

Program EVALPLOT  
(Version 2021-1)

by

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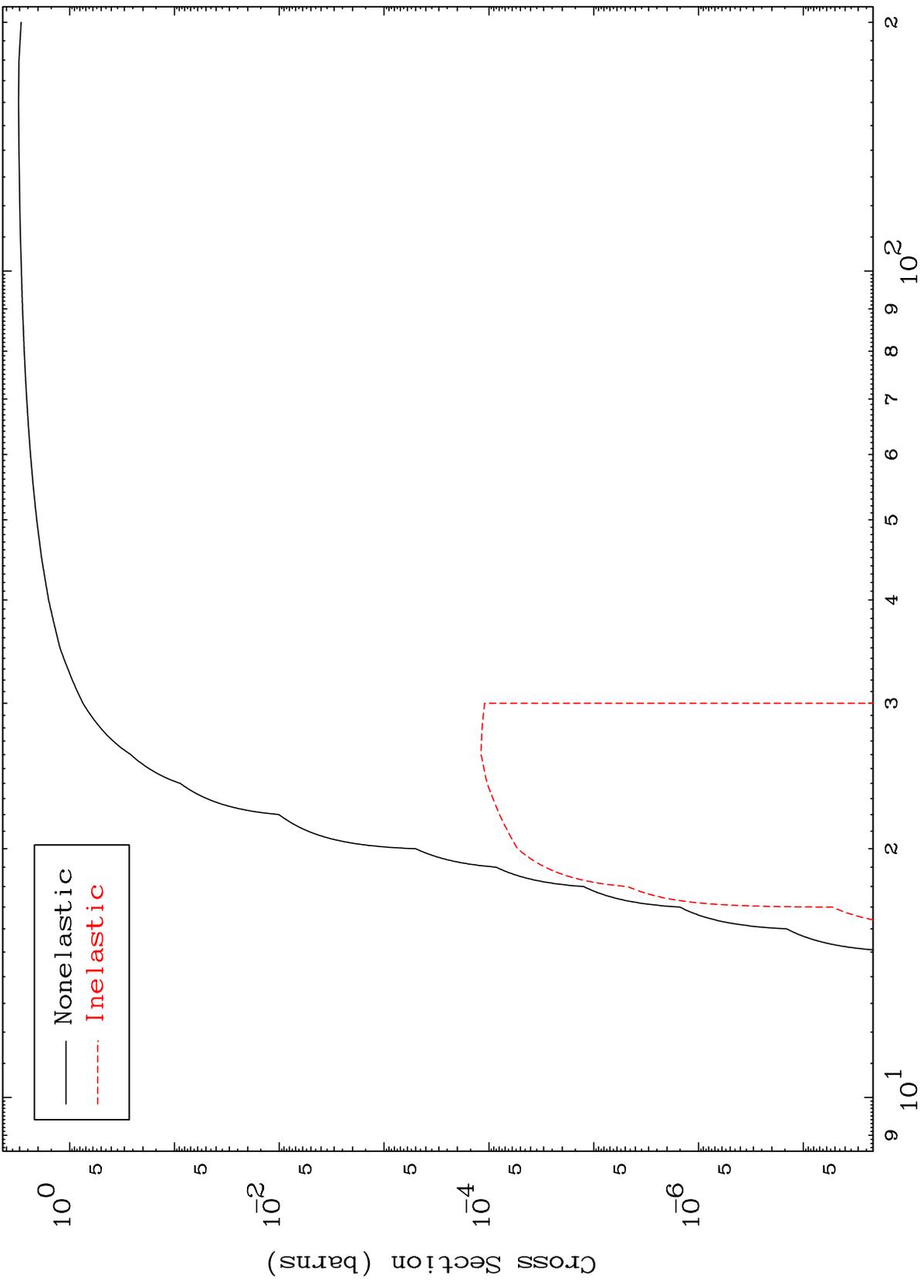
Press Mouse Button to Start

MAT 9933

$\alpha$  Major

100-Fm-252

0 Kelvin Cross Sections



— Nonelastic  
- - - Inelastic

100-Fm-252

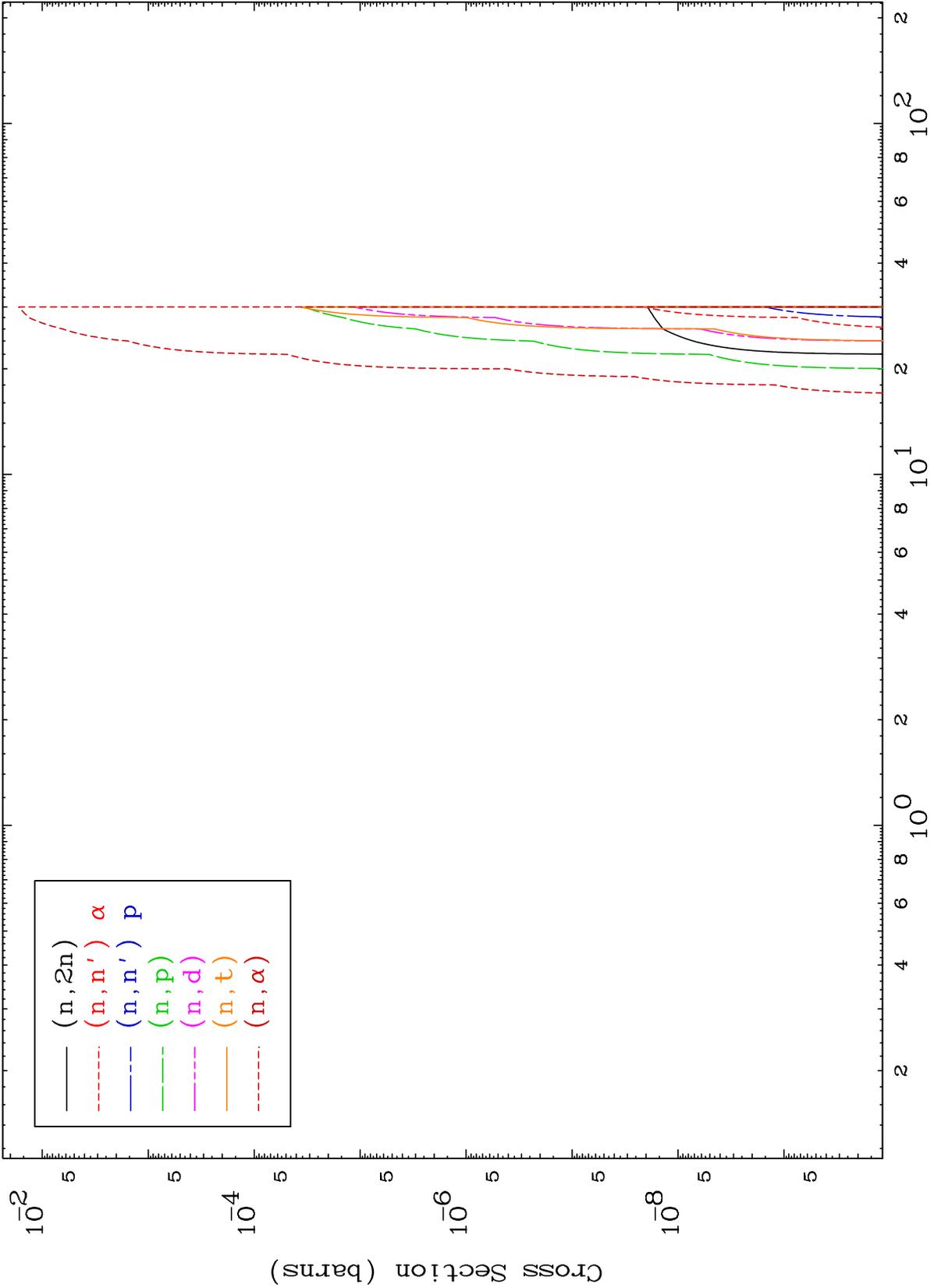
Incident Energy (MeV)

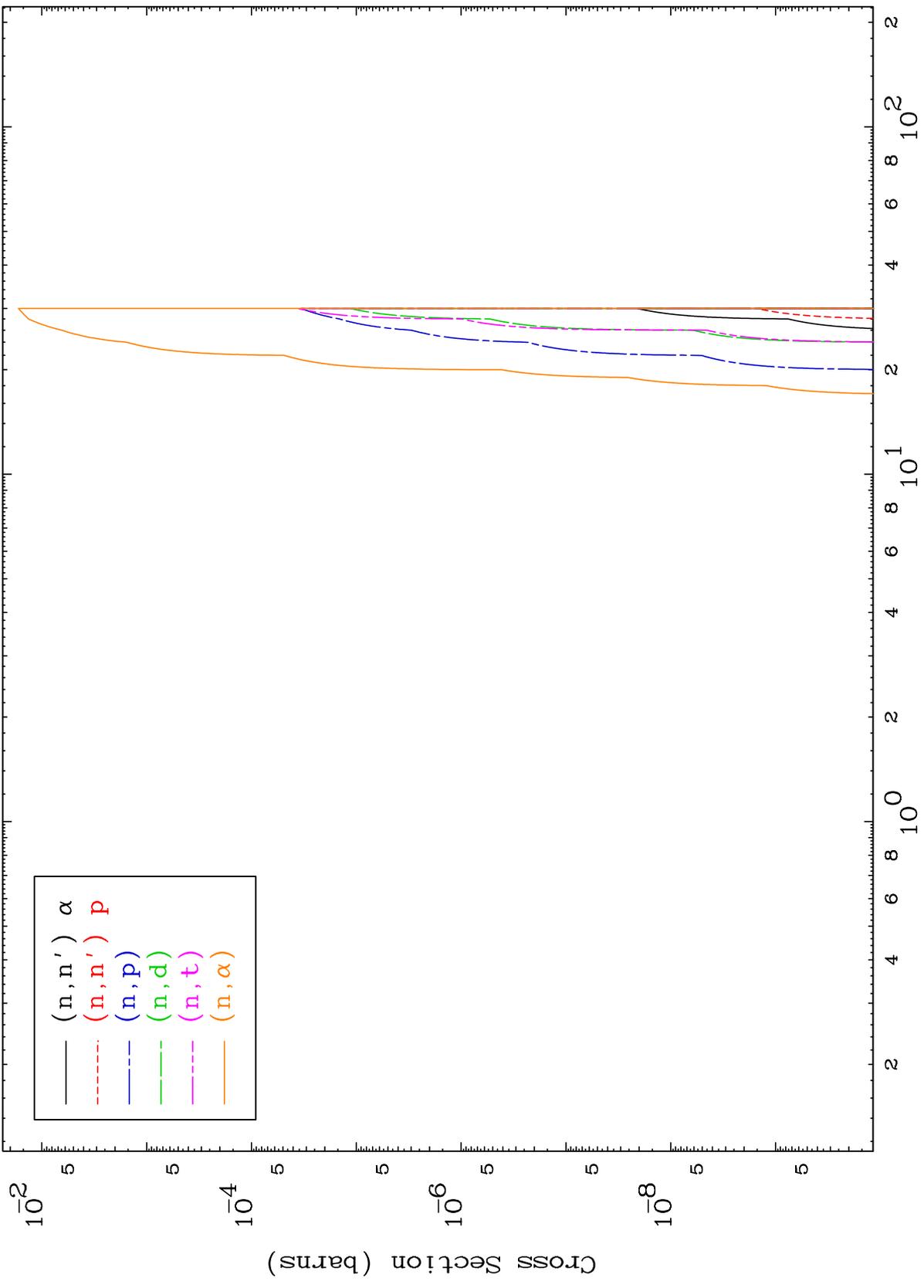
1

MAT 9933

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

100-Fm-252

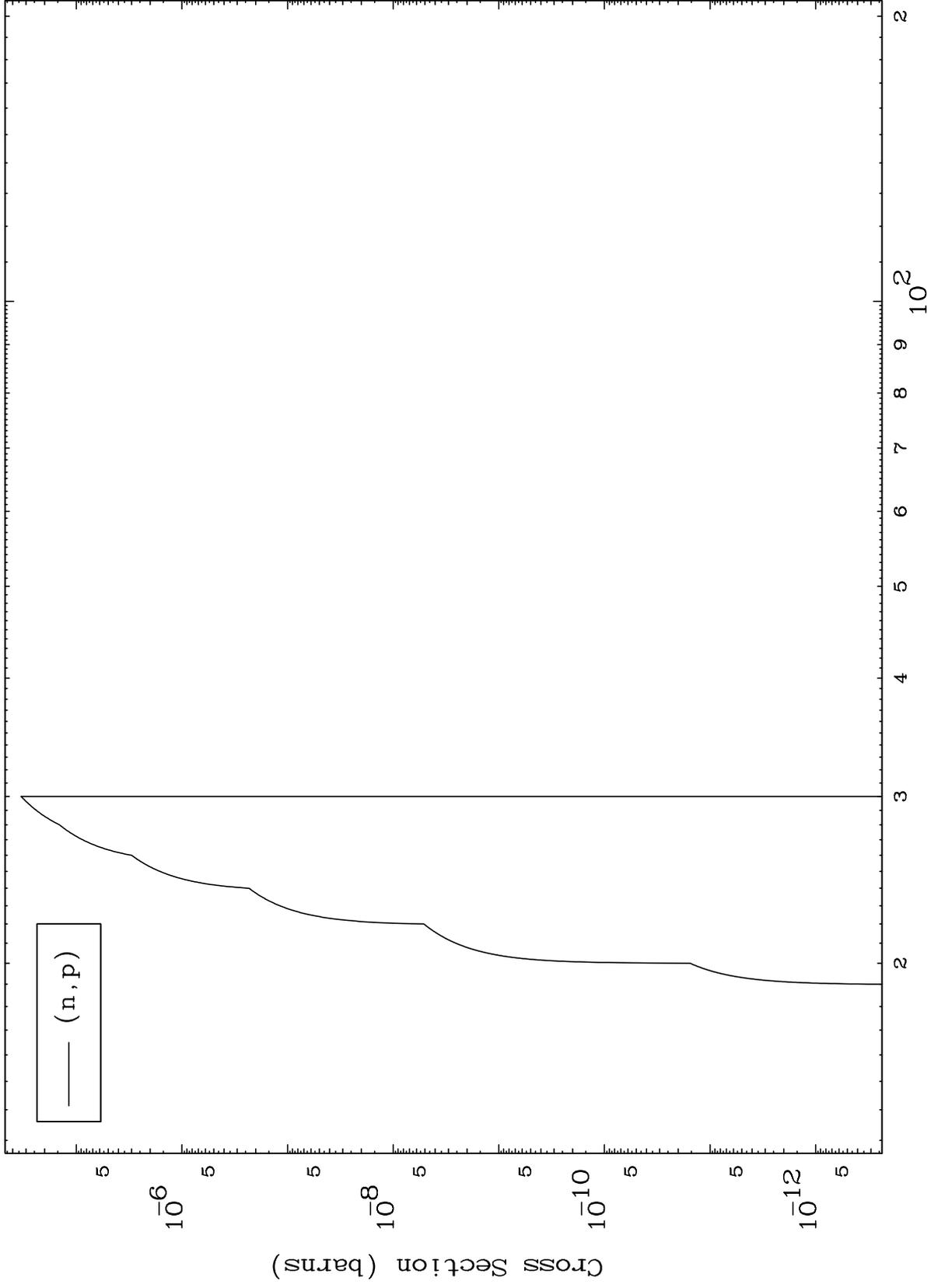


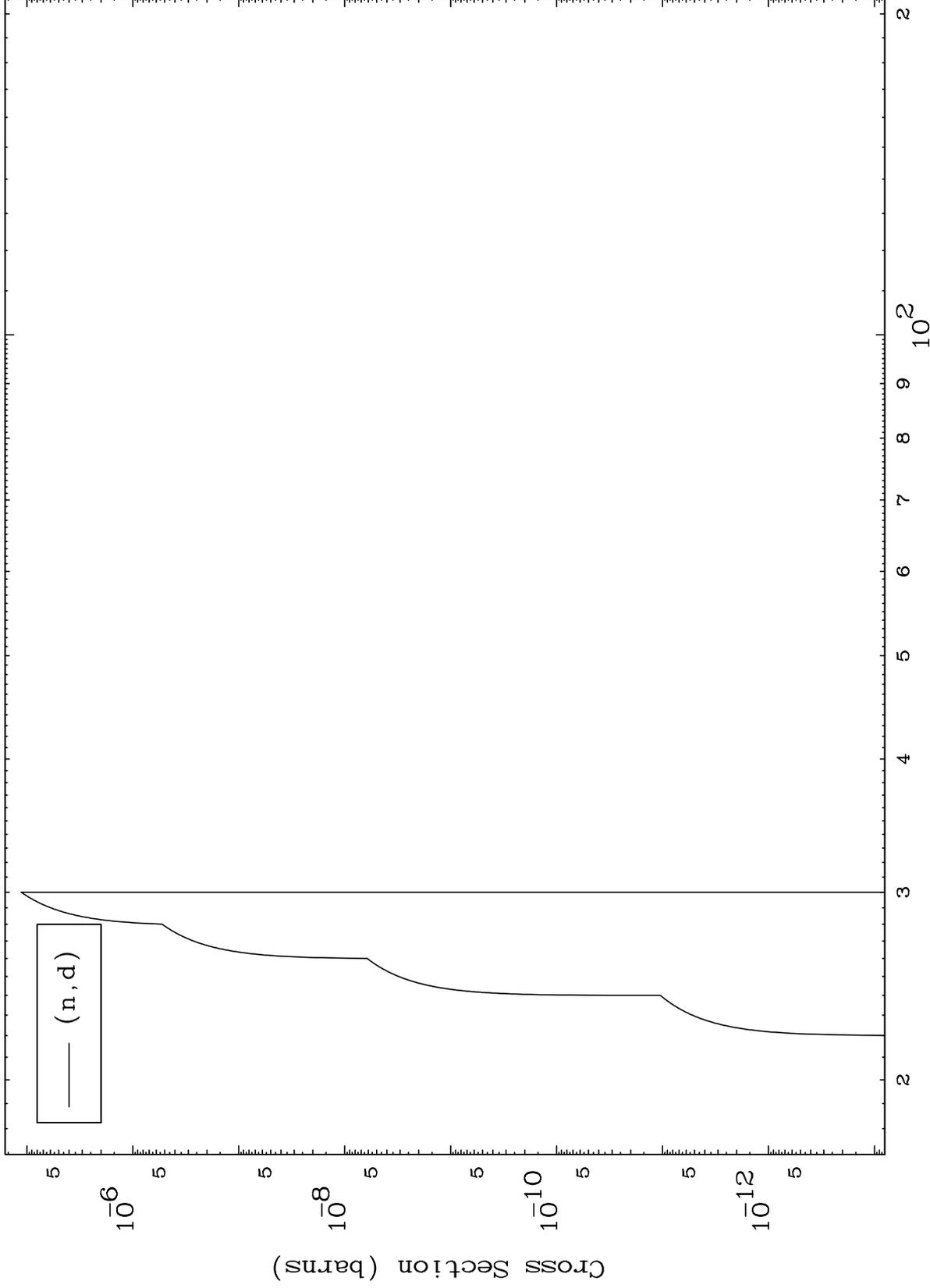


MAT 9933

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-252

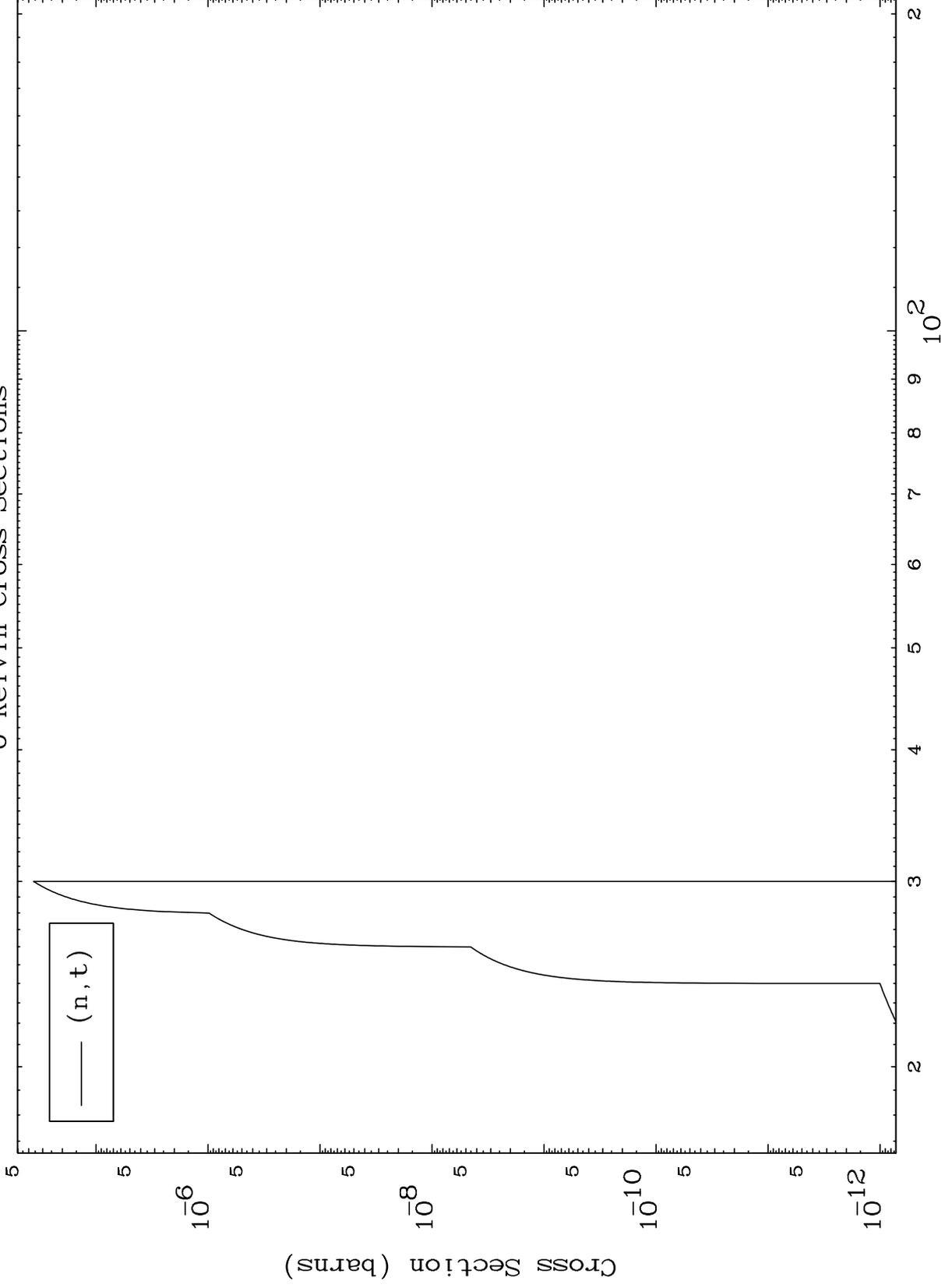




MAT 9933

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-252



6

Incident Energy (MeV)

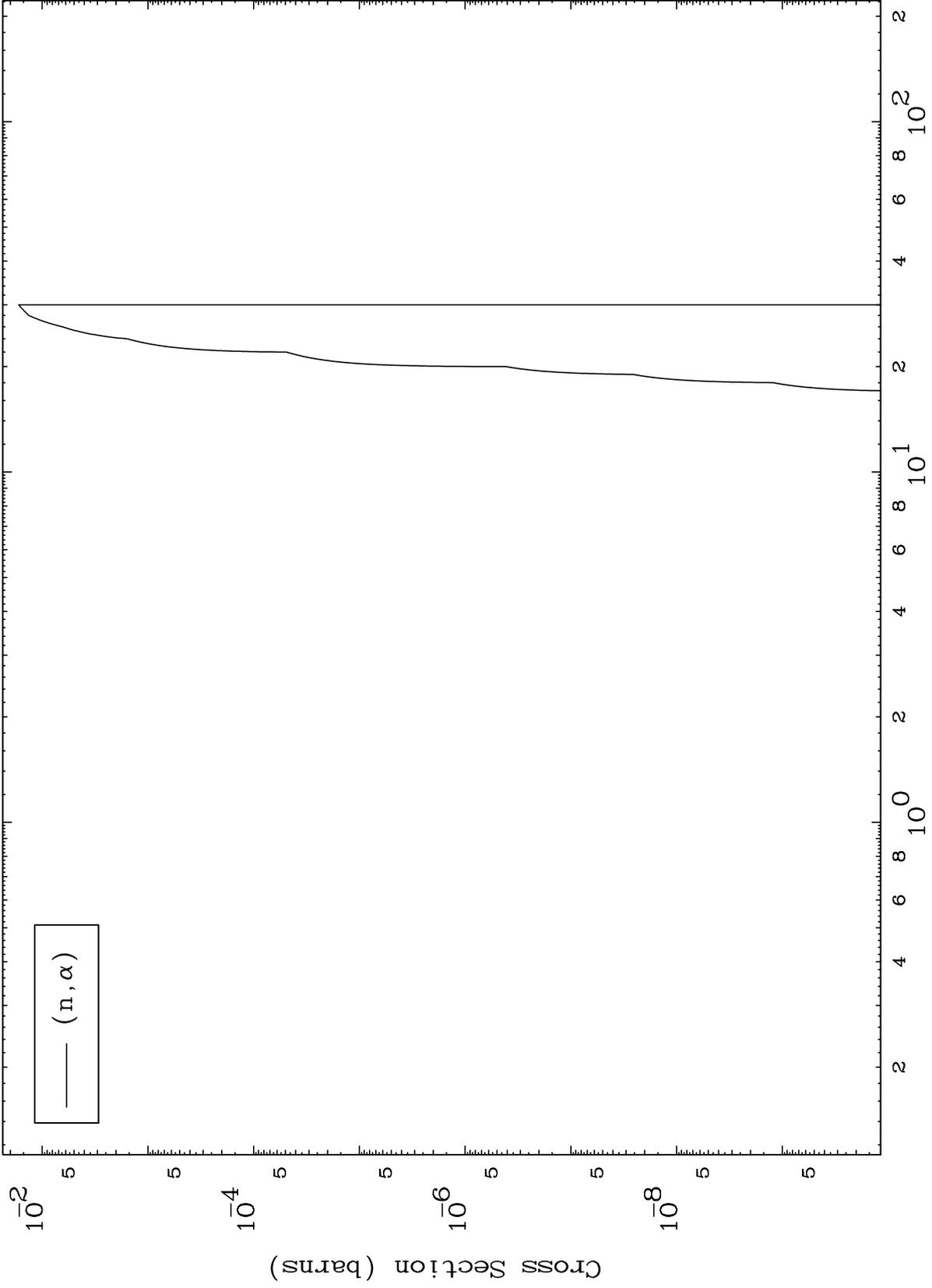
100-Fm-252

MAT 9933

( $\alpha, \alpha$ ) Levels

100-Fm-252

0 Kelvin Cross Sections

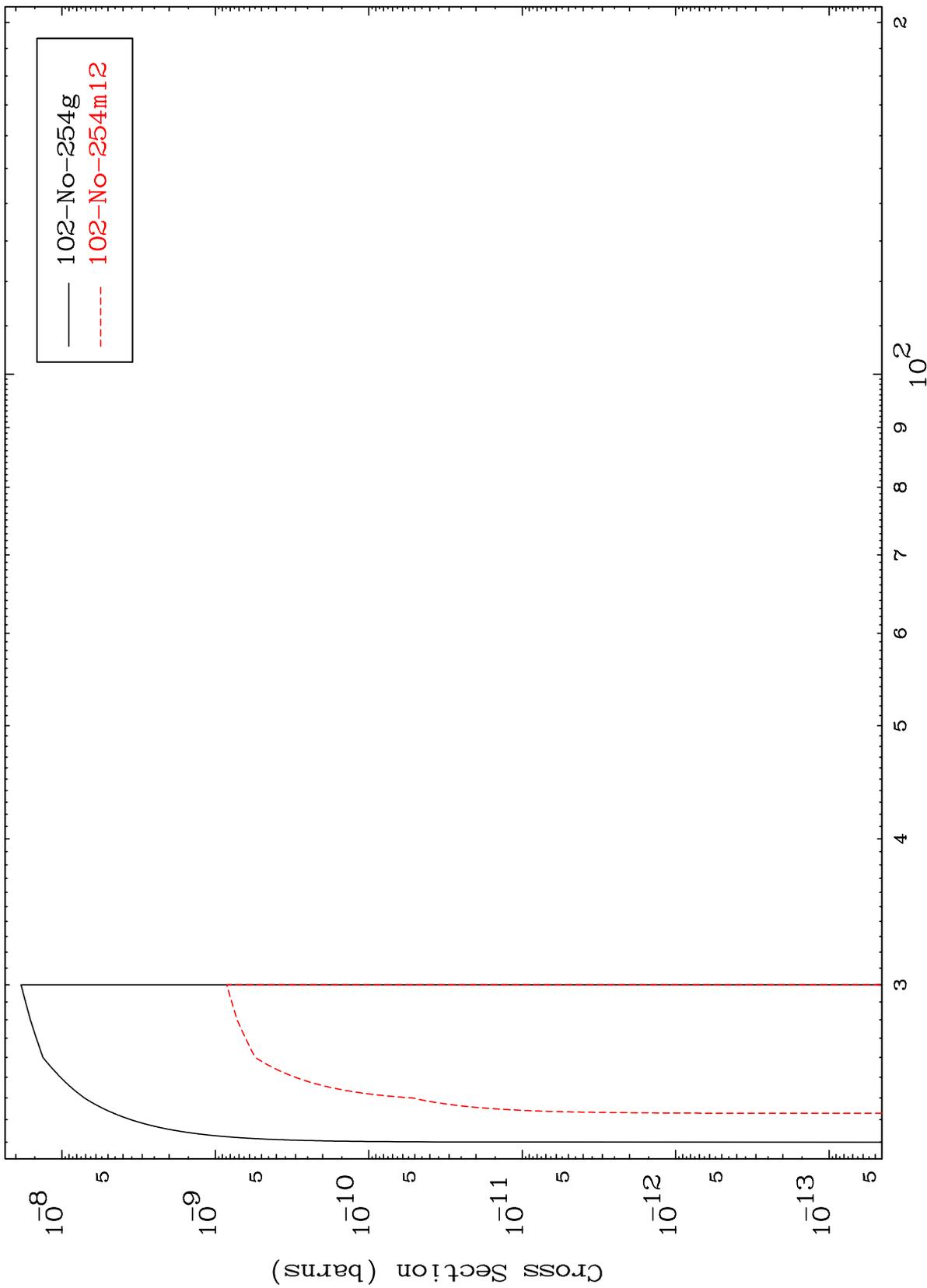


(n,  $\alpha$ )

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100-Fm-252

(n,2n)  
Radionuclide Production Cross Section



8

Incident Energy (MeV)

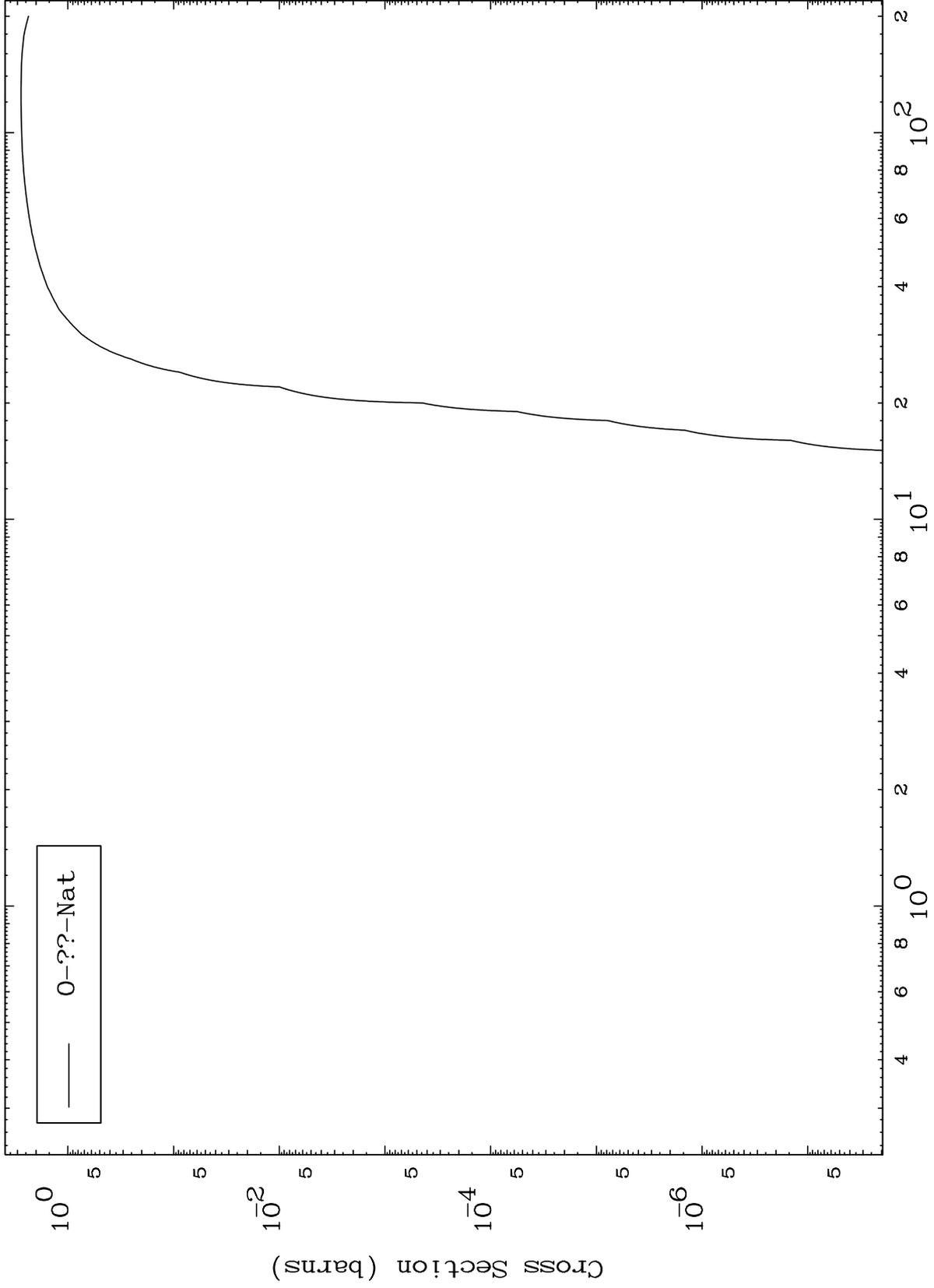
100-Fm-252

MAT 9933

Fission

100-Fm-252

Radionuclide Production Cross Section

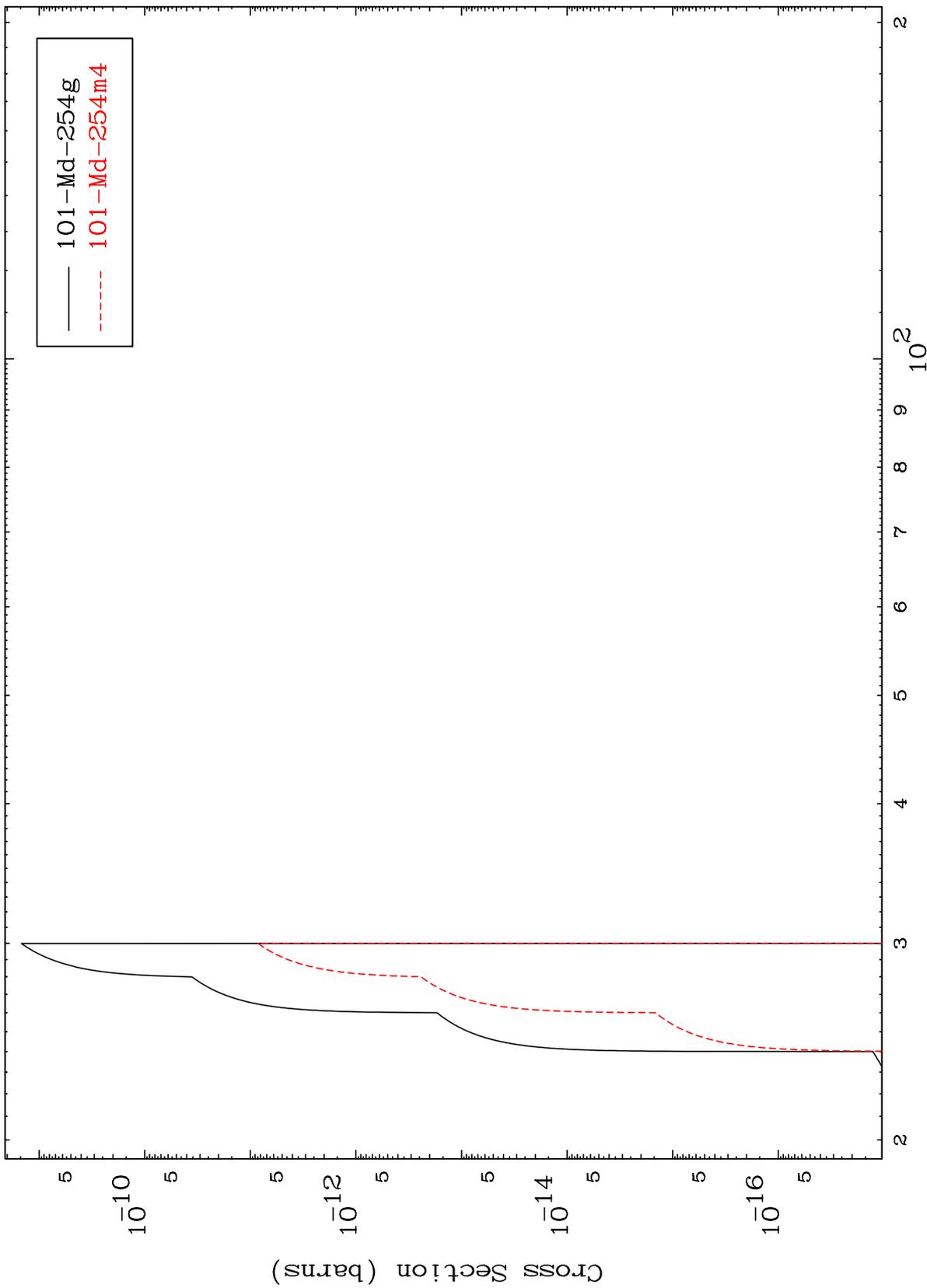


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(n,n') p

100-Fm-252

Radionuclide Production Cross Section



101-Md-254g  
101-Md-254m4

10

Incident Energy (MeV)

100-Fm-252

(n,d)  
Radionuclide Production Cross Section

