

Program EVALPLOT  
(Version 2021-1)

by

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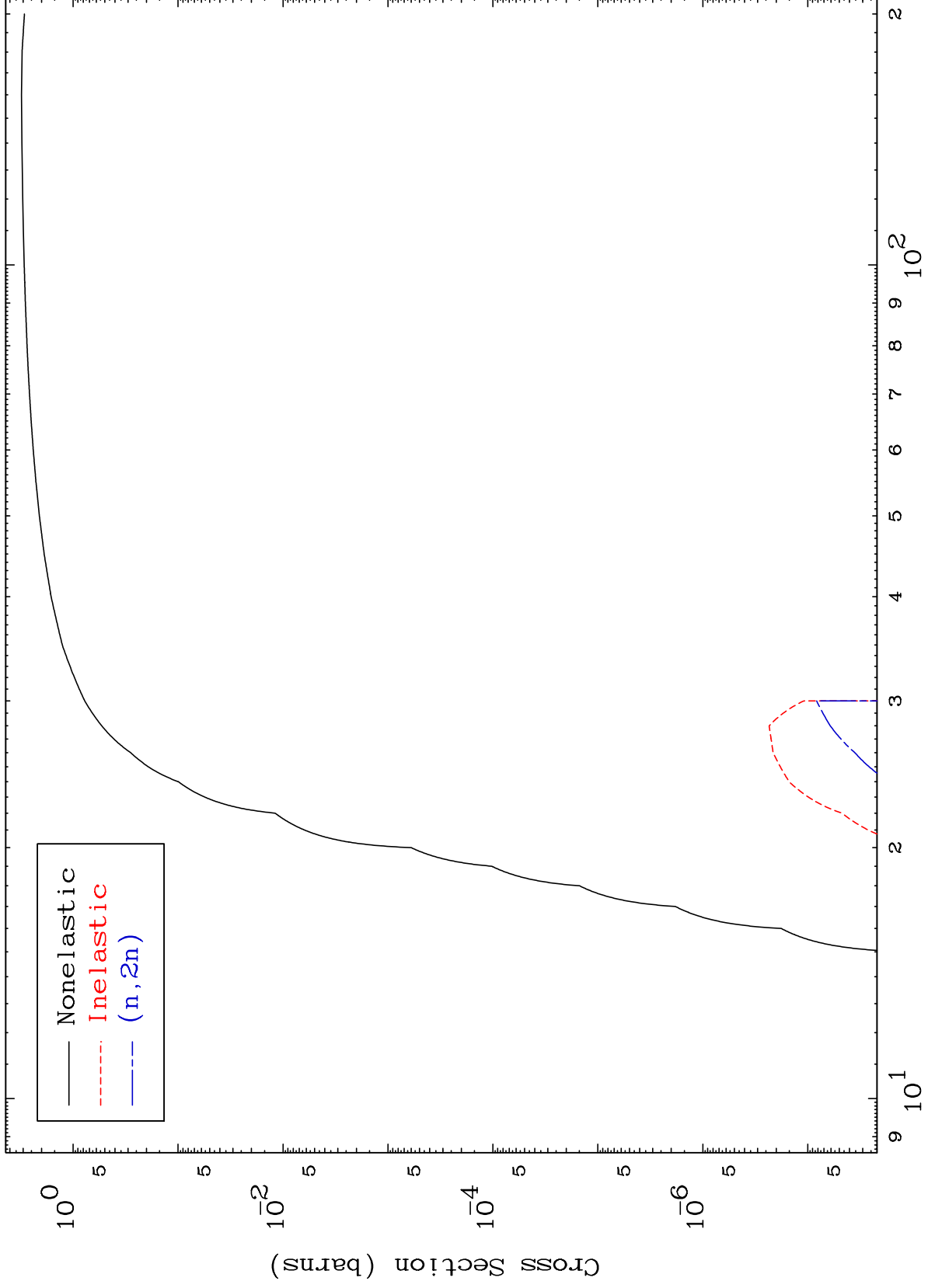
Press Mouse Button to Start

MAT 9938

0 Kelvin Cross Sections

$\alpha$  Major

100-Fm-257



1

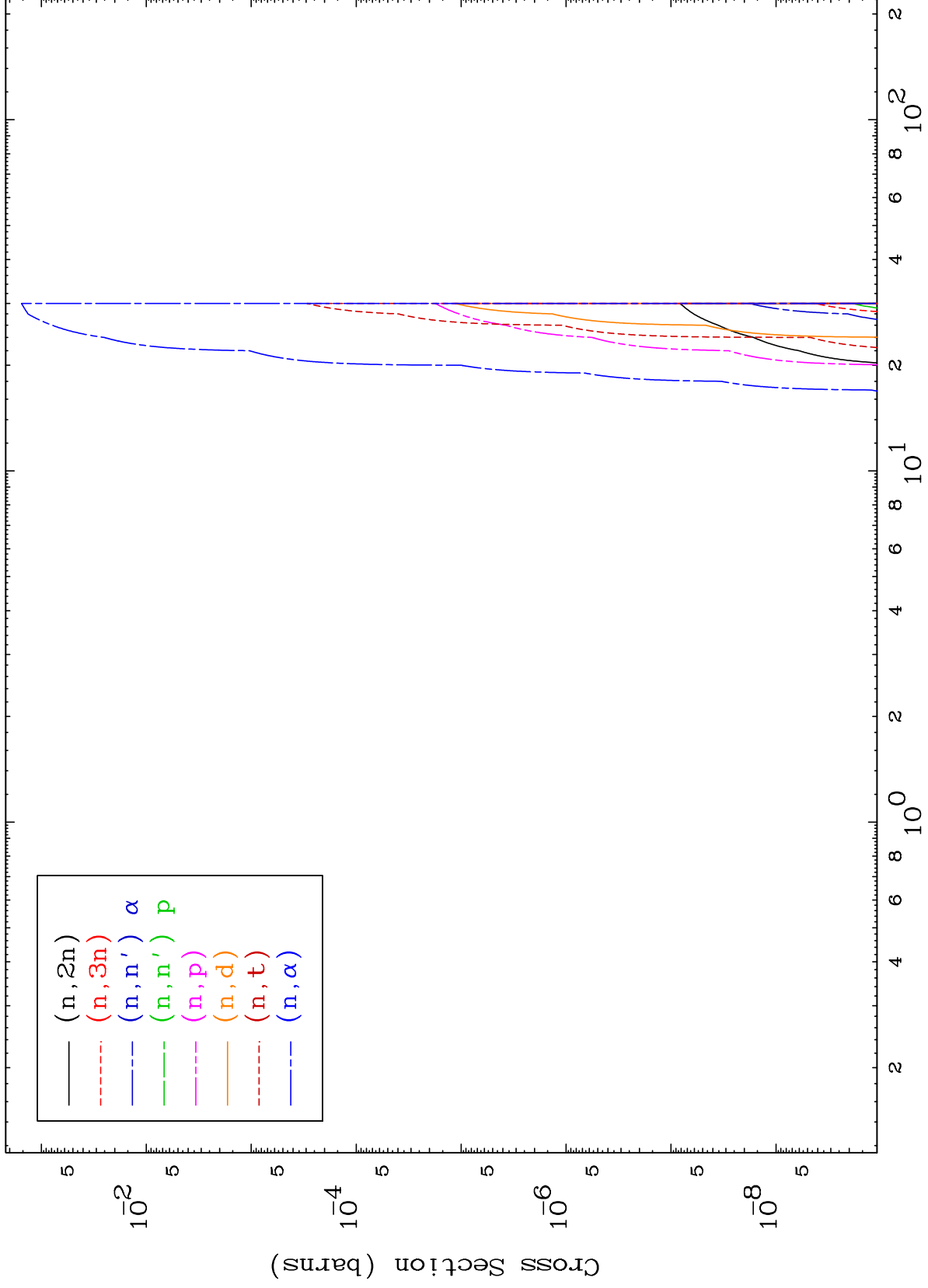
Incident Energy (MeV)

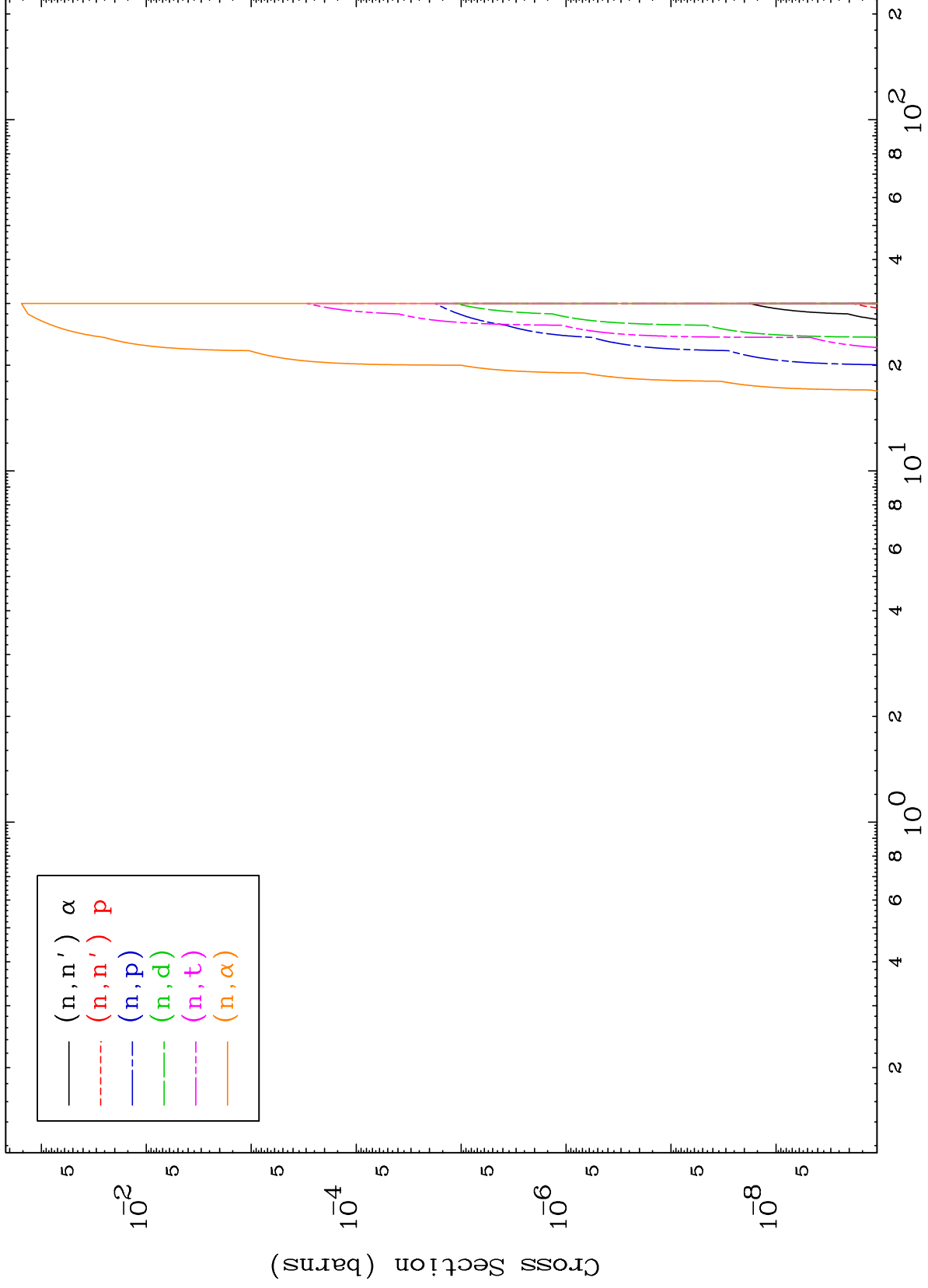
100-Fm-257

MAT 9938

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

100-Fm-257

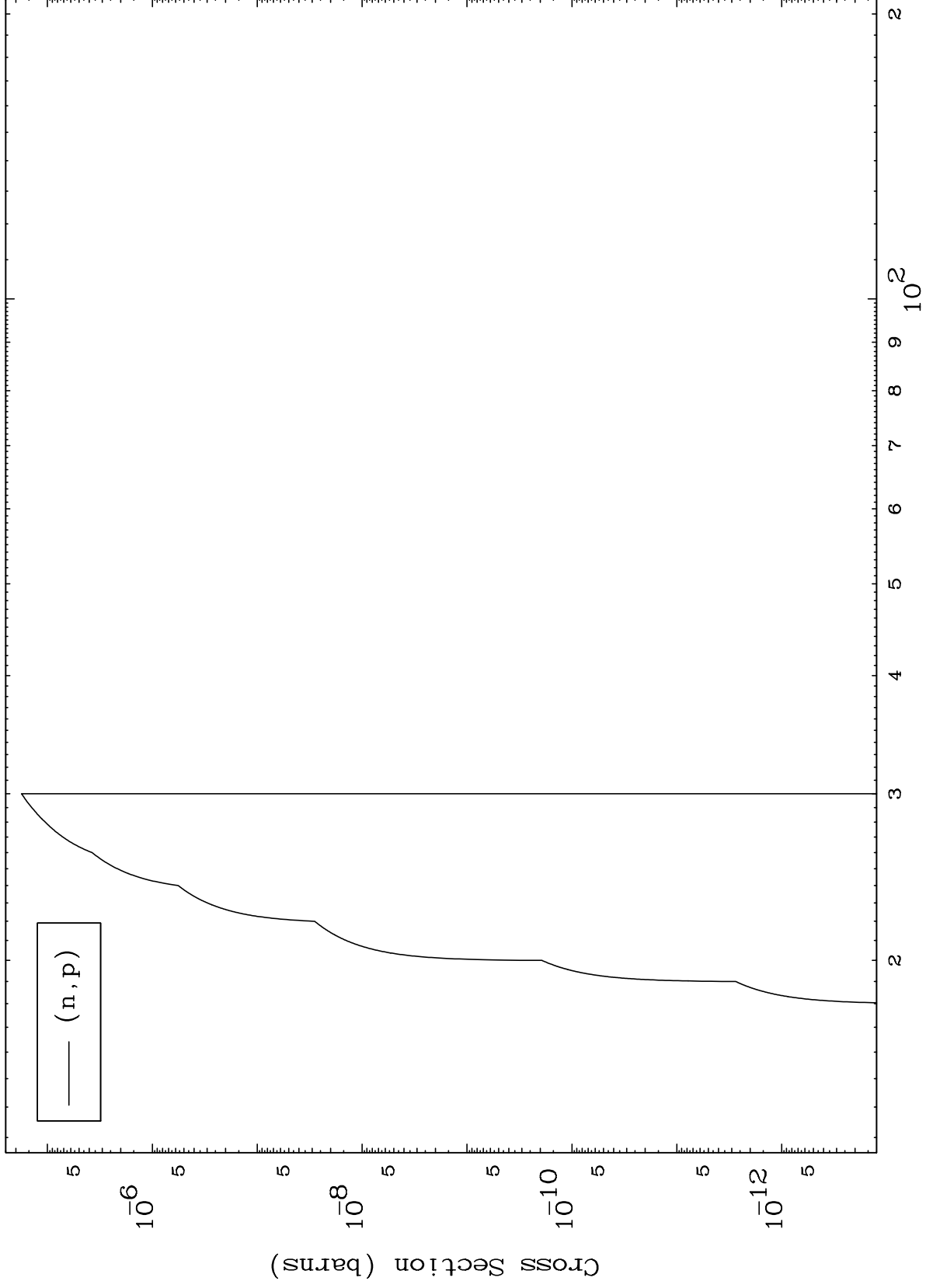




MAT 9938

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

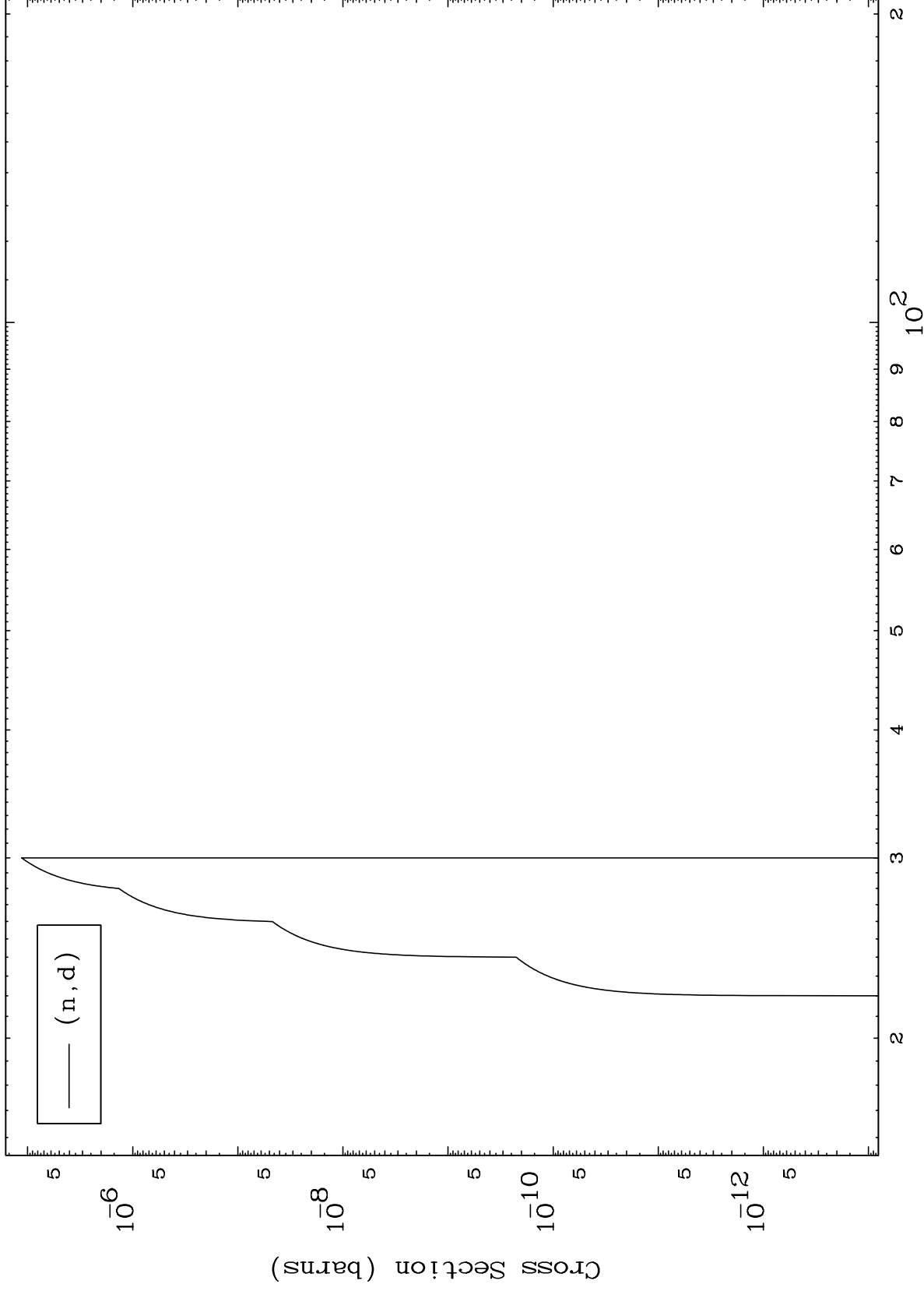
100-Fm-257



MAT 9938

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-257



5

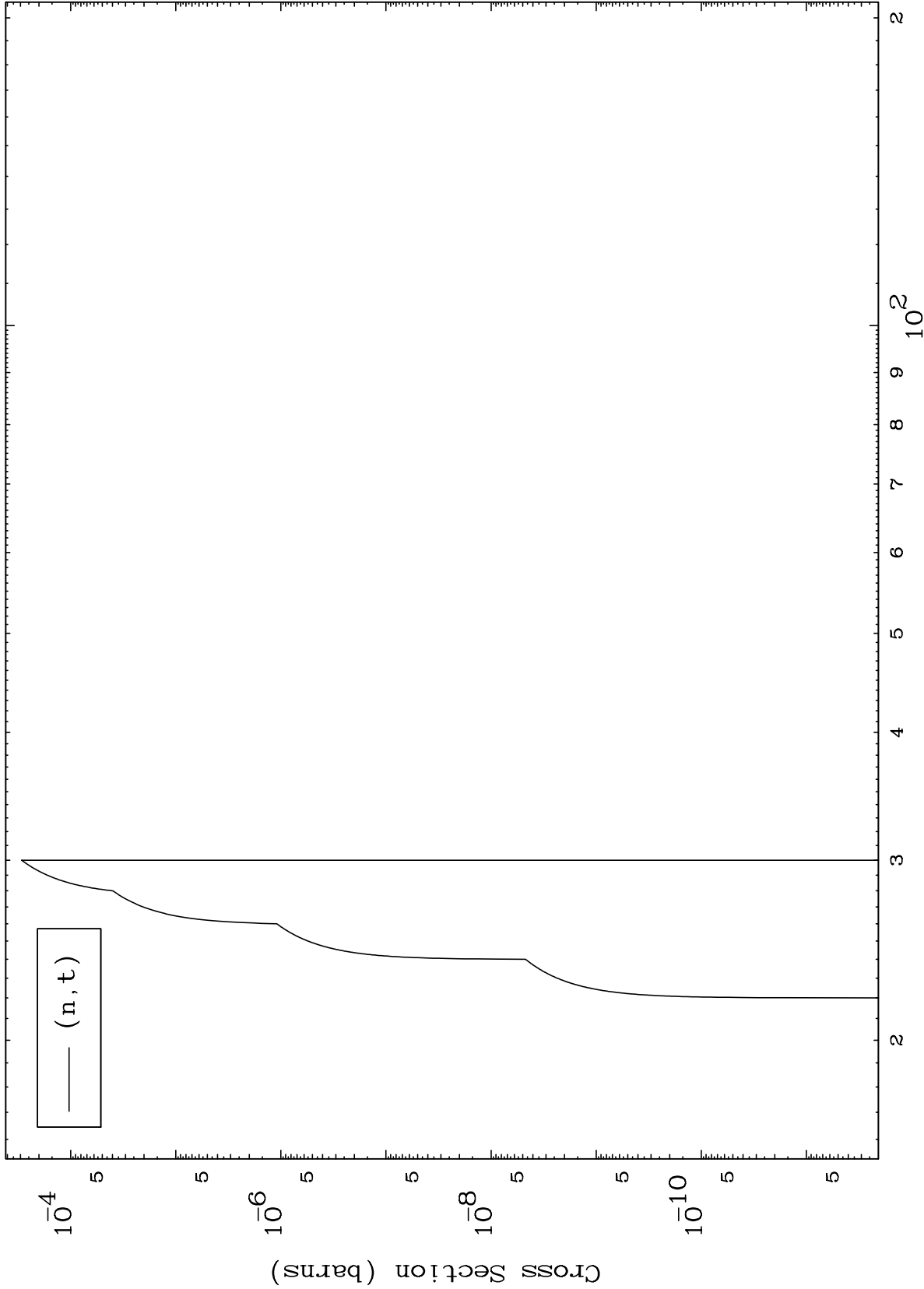
Incident Energy (MeV)

100-Fm-257

MAT 9938

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-257



6

Incident Energy (MeV)

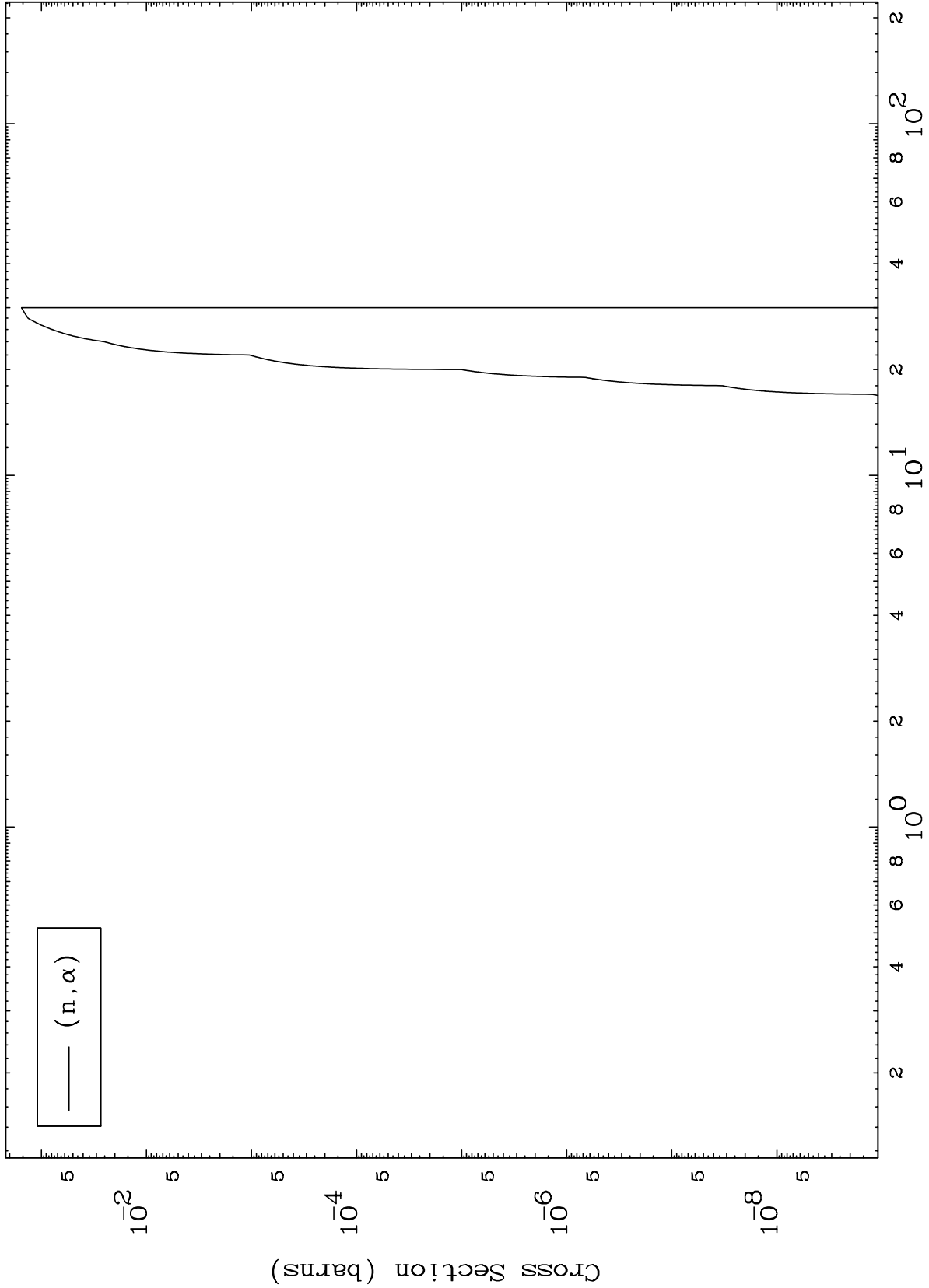
100-Fm-257

MAT 9938

( $\alpha, \alpha$ ) Levels

100-Fm-257

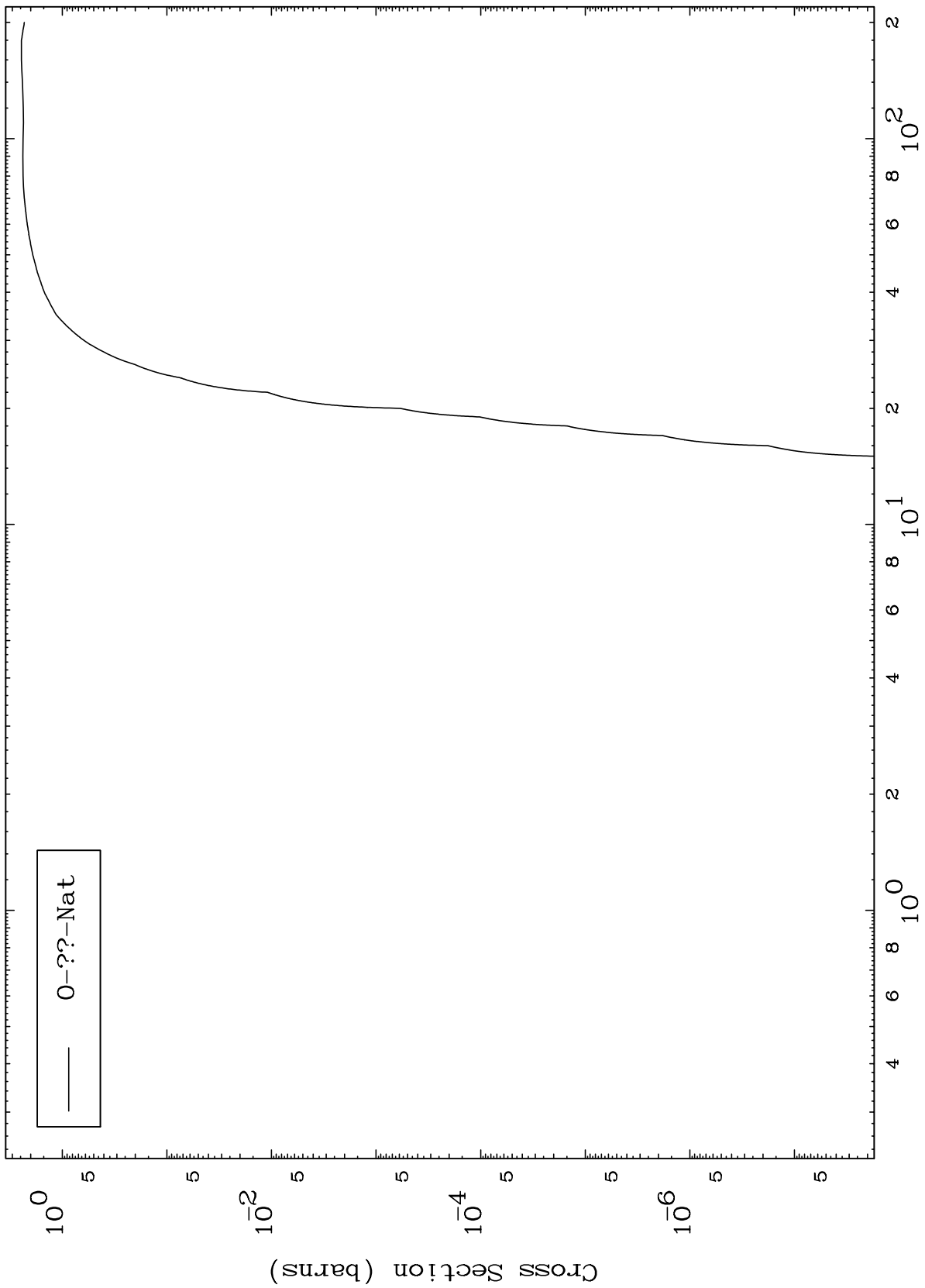
0 Kelvin Cross Sections



MAT 9938

100-Fm-257

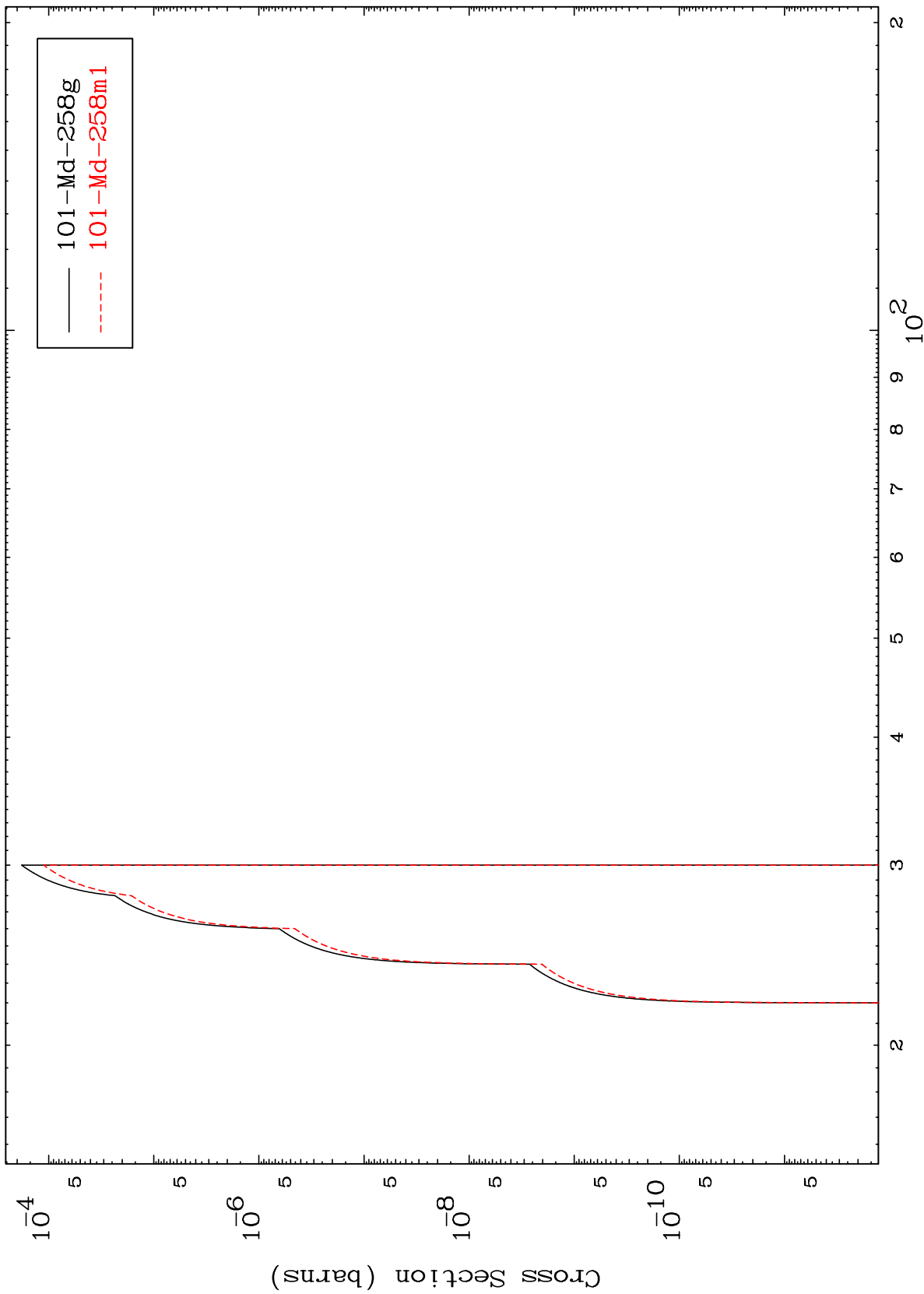
Fission  
Radionuclide Production Cross Section



MAT 9938

100-Fm-257

(n,t)  
Radionuclide Production Cross Section



9

Incident Energy (MeV)

100-Fm-257