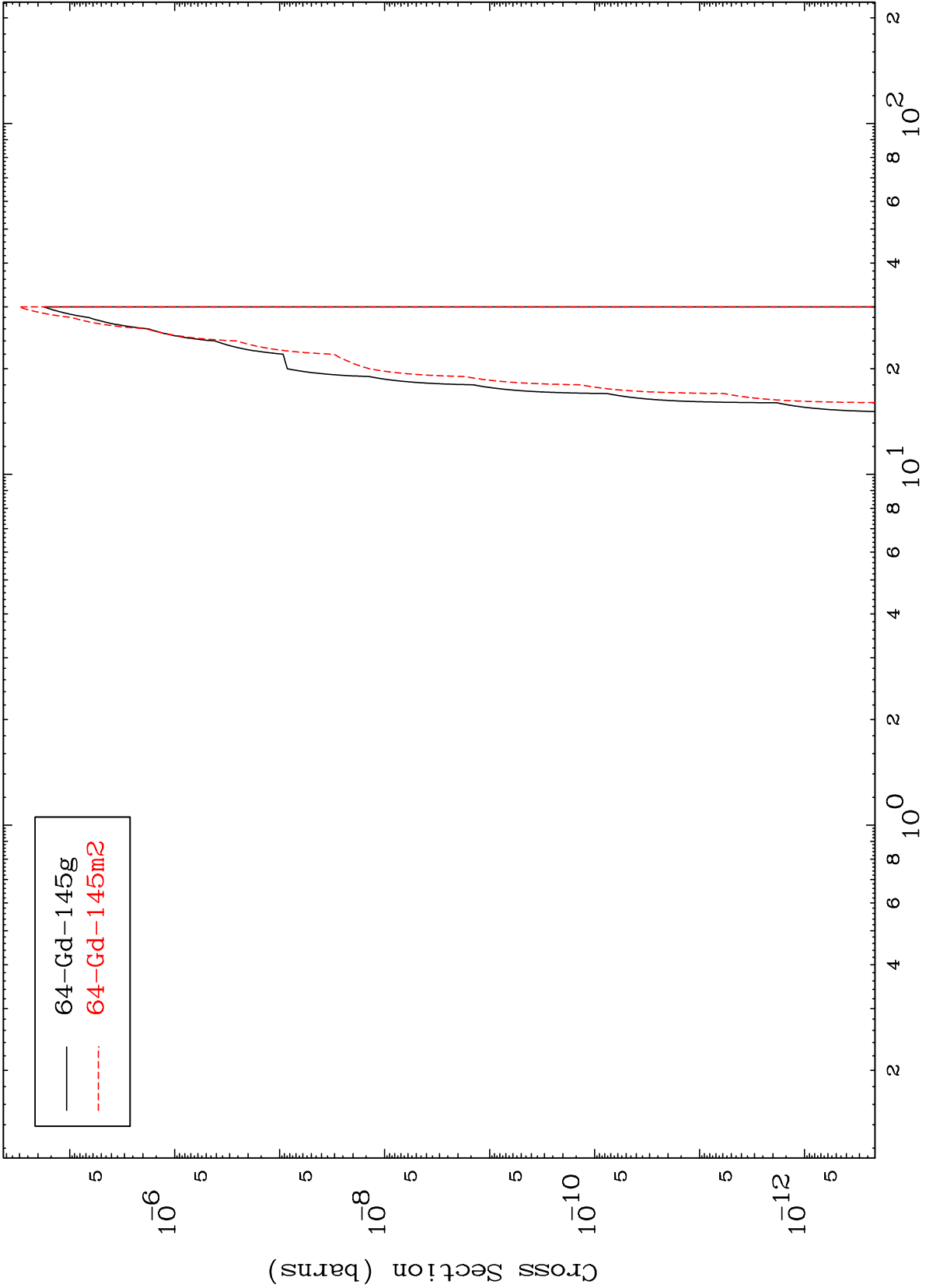
66-Dy-149

MAT 6604

(n,2α)

66-Dy-149

Radionuclide Production Cross Section

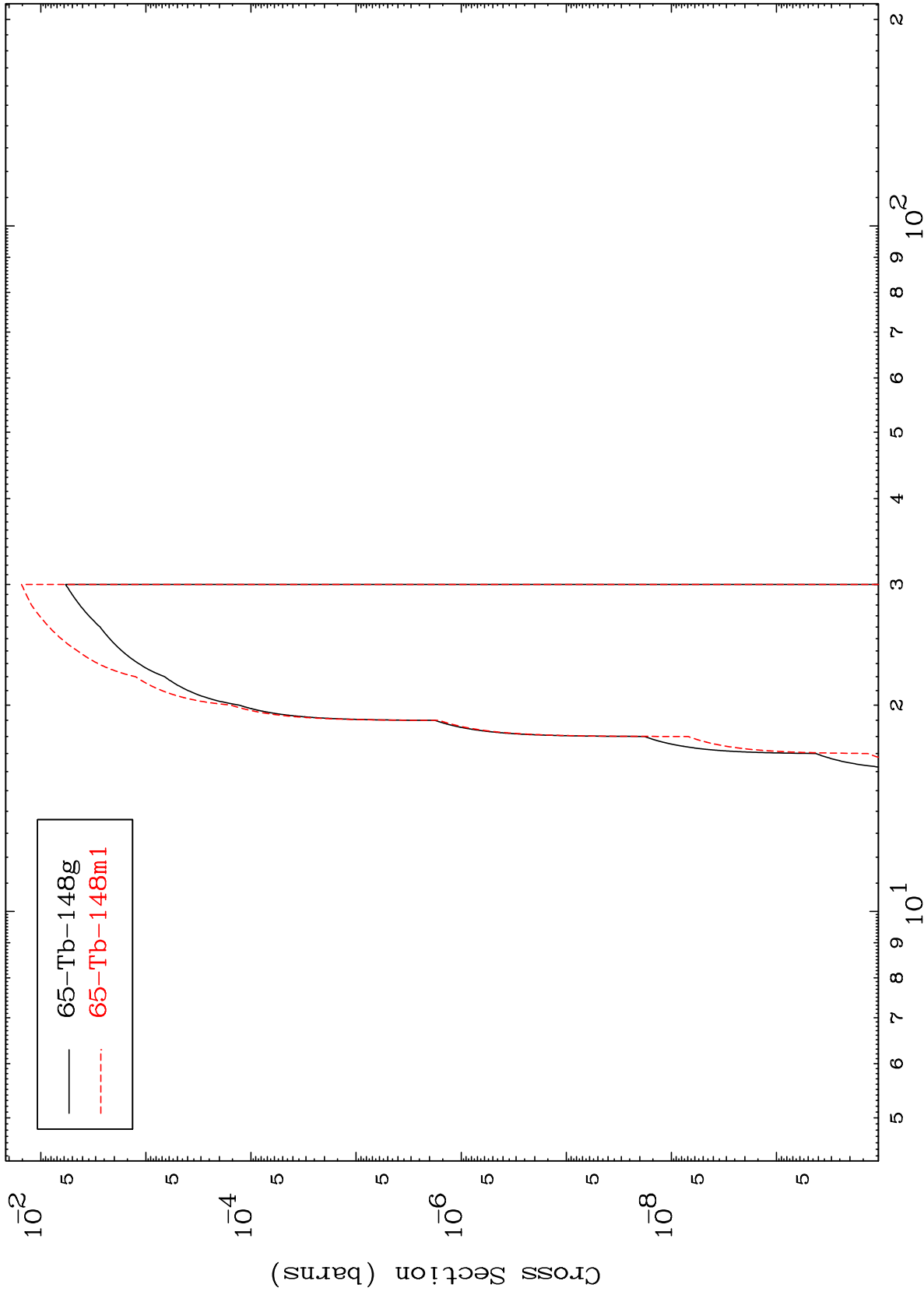


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(n,p)  $\alpha$

66-Dy-149

Radionuclide Production Cross Section

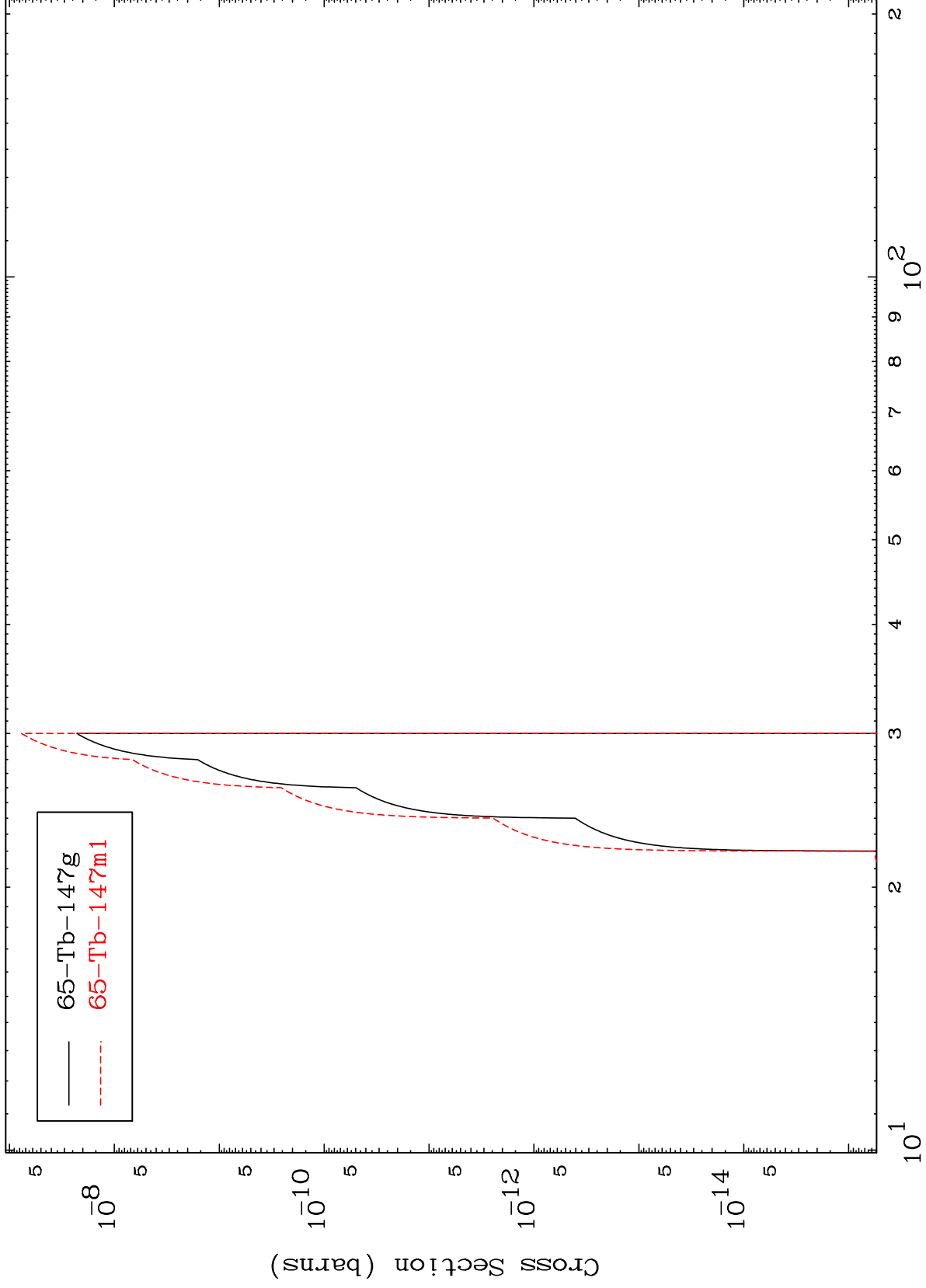


MAT 6604

(n,d)  $\alpha$

66-Dy-149

Radionuclide Production Cross Section



22

Incident Energy (MeV)

66-Dy-149