

Program EVALPLOT  
(Version 2021-1)

by

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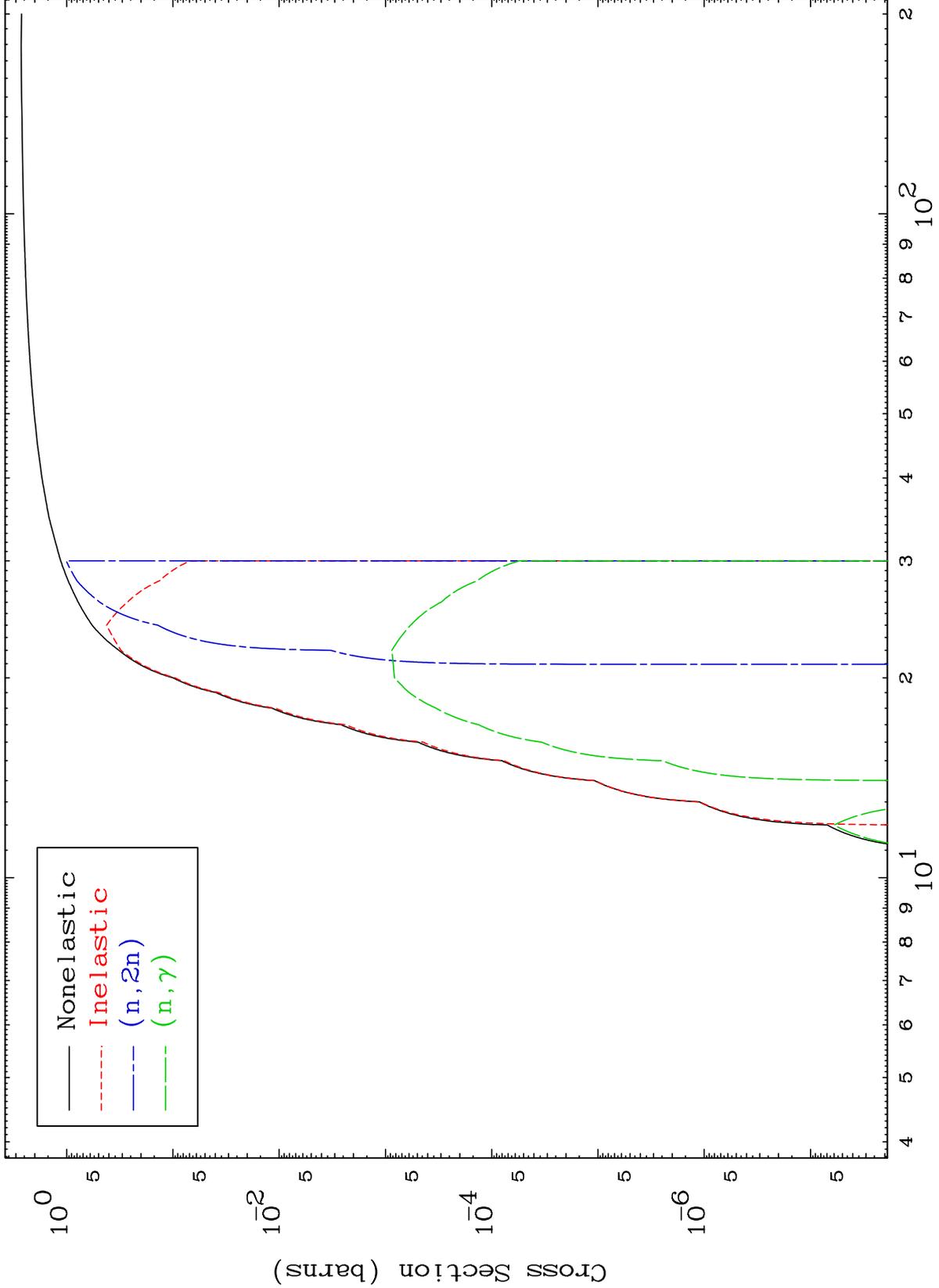
Press Mouse Button to Start

MAT 7710

0 Kelvin

$\alpha$  Major Cross Sections

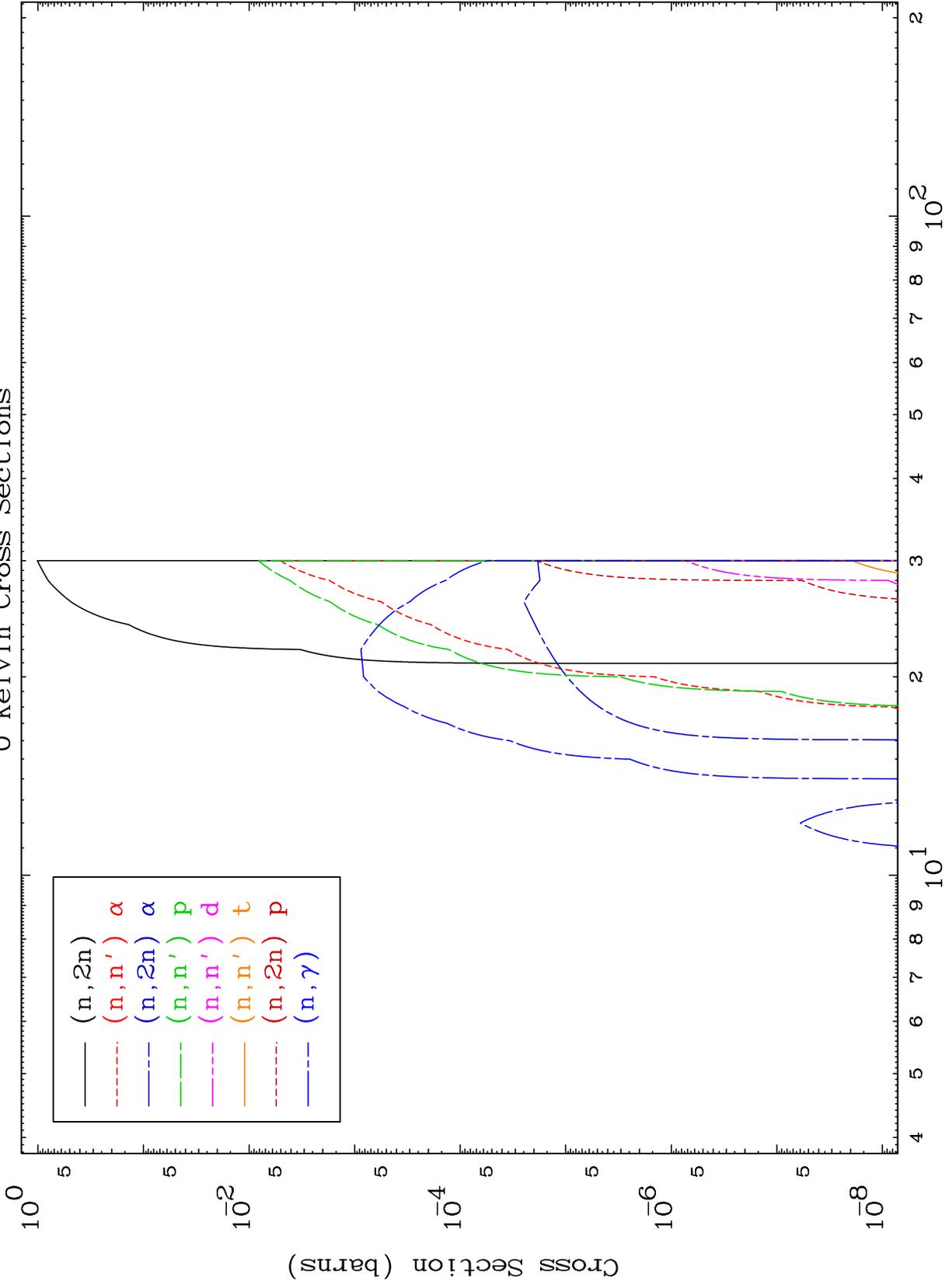
77-Ir-186



1

Incident Energy (MeV)

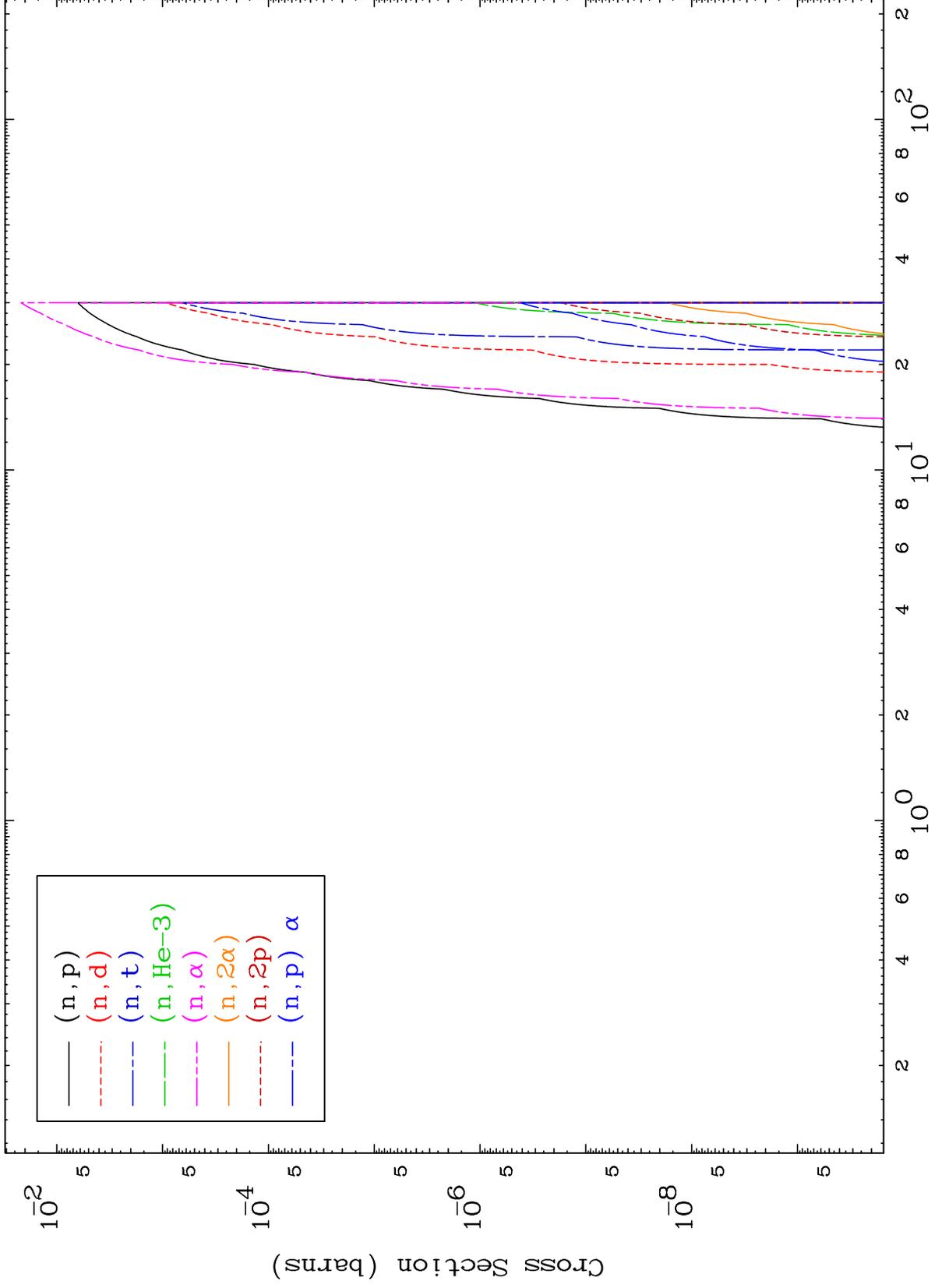
77-Ir-186



MAT 7710

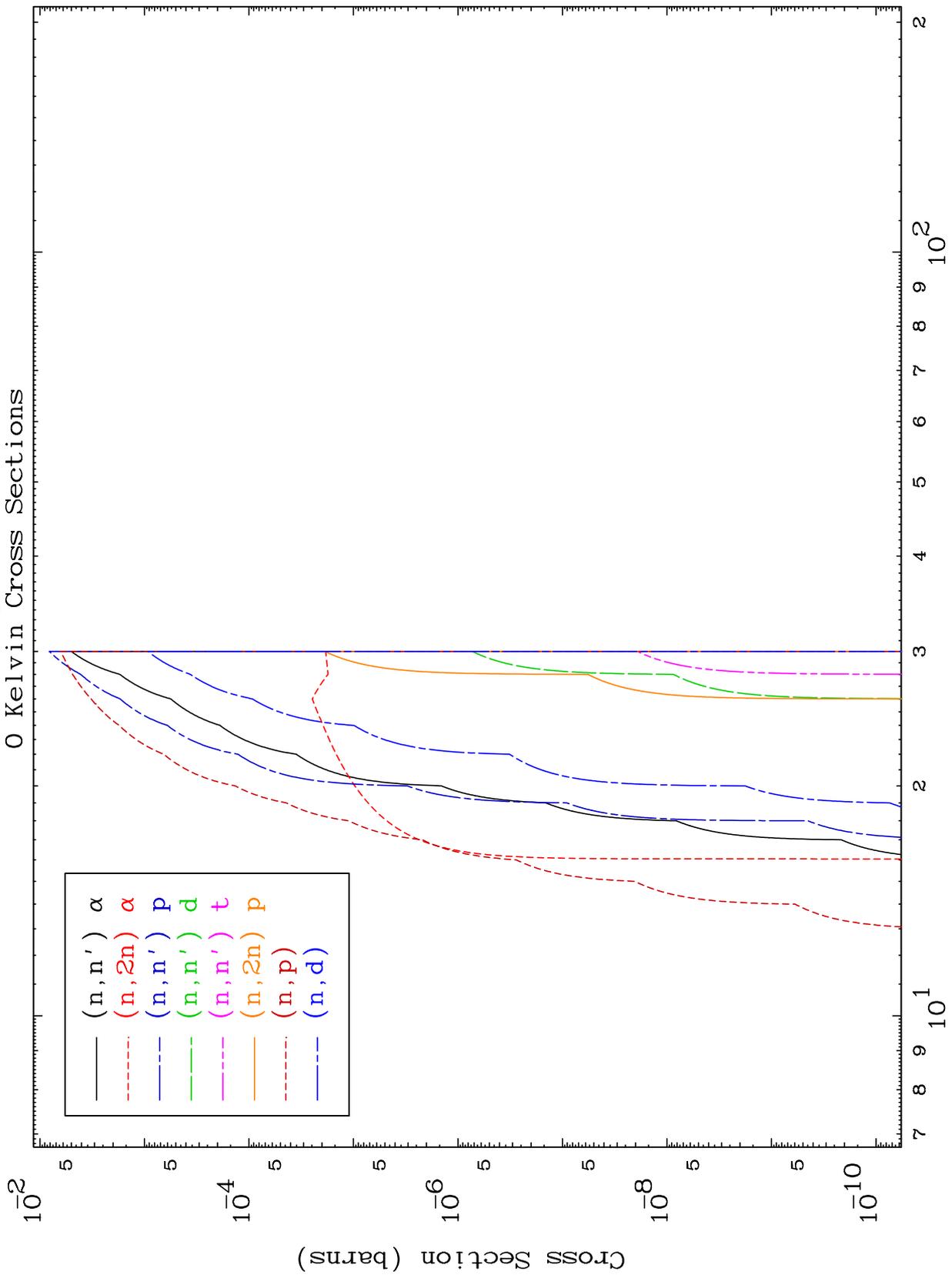
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

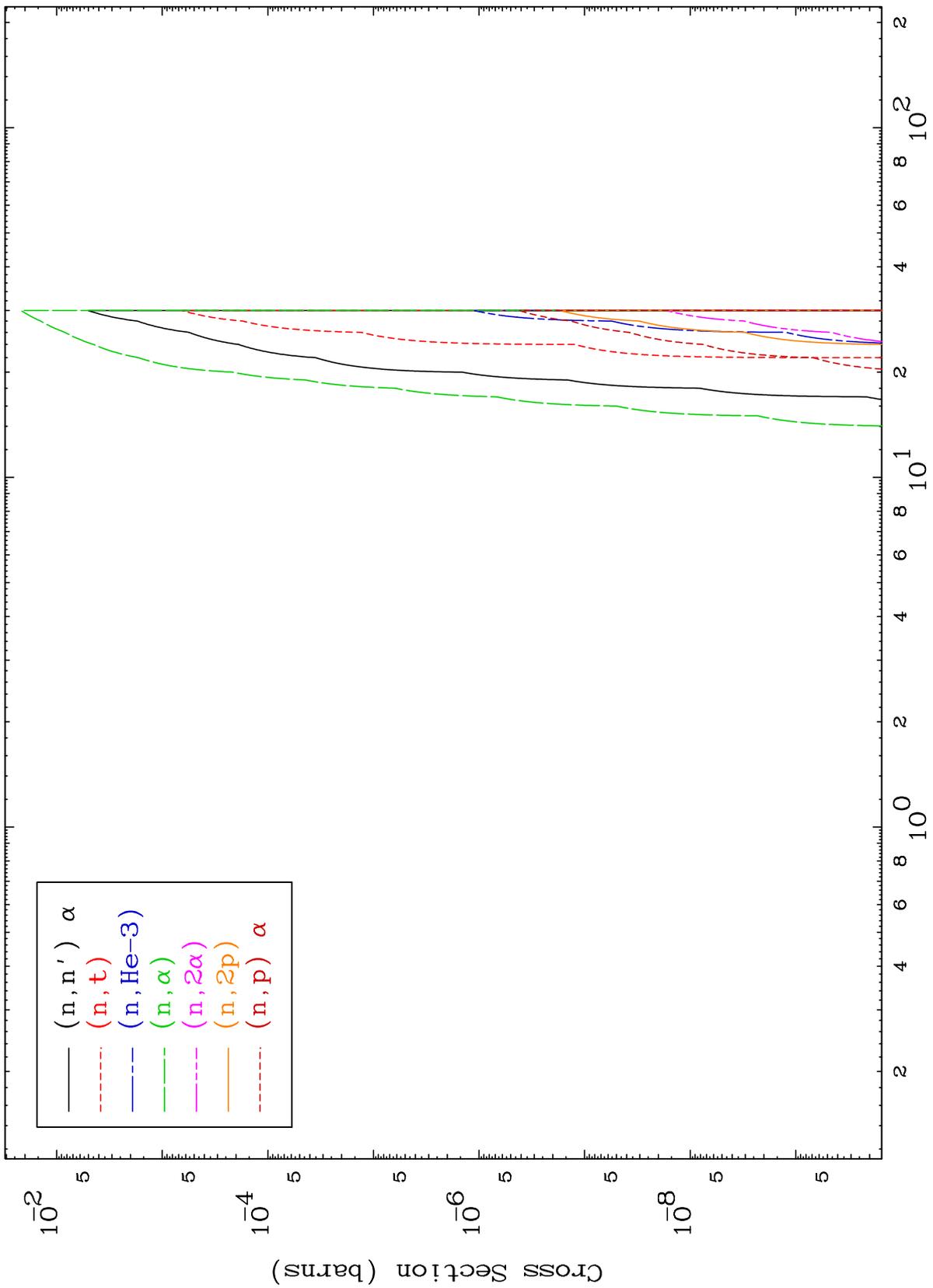
77-Ir-186



77-Ir-186

Incident Energy (MeV)



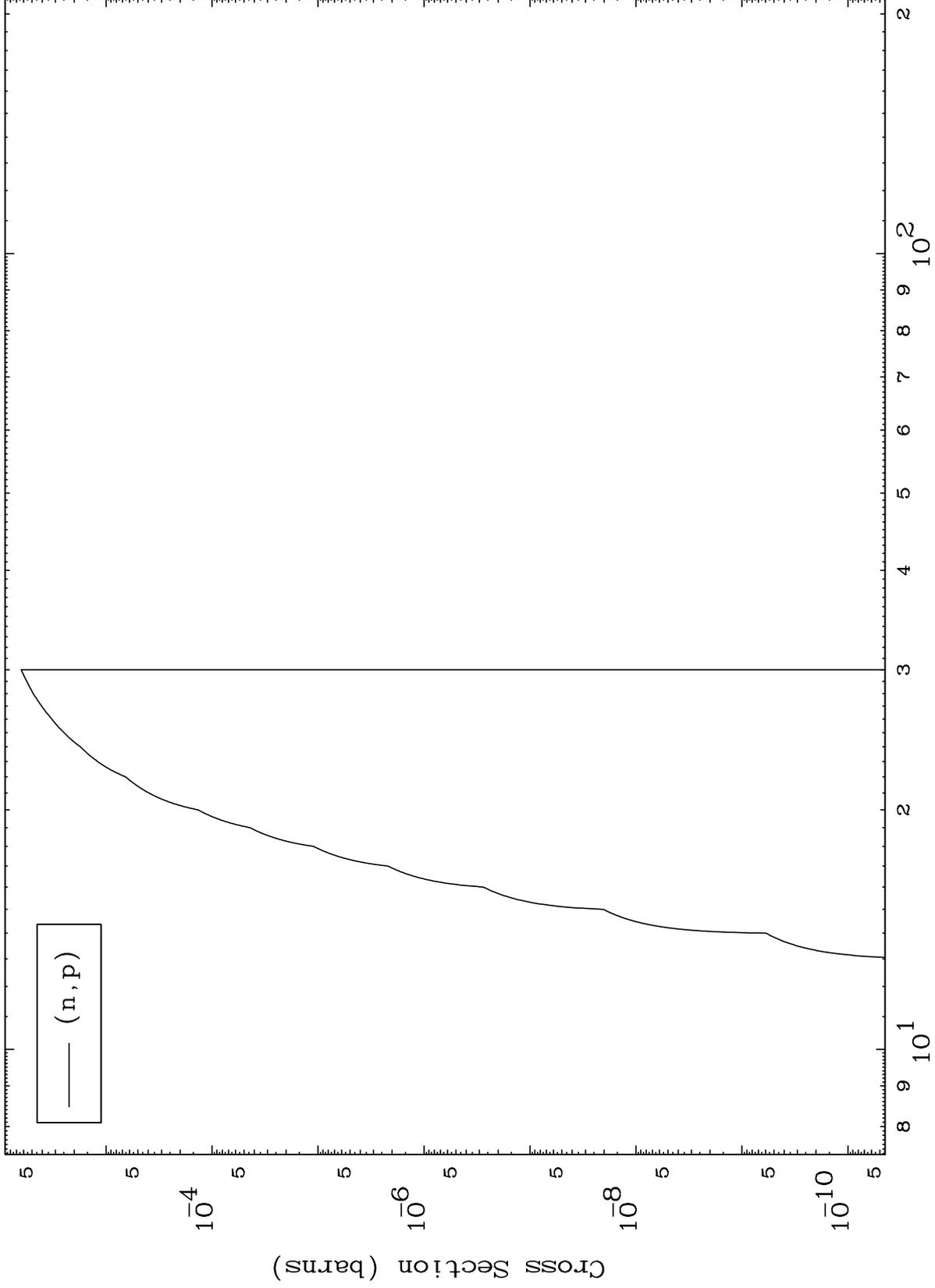


MAT 7710

( $\alpha, p$ ) Levels

77-Ir-186

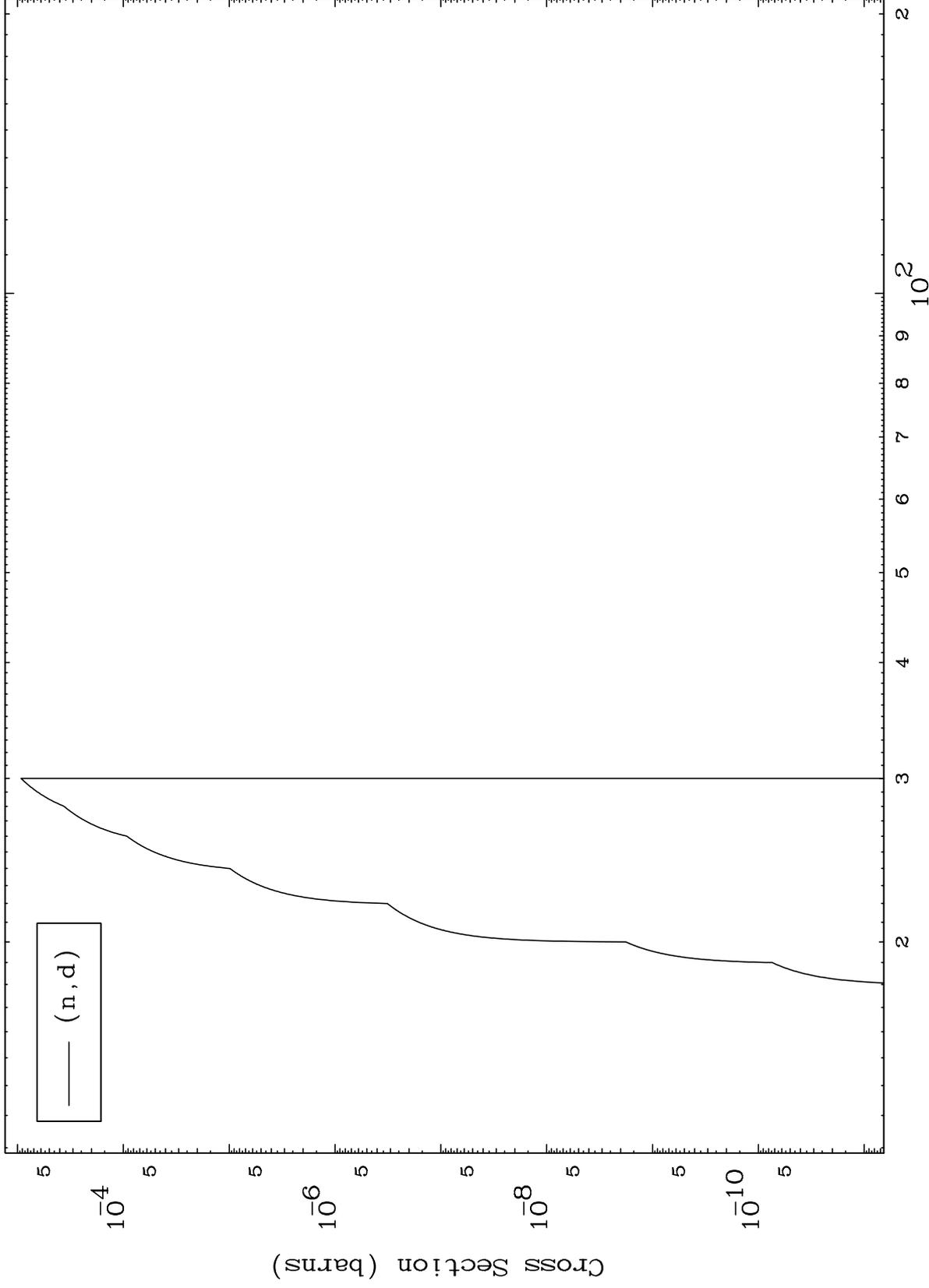
0 Kelvin Cross Sections

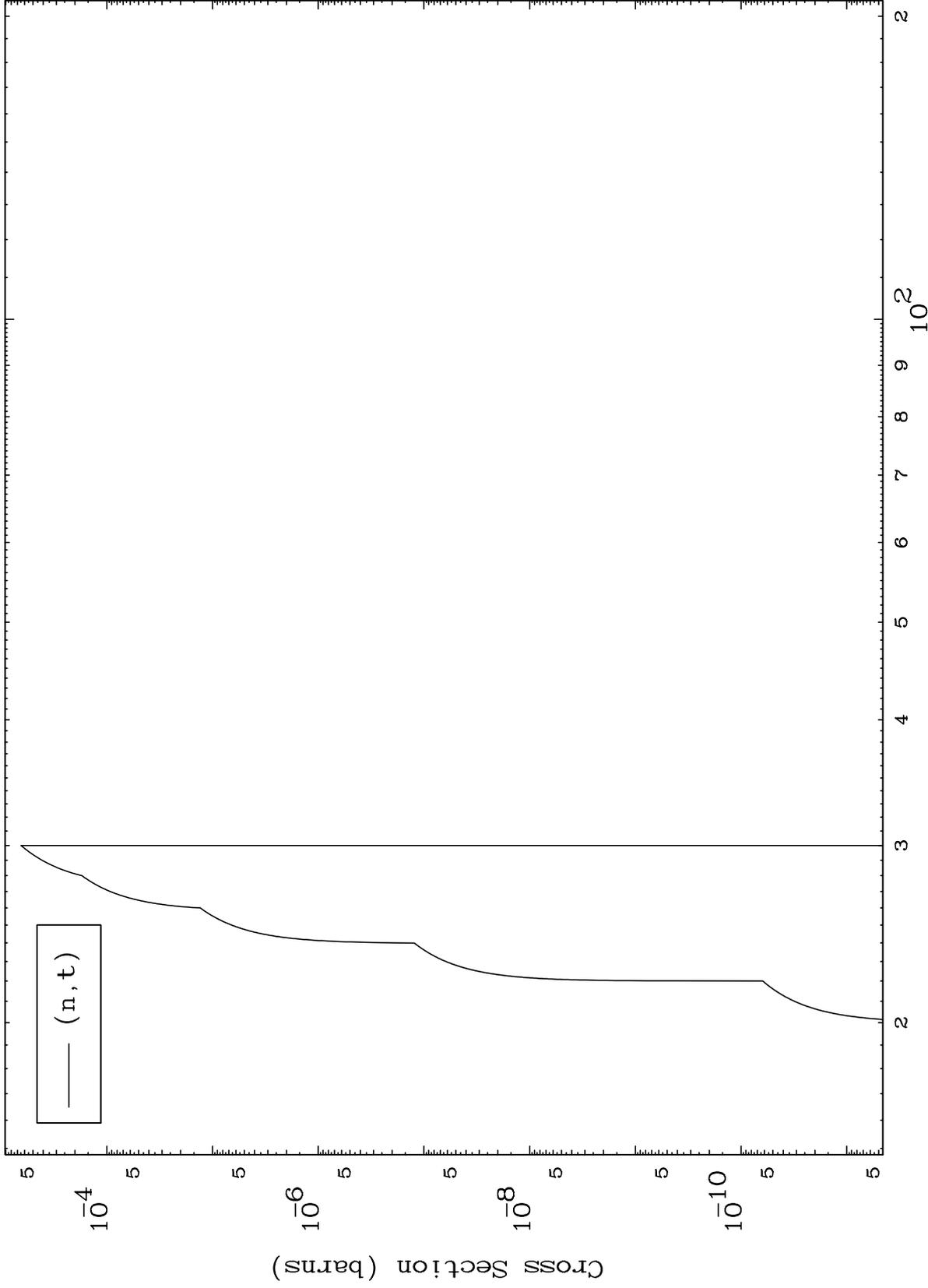


6

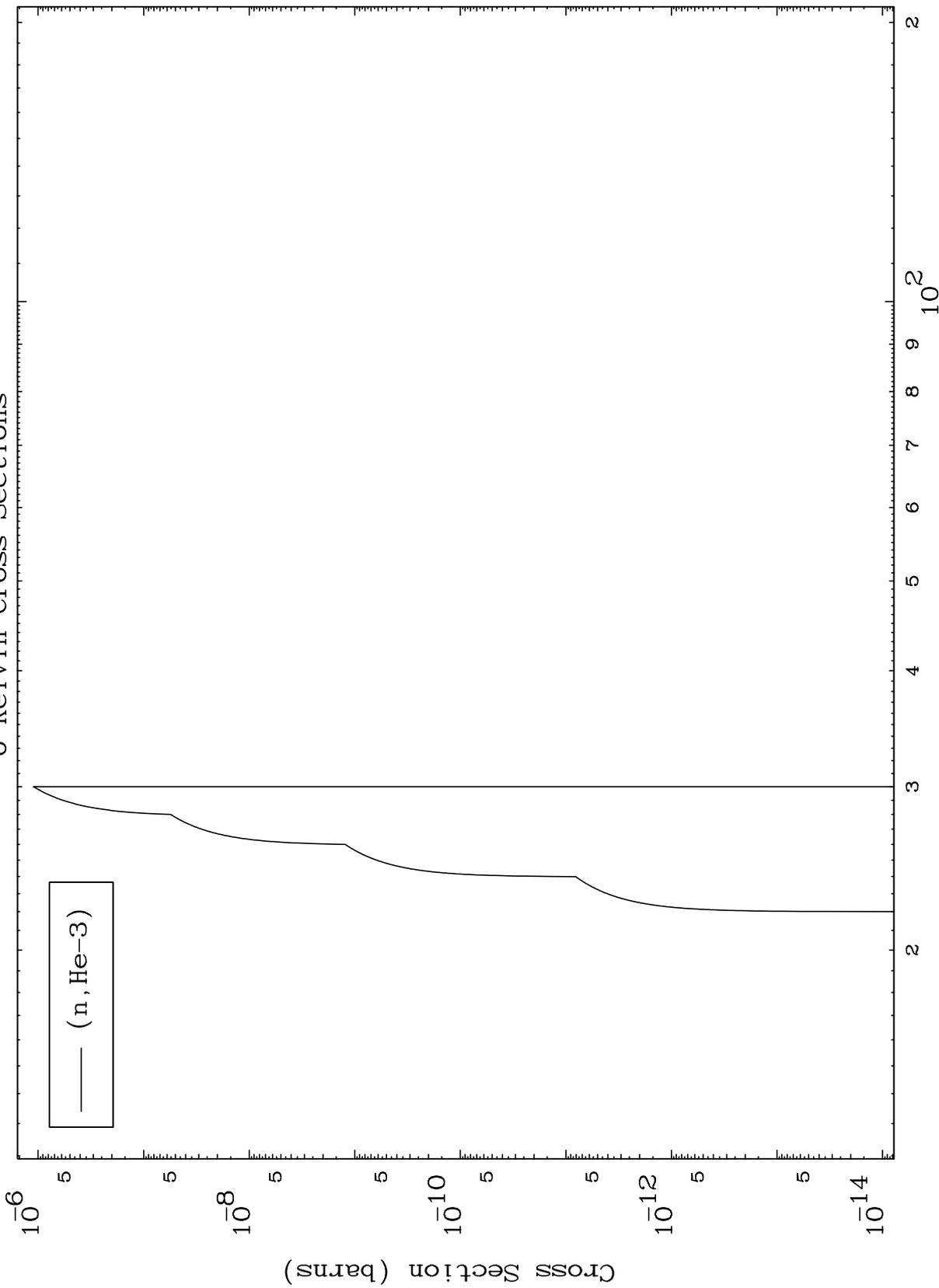
Incident Energy (MeV)

77-Ir-186





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

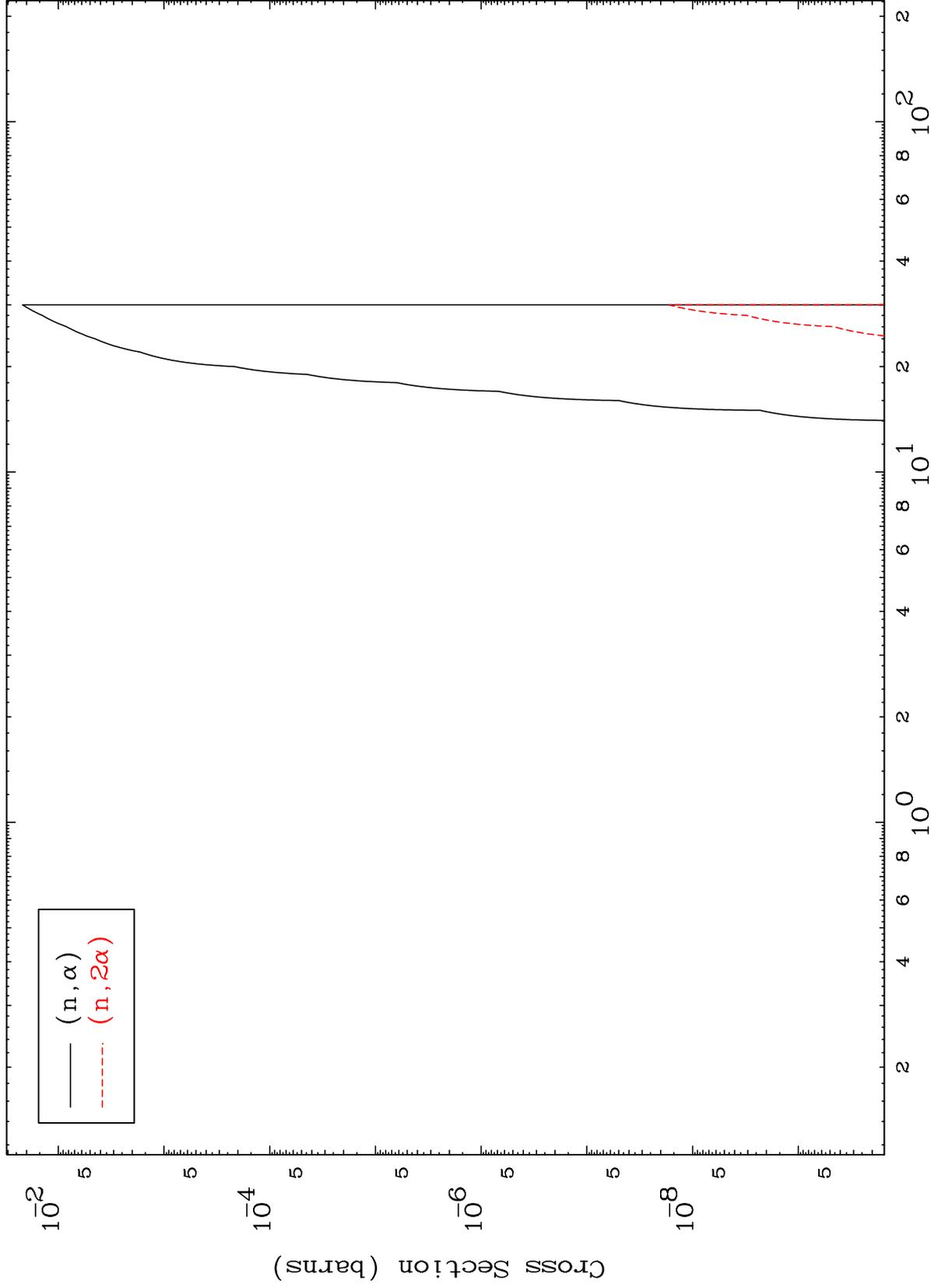


MAT 7710

( $\alpha, \alpha$ ) Levels

<sup>77</sup>Ir-186

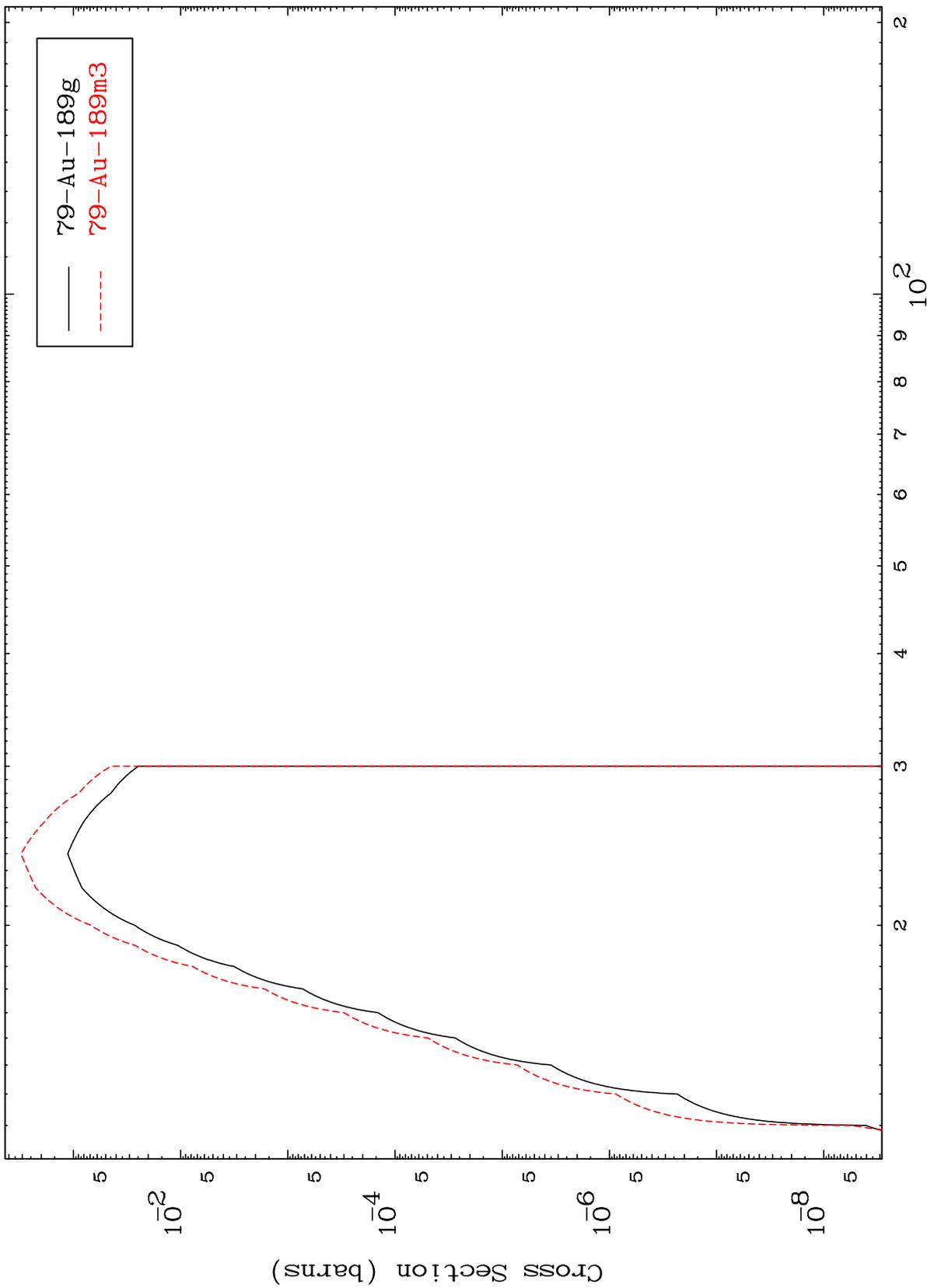
0 Kelvin Cross Sections



10

Incident Energy (MeV)

<sup>77</sup>Ir-186

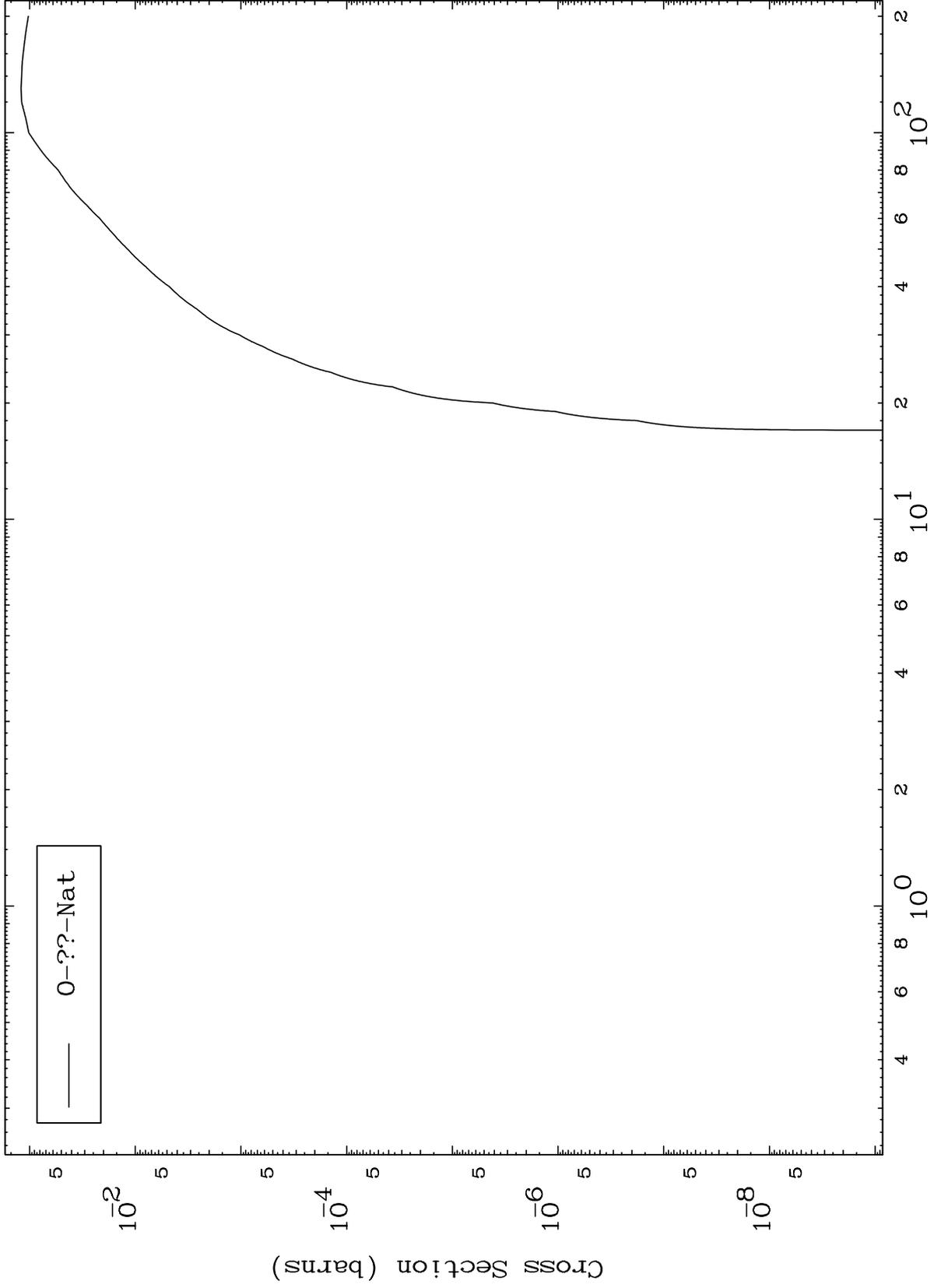


MAT 7710

Fission

<sup>77</sup>Ir-186

Radionuclide Production Cross Section

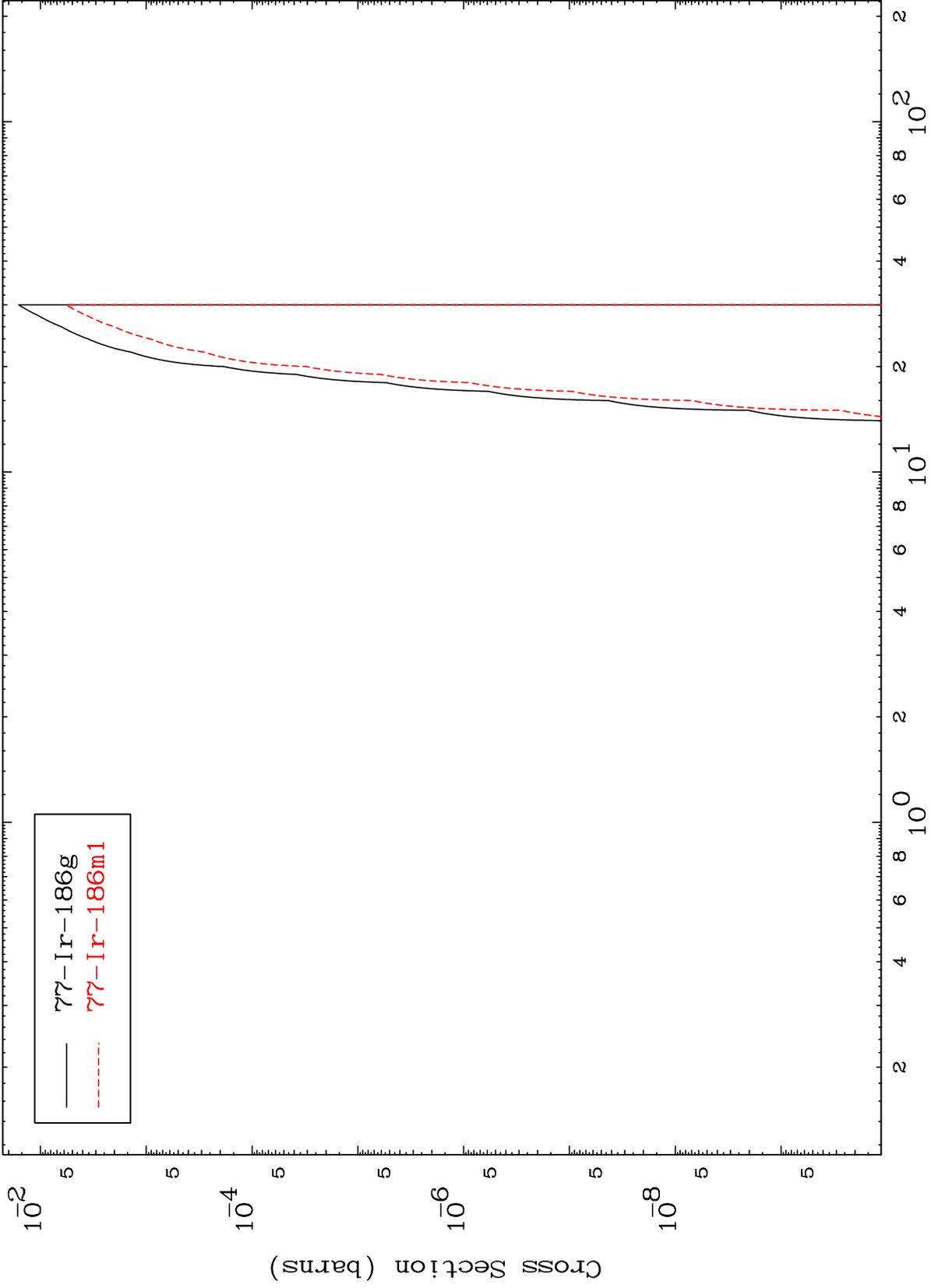


MAT 7710

(n,  $\alpha$ )

<sup>77</sup>Ir-186

Radionuclide Production Cross Section



13

Incident Energy (MeV)

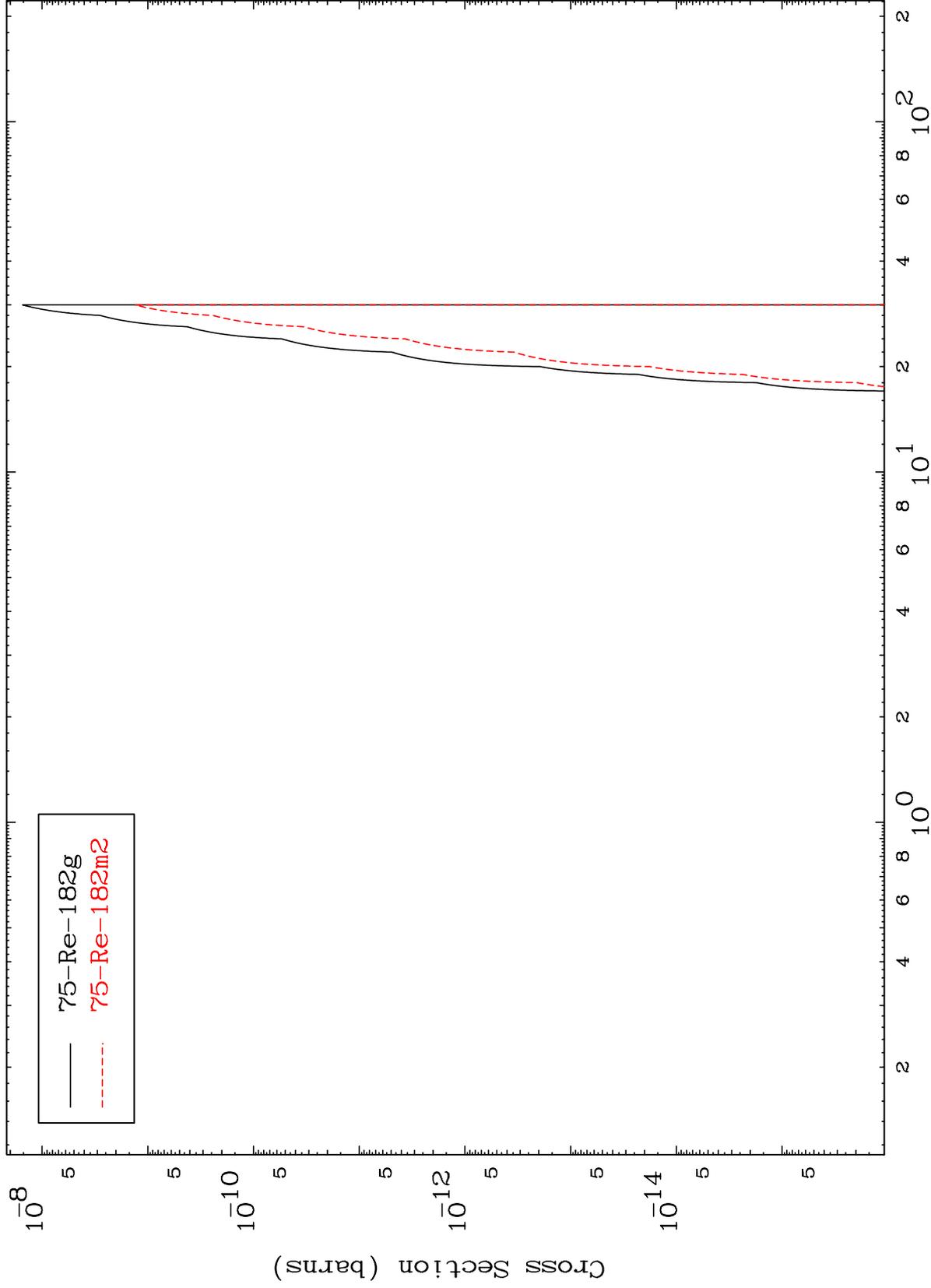
<sup>77</sup>Ir-186

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(n,2α)

<sup>77</sup>Ir-186

Radionuclide Production Cross Section



14

Incident Energy (MeV)

<sup>77</sup>Ir-186