

Program EVALPLOT  
(Version 2021-1)

by

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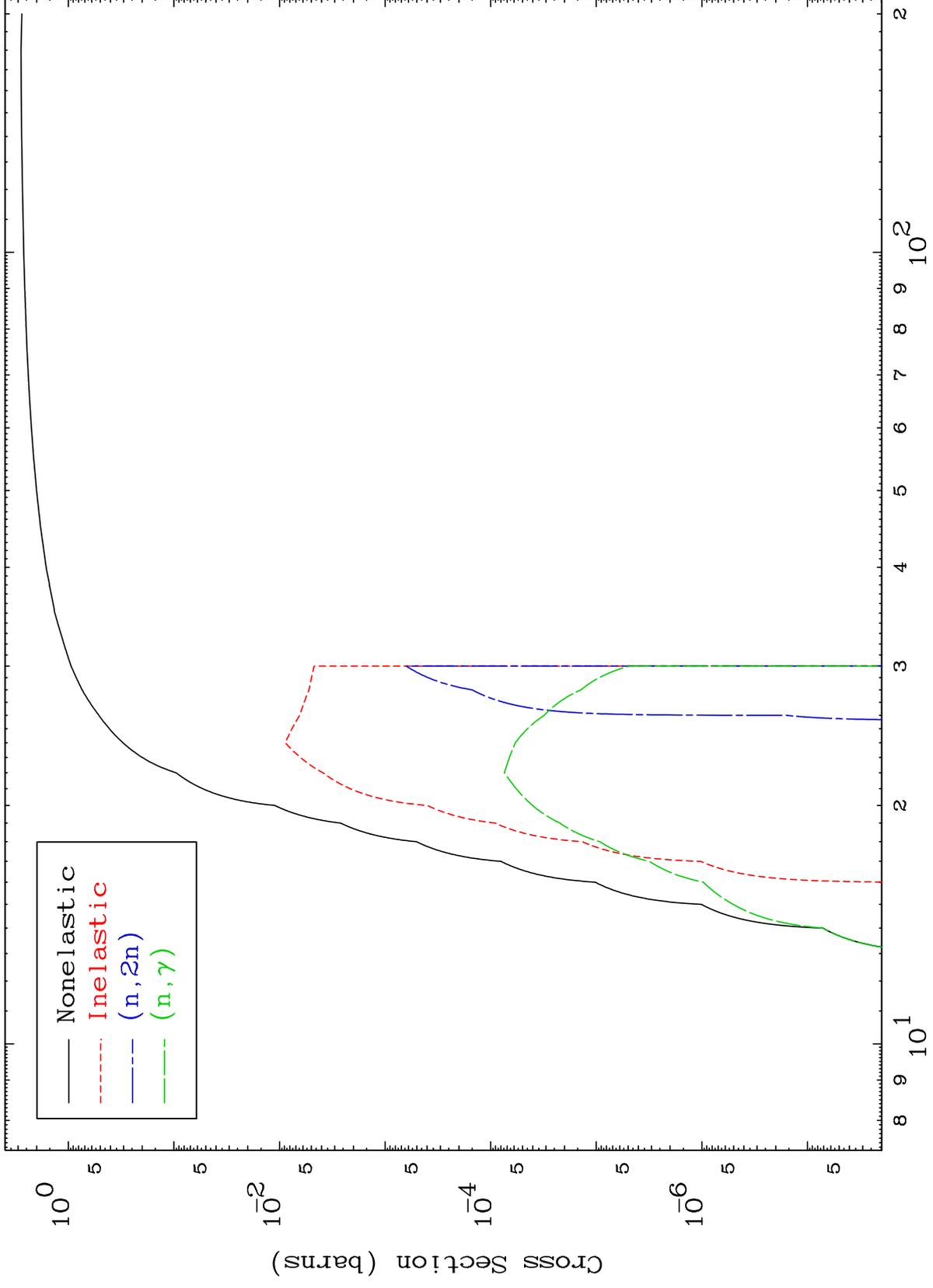
Press Mouse Button to Start

MAT 8713

$\alpha$  Major

87-Fr-208

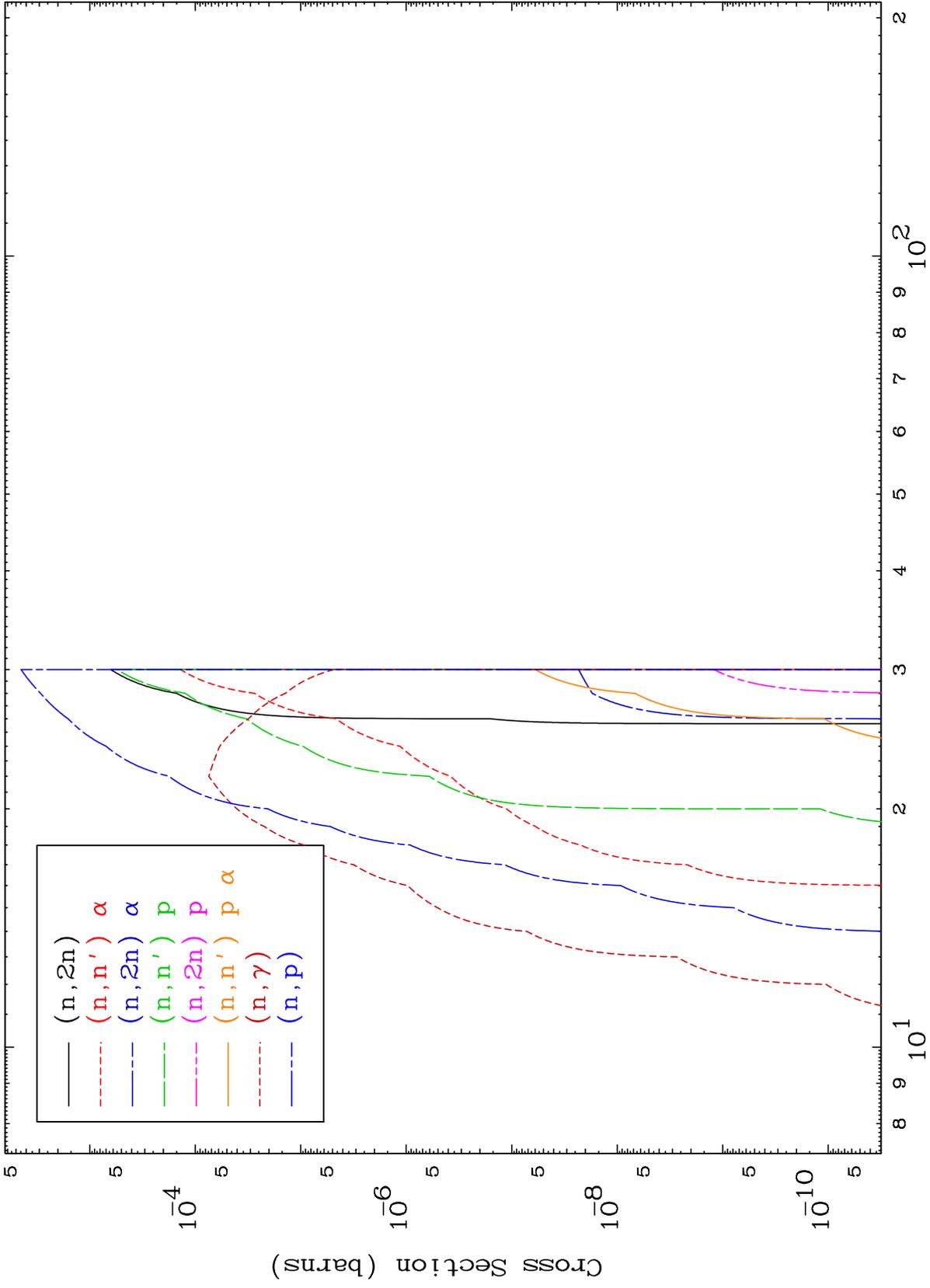
0 Kelvin Cross Sections



1

Incident Energy (MeV)

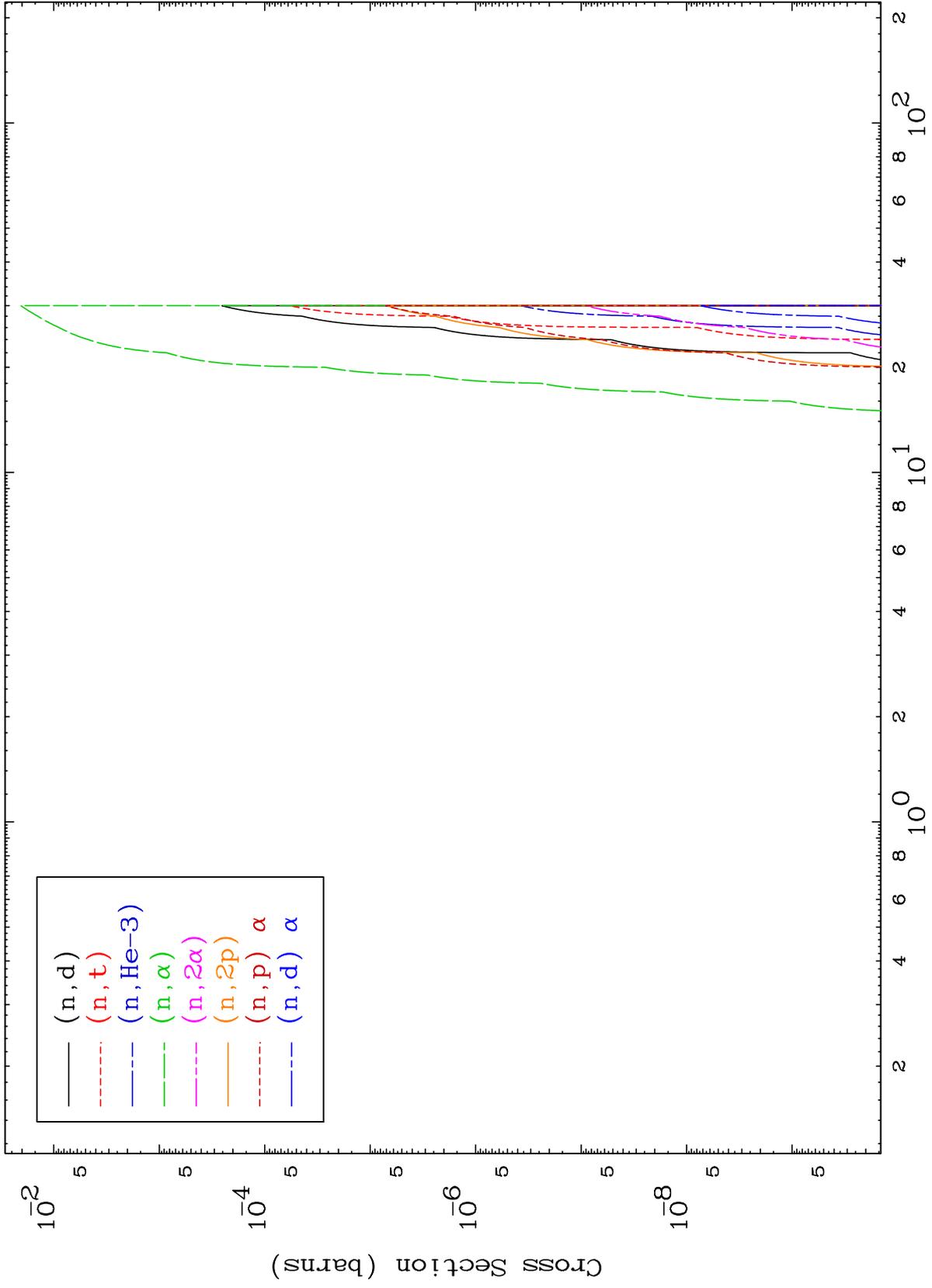
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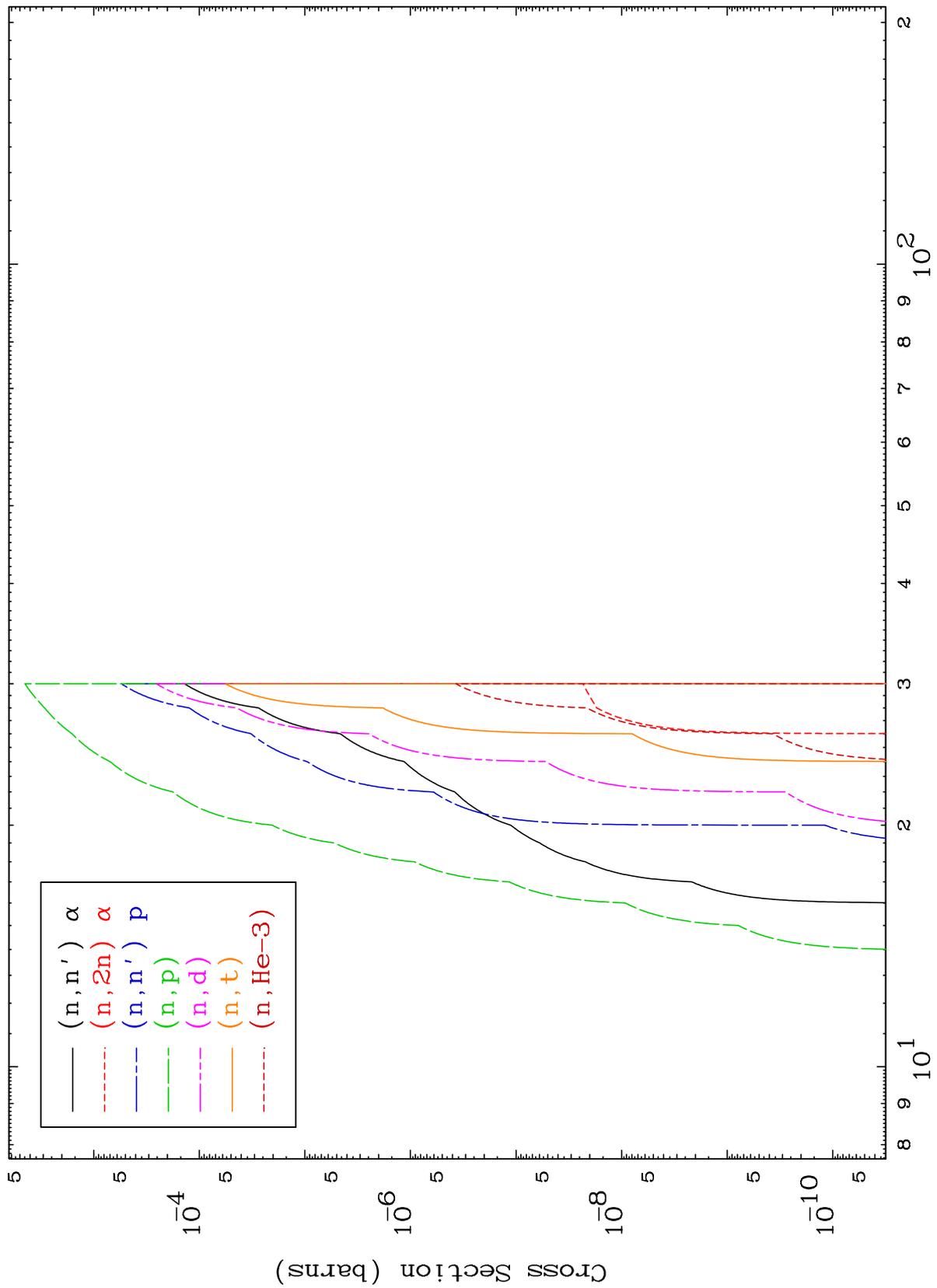


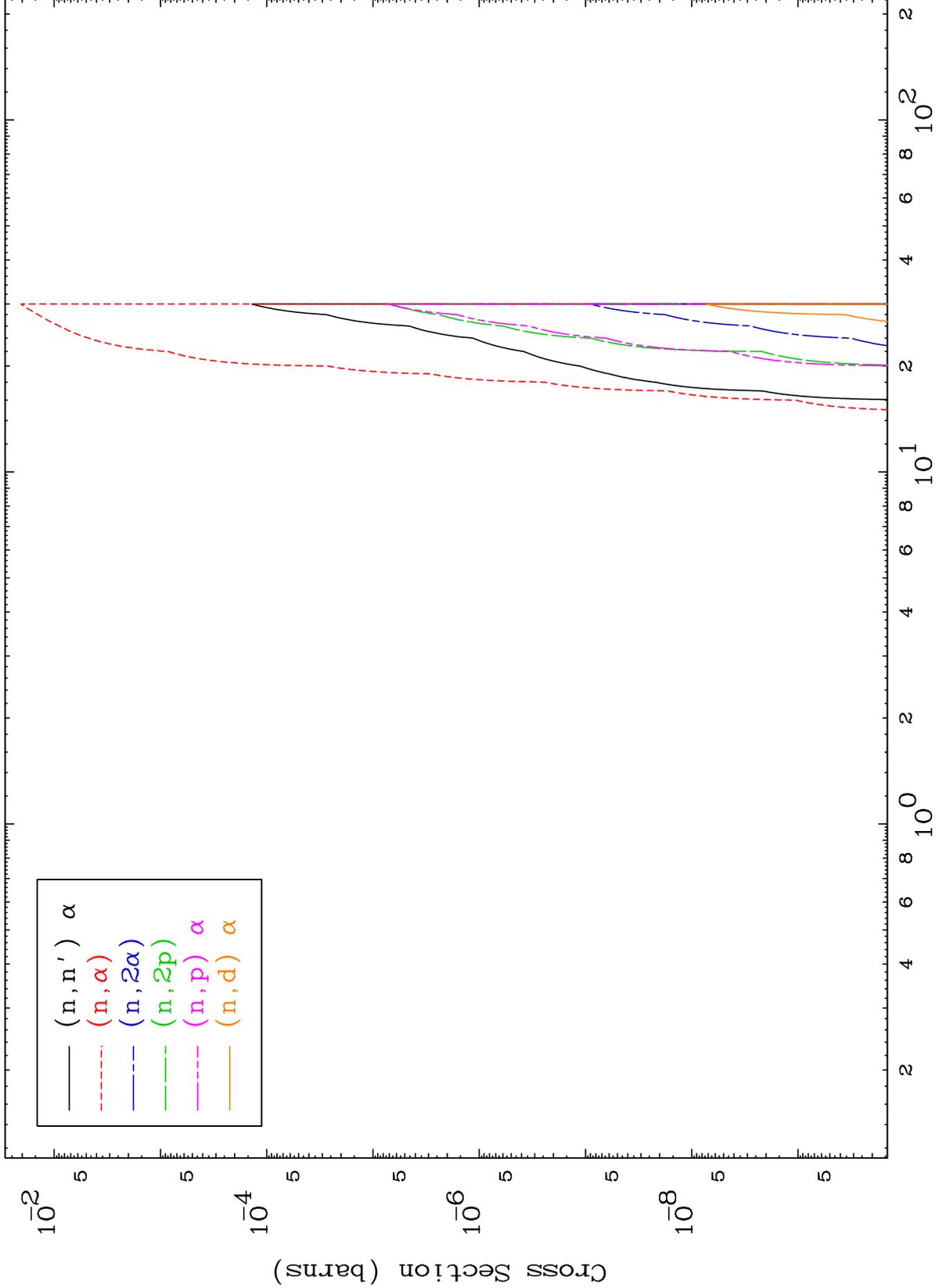
MAT 8713

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

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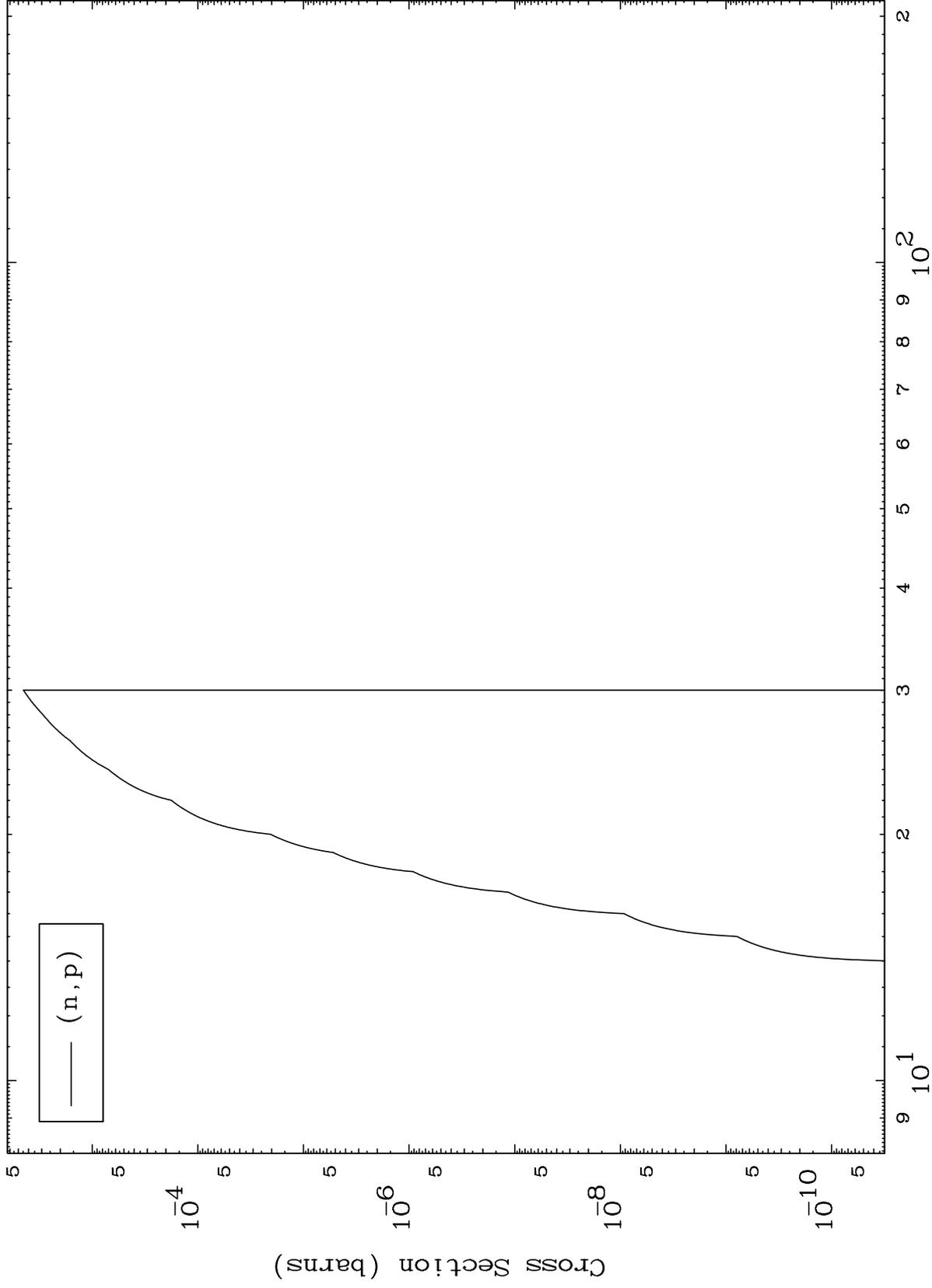




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( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-208



Incident Energy (MeV)

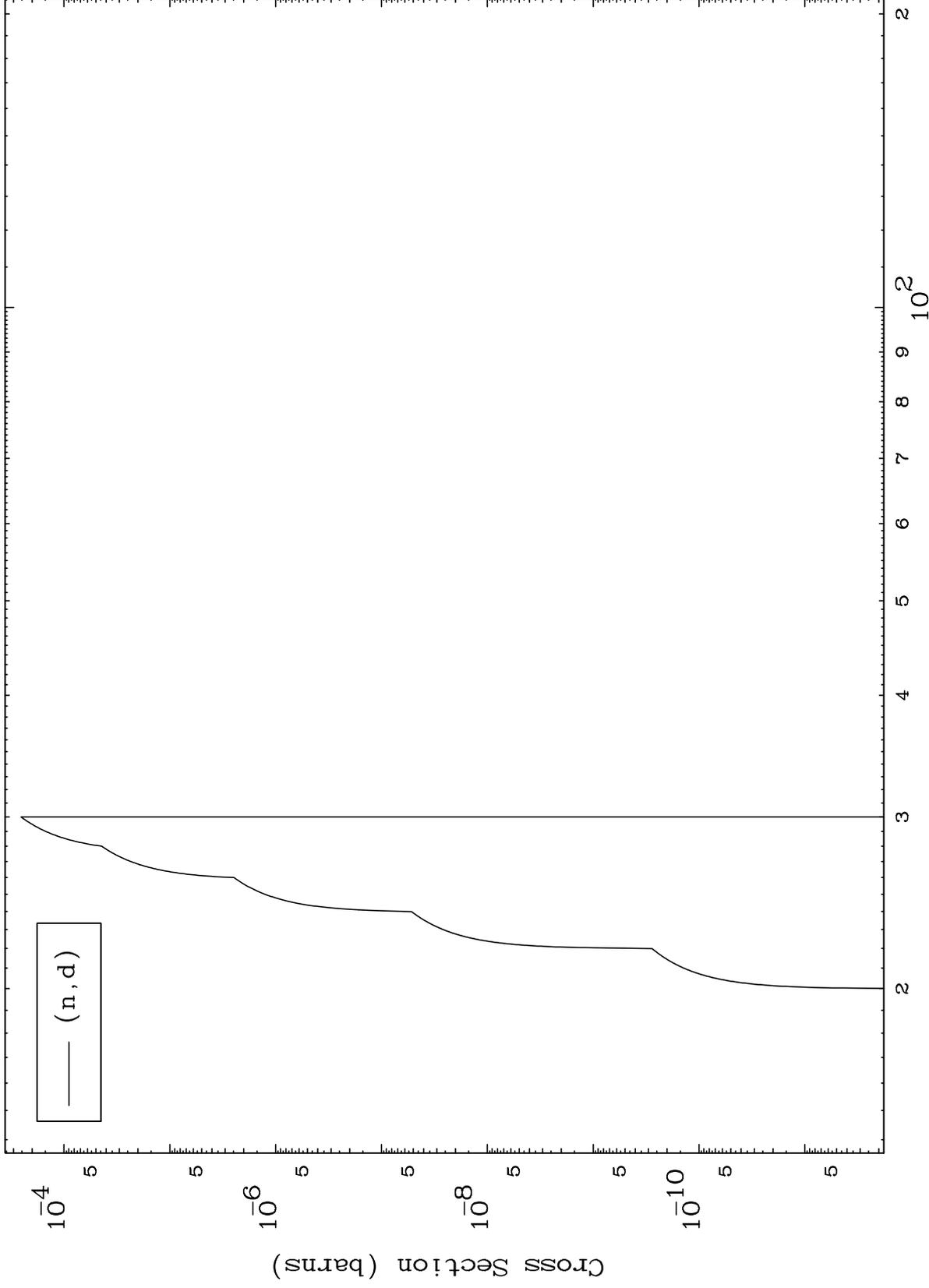
87-Fr-208

6

MAT 8713

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

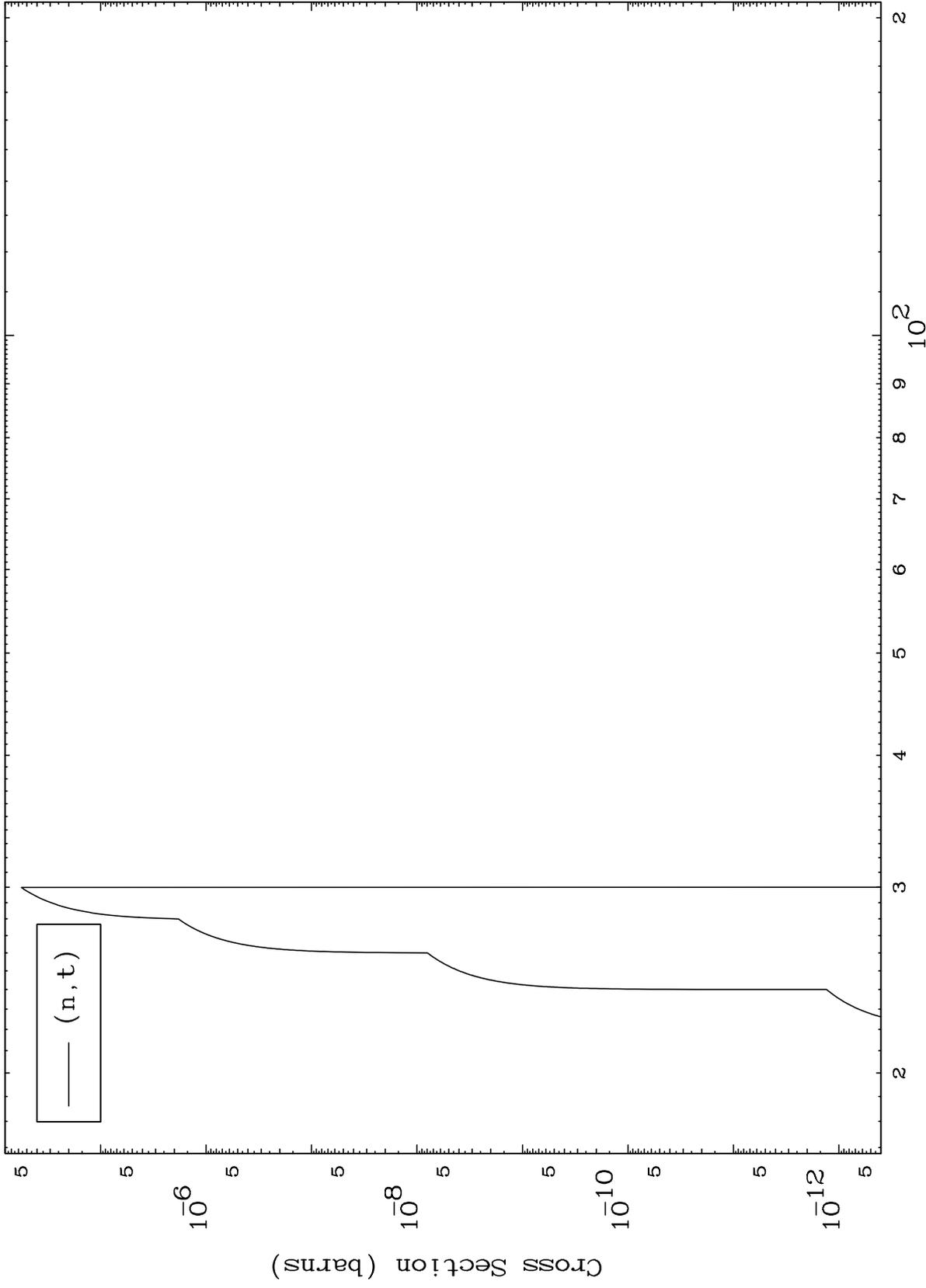
87-Fr-208



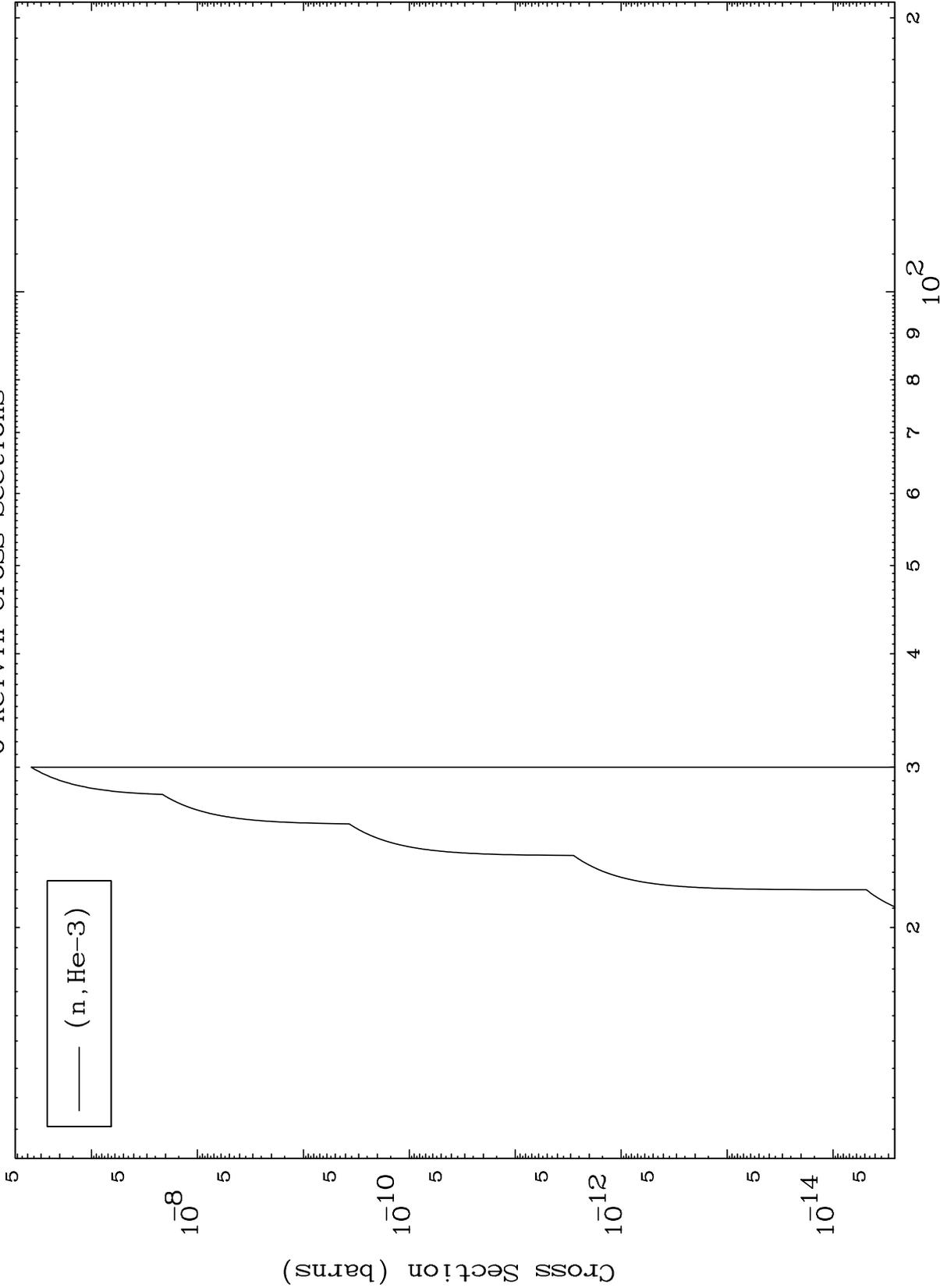
7

Incident Energy (MeV)

87-Fr-208



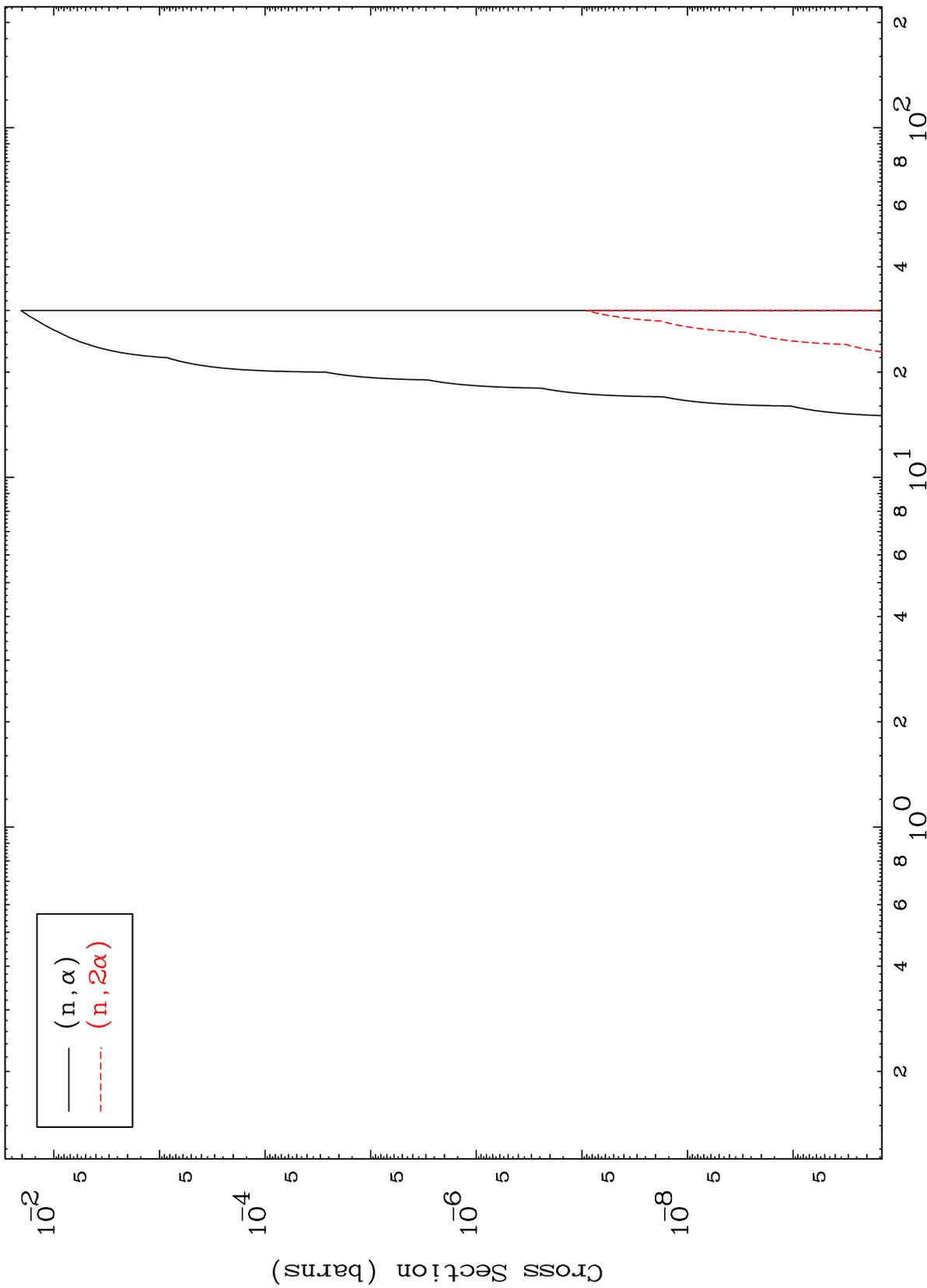
( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



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87-Fr-208

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



10

87-Fr-208

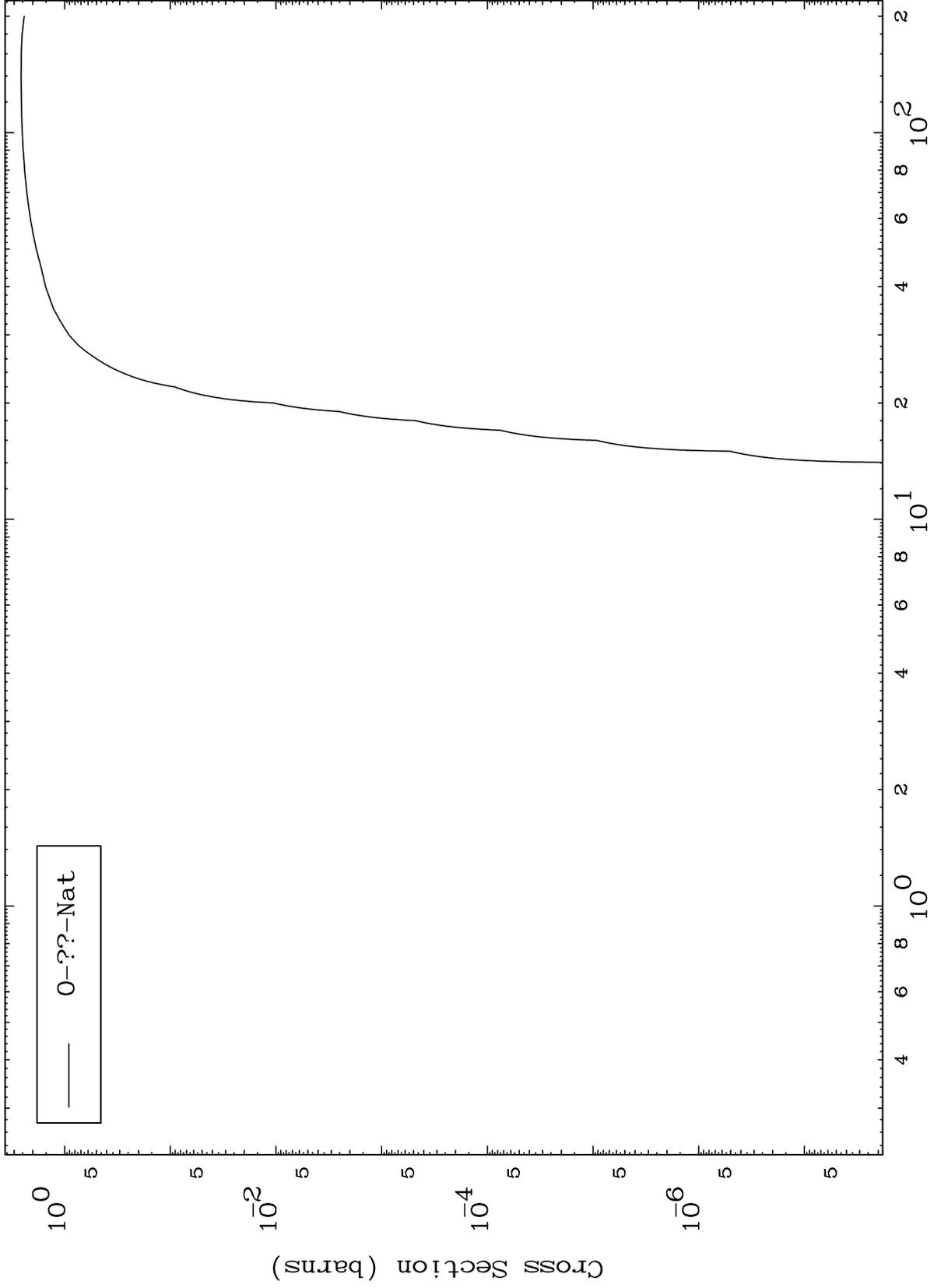
Incident Energy (MeV)

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Fission

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Radionuclide Production Cross Section

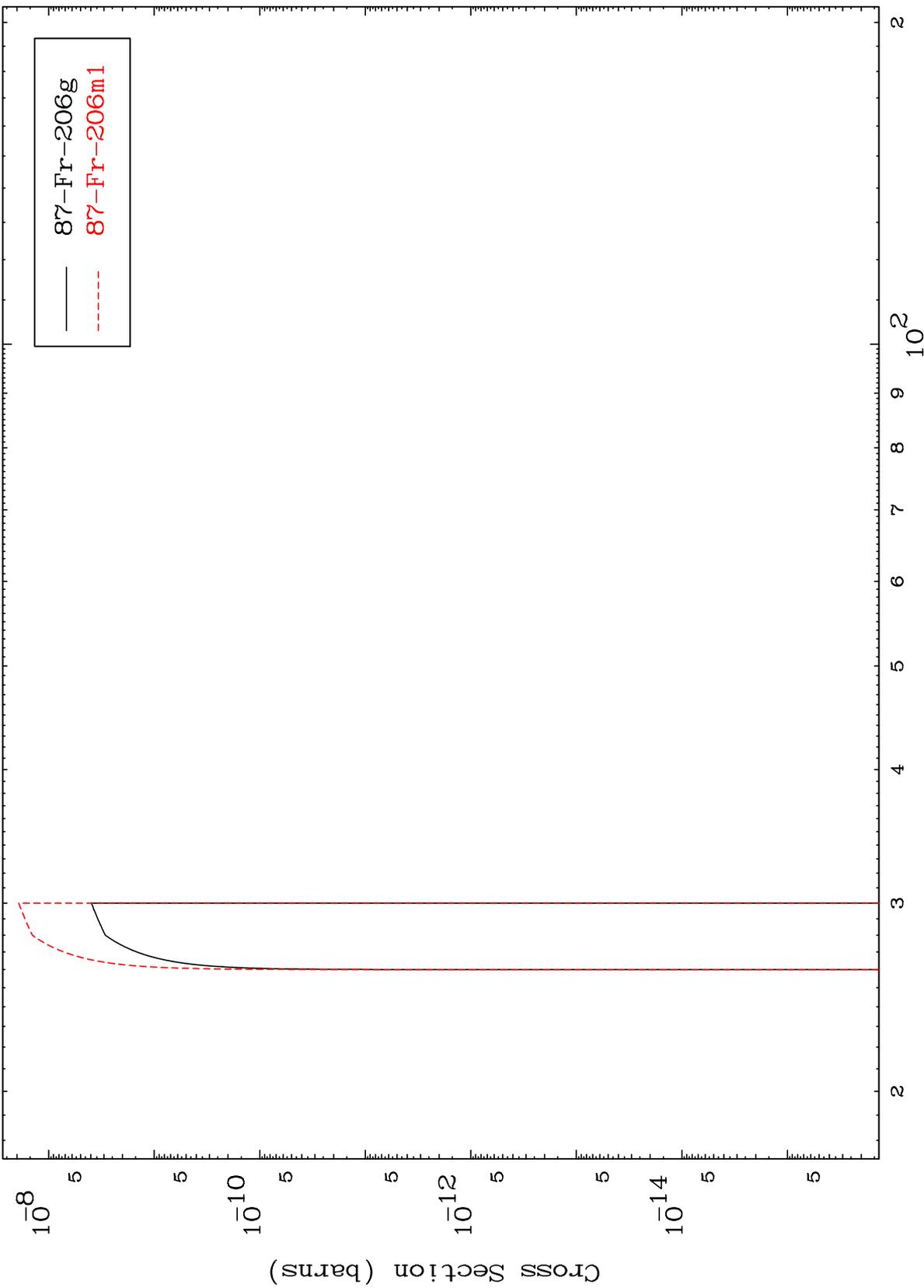


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87-Fr-208

(n,2n)  $\alpha$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

87-Fr-208

MAT 8713

(n,2α)

87-Fr-208

Radionuclide Production Cross Section

