

Program EVALPLOT  
(Version 2021-1)

by

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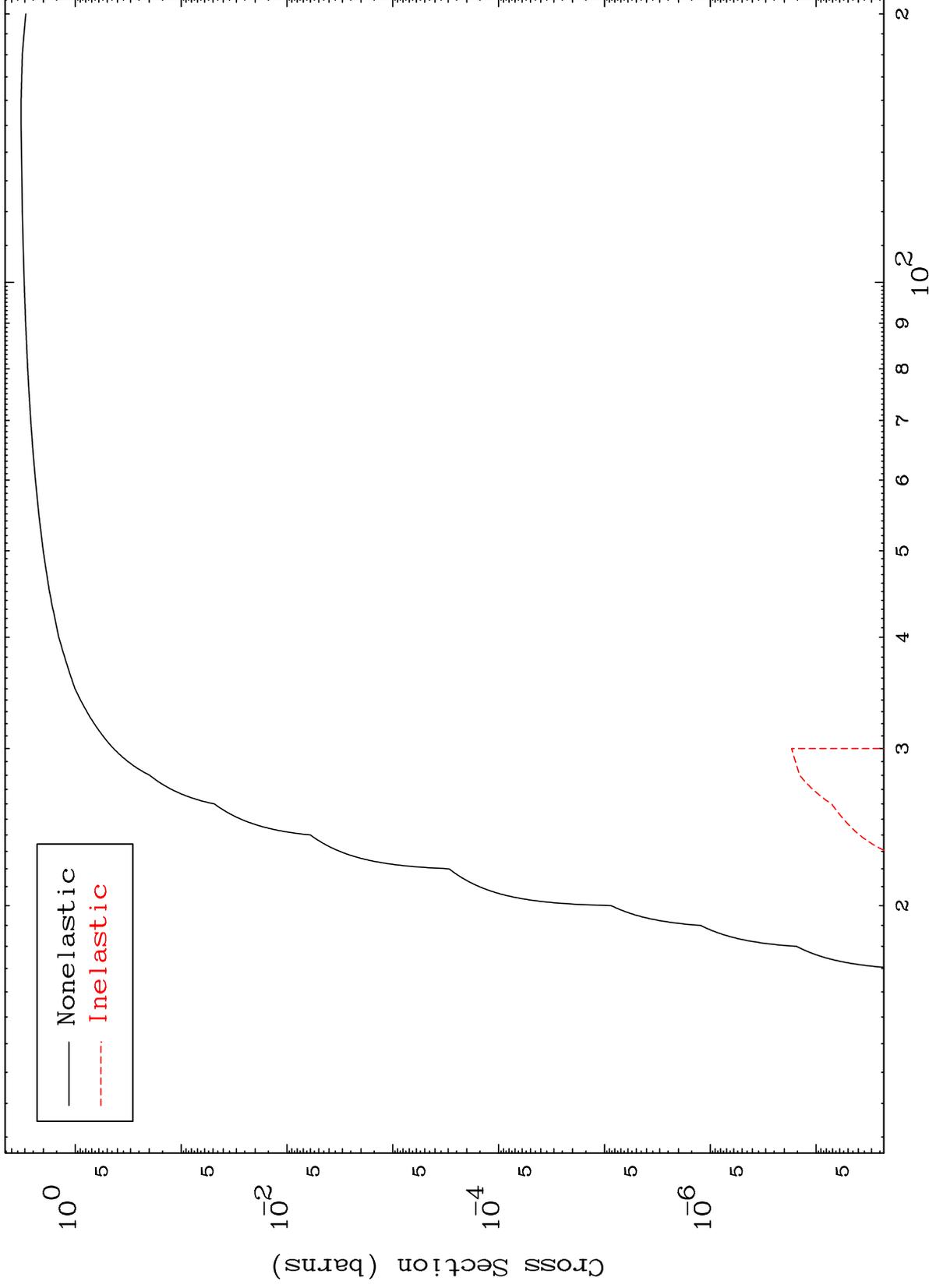
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Press Mouse Button to Start

MAT 1489

$\alpha$  Major  
0 Kelvin Cross Sections

114-F1-289



1

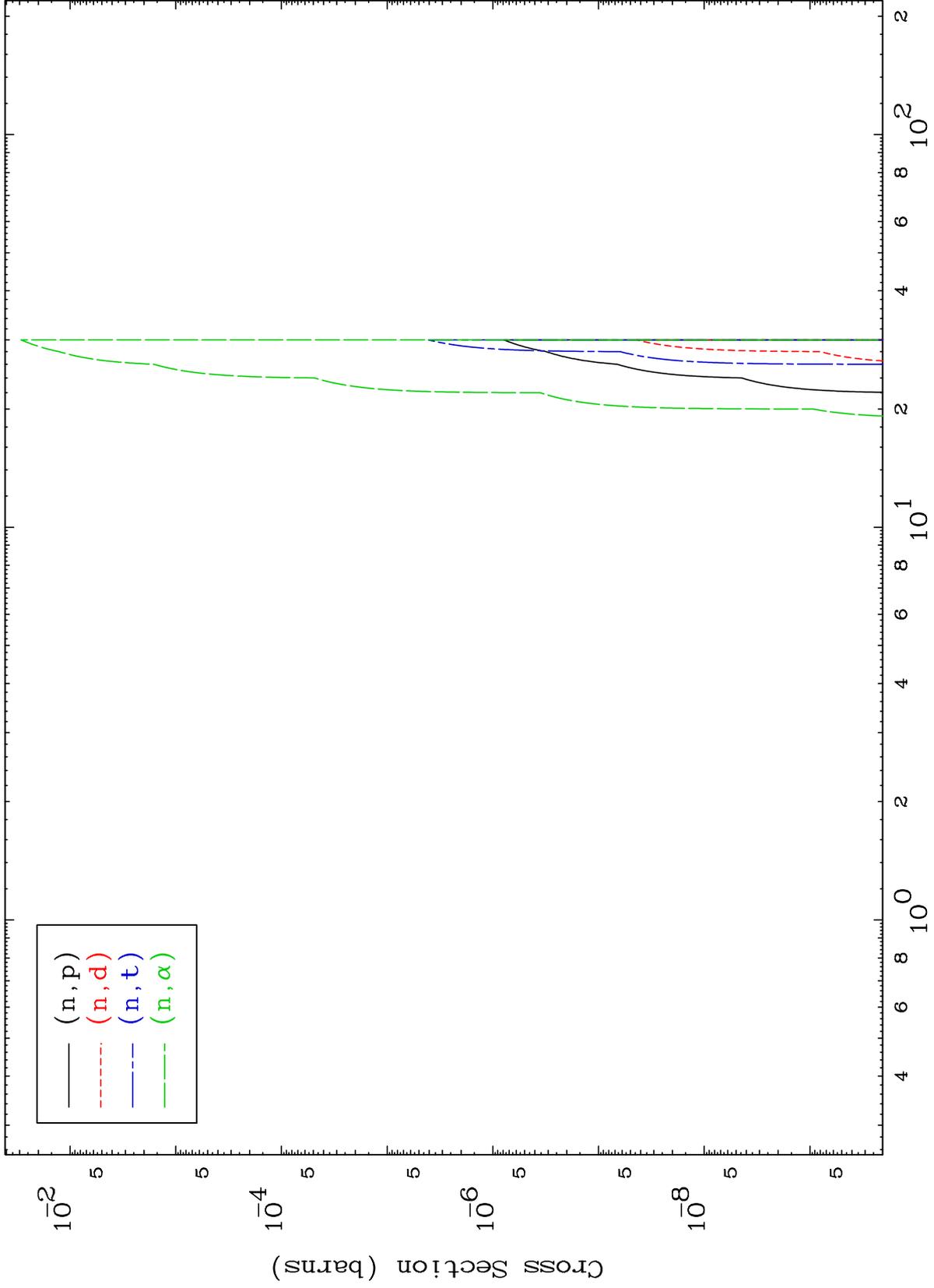
Incident Energy (MeV)

114-F1-289

MAT 1489

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

114-F1-289

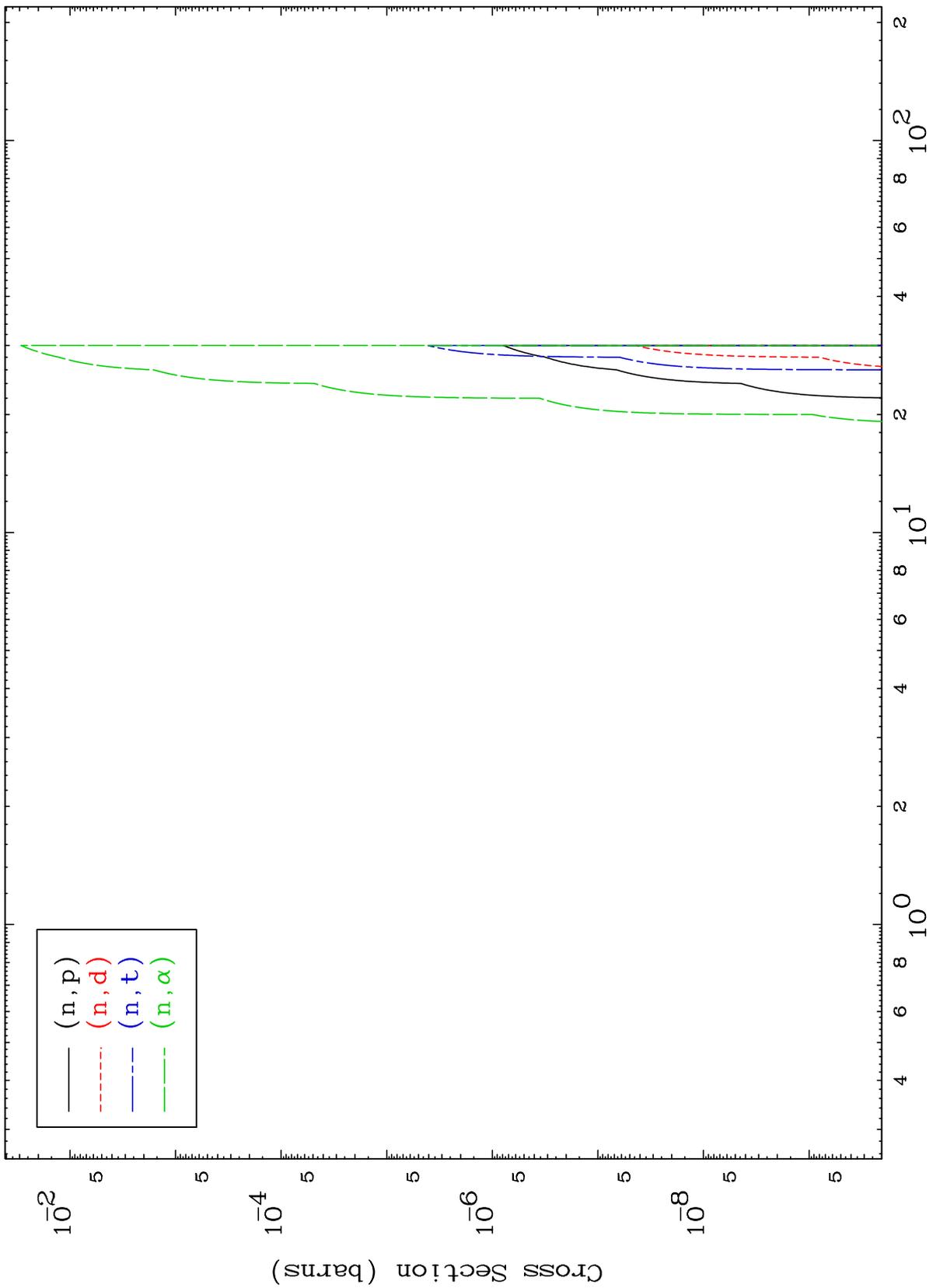


2

Incident Energy (MeV)

114-F1-289

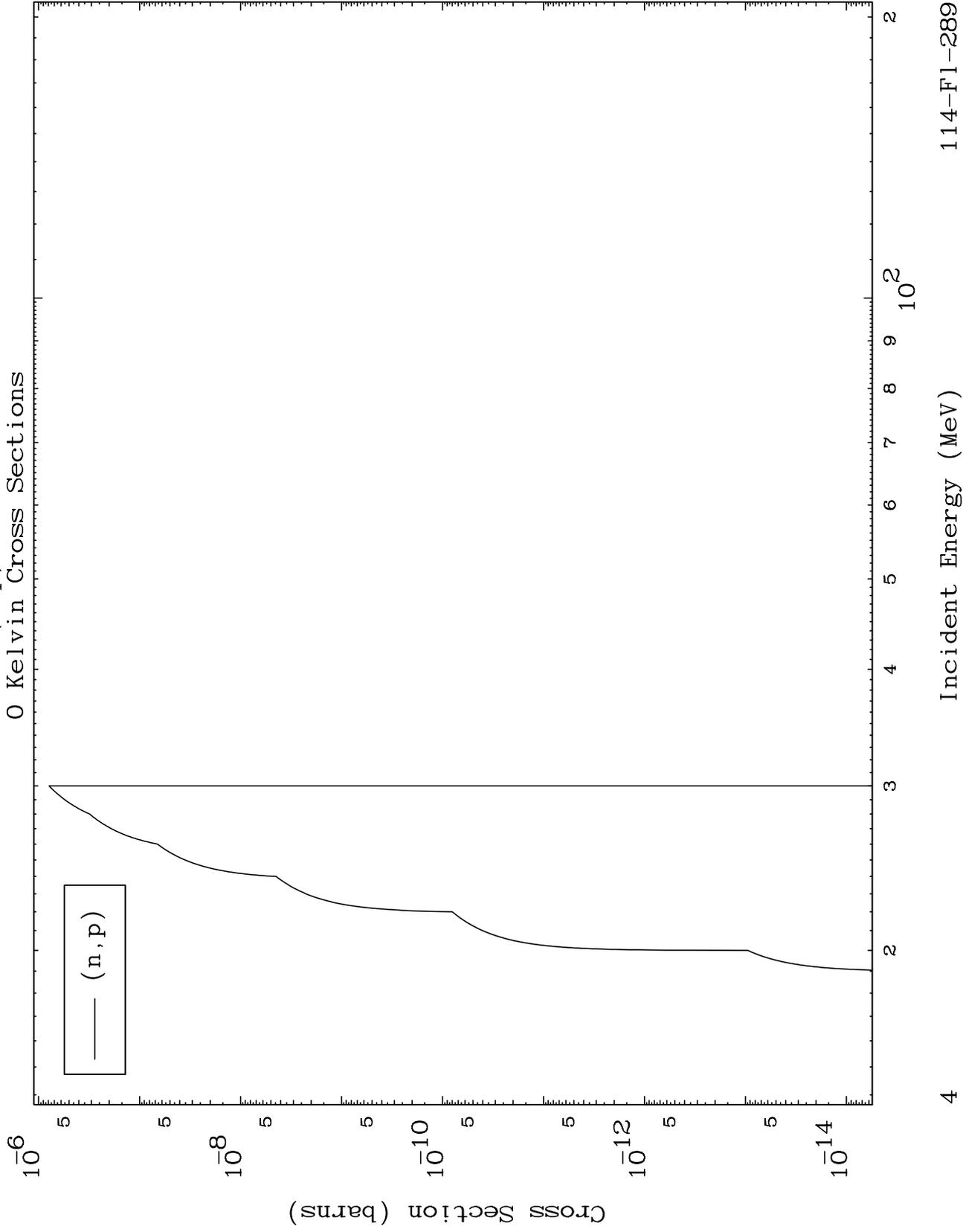
0 Kelvin Cross Sections

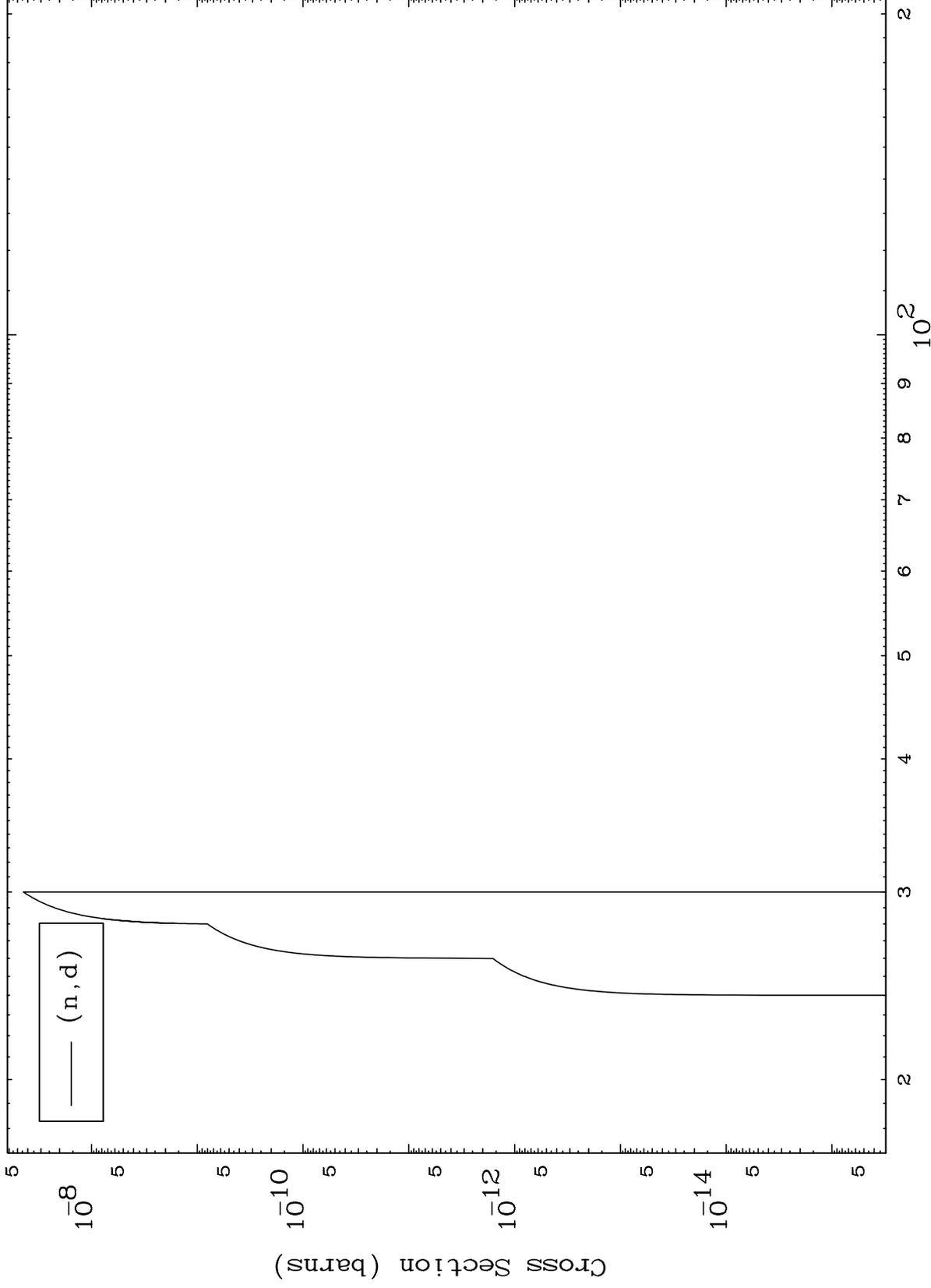


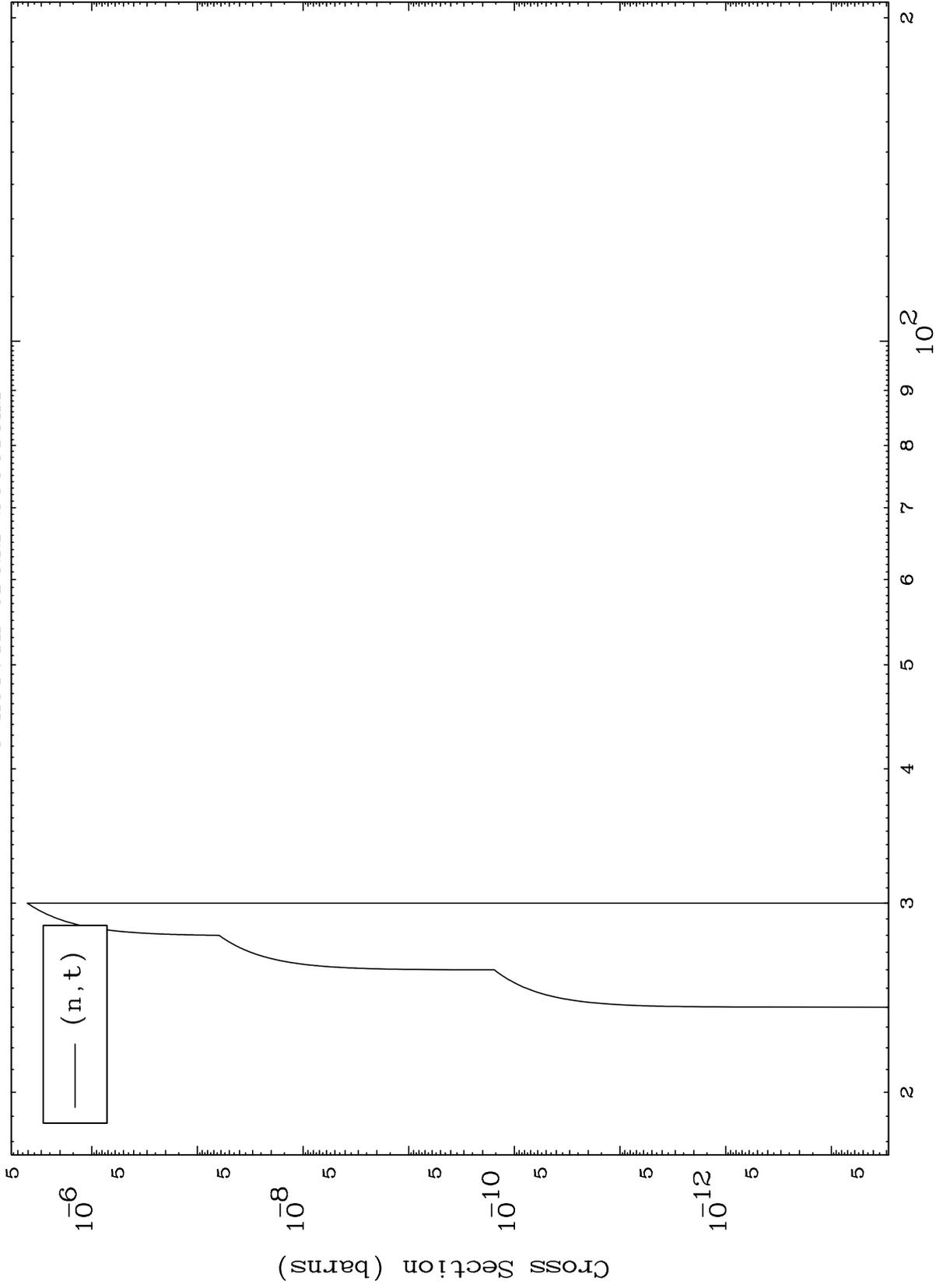
MAT 1489

( $\alpha, p$ ) Levels

114-F1-289





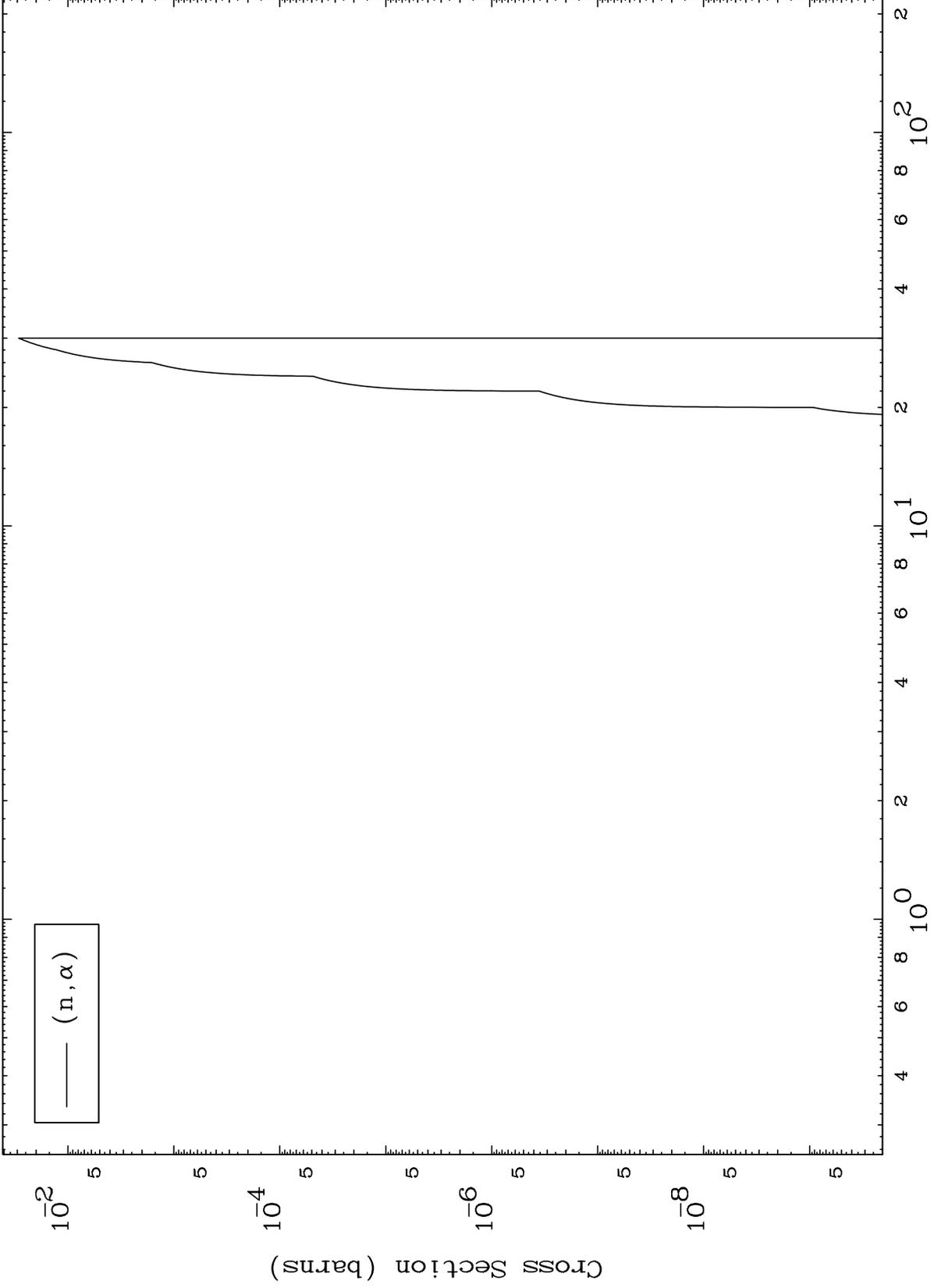


MAT 1489

( $\alpha, \alpha$ ) Levels

114-F1-289

0 Kelvin Cross Sections



MAT 1489

Fission

114-F1-289

Radionuclide Production Cross Section

