

Program EVALPLOT  
(Version 2021-1)

by

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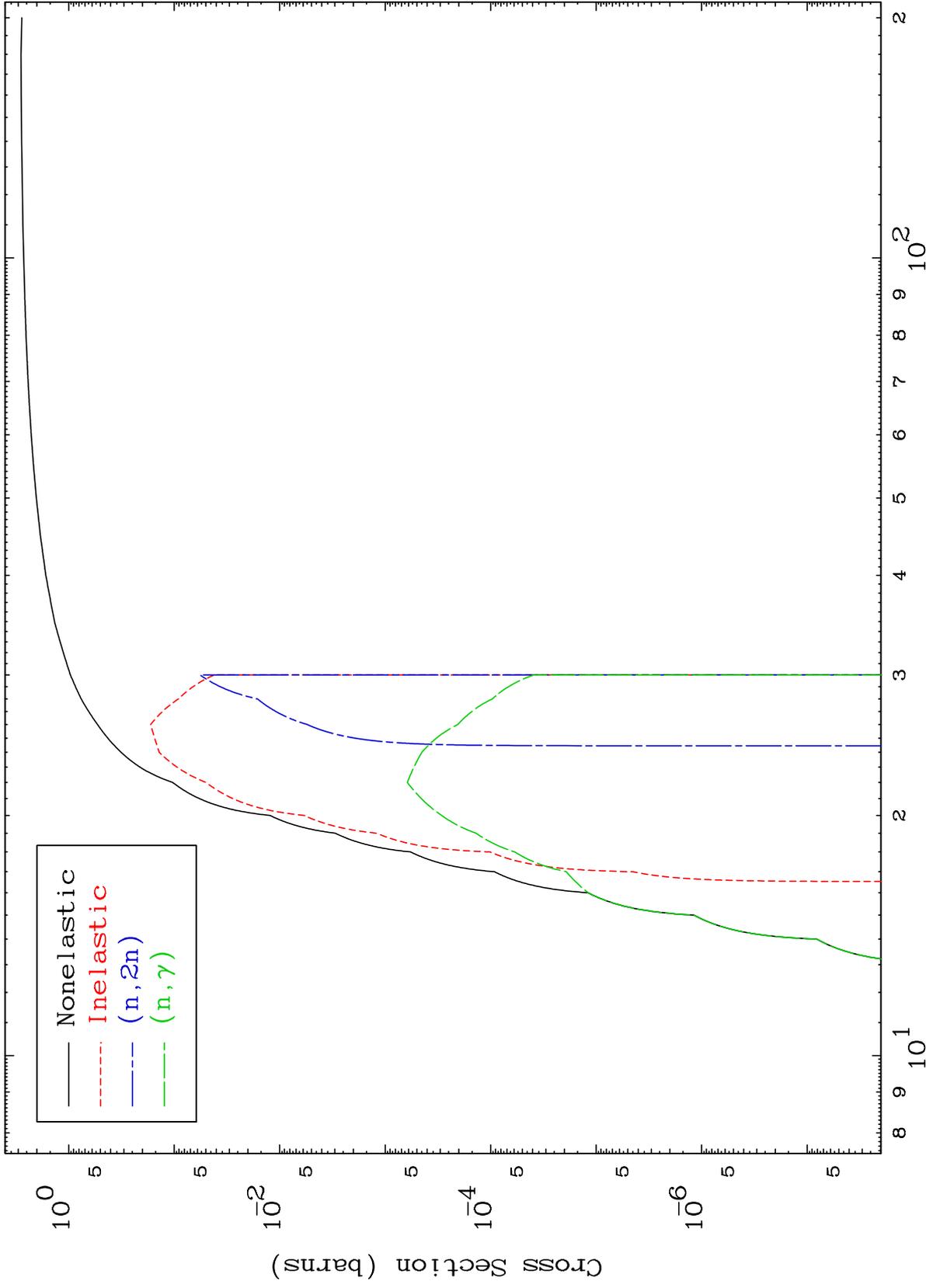
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Press Mouse Button to Start

MAT 8722

0 Kelvin Cross Sections

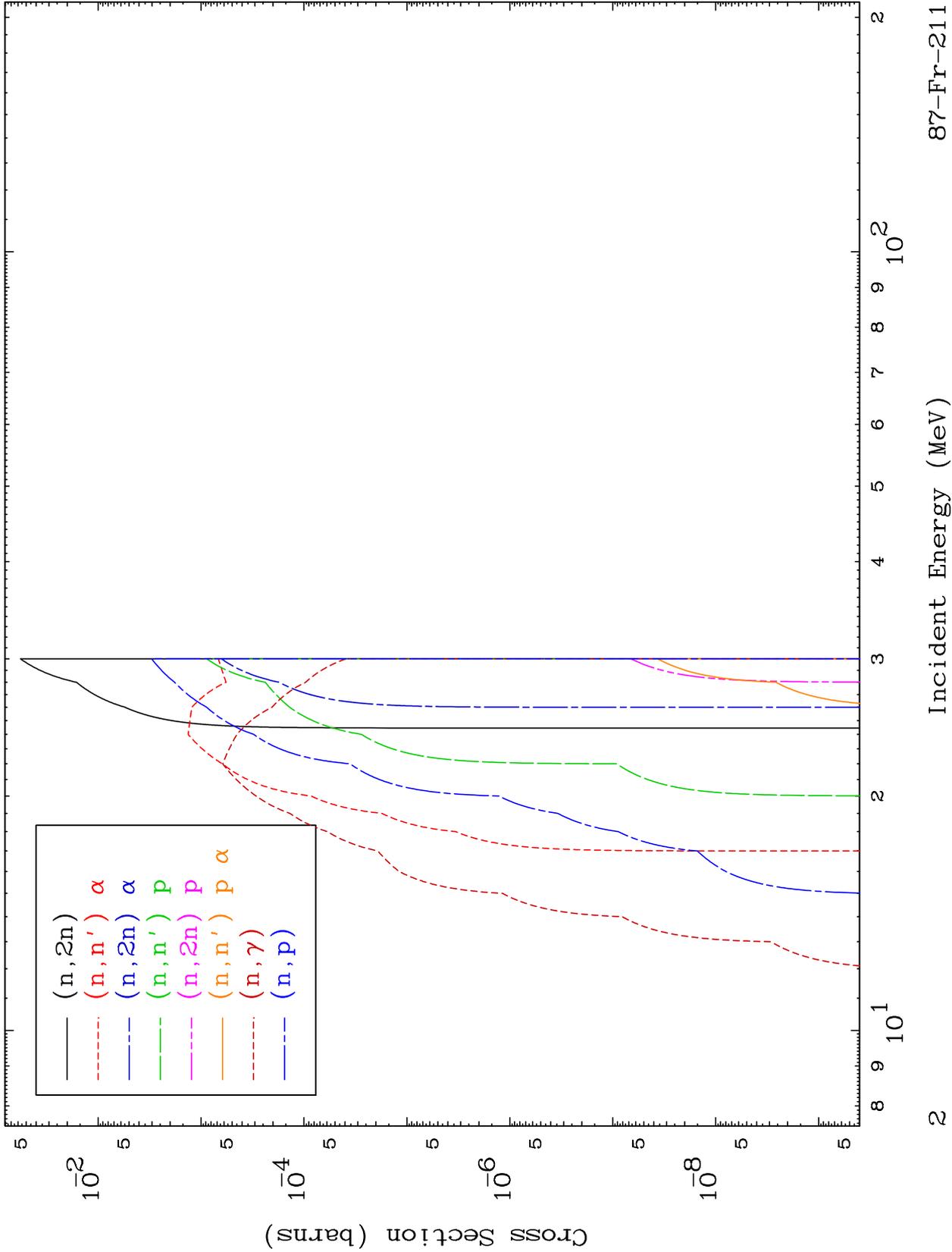
87-Fr-211



87-Fr-211

Incident Energy (MeV)

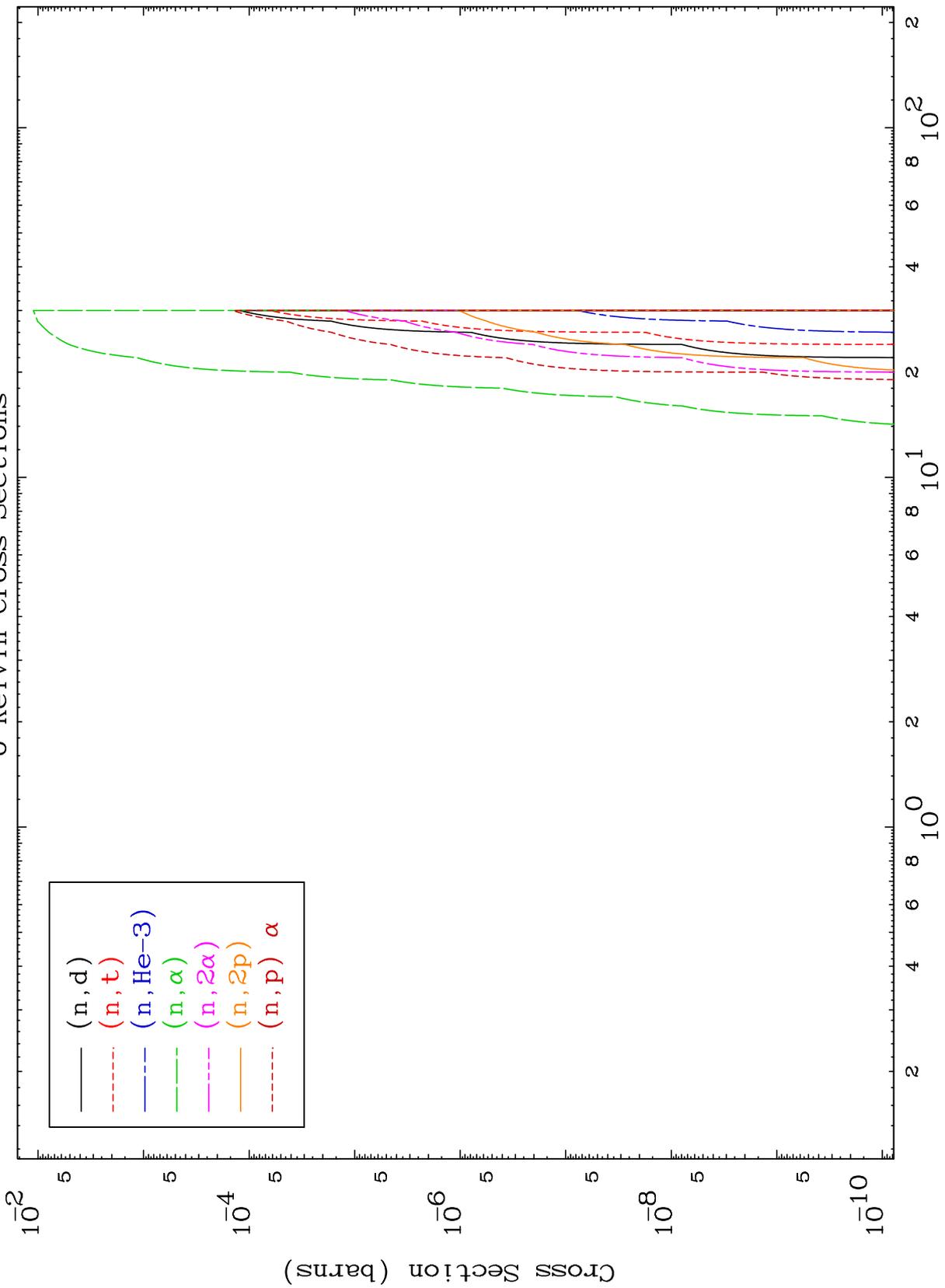
1

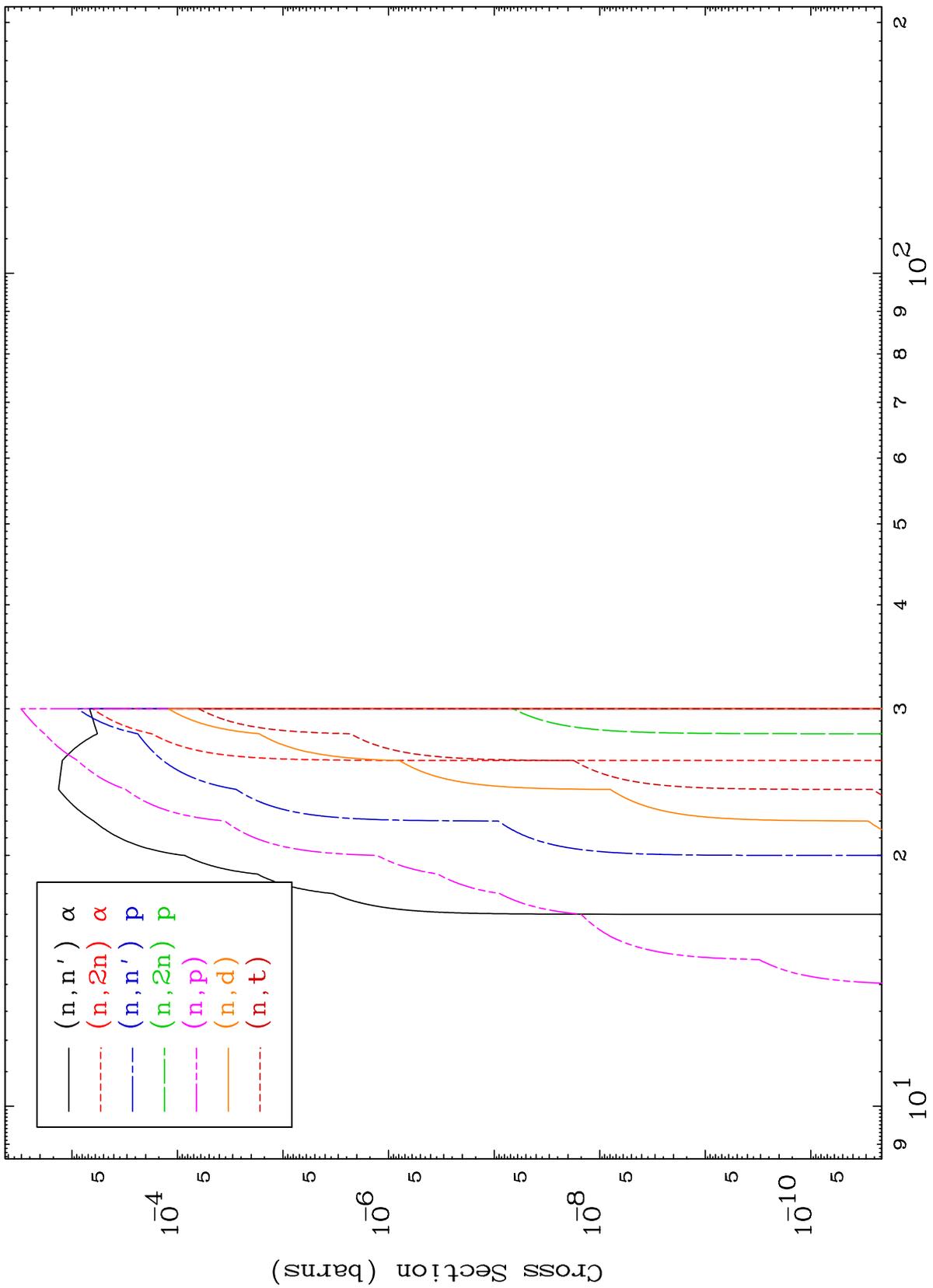


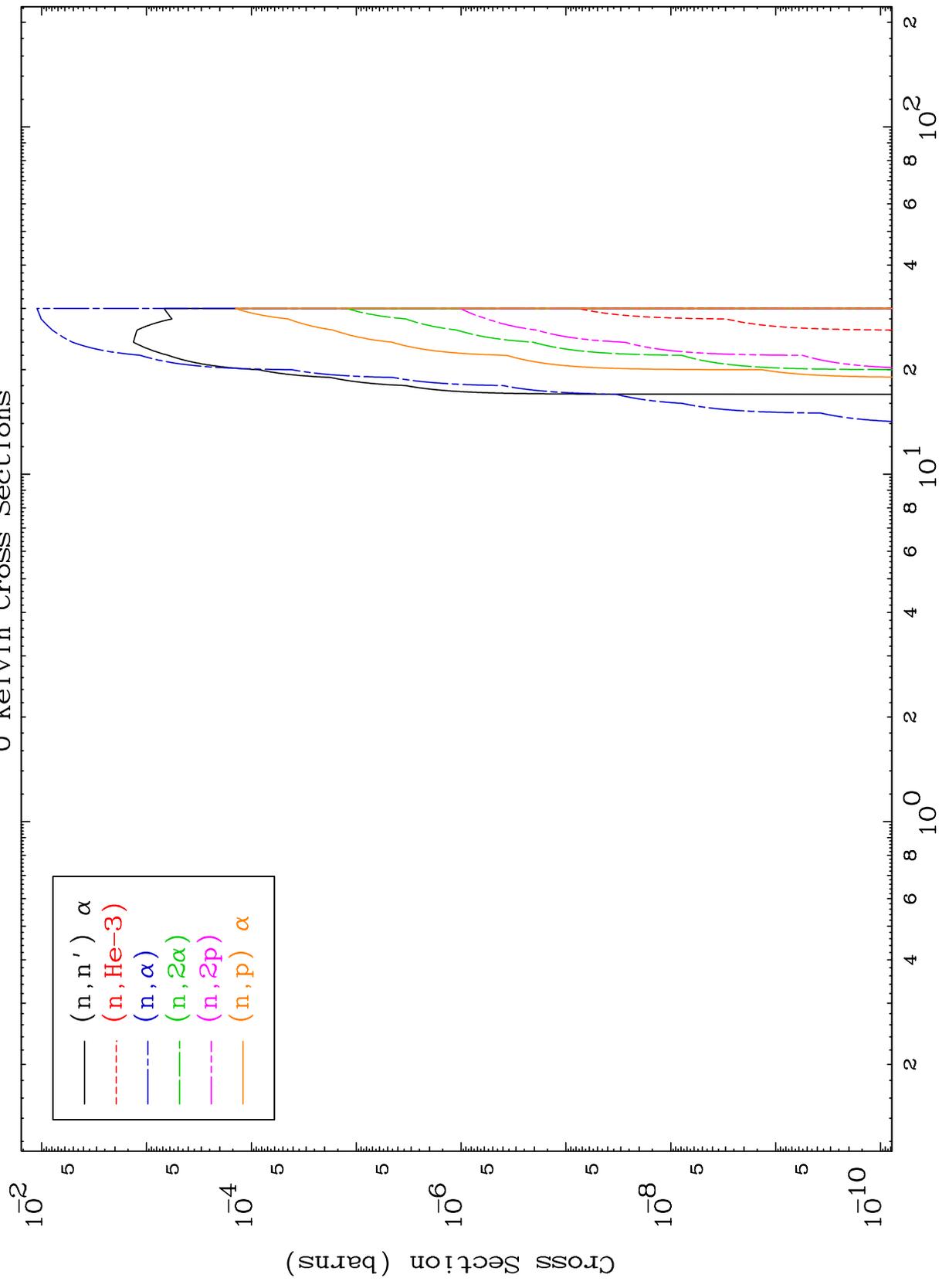
MAT 8722

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-211



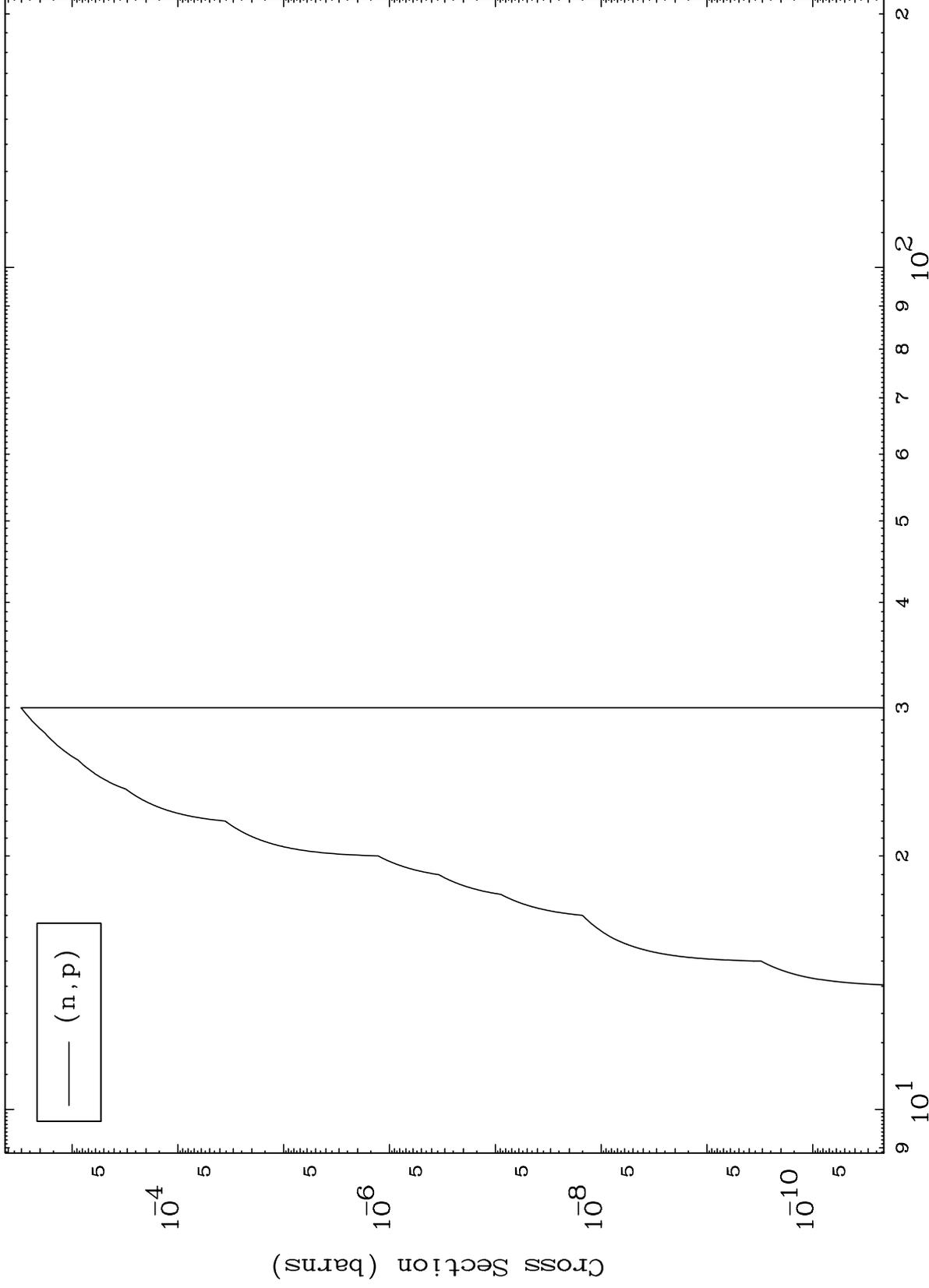




MAT 8722

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-211



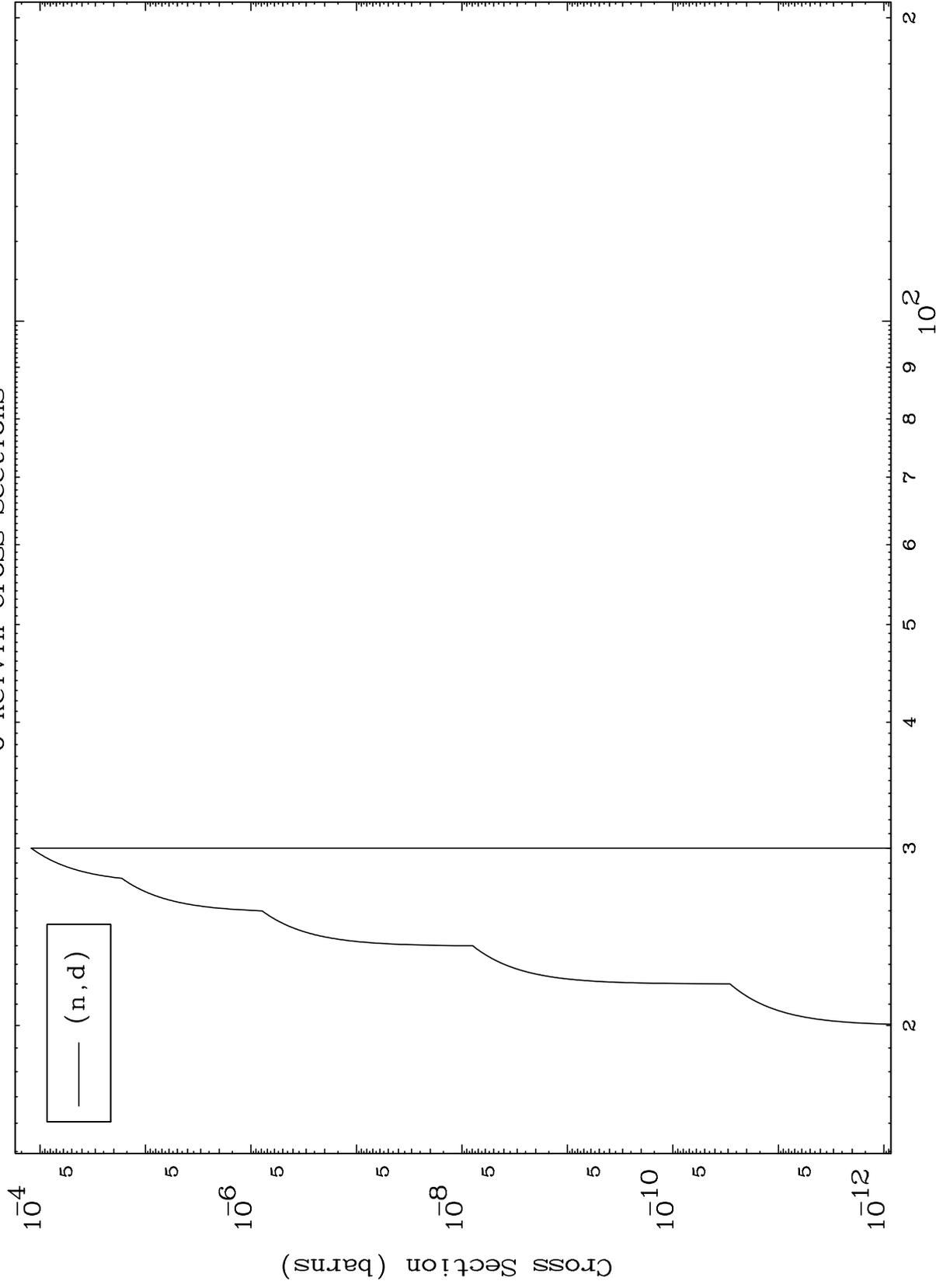
Incident Energy (MeV)

87-Fr-211

MAT 8722

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

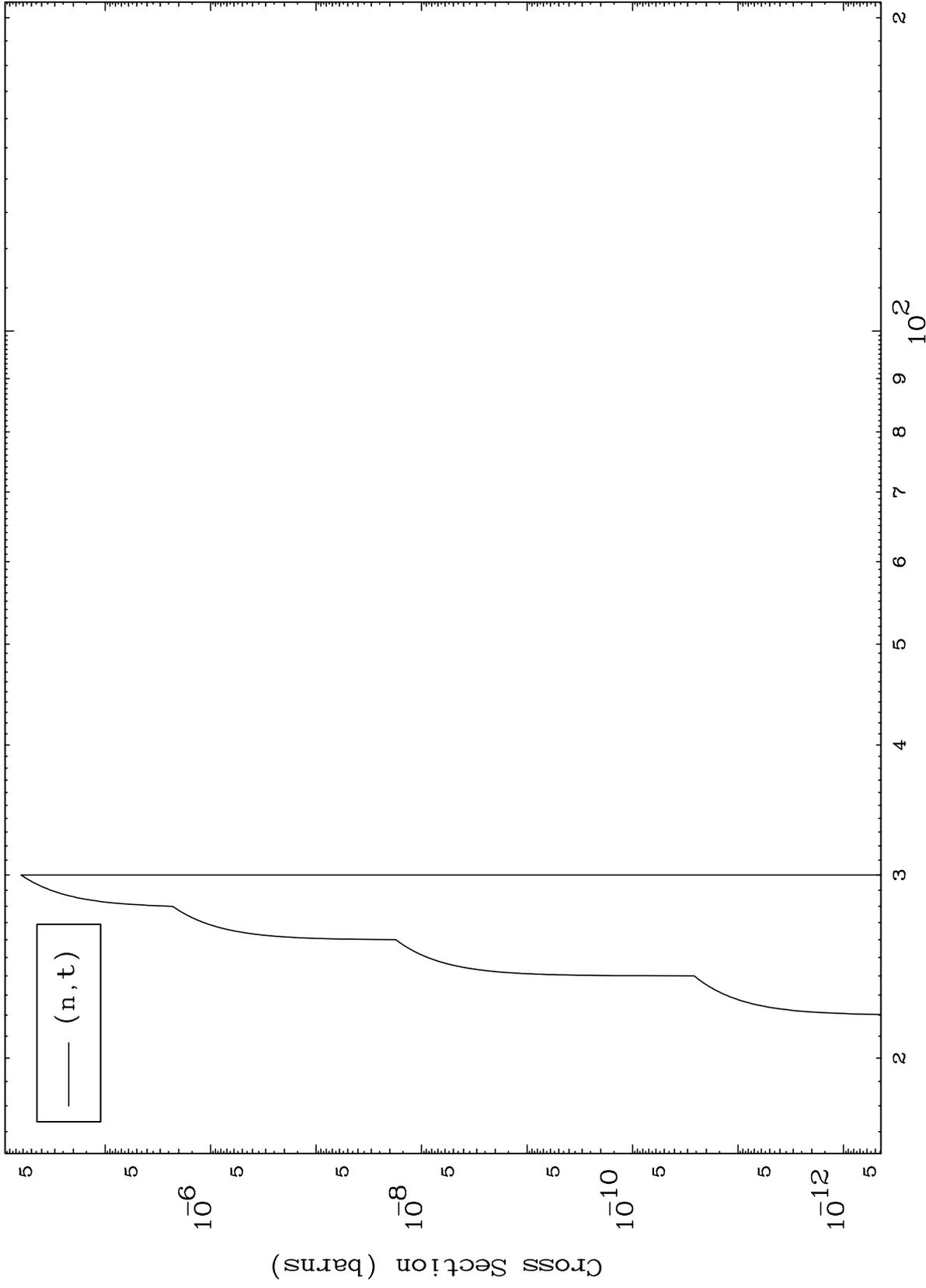
87-Fr-211



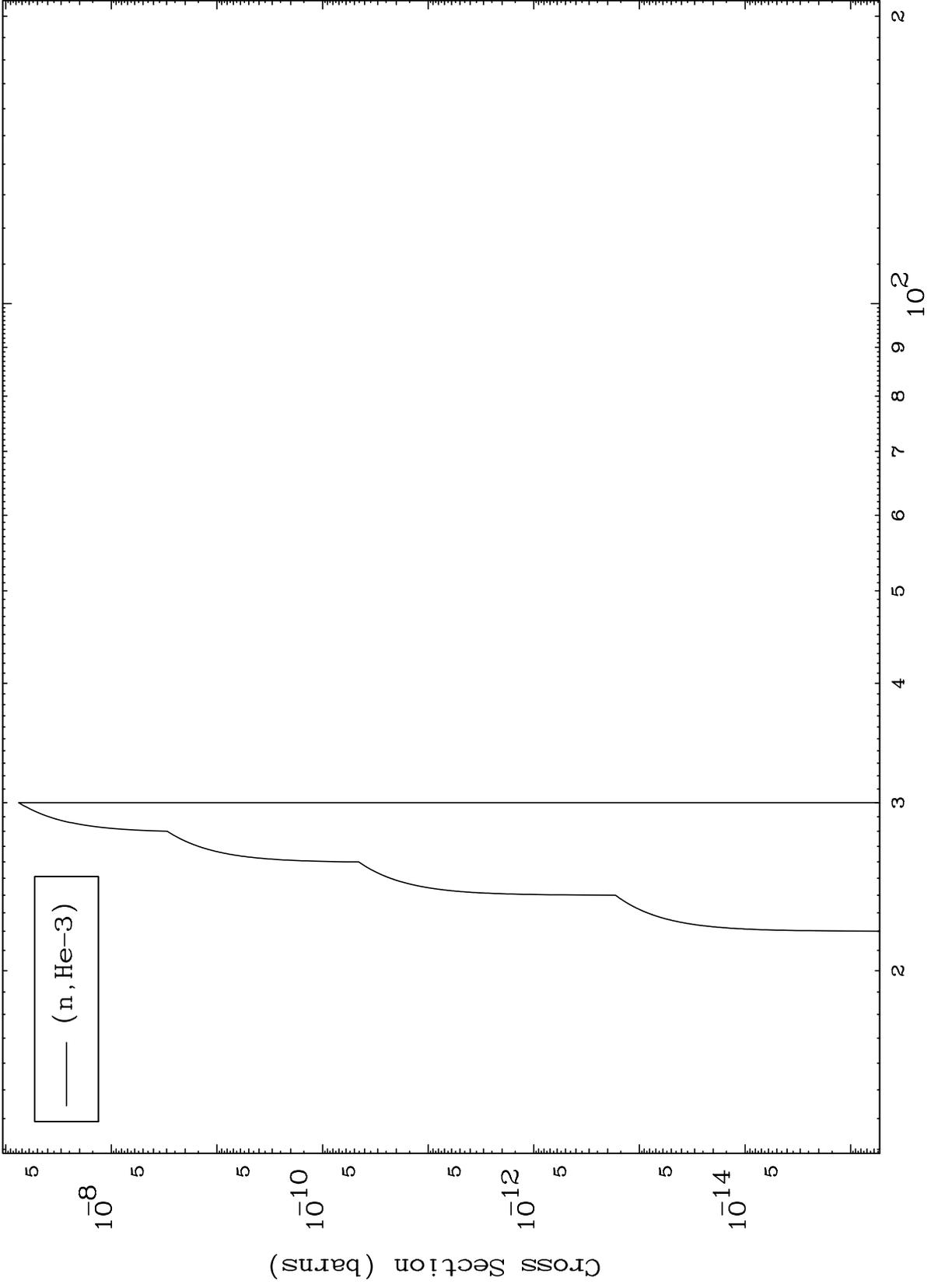
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Incident Energy (MeV)

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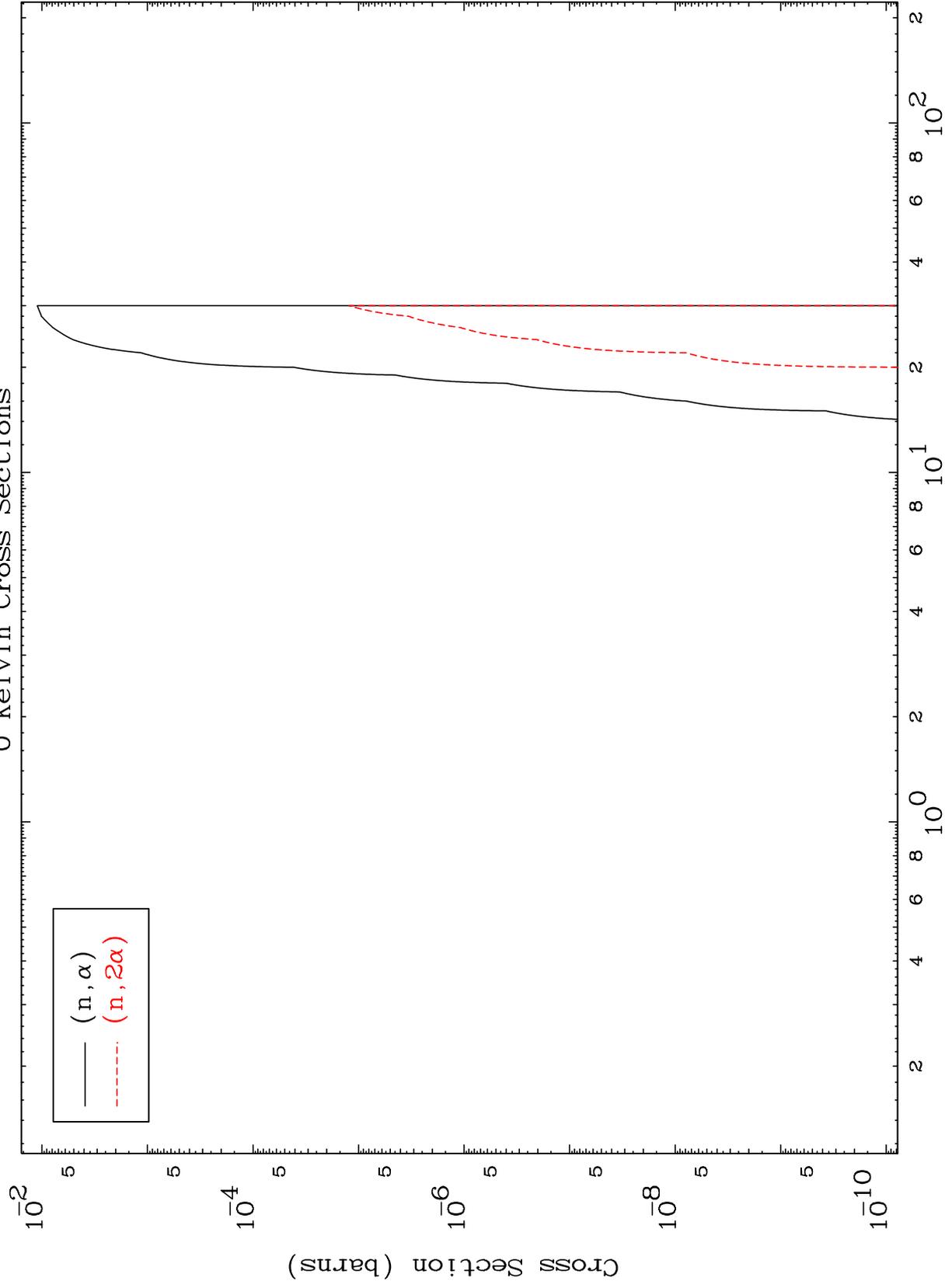
( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



MAT 8722

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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



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Incident Energy (MeV)

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Fission

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Radionuclide Production Cross Section

