

Program EVALPLOT  
(Version 2021-1)

by

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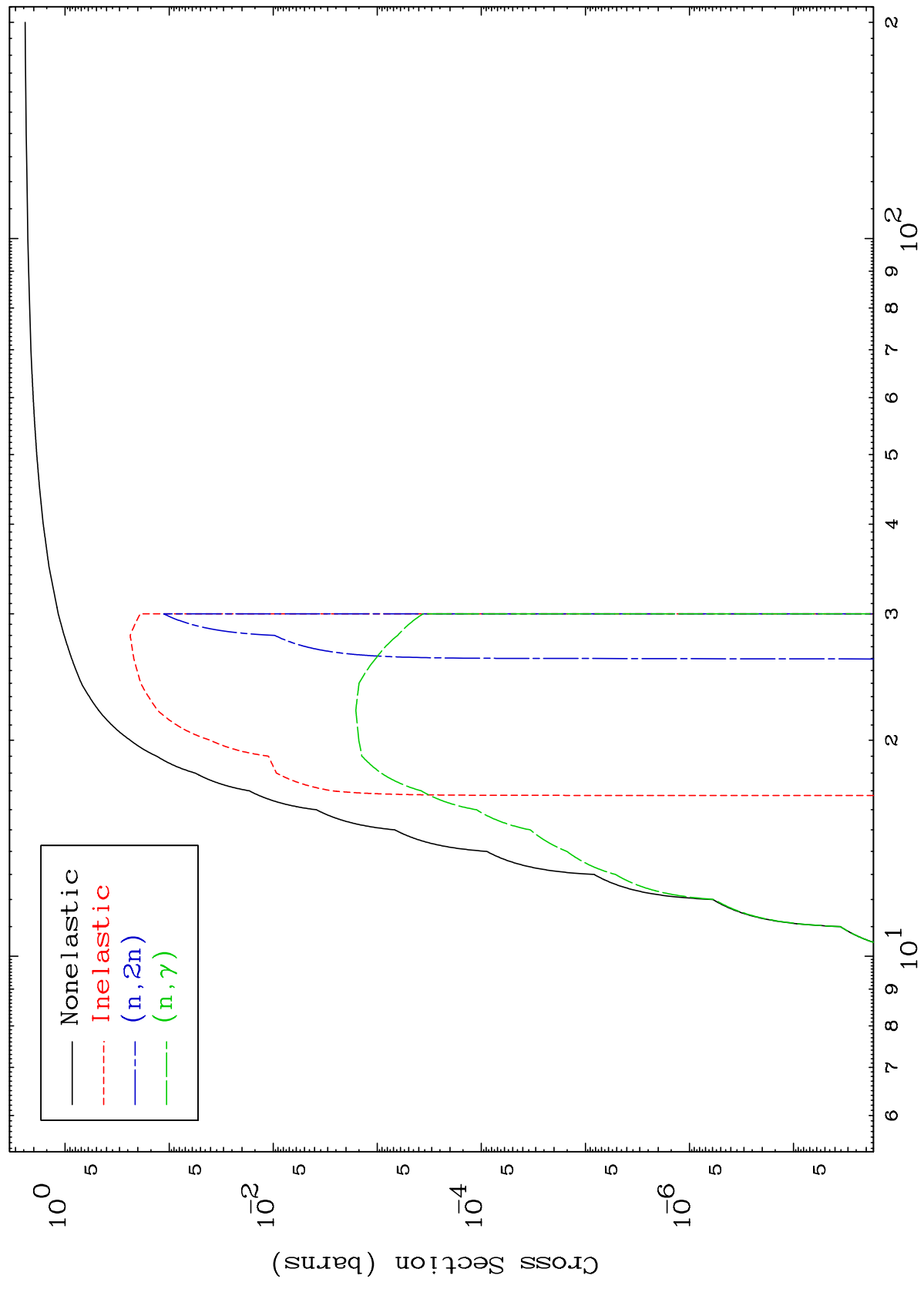
Press Mouse Button to Start

MAT 6789

0 Kelvin

$\alpha$  Major Cross Sections

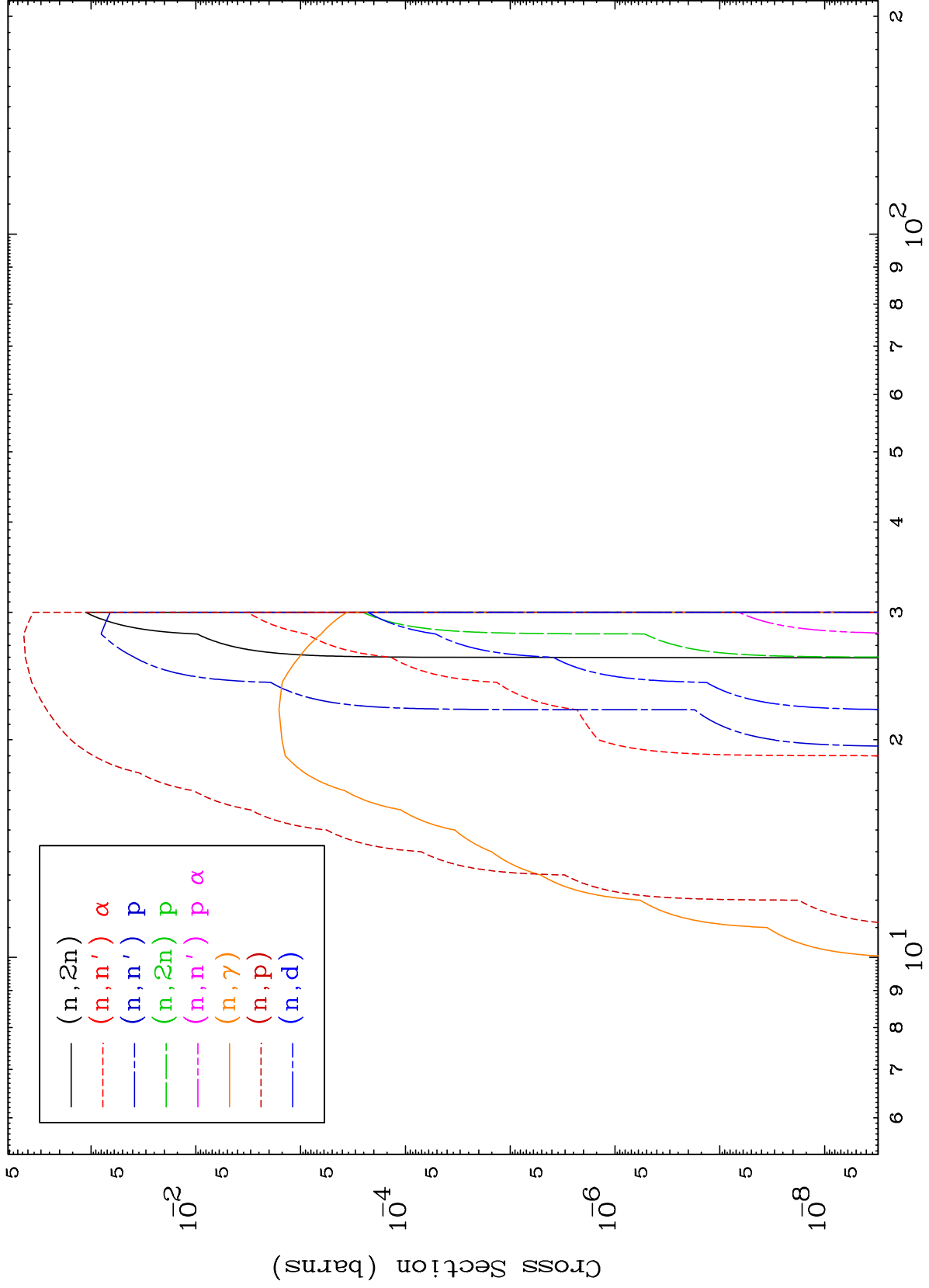
68-Er-150



1

Incident Energy (MeV)

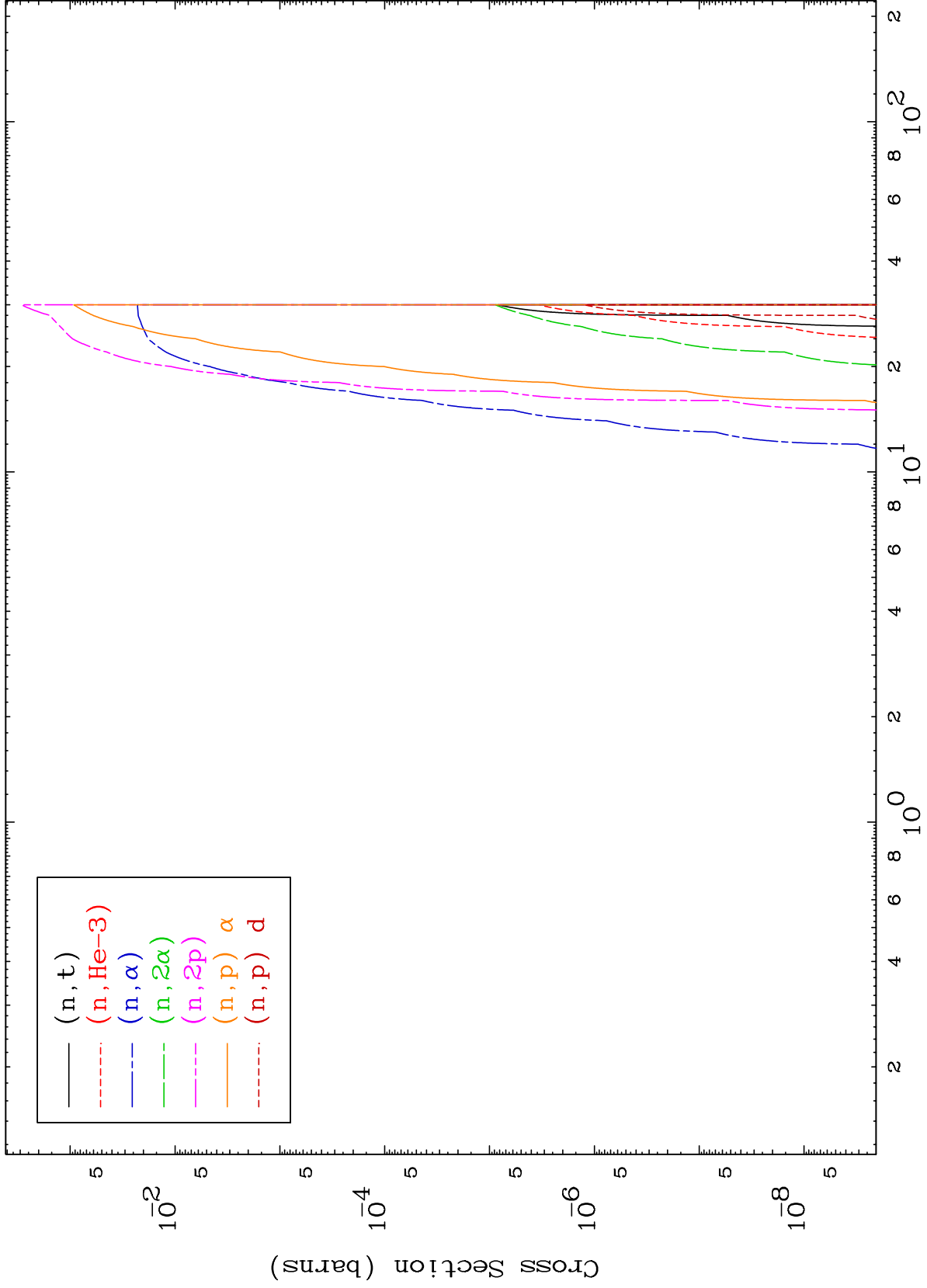
68-Er-150

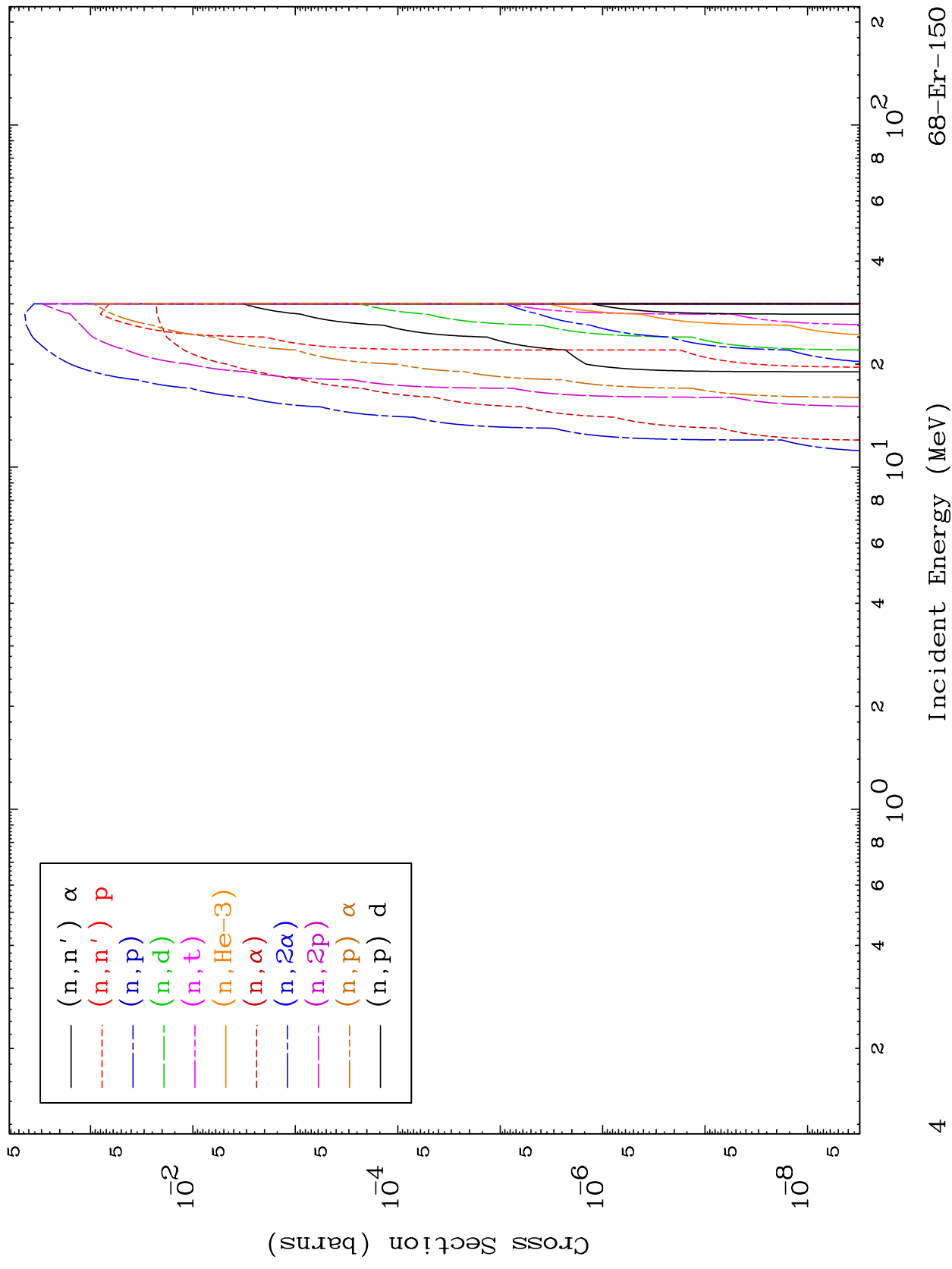


MAT 6789

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

68-Er-150

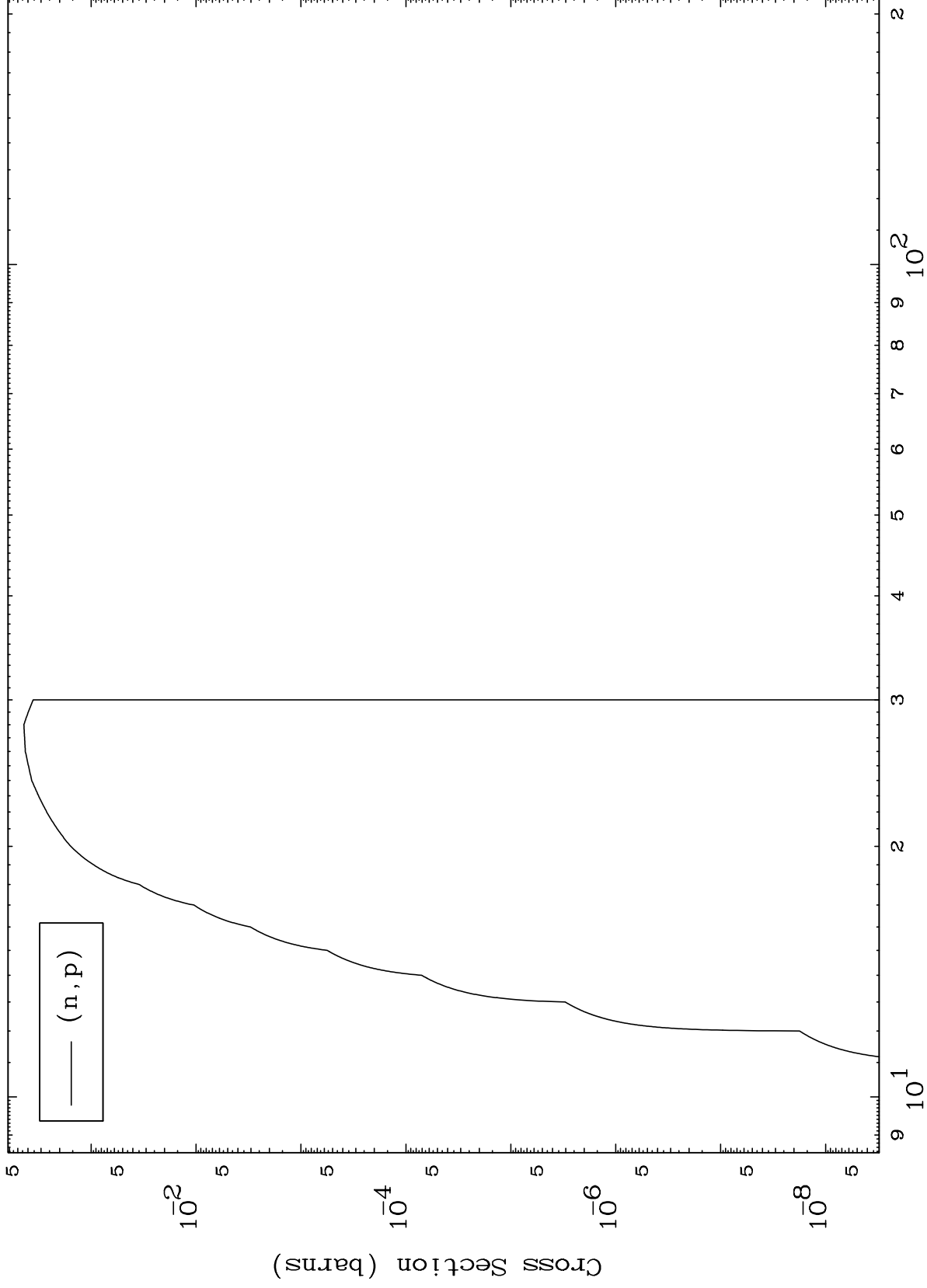




MAT 6789

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

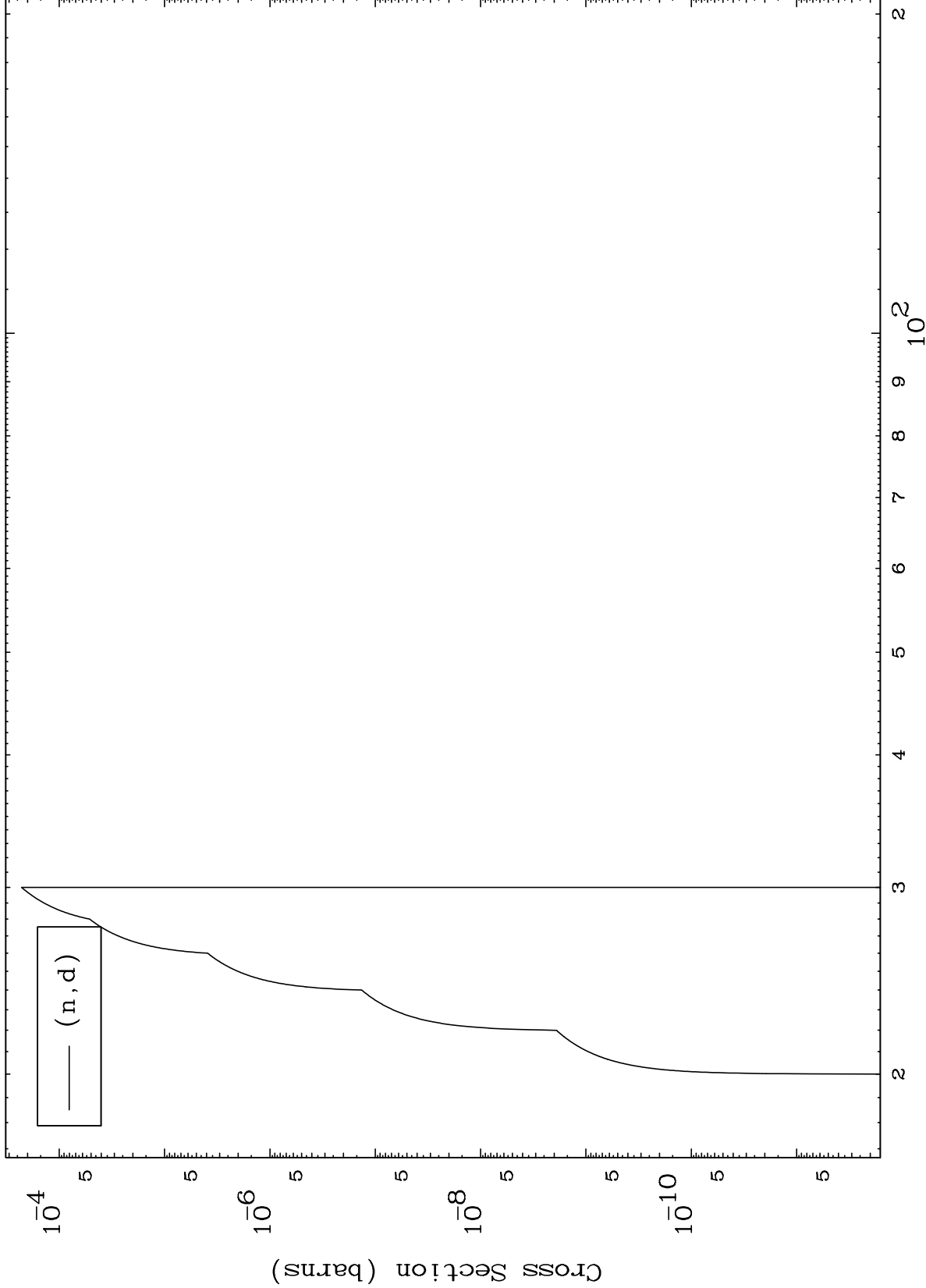
68-Er-150



Incident Energy (MeV)

68-Er-150

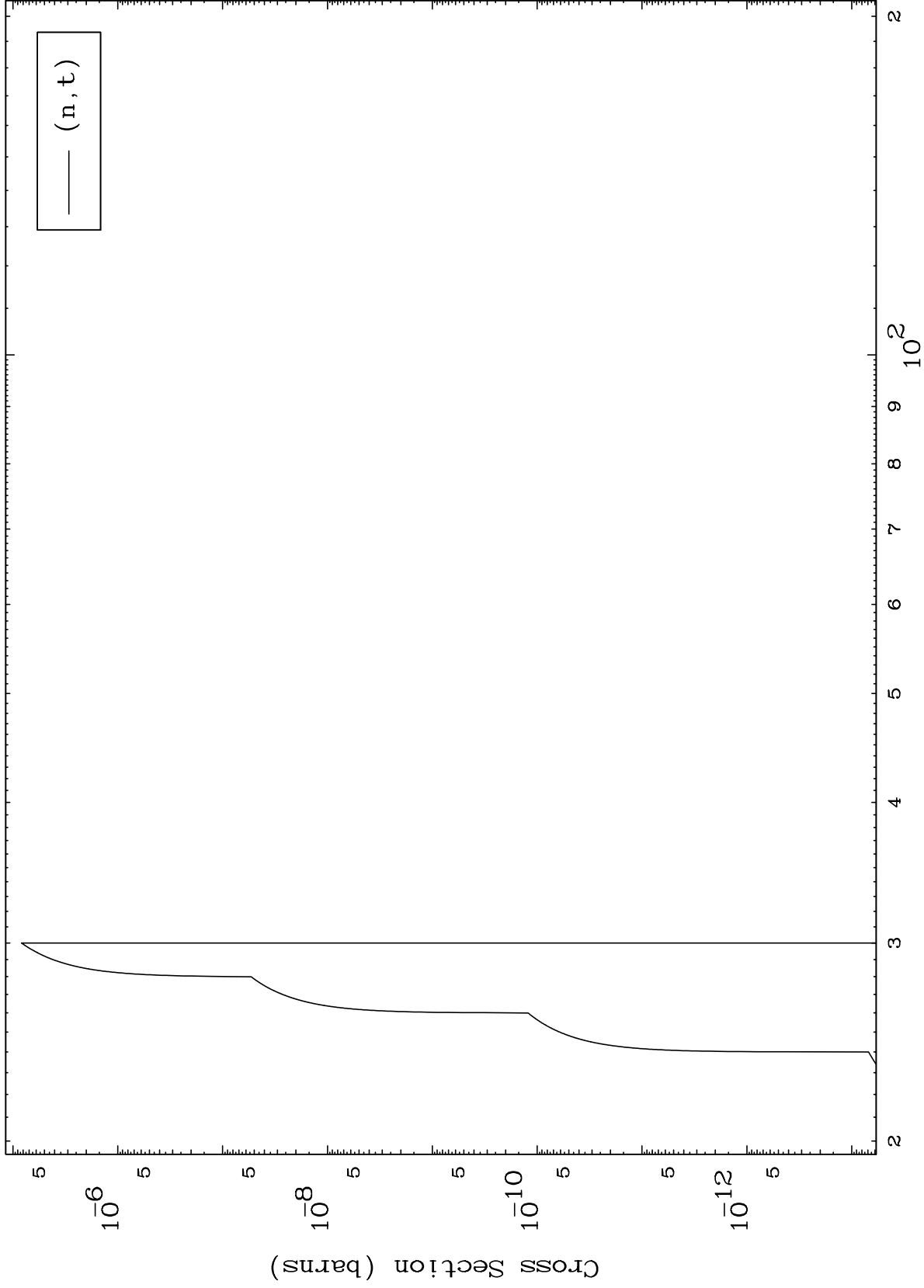
5



MAT 6789

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

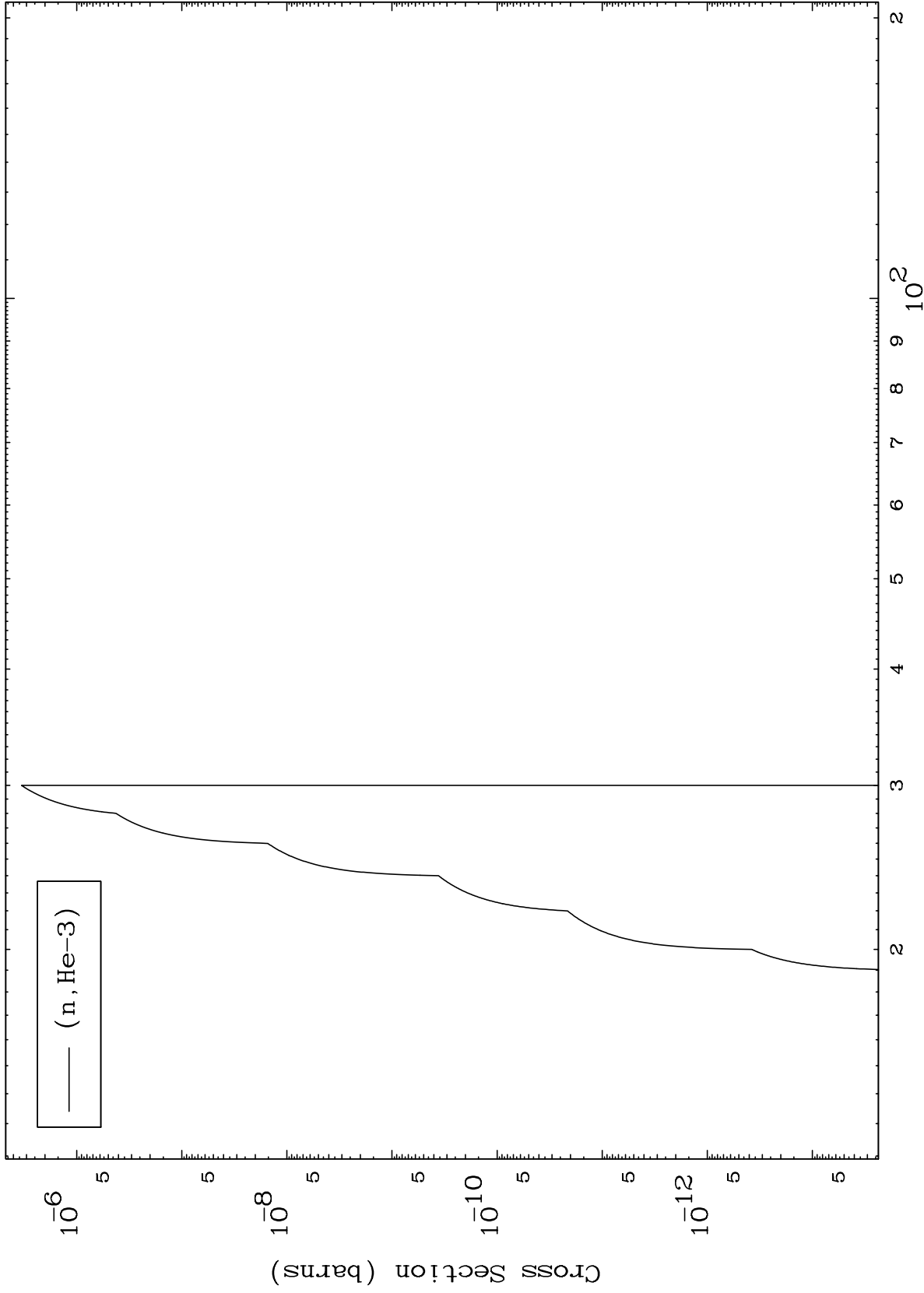
68-Er-150



7

Incident Energy (MeV)

68-Er-150

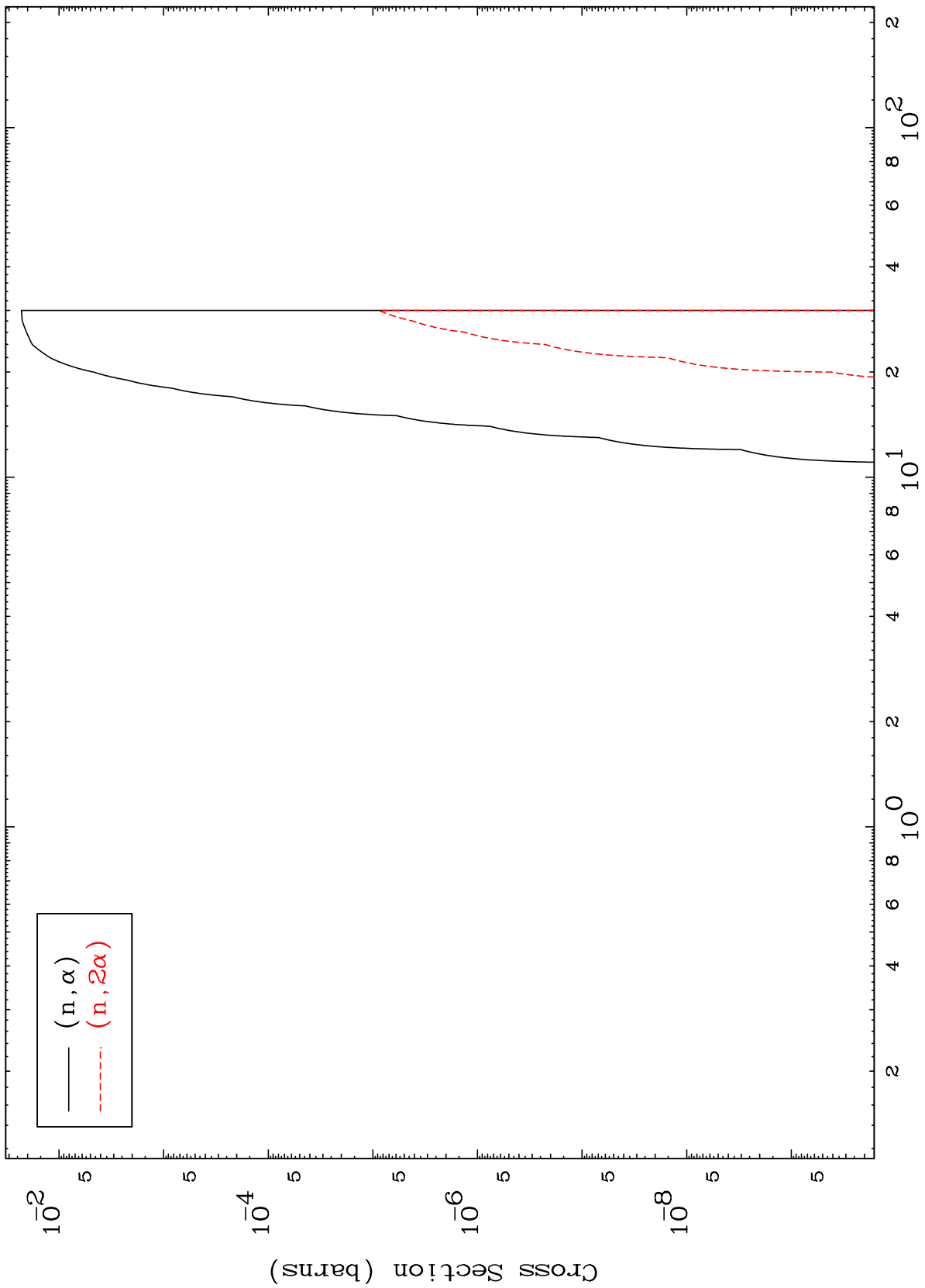


MAT 6789

( $\alpha, \alpha$ ) Levels

68-Er-150

0 Kelvin Cross Sections

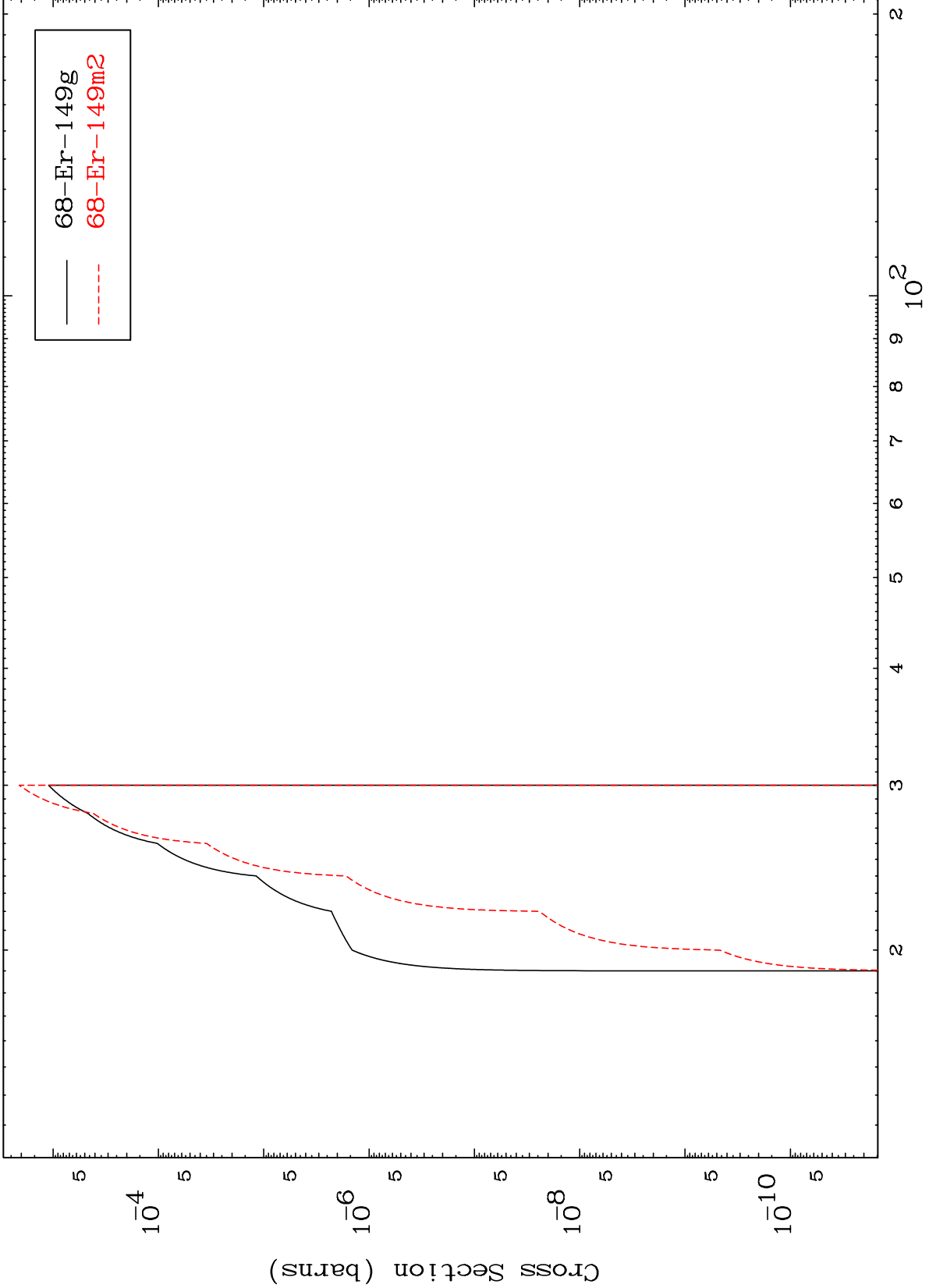


MAT 6789

$(n, n') \alpha$

68-Er-150

Radionuclide Production Cross Section

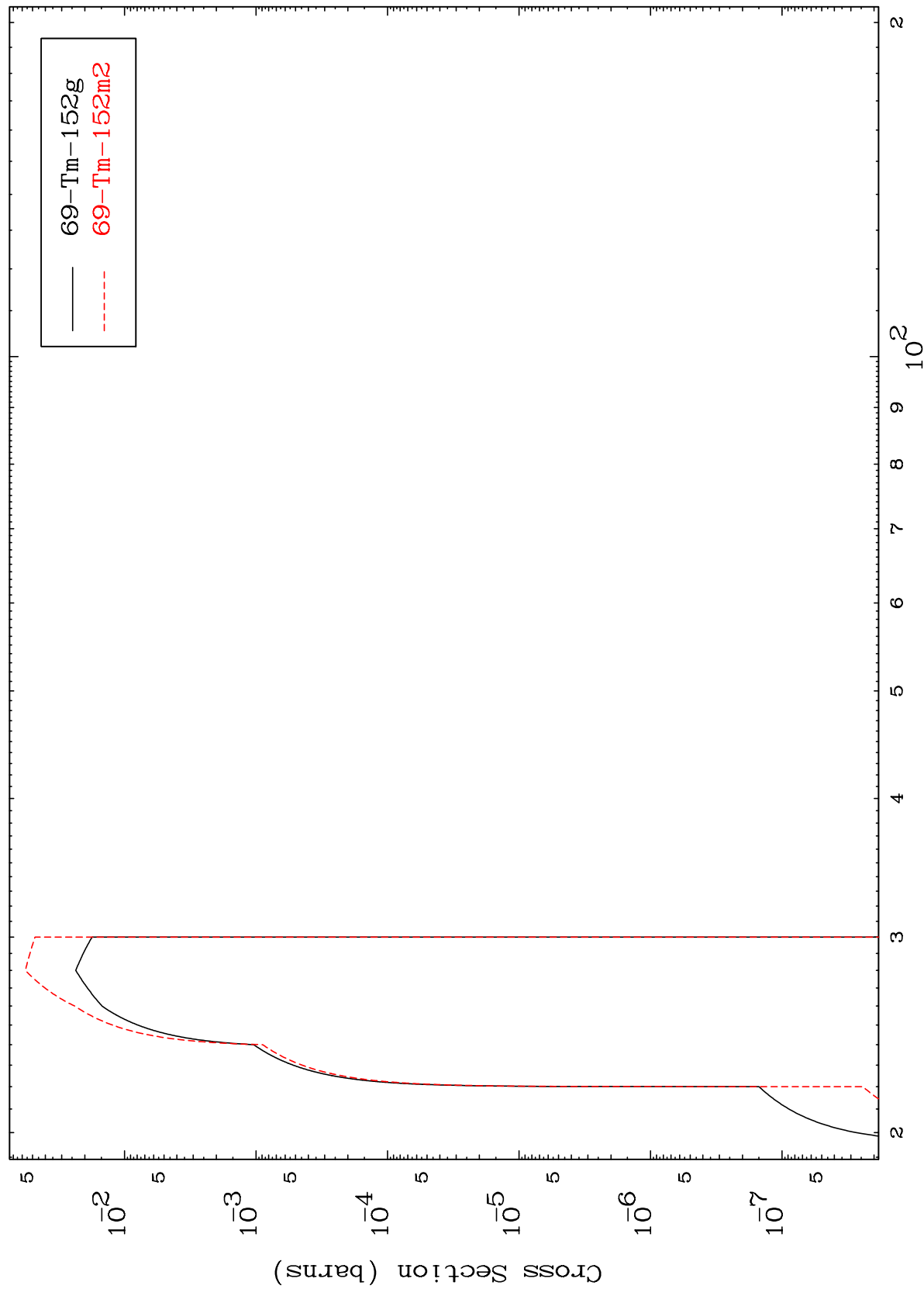


10

Incident Energy (MeV)

68-Er-150

Radionuclide Production Cross Section



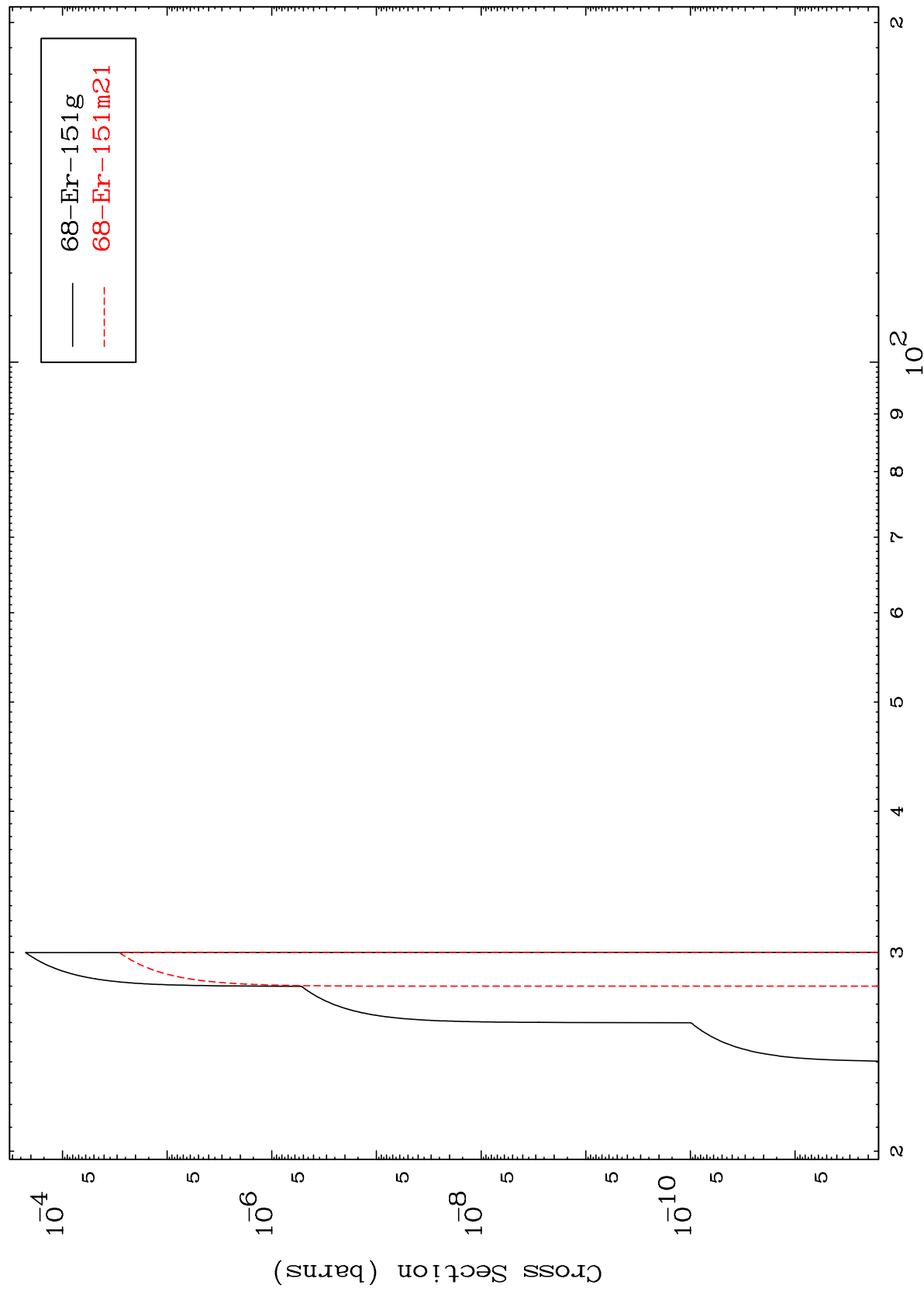
69-Tm-152g  
69-Tm-152m2

MAT 6789

(n,2n) p

68-Er-150

Radionuclide Production Cross Section

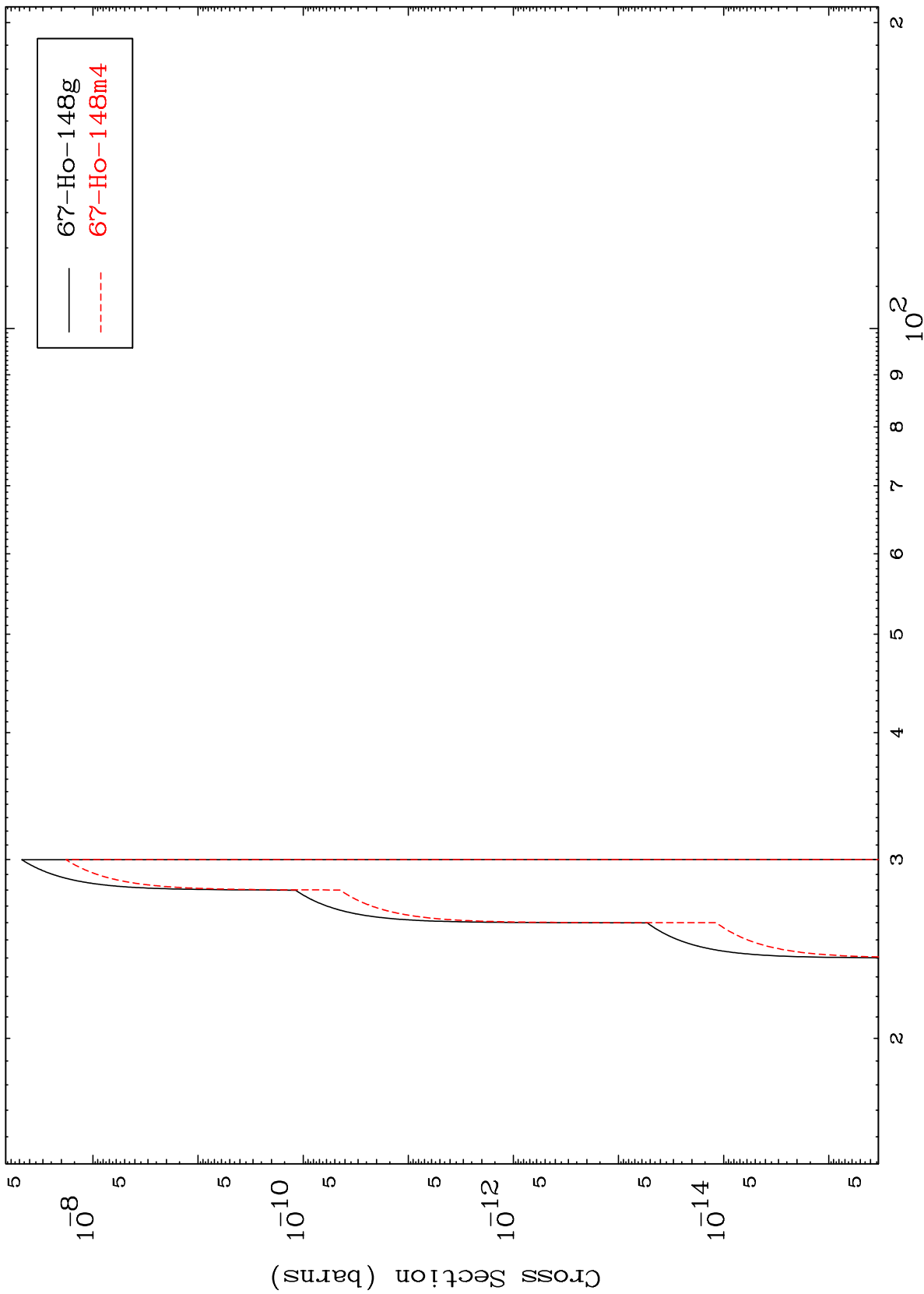


12

Incident Energy (MeV)

68-Er-150

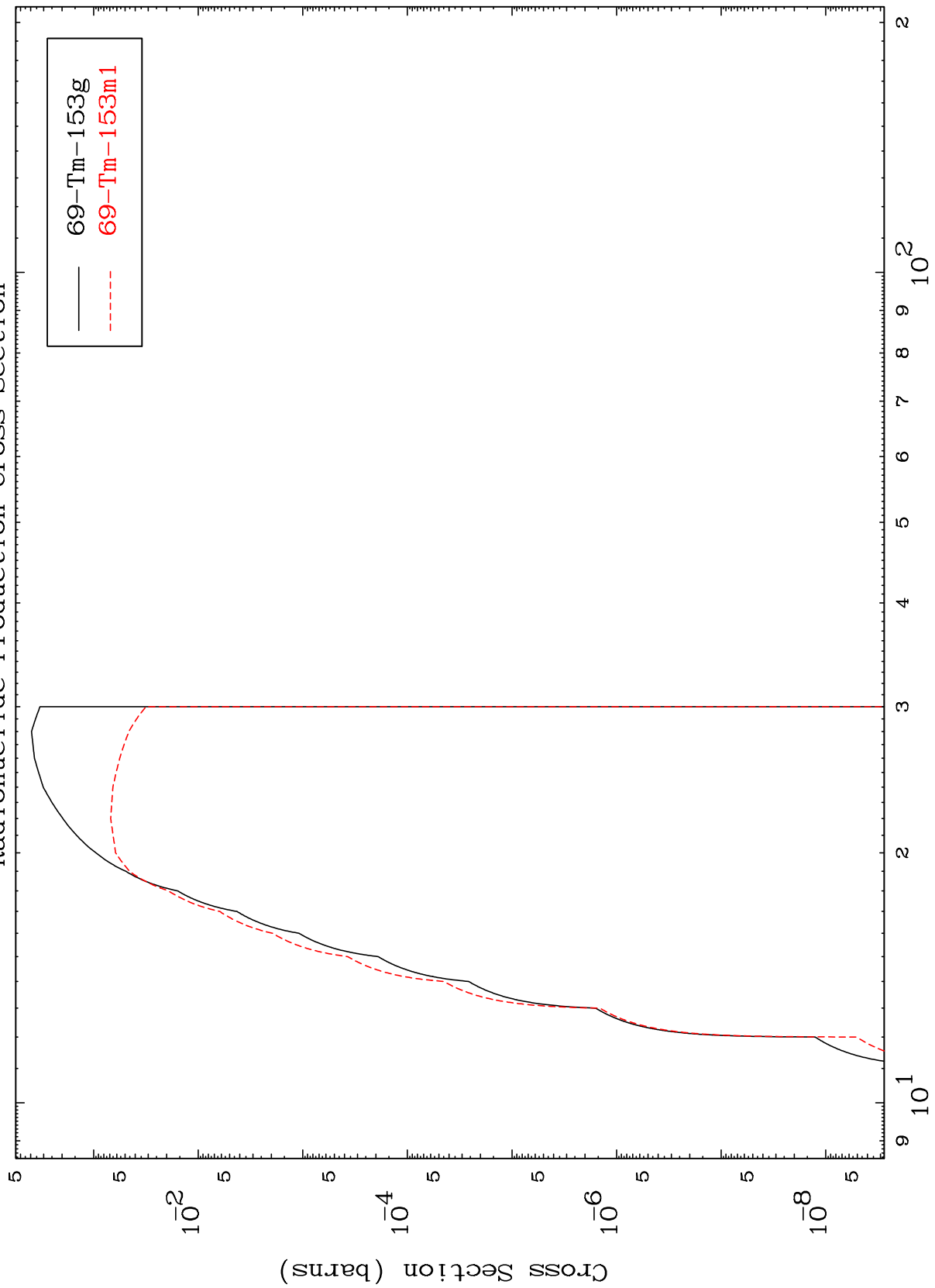
(n,n') p  $\alpha$   
Radionuclide Production Cross Section



MAT 6789

68-Er-150

(n,p)  
Radionuclide Production Cross Section

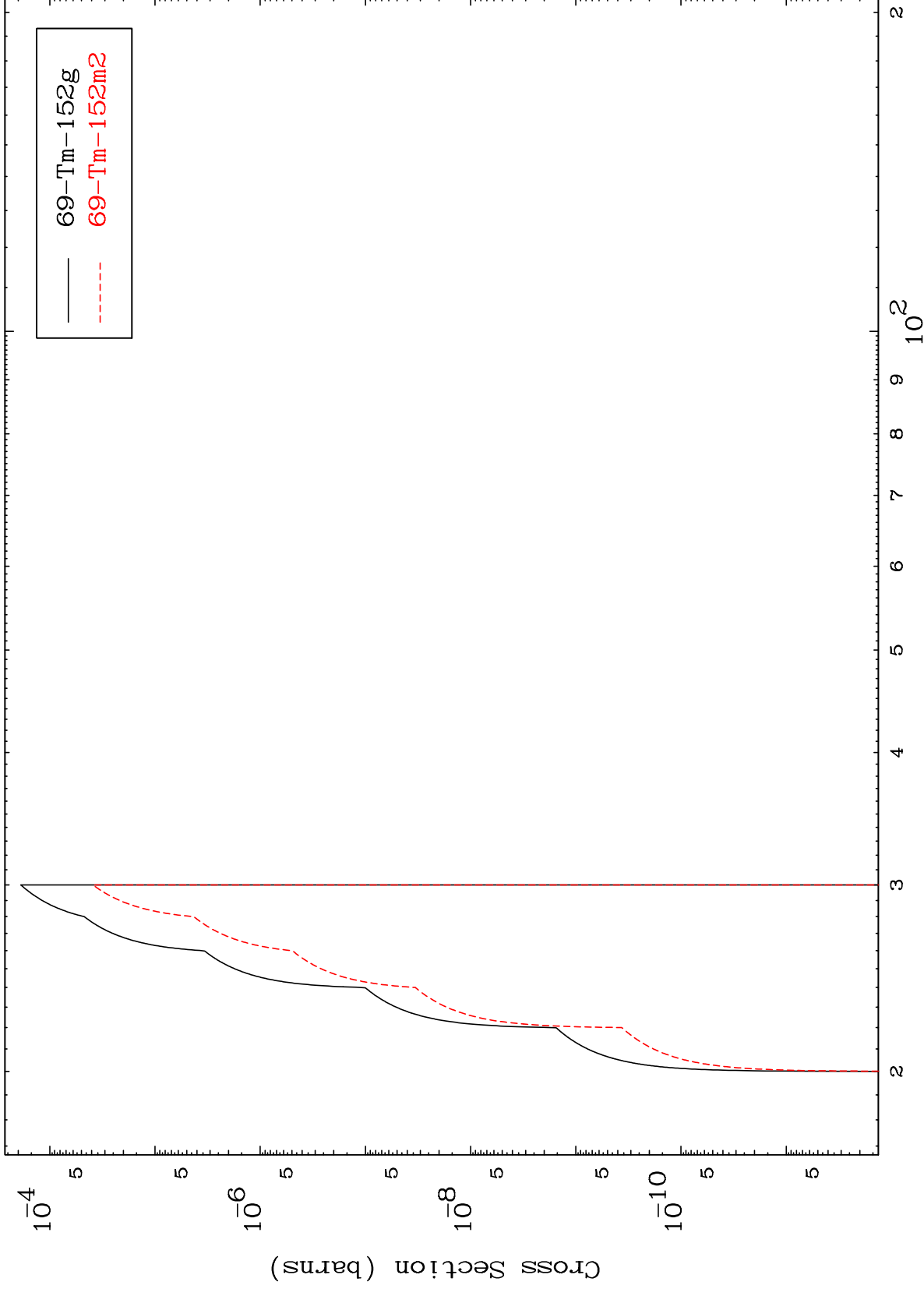


68-Er-150

Incident Energy (MeV)

14

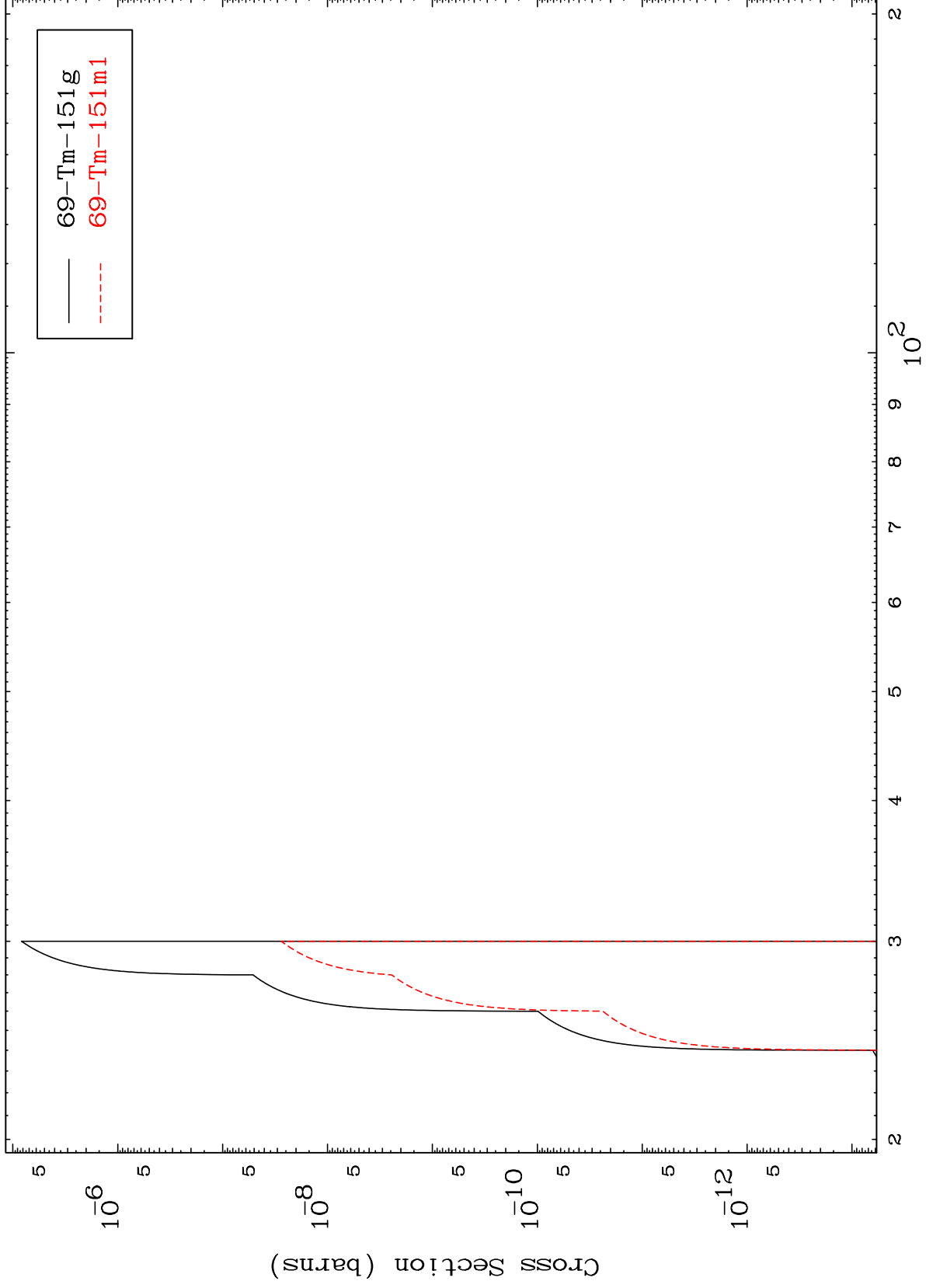
Radionuclide Production Cross Section



MAT 6789

68-Er-150

(n,t)  
Radionuclide Production Cross Section

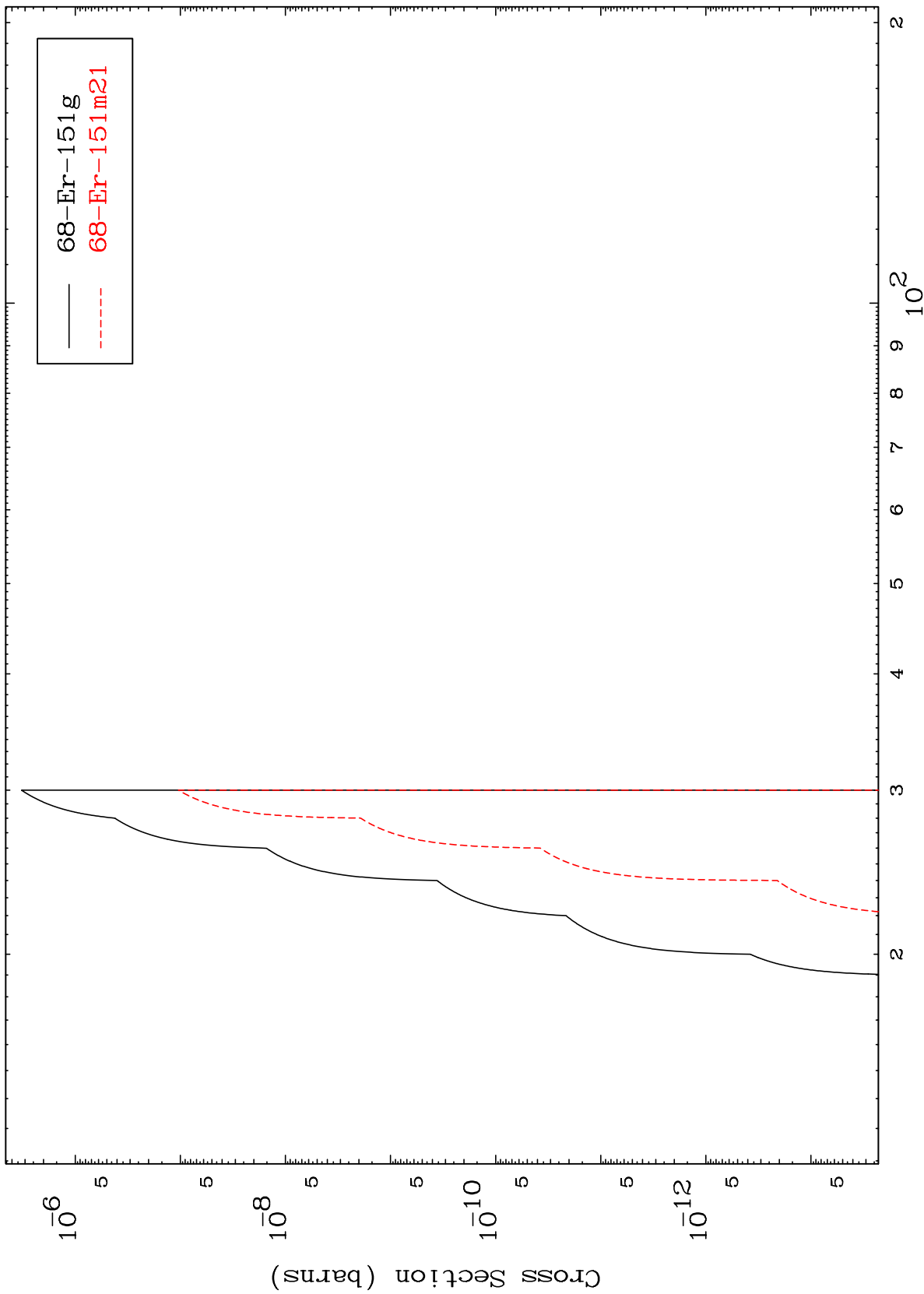


68-Er-150

Incident Energy (MeV)

16

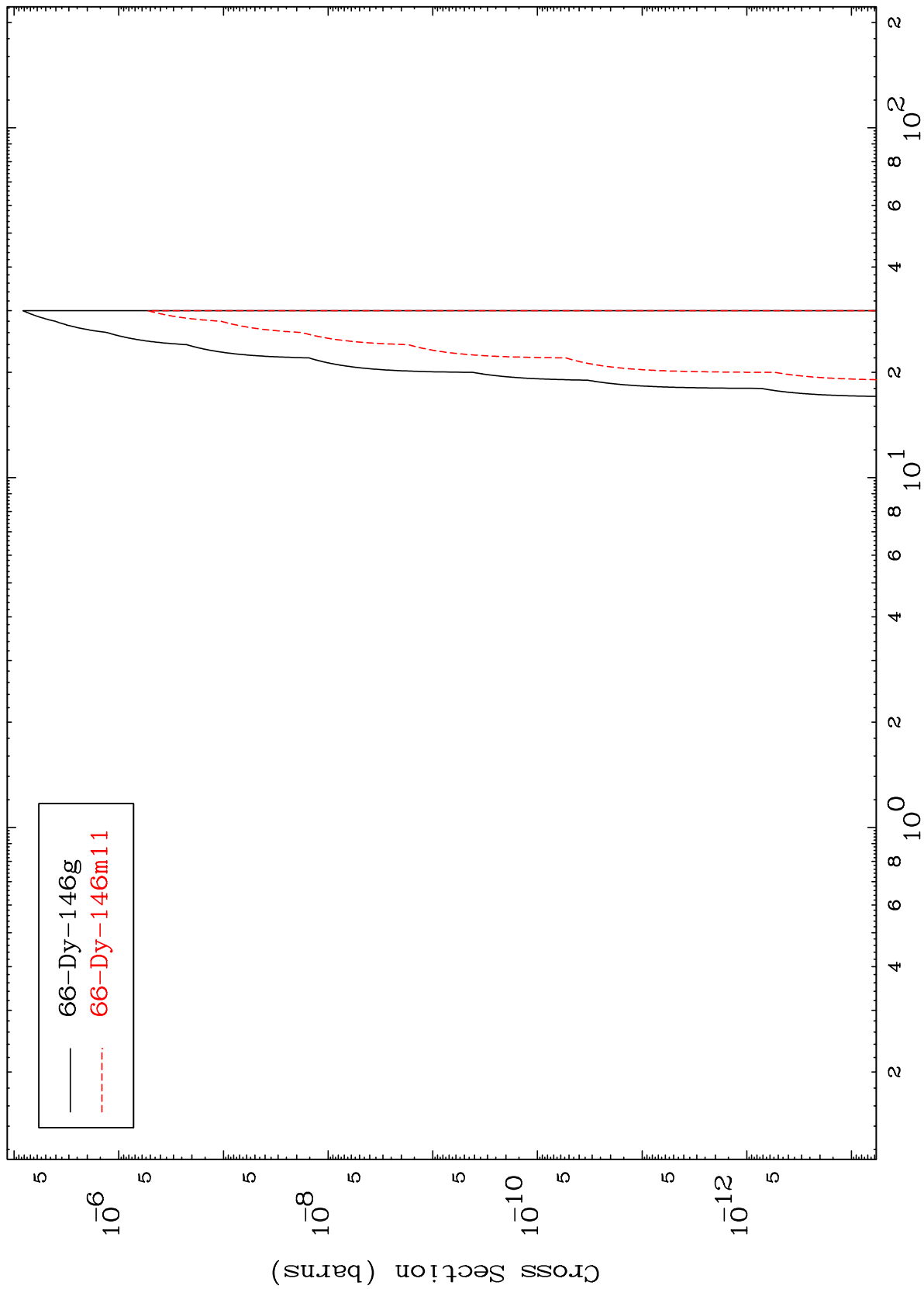
(n,He-3)  
Radionuclide Production Cross Section



MAT 6789

68-Er-150

Radionuclide Production Cross Section  
(n,2α)



18

68-Er-150

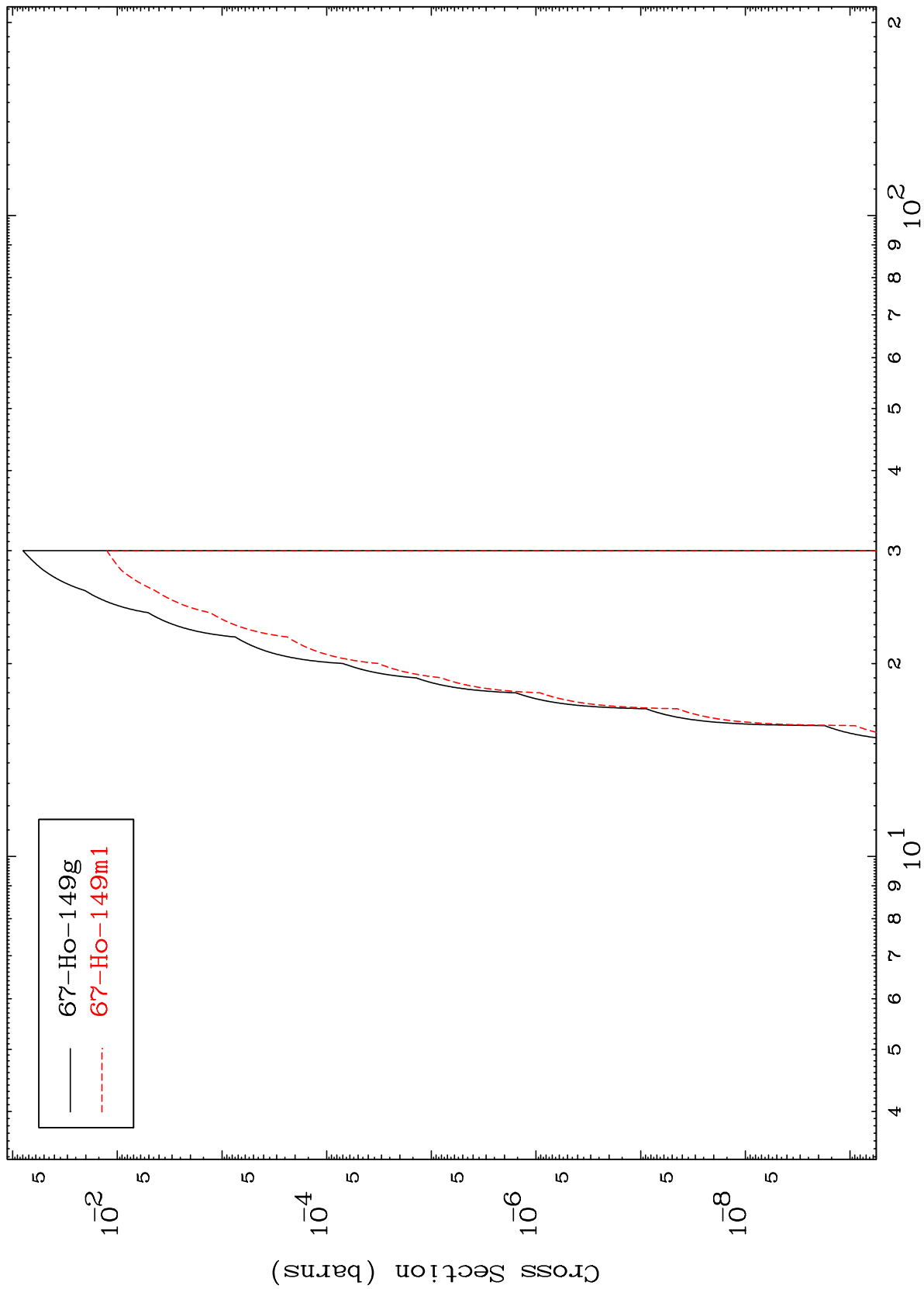
Incident Energy (MeV)

MAT 6789

(n,p)  $\alpha$

68-Er-150

Radionuclide Production Cross Section



19

Incident Energy (MeV)

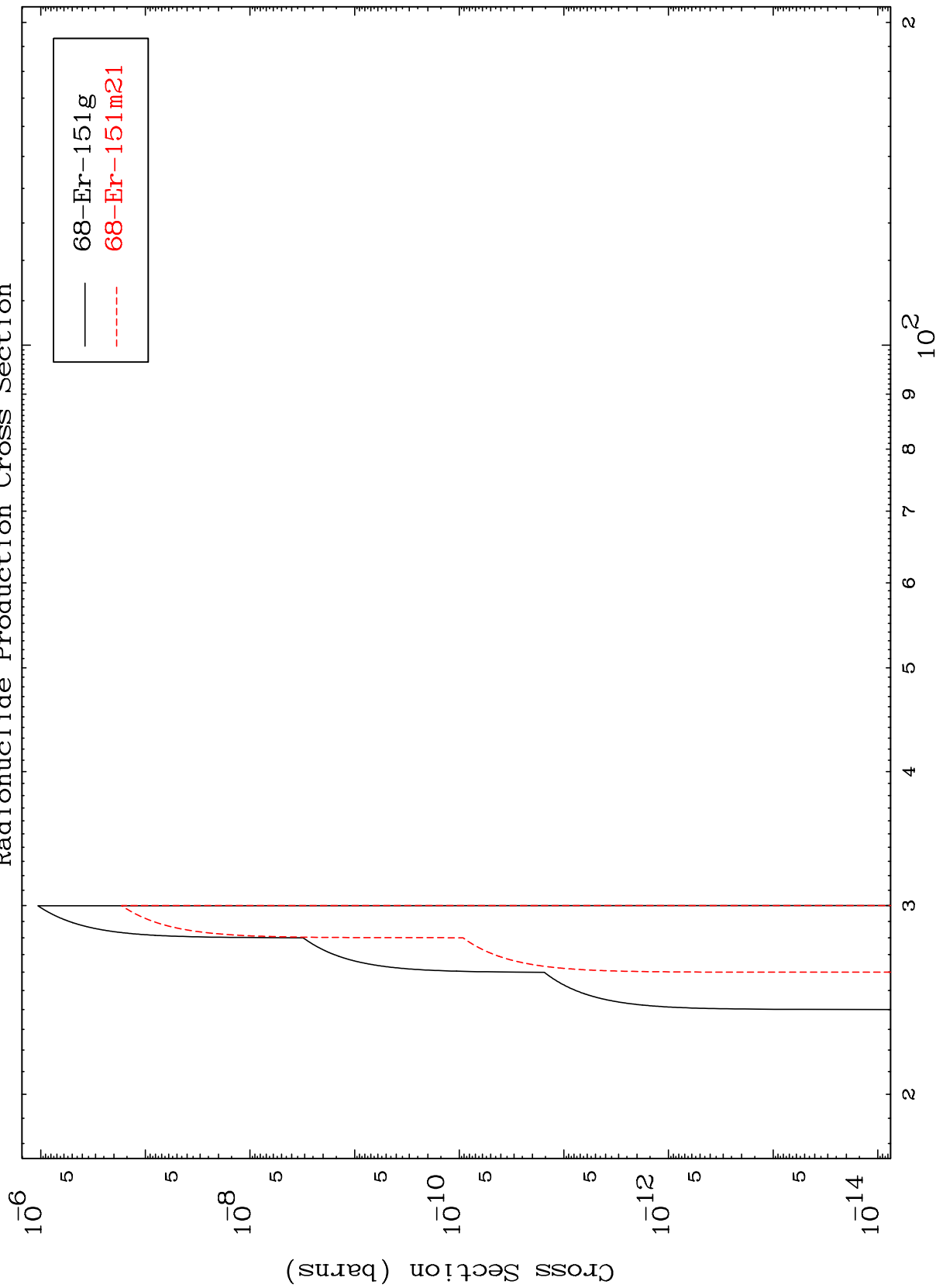
68-Er-150

MAT 6789

(n,p) d

68-Er-150

Radionuclide Production Cross Section



20

Incident Energy (MeV)

68-Er-150