

Program EVALPLOT  
(Version 2021-1)

by

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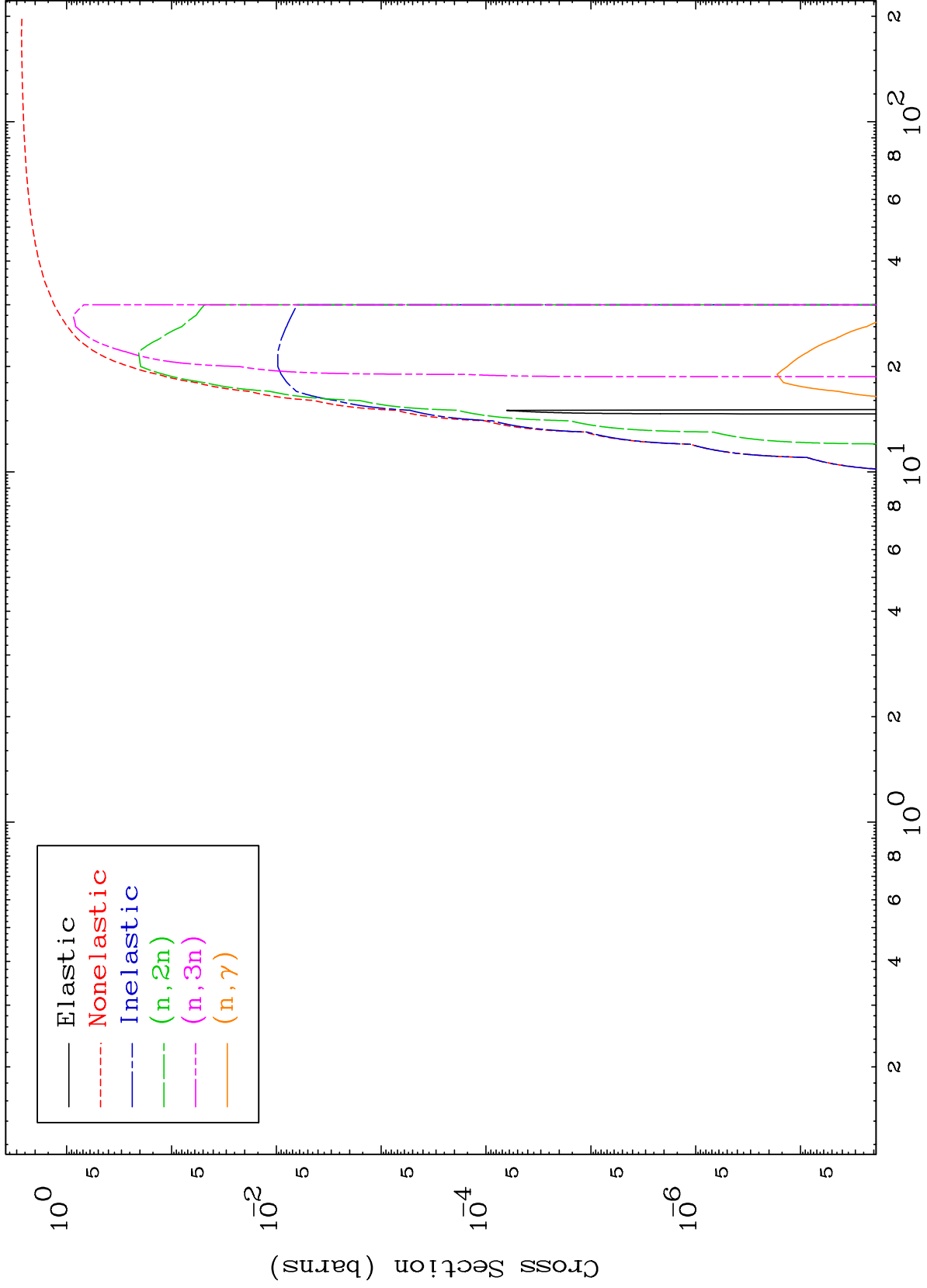
Press Mouse Button to Start

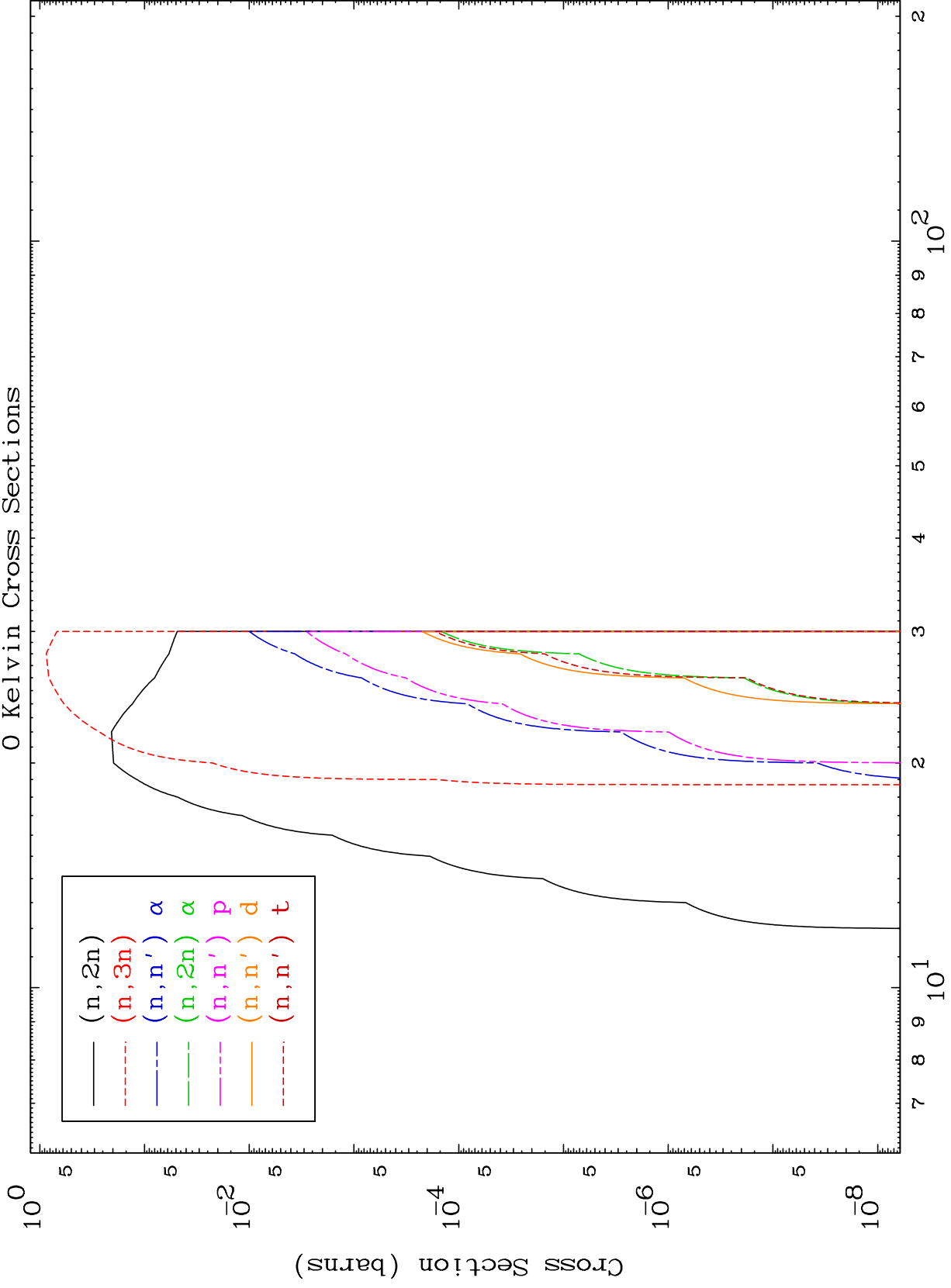
MAT 7261

$\alpha$  Major

72-Hf-186

0 Kelvin Cross Sections

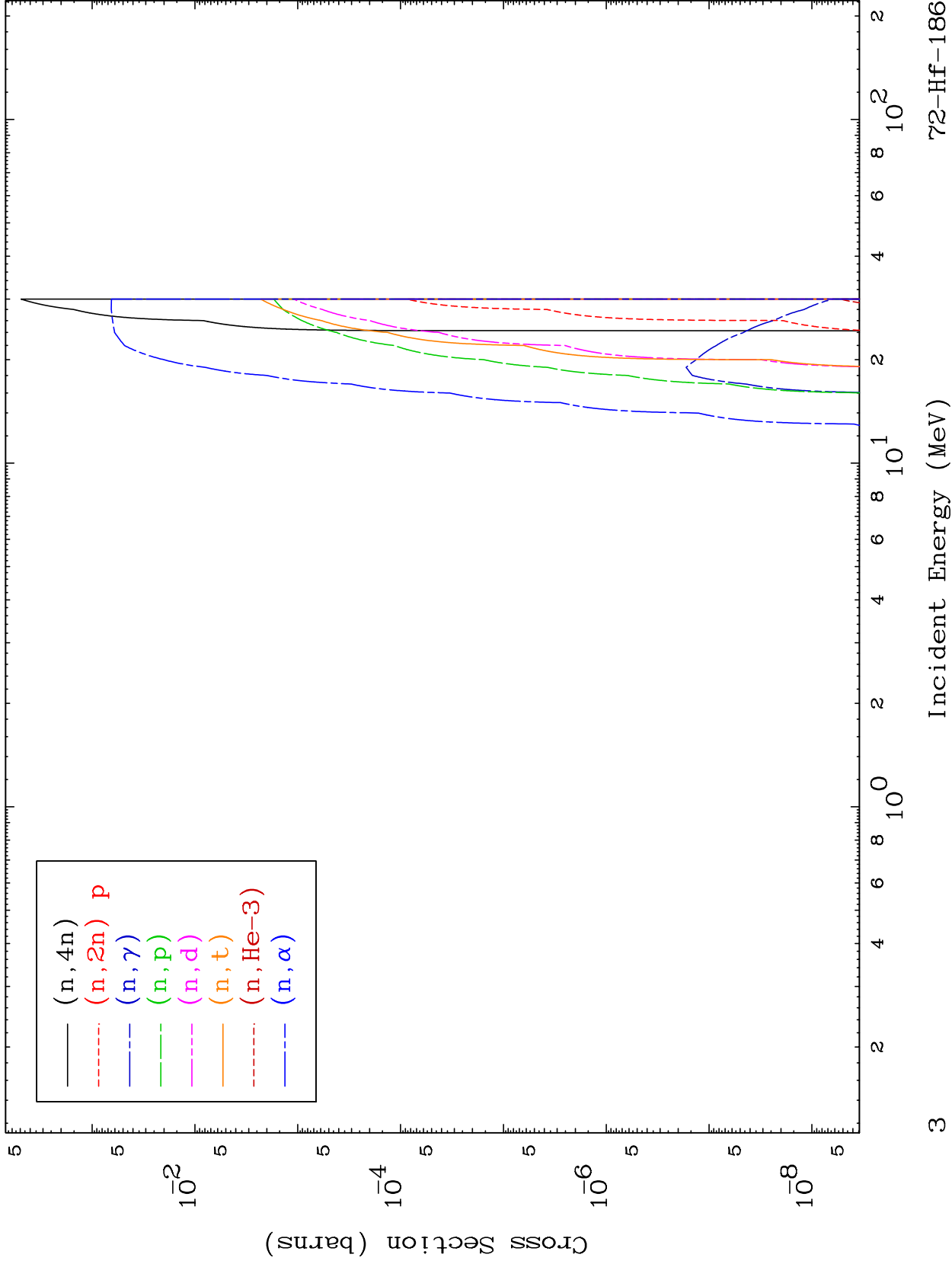


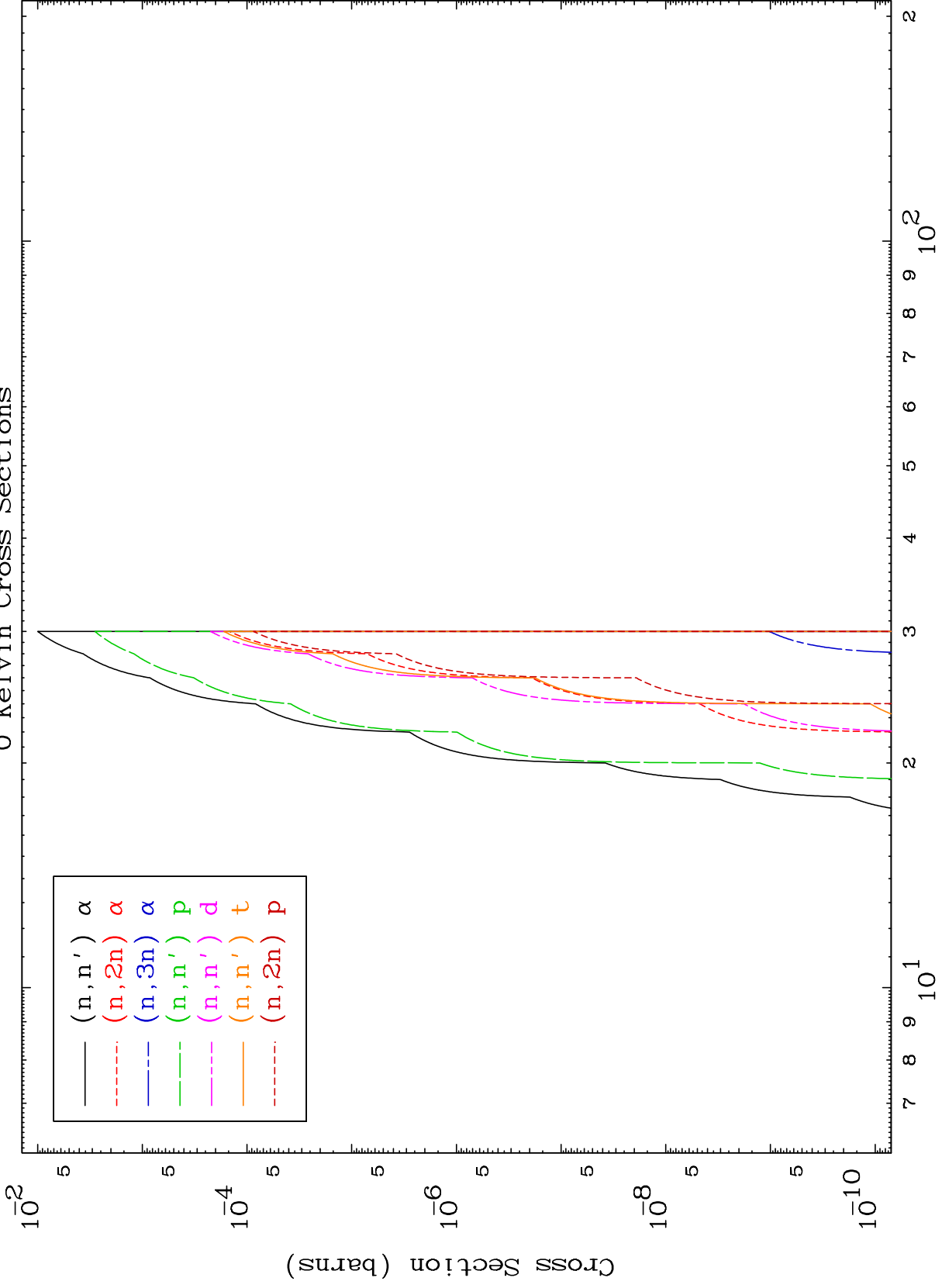


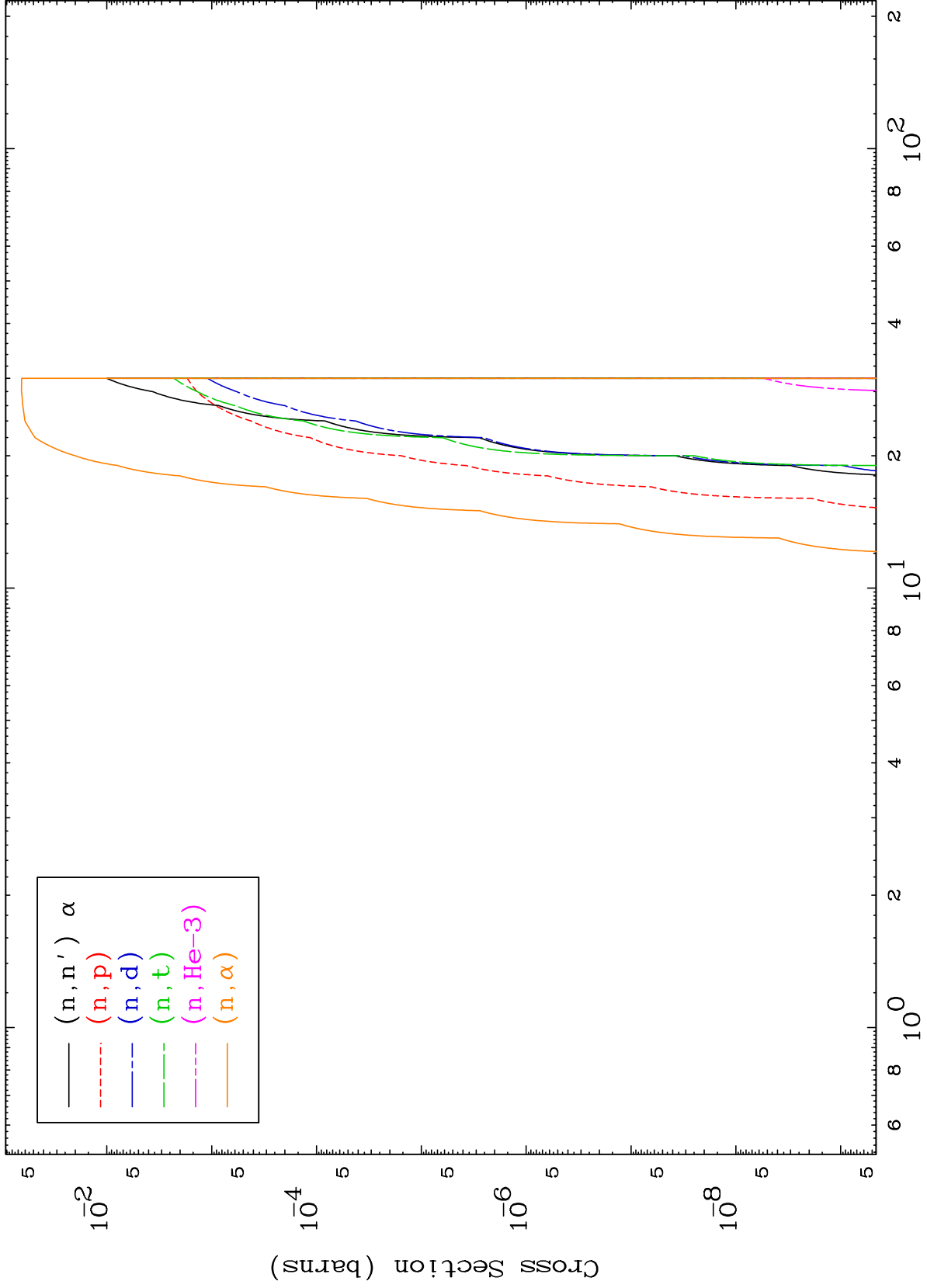
MAT 7261

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

72-Hf-186





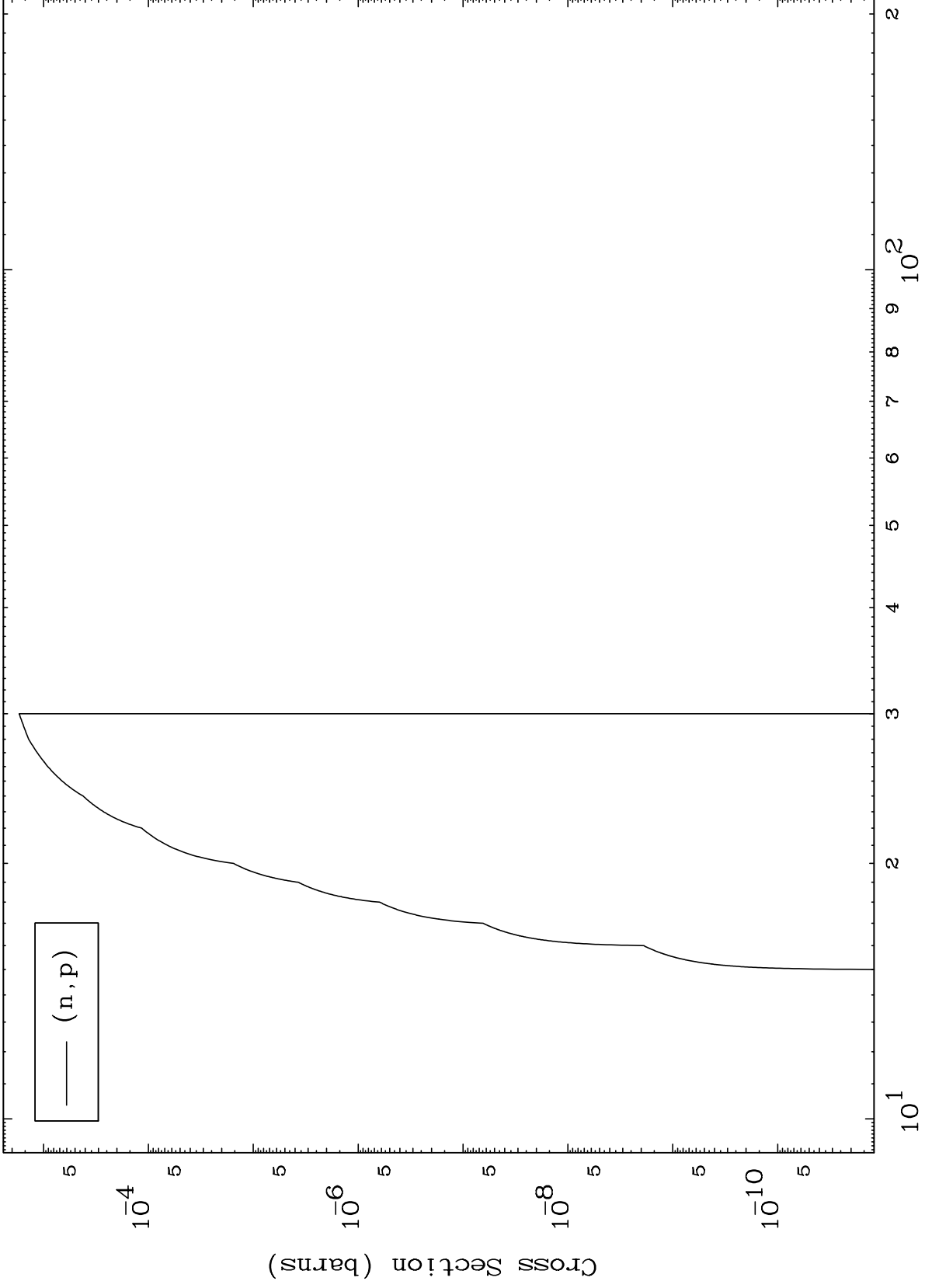


MAT 7261

( $\alpha, p$ ) Levels

72-Hf-186

0 Kelvin Cross Sections



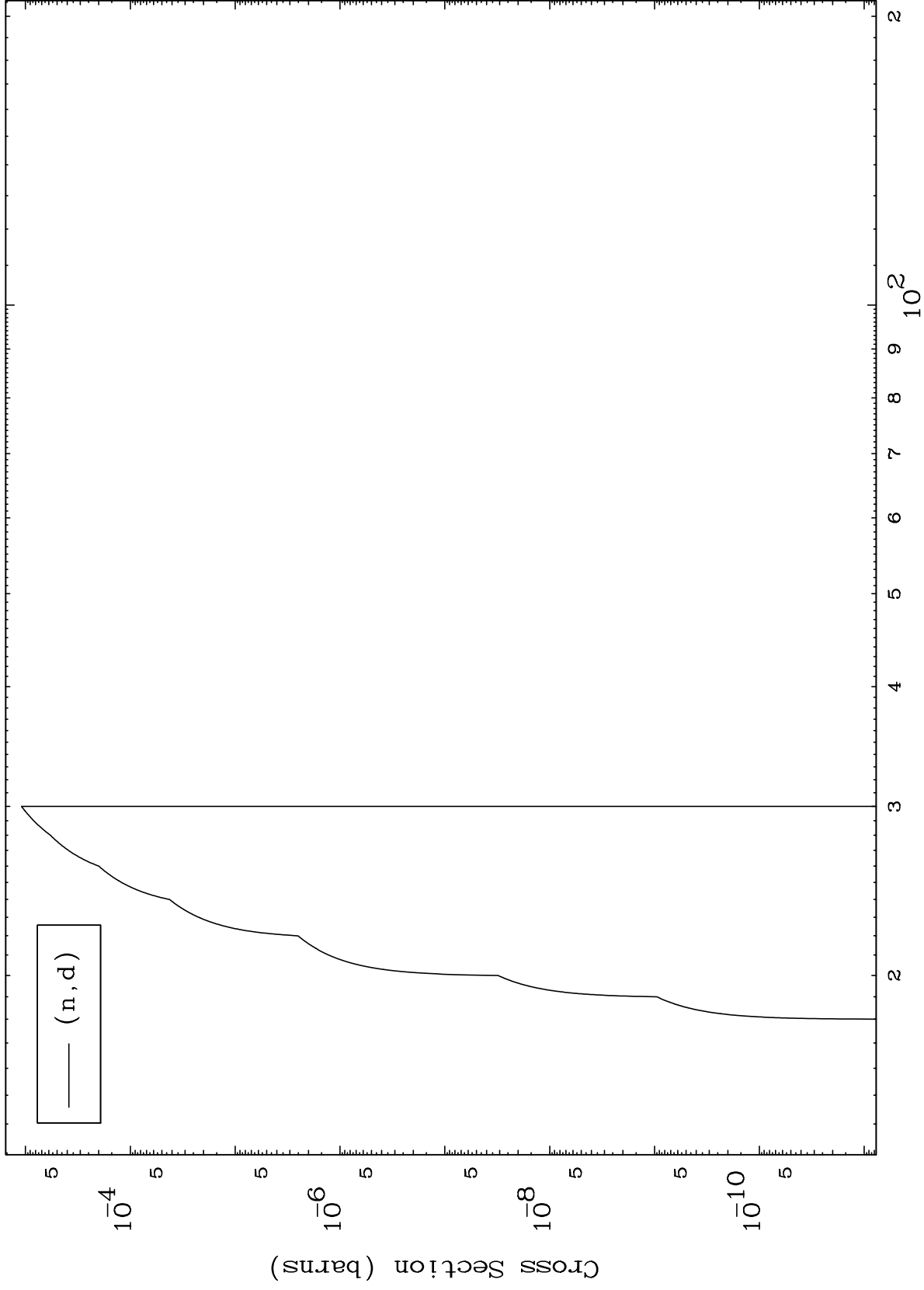
Incident Energy (MeV)

72-Hf-186

MAT 7261

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

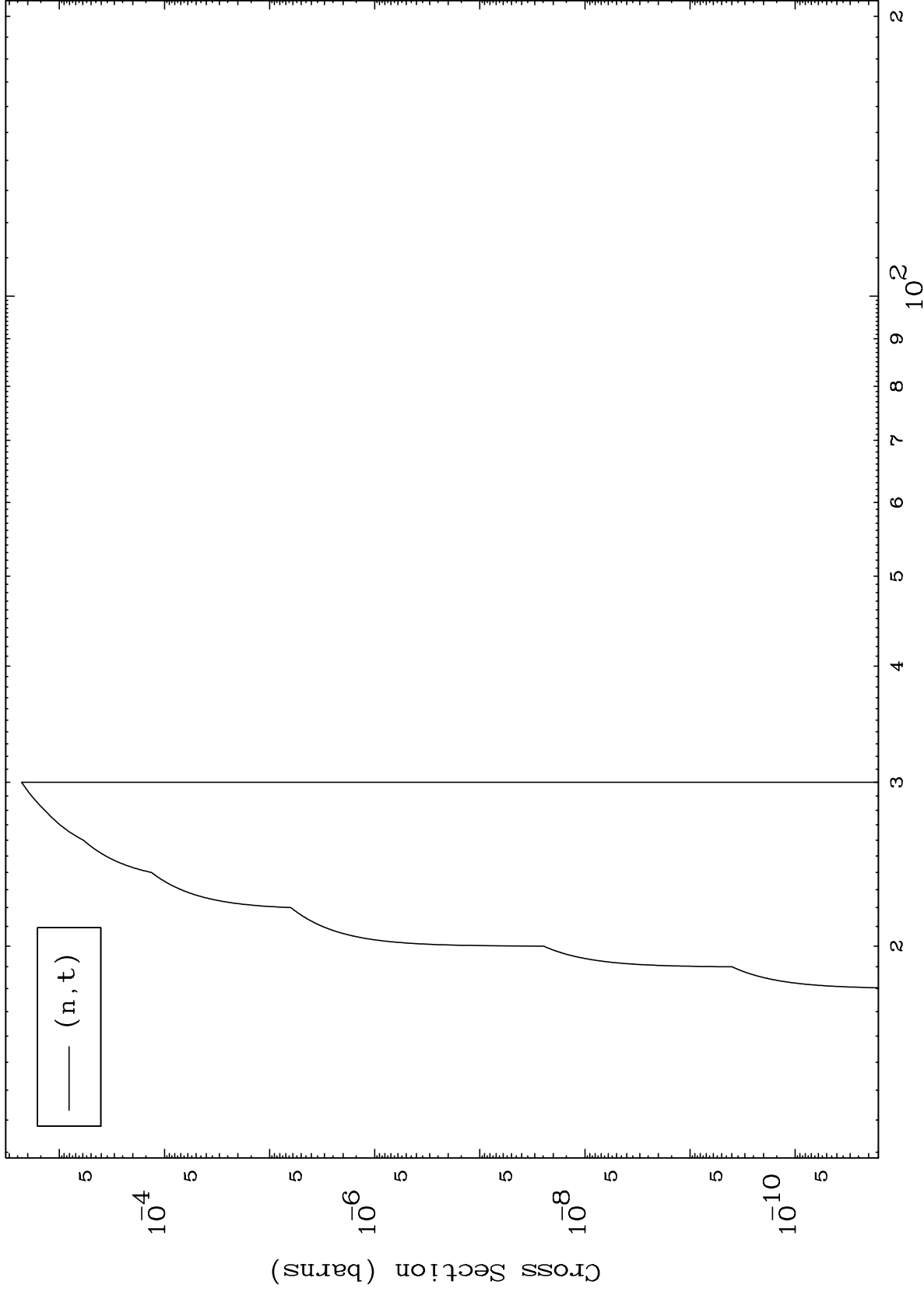
72-Hf-186



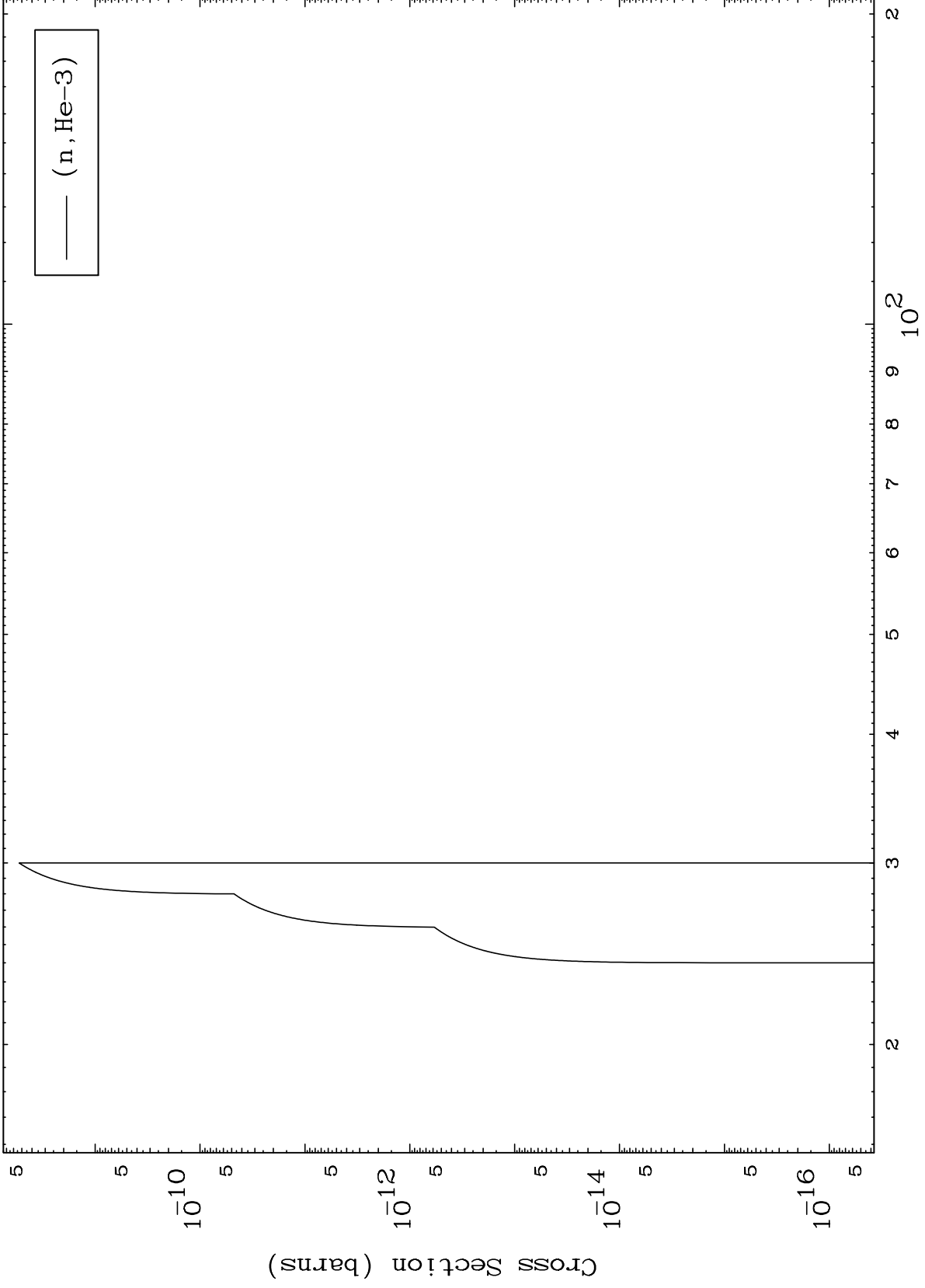
7

Incident Energy (MeV)

72-Hf-186



( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

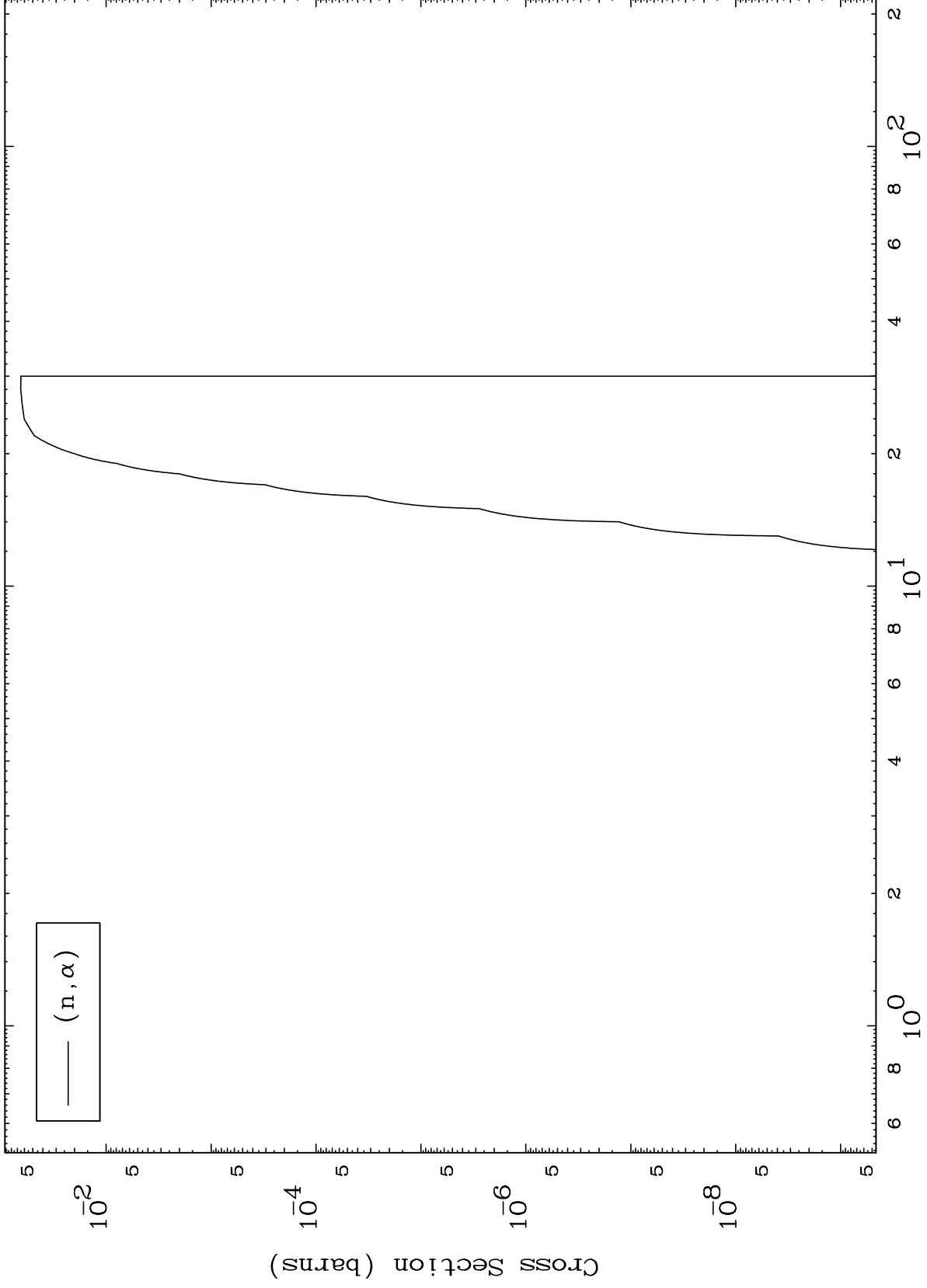


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( $\alpha, \alpha$ ) Levels

72-Hf-186

0 Kelvin Cross Sections



10

Incident Energy (MeV)

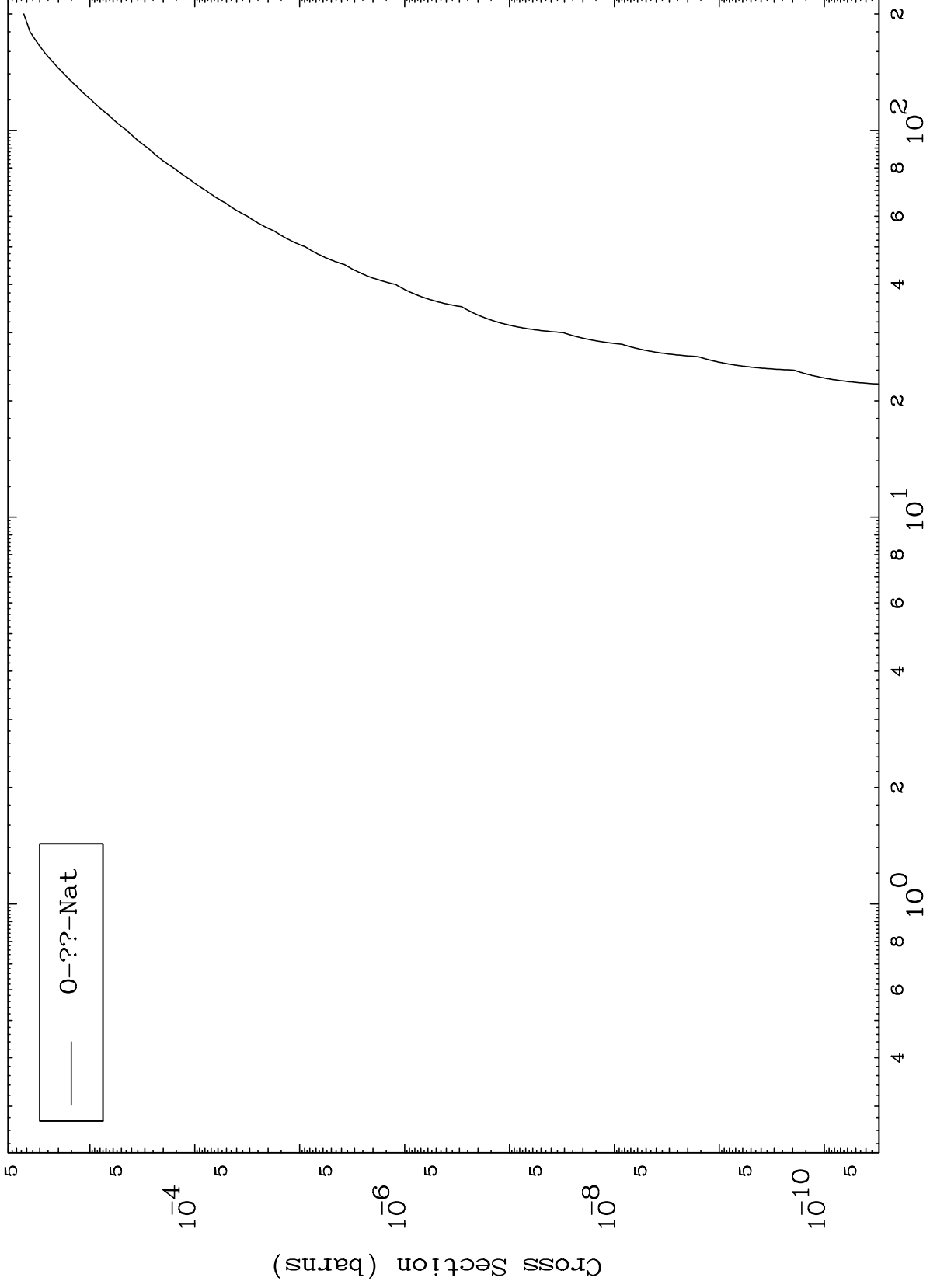
72-Hf-186

MAT 7261

Fission

72-Hf-186

Radionuclide Production Cross Section

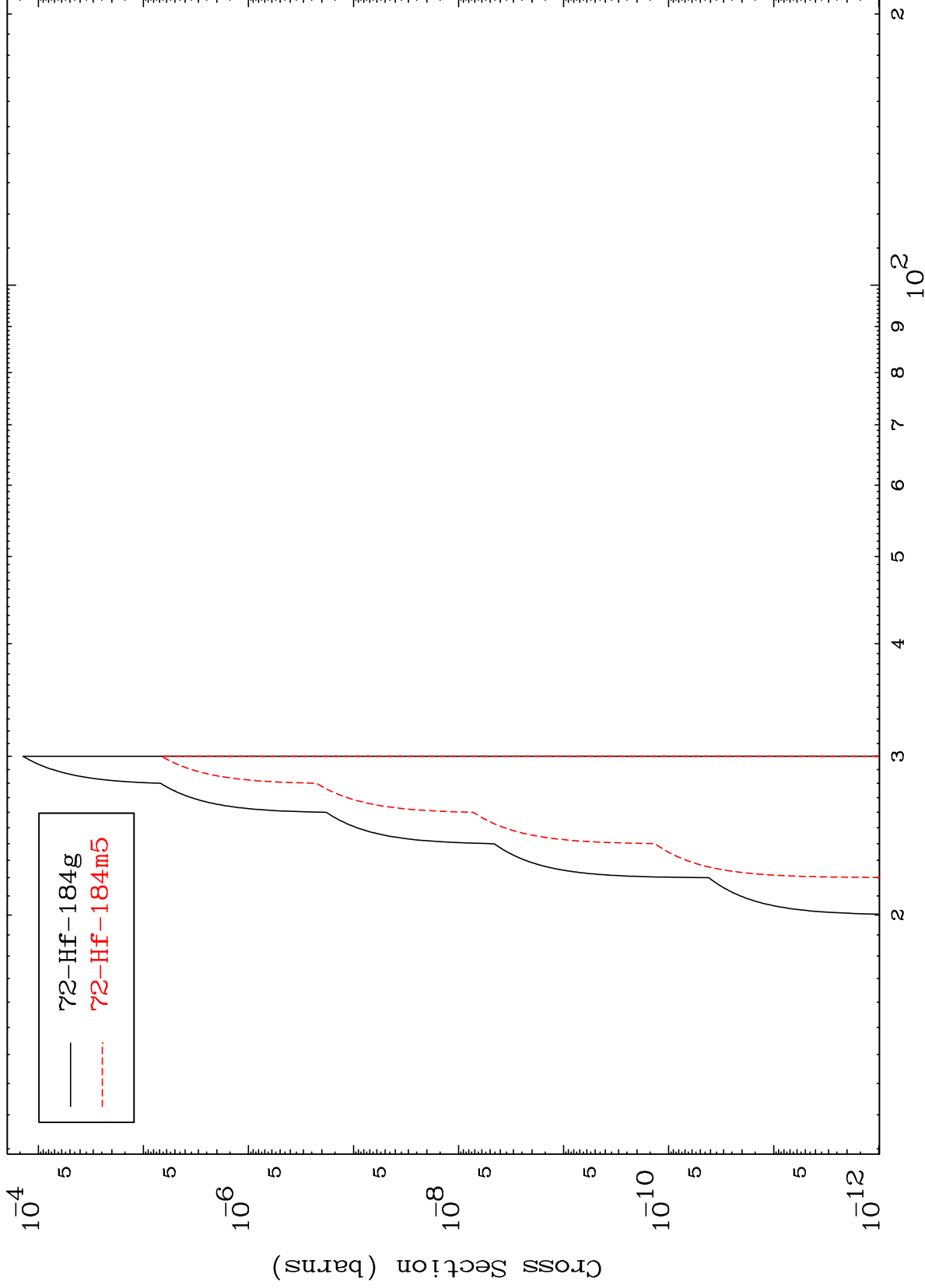


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(n,2n)  $\alpha$

<sup>72</sup>Hf-186

Radionuclide Production Cross Section

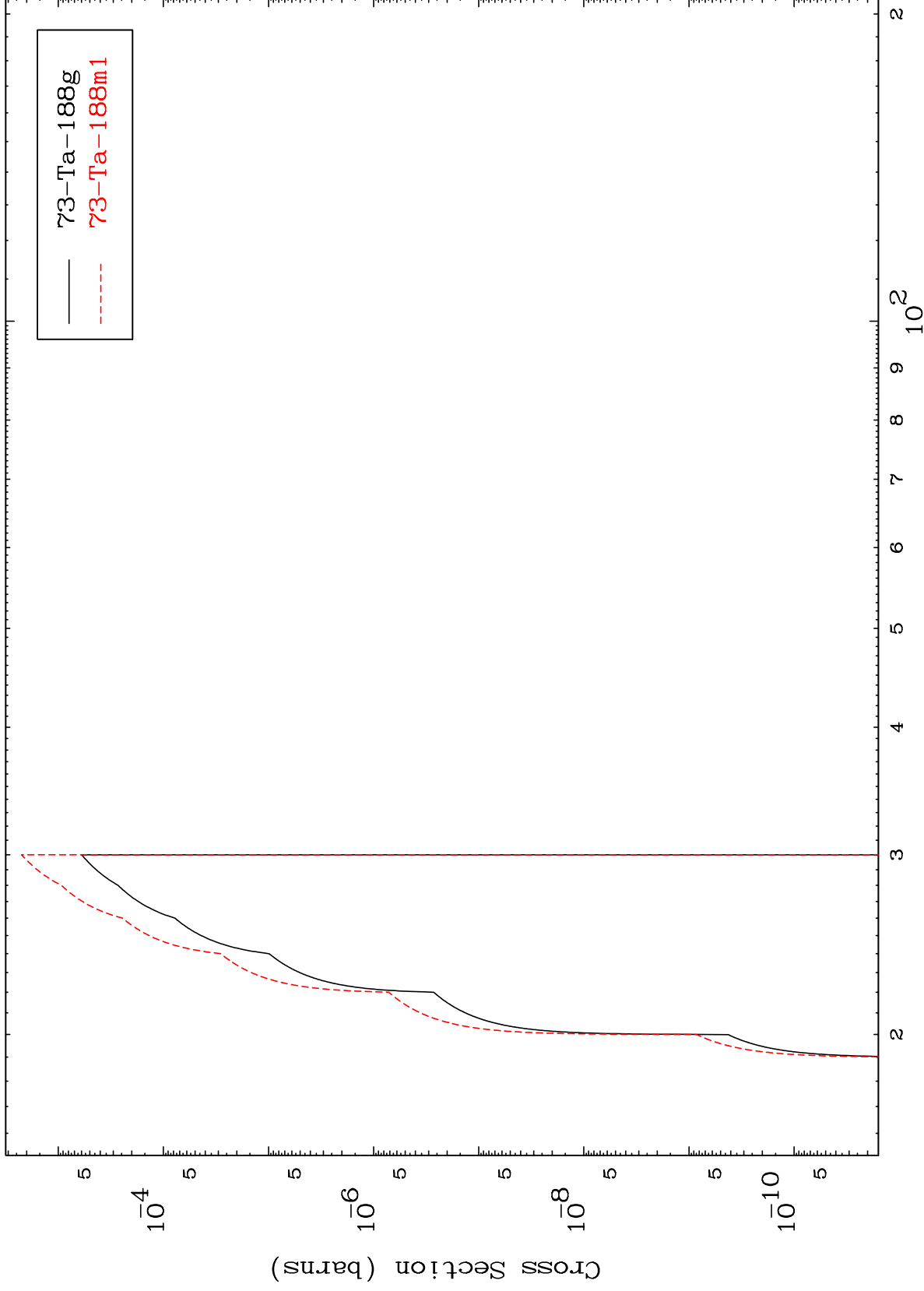


12

Incident Energy (MeV)

<sup>72</sup>Hf-186

$(n, n')$  p  
Radionuclide Production Cross Section



(n,d)  
Radionuclide Production Cross Section

