

Program EVALPLOT
(Version 2021-1)

by

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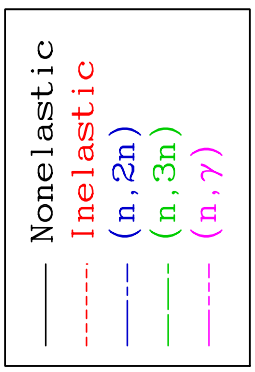
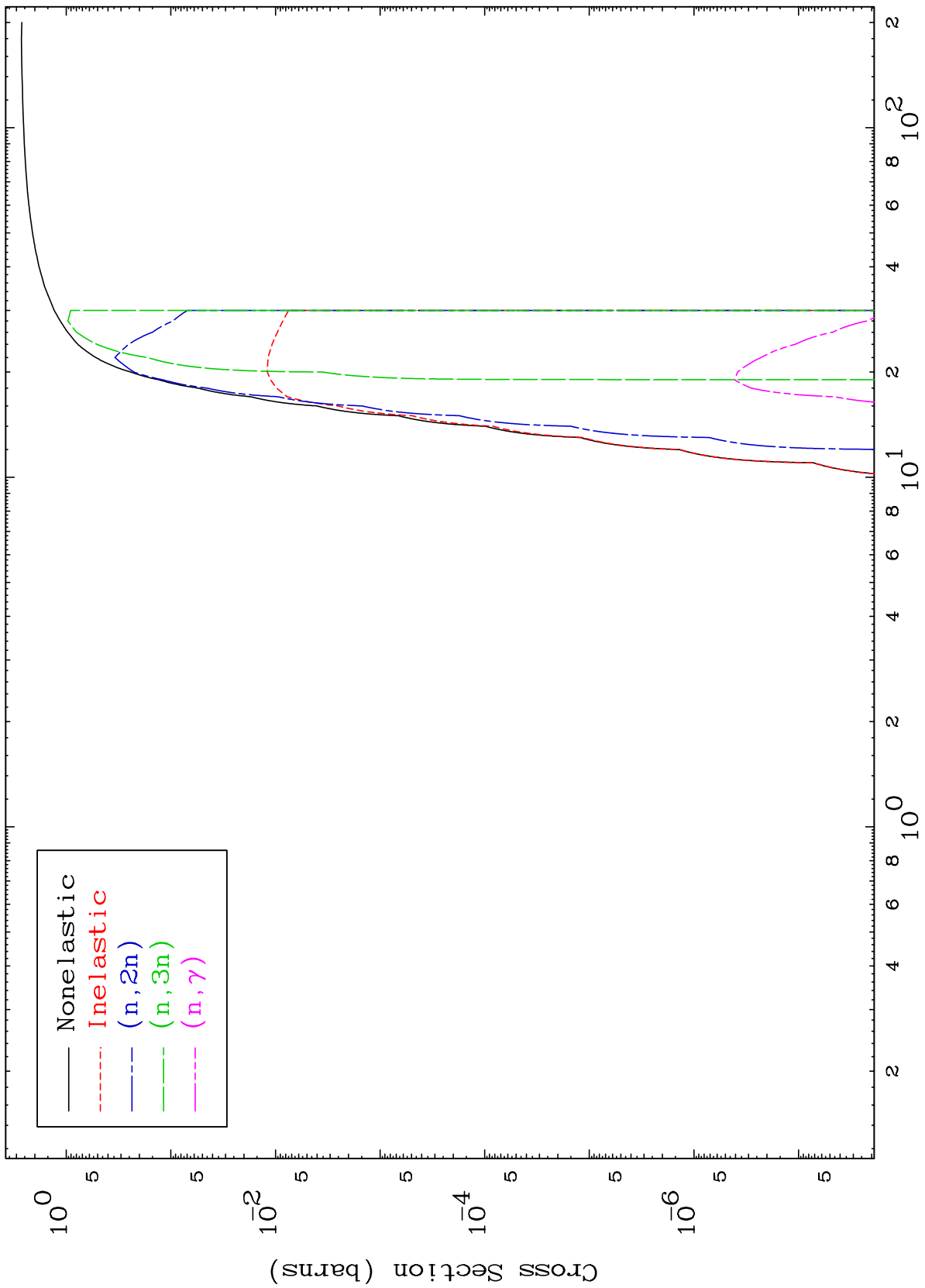
Press Mouse Button to Start

MAT 7256

0 Kelvin

α Major Cross Sections

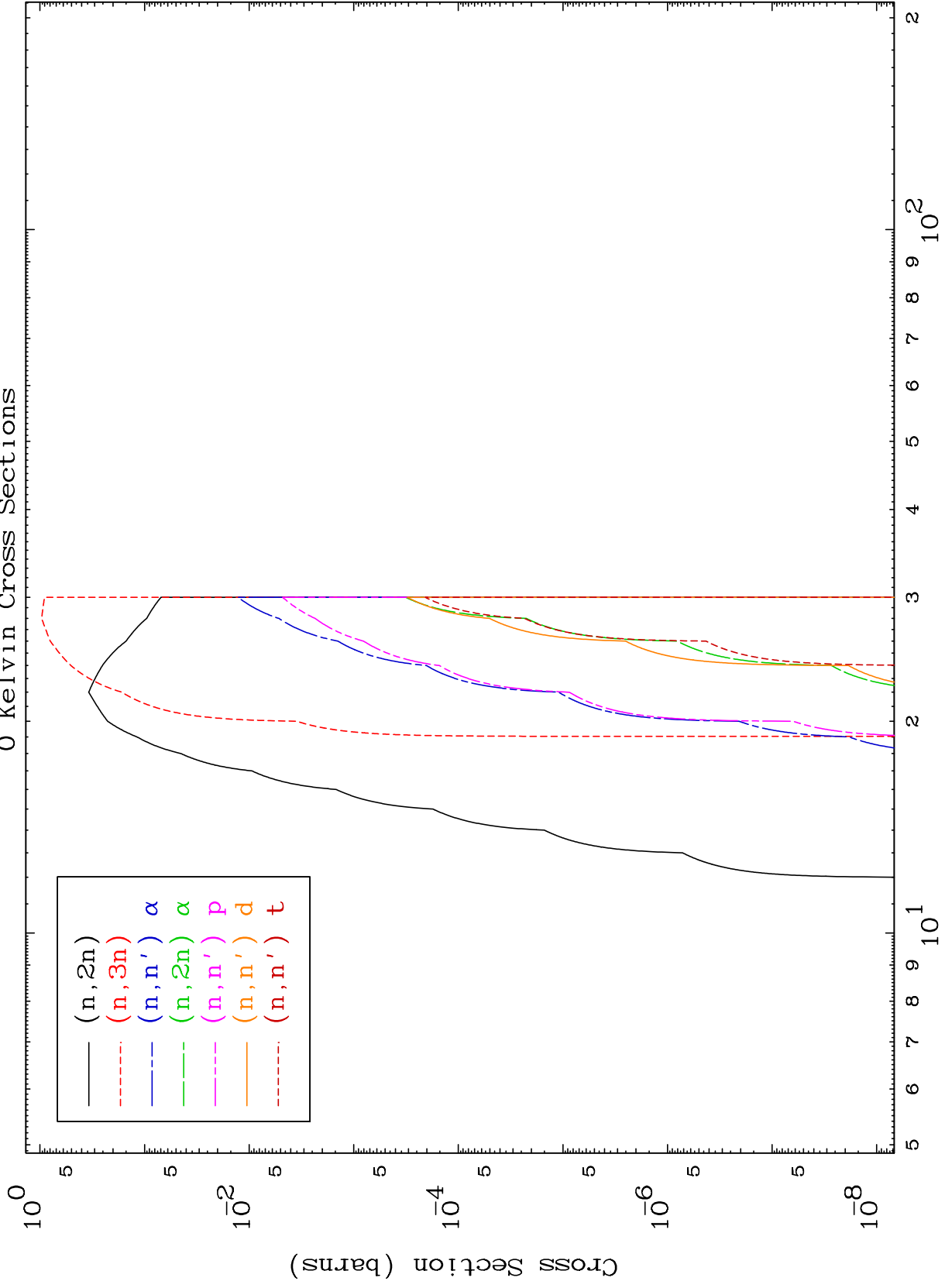
72-Hf-184m



MAT 7256

α Neutron Absorption
0 Kelvin Cross Sections

$^{72}\text{Hf}-184\text{m}$



2

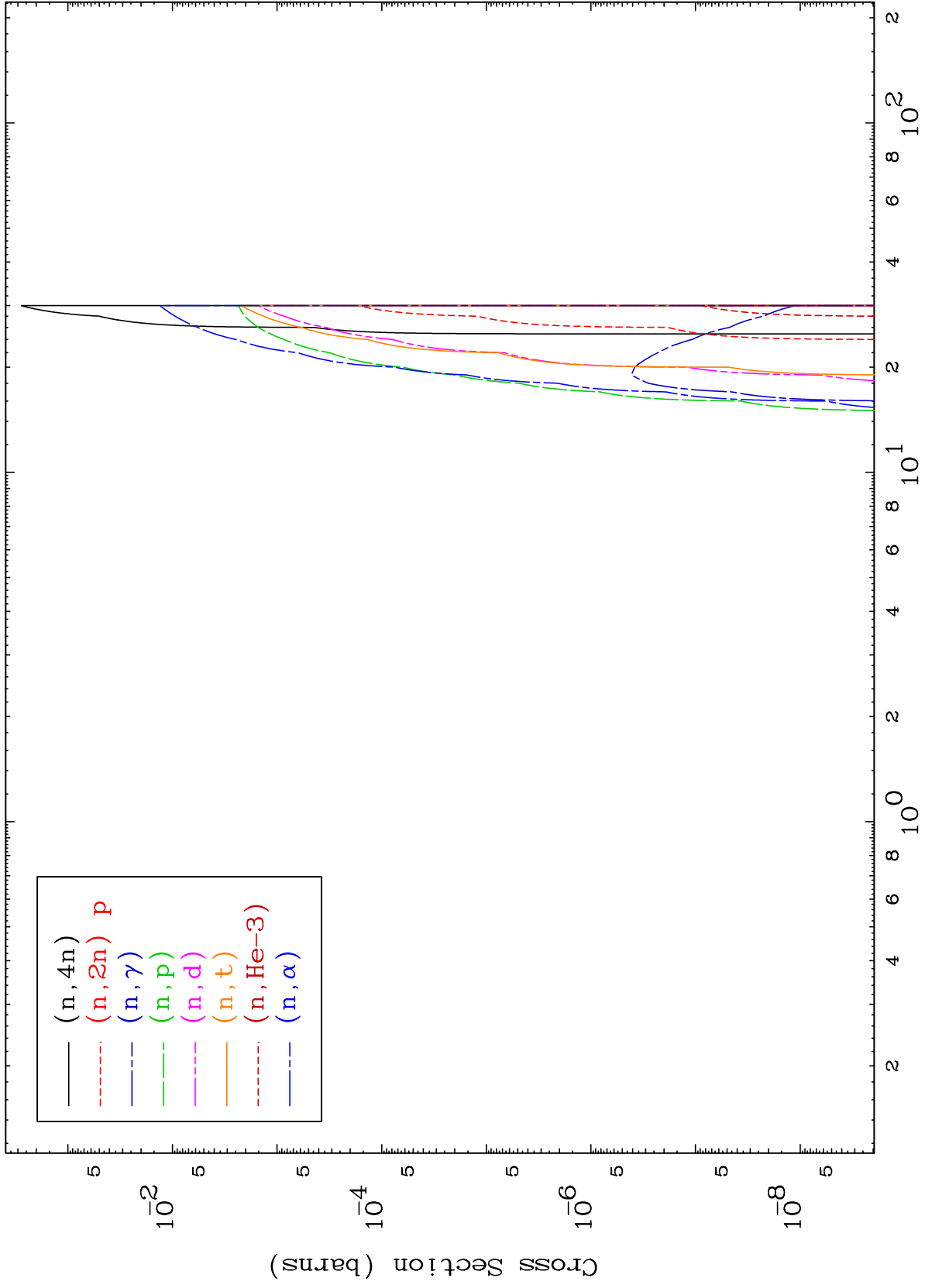
Incident Energy (MeV)

$^{72}\text{Hf}-184\text{m}$

MAT 7256

α Neutron Absorption
0 Kelvin Cross Sections

⁷²Hf-184m



Incident Energy (MeV)

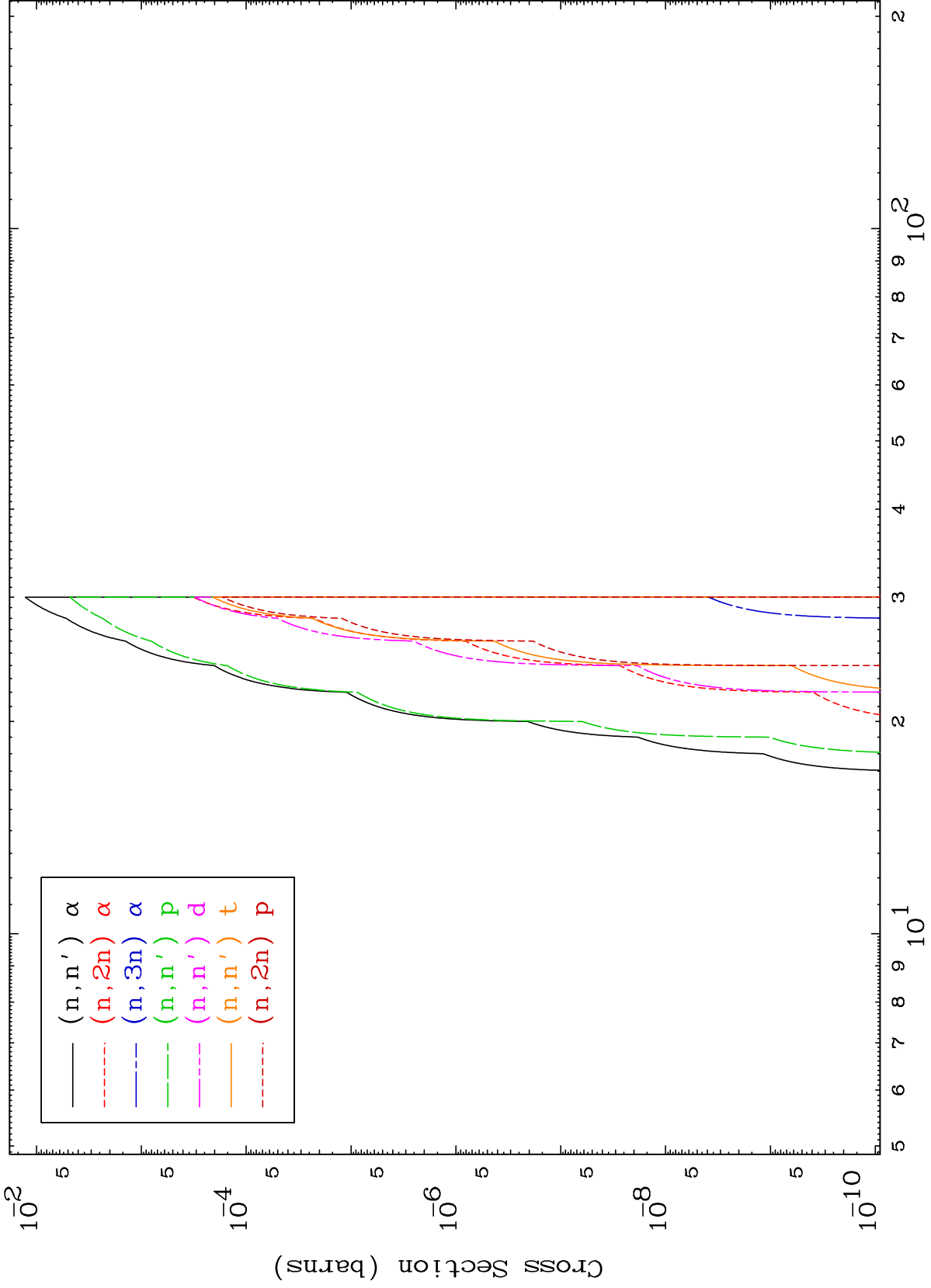
⁷²Hf-184m

MAT 7256

α Charged Particle

72-Hf-184m

0 Kelvin Cross Sections



4

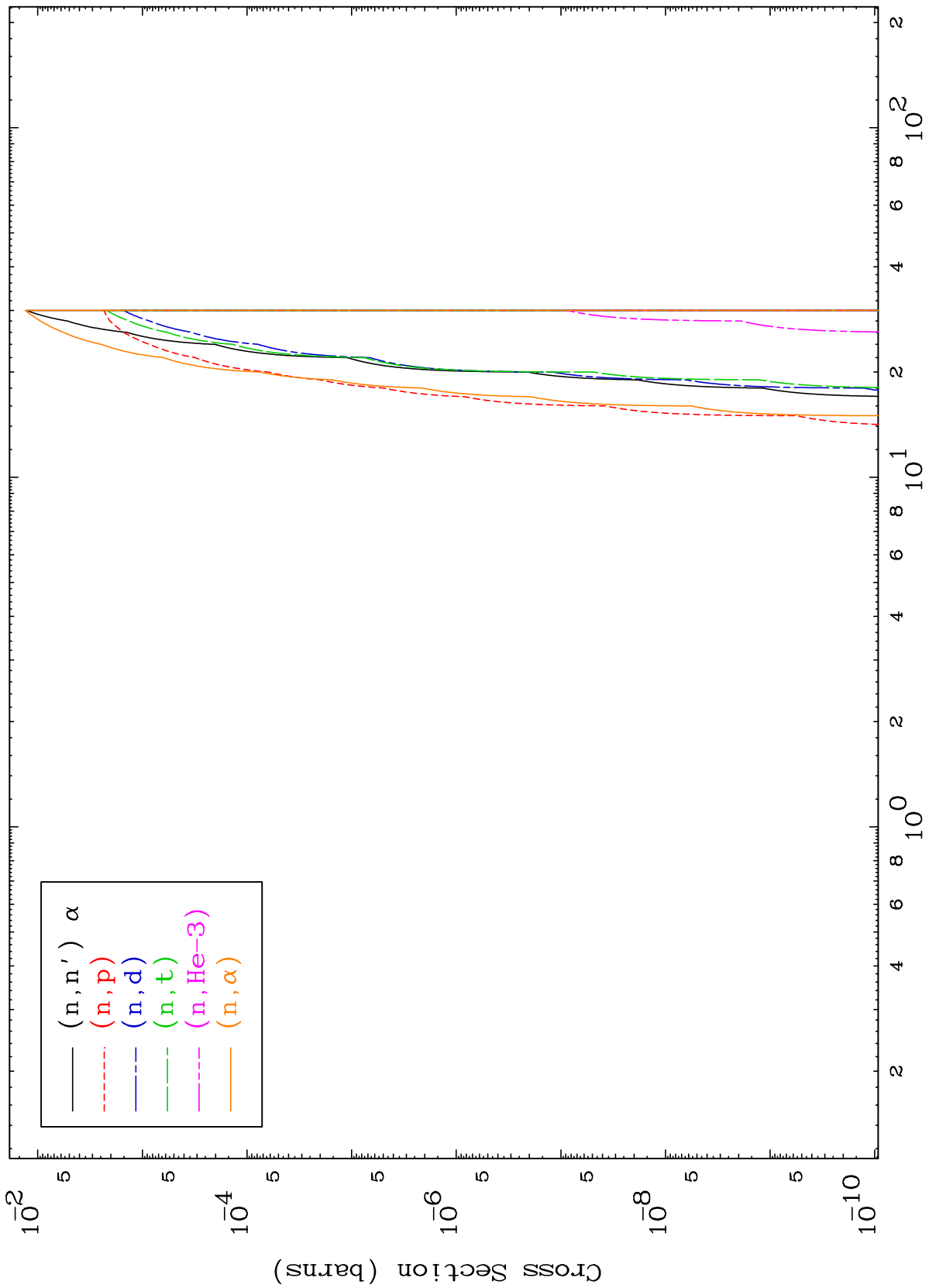
Incident Energy (MeV)

72-Hf-184m

MAT 7256

α Charged Particle
0 Kelvin Cross Sections

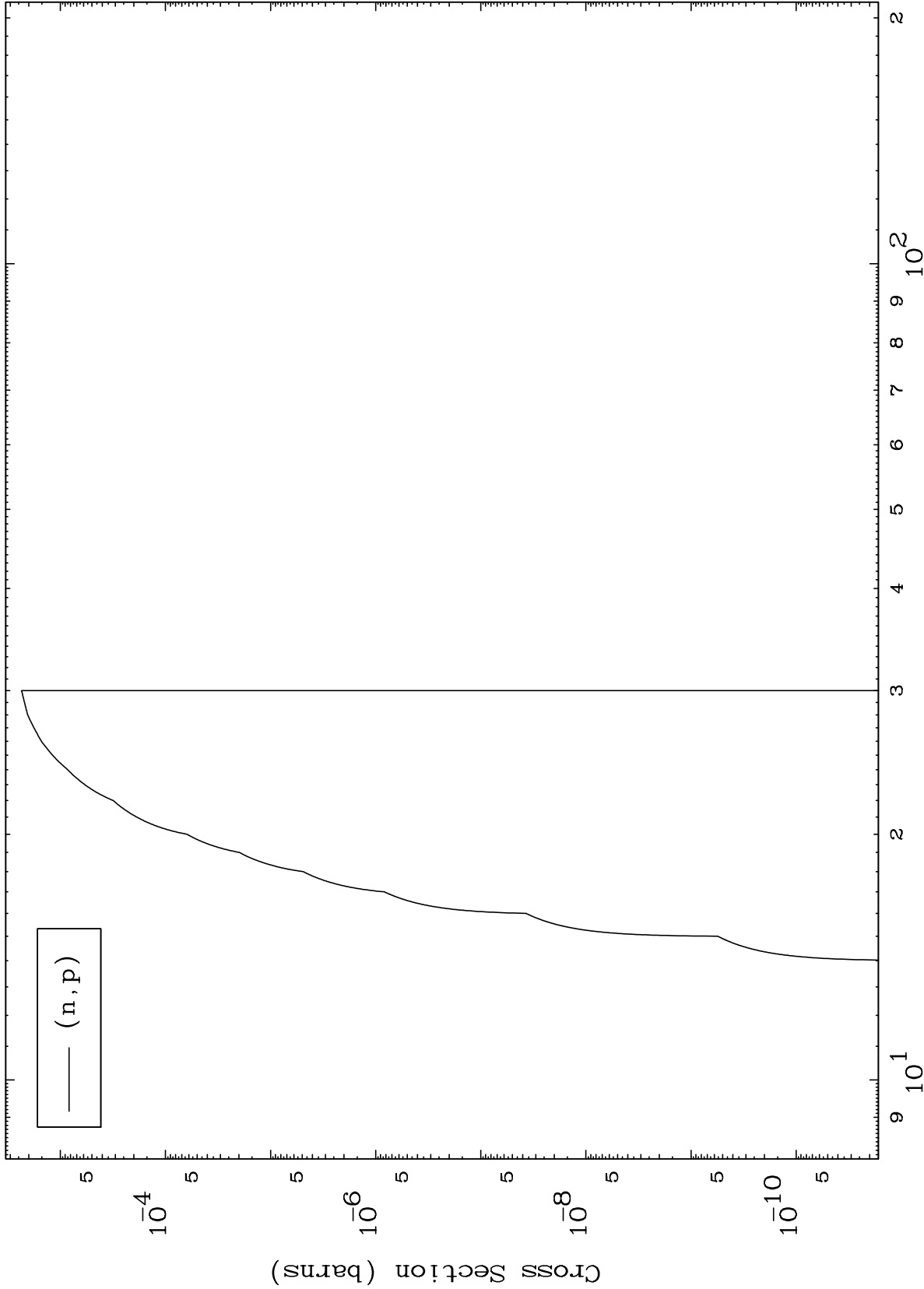
72-Hf-184m



MAT 7256

(α, p) Levels
0 Kelvin Cross Sections

$^{72}\text{Hf}-184\text{m}$



Incident Energy (MeV)

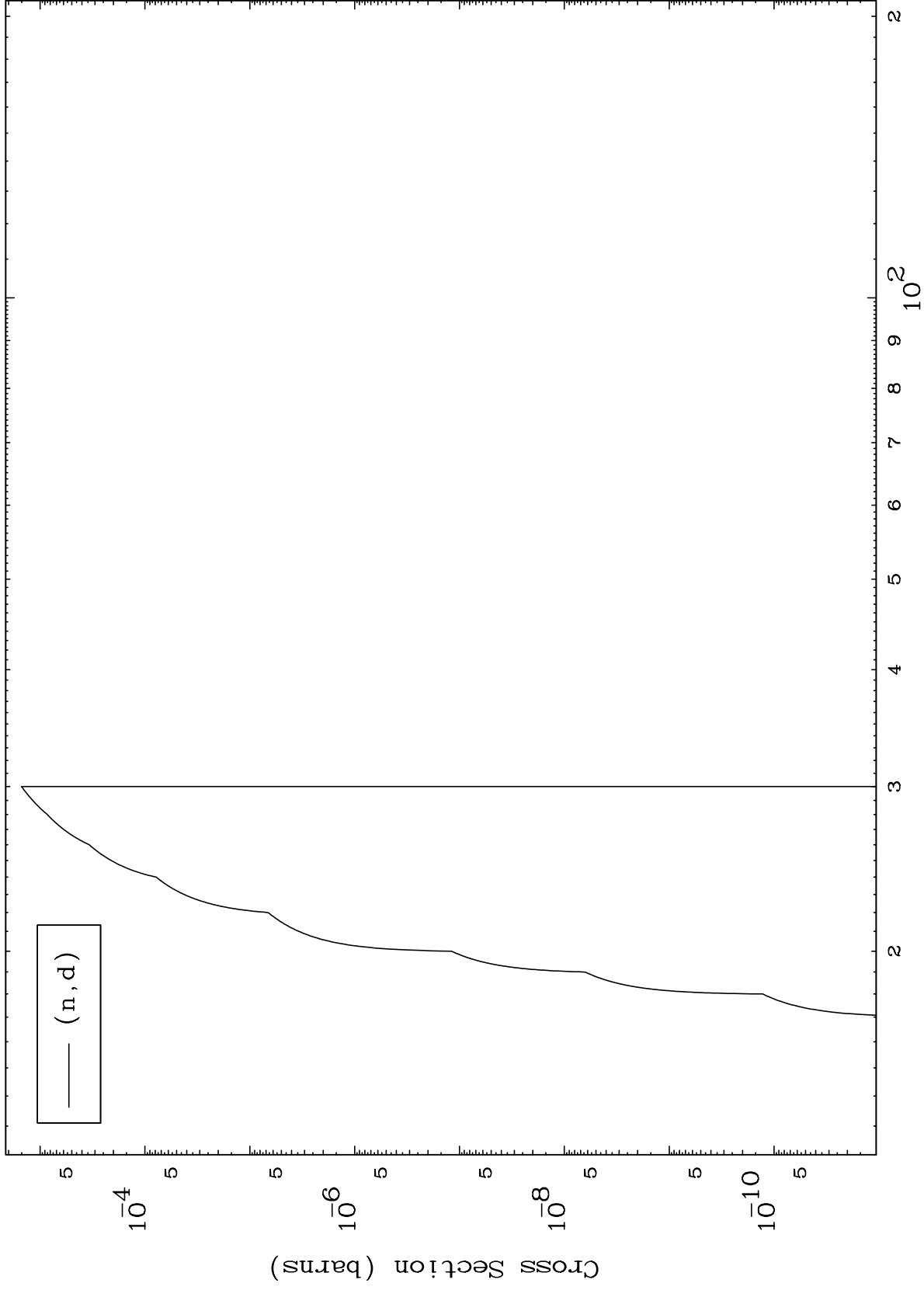
$^{72}\text{Hf}-184\text{m}$

6

MAT 7256

(α, d) Levels
0 Kelvin Cross Sections

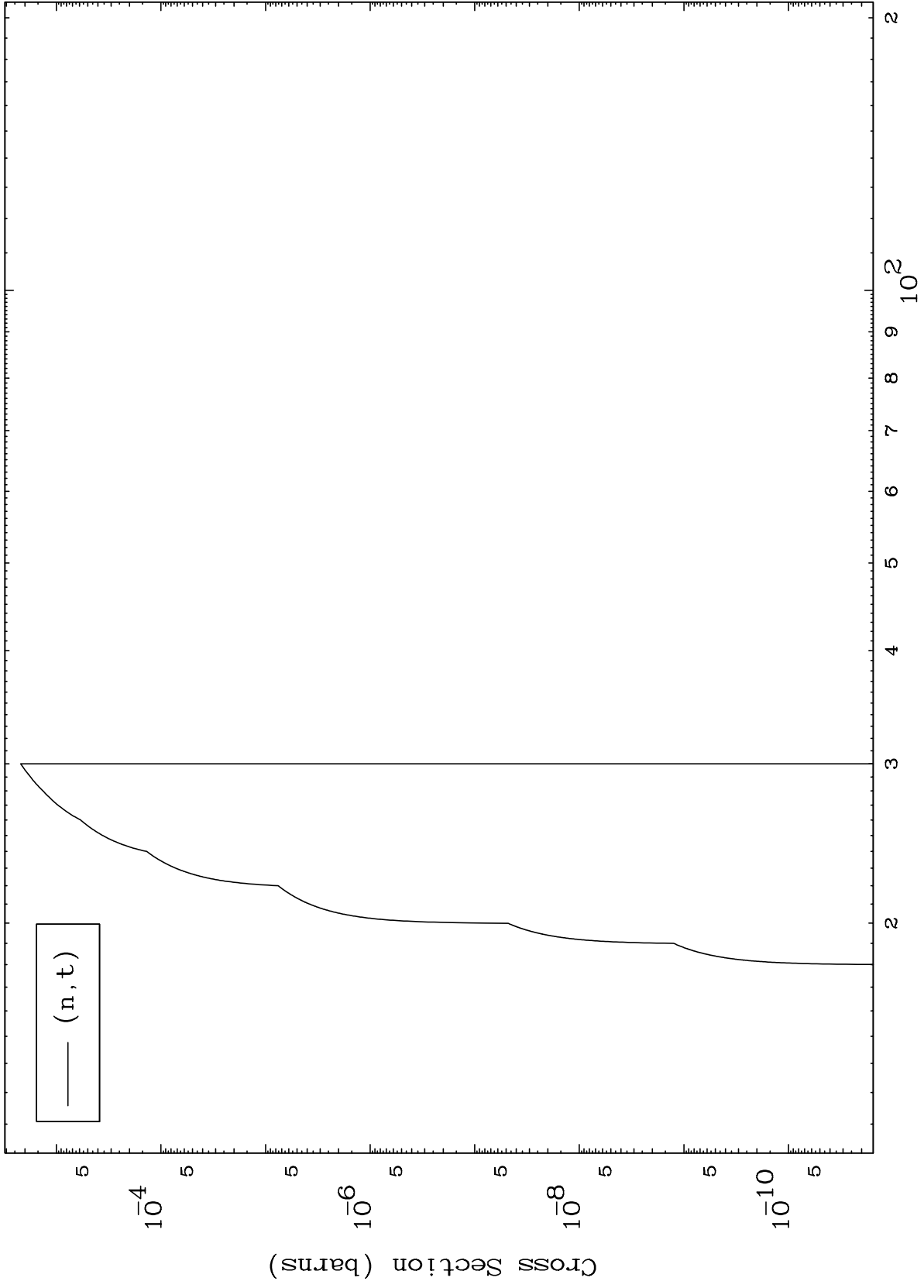
⁷²Hf-184m



7

Incident Energy (MeV)

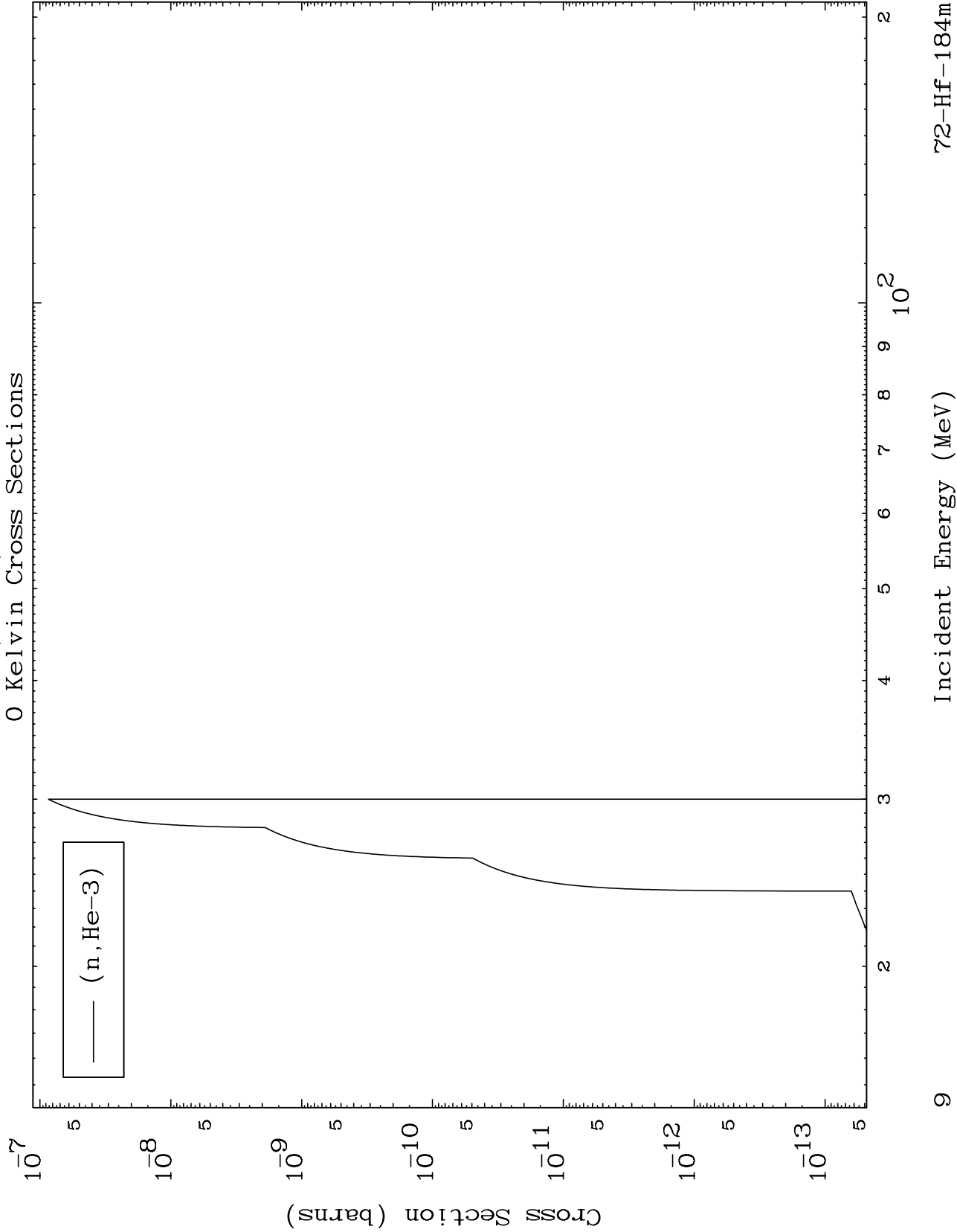
⁷²Hf-184m



MAT 7256

(α , He3) Levels

72-Hf-184m

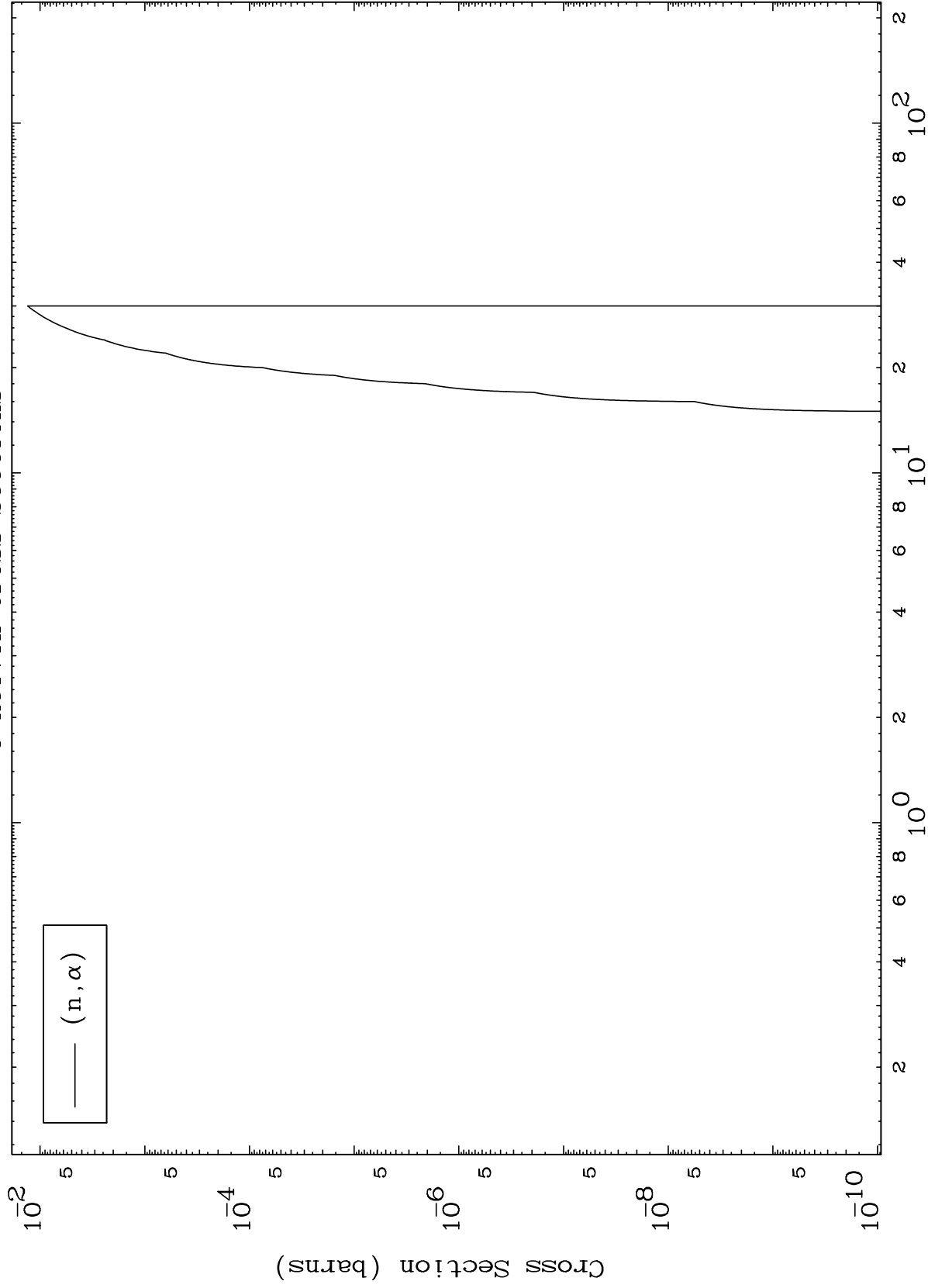


MAT 7256

(α, α) Levels

$^{72}\text{Hf}-184\text{m}$

0 Kelvin Cross Sections



10

Incident Energy (MeV)

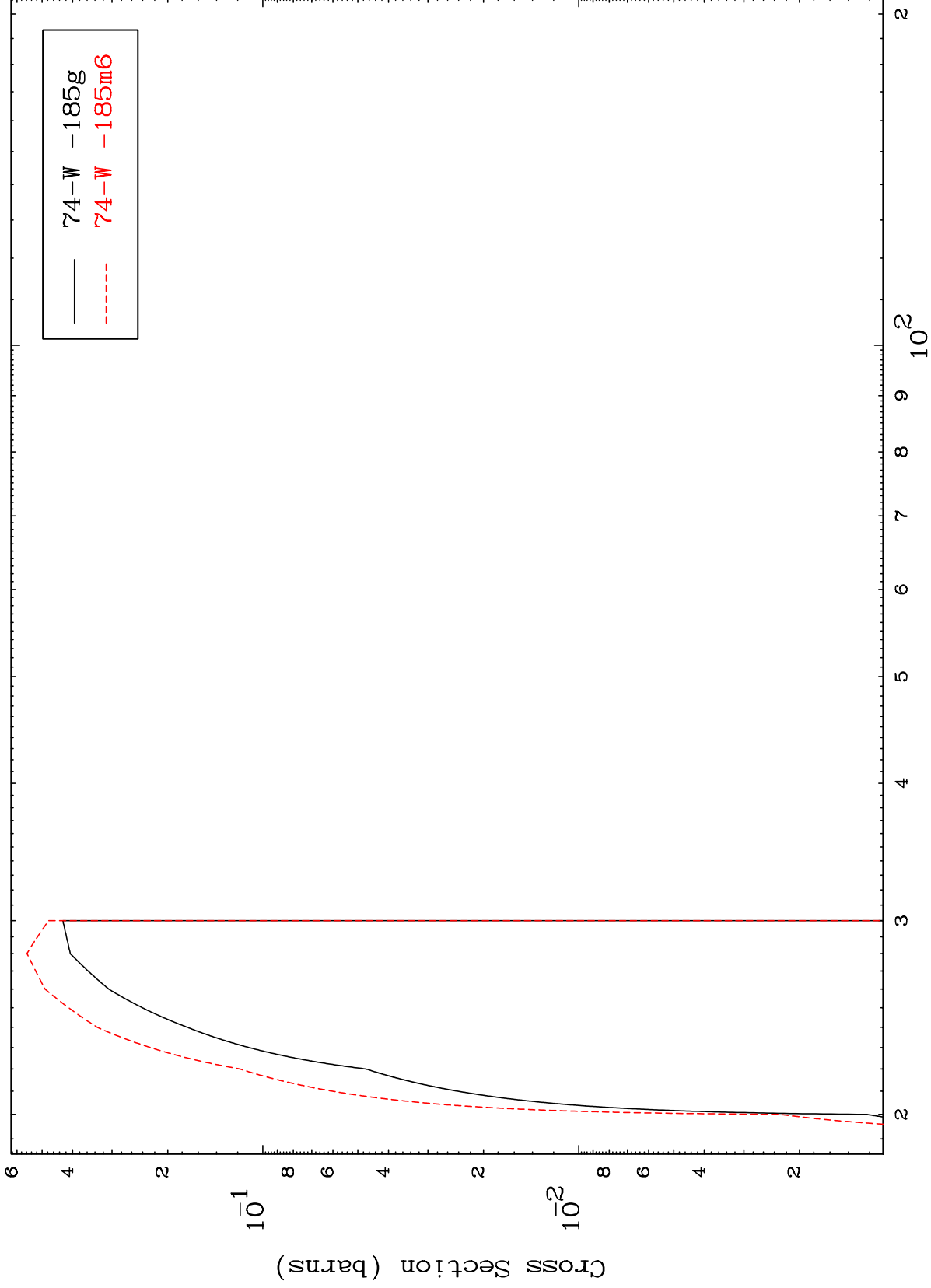
$^{72}\text{Hf}-184\text{m}$

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(n,3n)

72-Hf-184m

Radionuclide Production Cross Section



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Incident Energy (MeV)

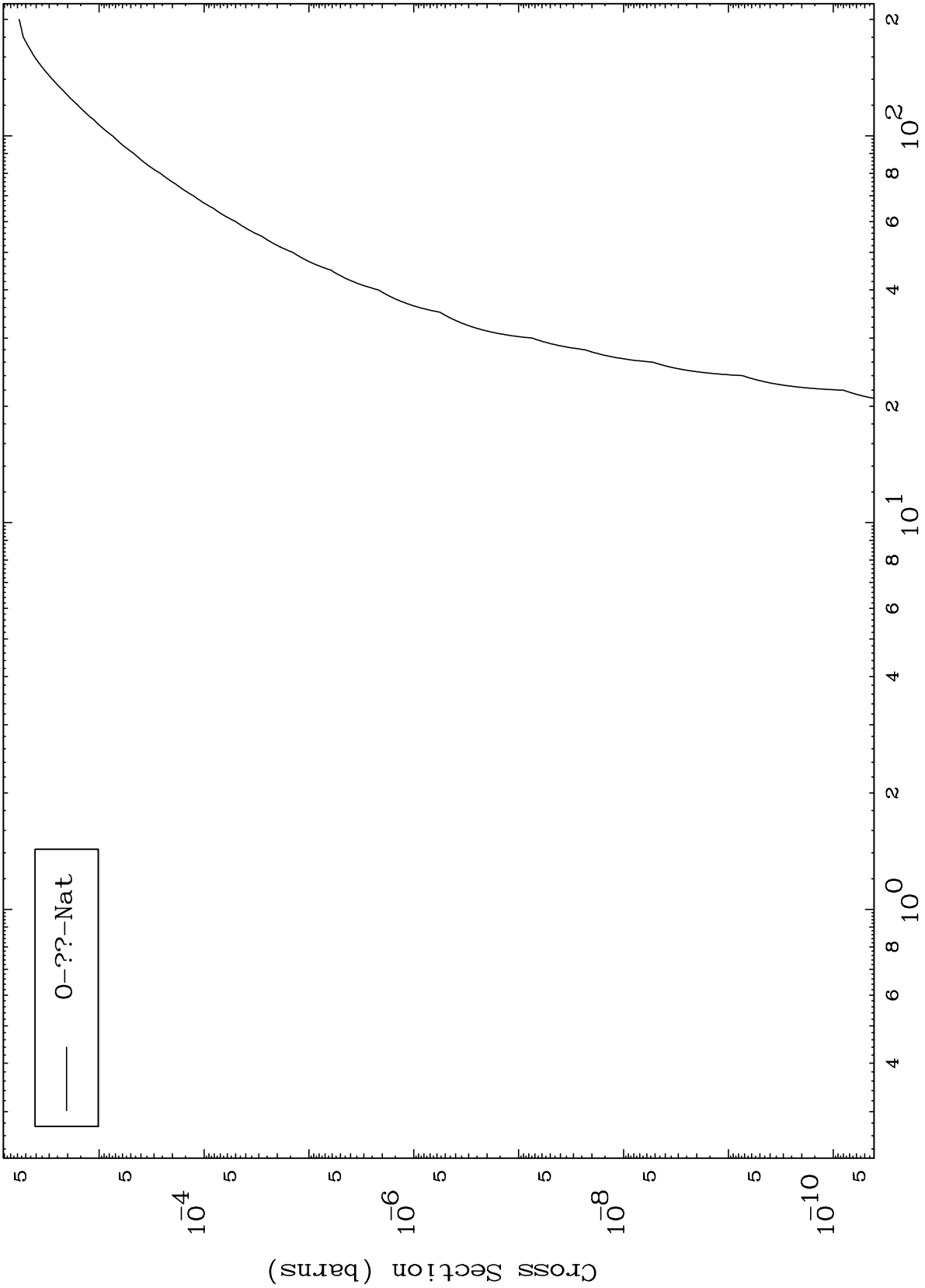
72-Hf-184m

MAT 7256

Fission

⁷²Hf-184m

Radionuclide Production Cross Section

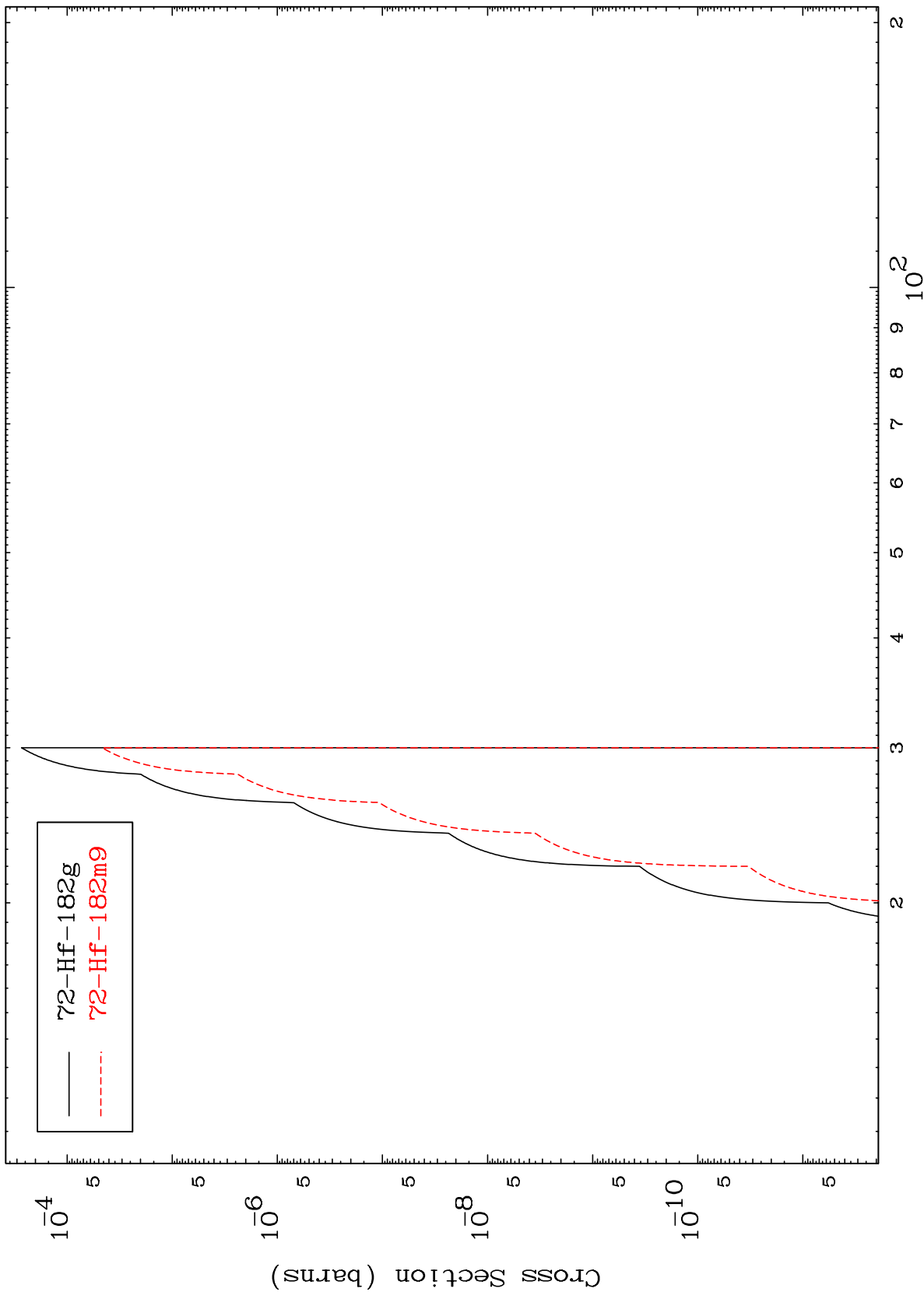


MAT 7256

(n,2n) α

⁷²Hf-184m

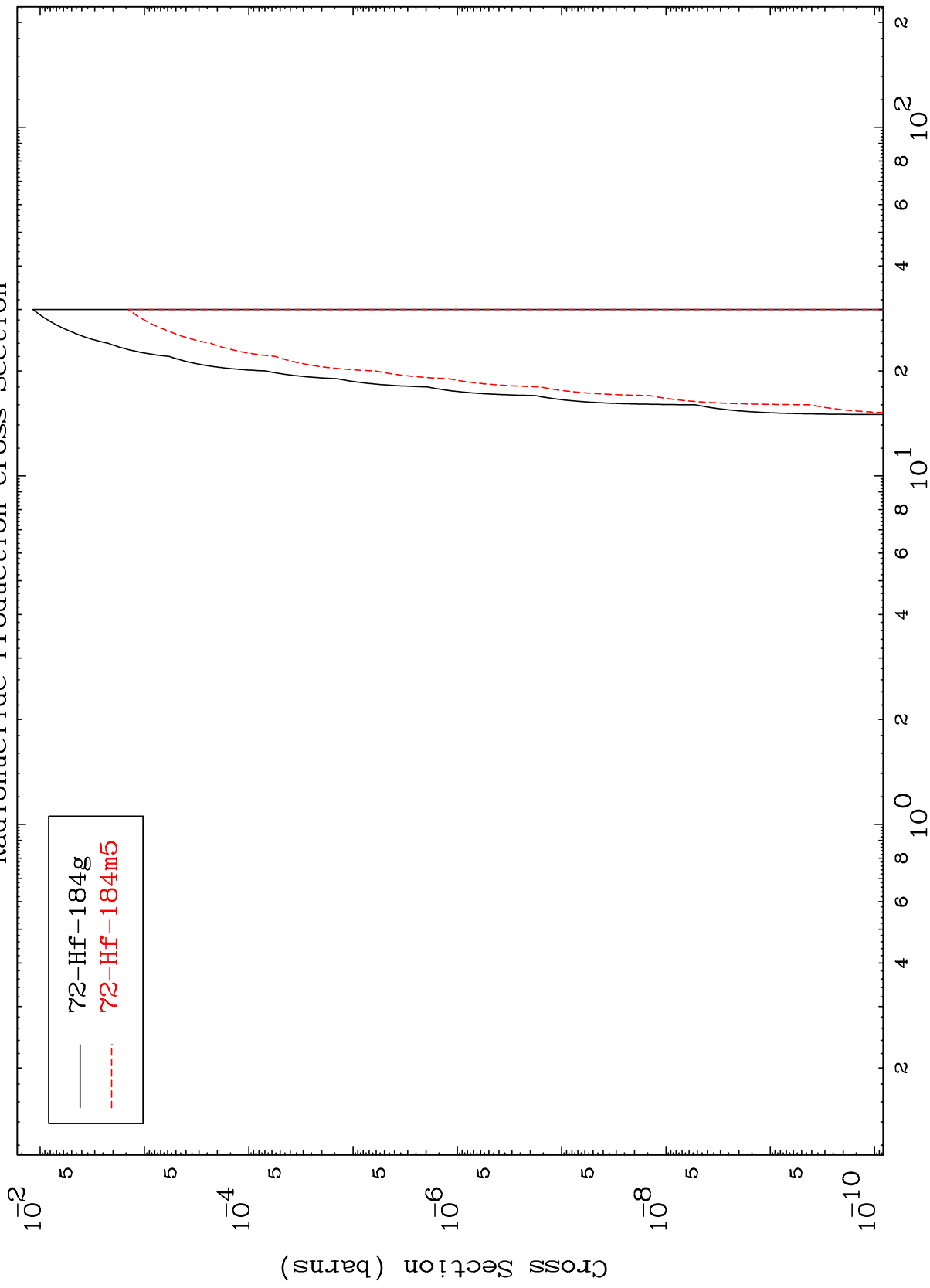
Radionuclide Production Cross Section



MAT 7256

$^{72}\text{Hf}-184\text{m}$

Radionuclide Production Cross Section



$^{72}\text{Hf}-184\text{m}$

Incident Energy (MeV)

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