

Program EVALPLOT  
(Version 2021-1)

by

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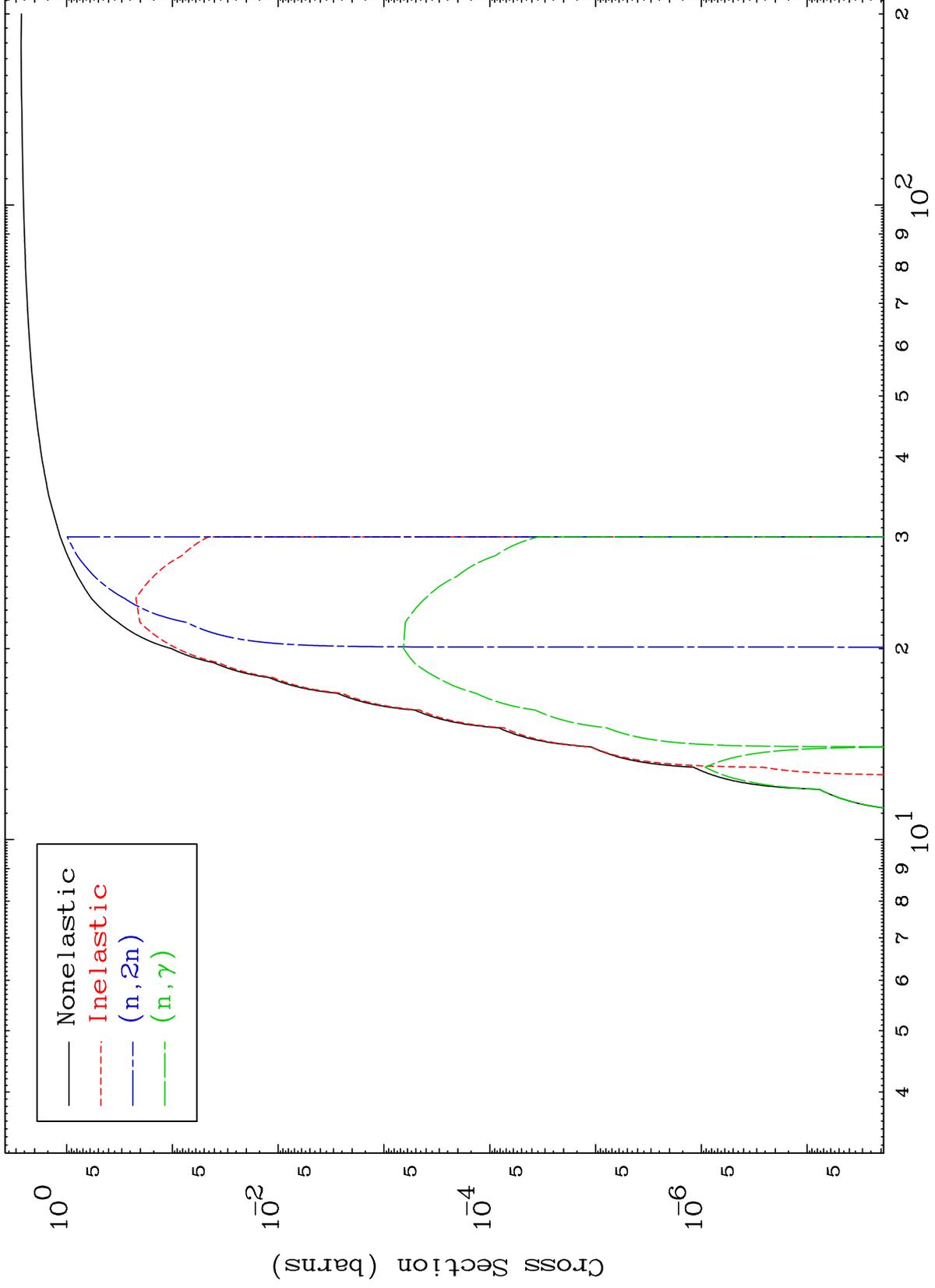
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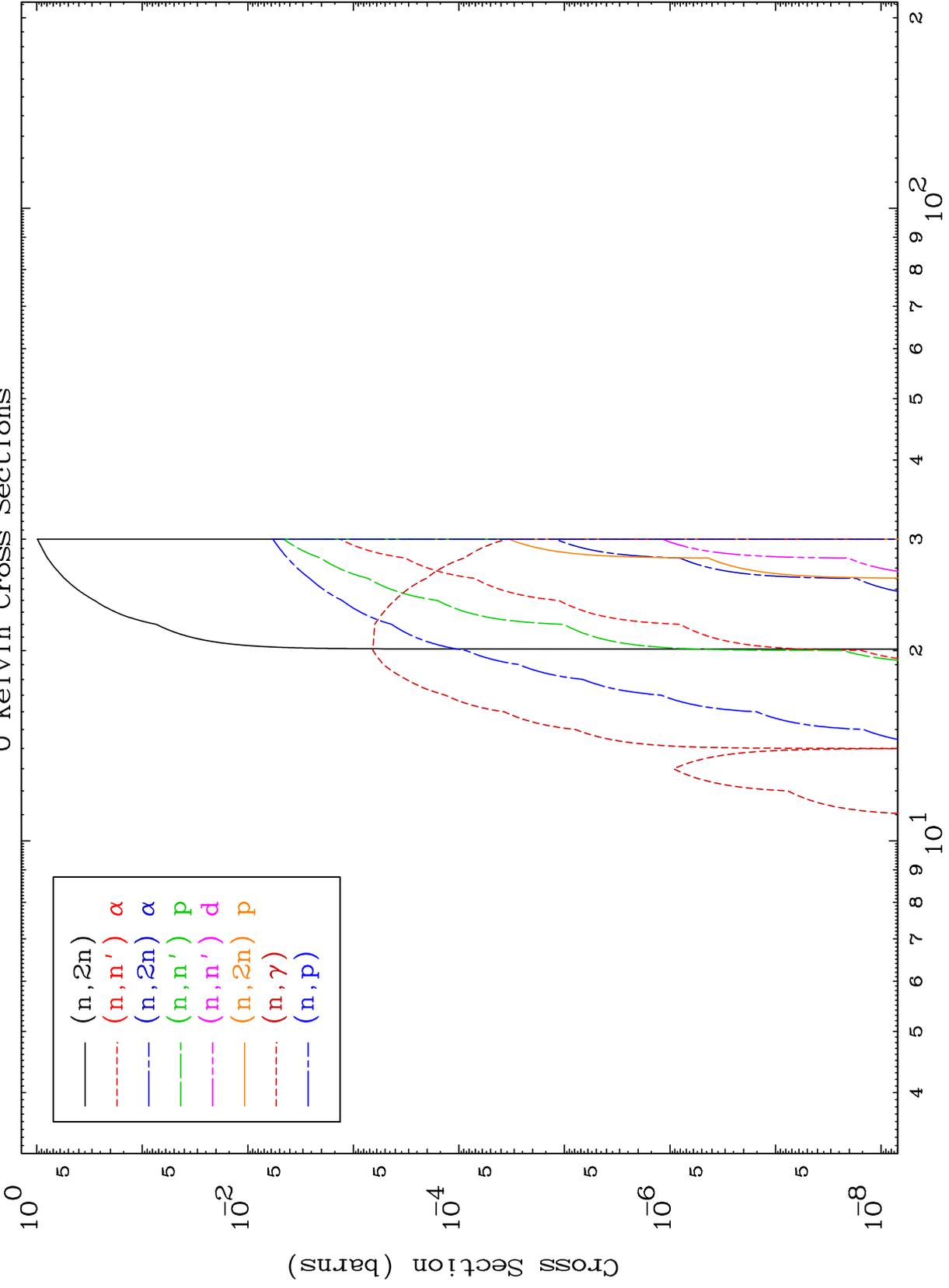
Press Mouse Button to Start

MAT 7713

0 Kelvin Cross Sections

77-Ir-187

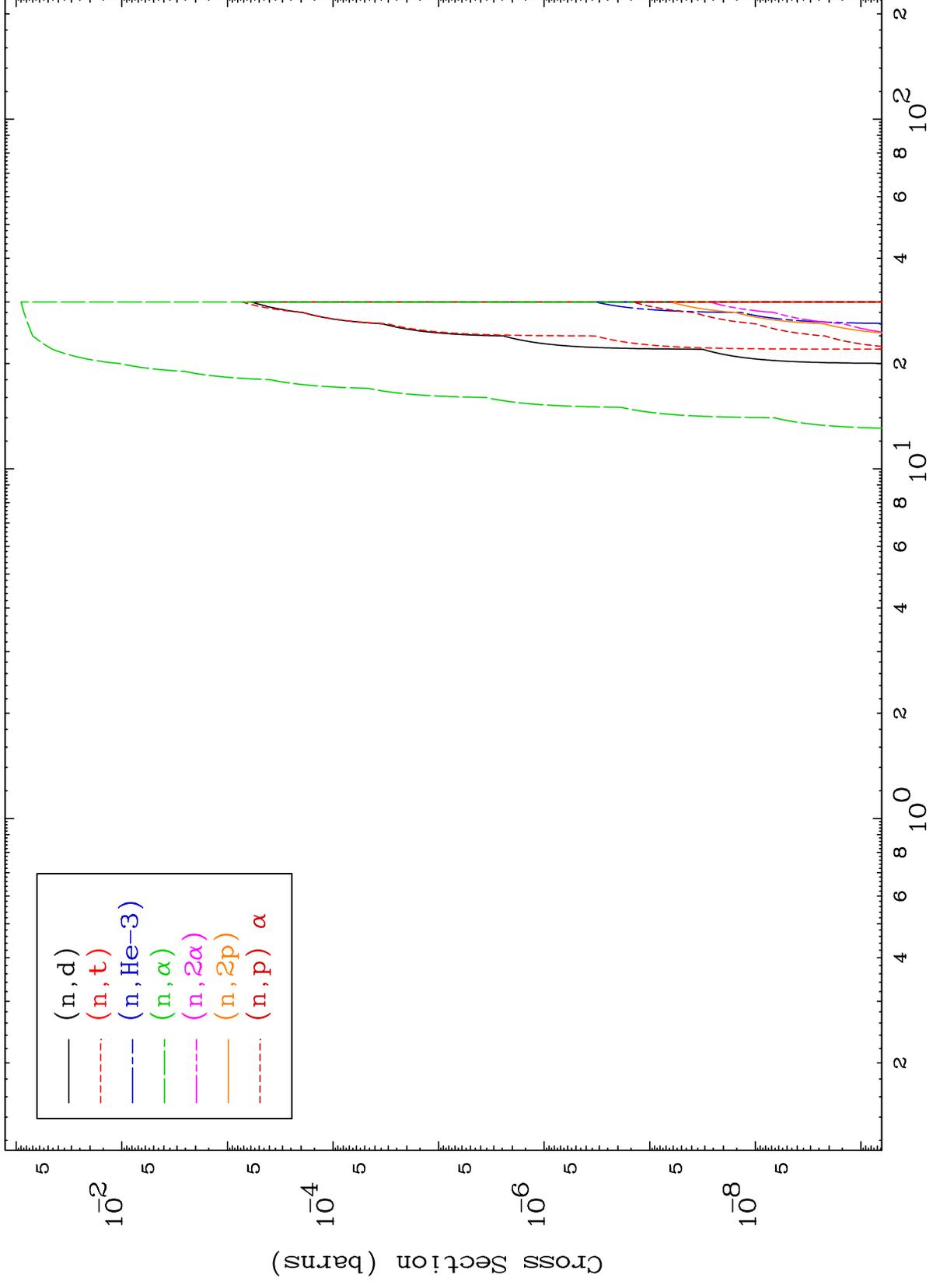




MAT 7713

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

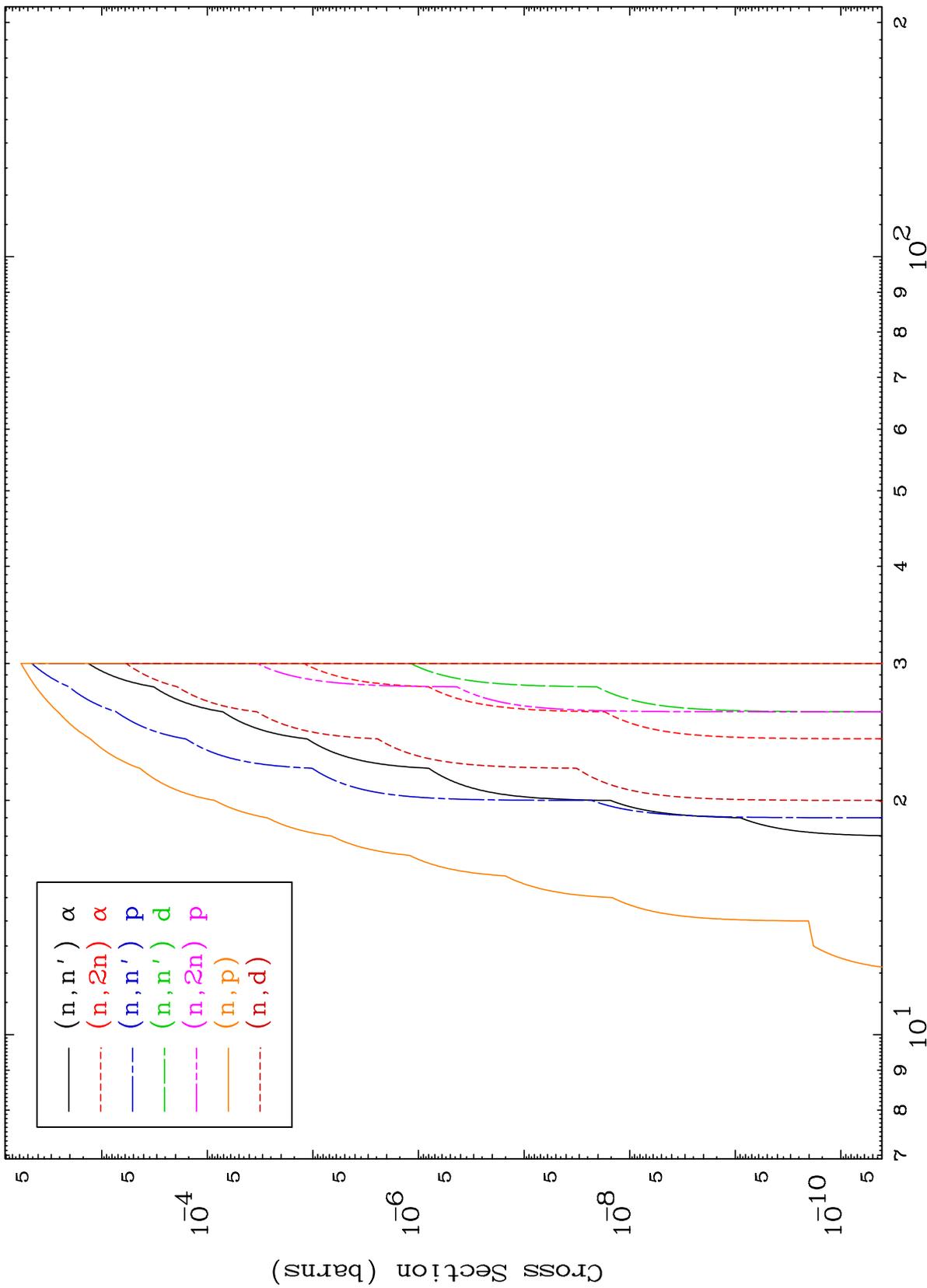
77-Ir-187

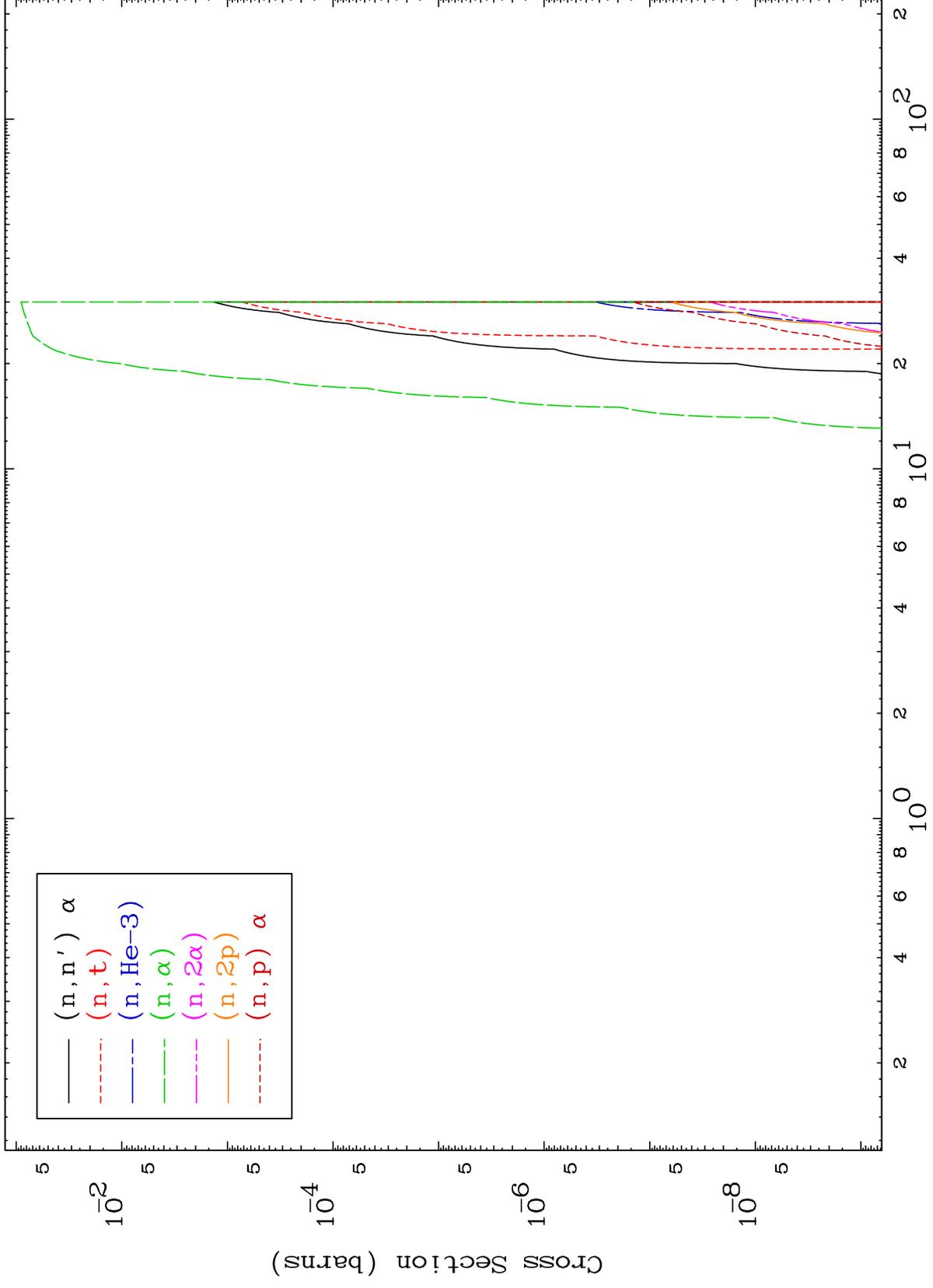


77-Ir-187

Incident Energy (MeV)

3

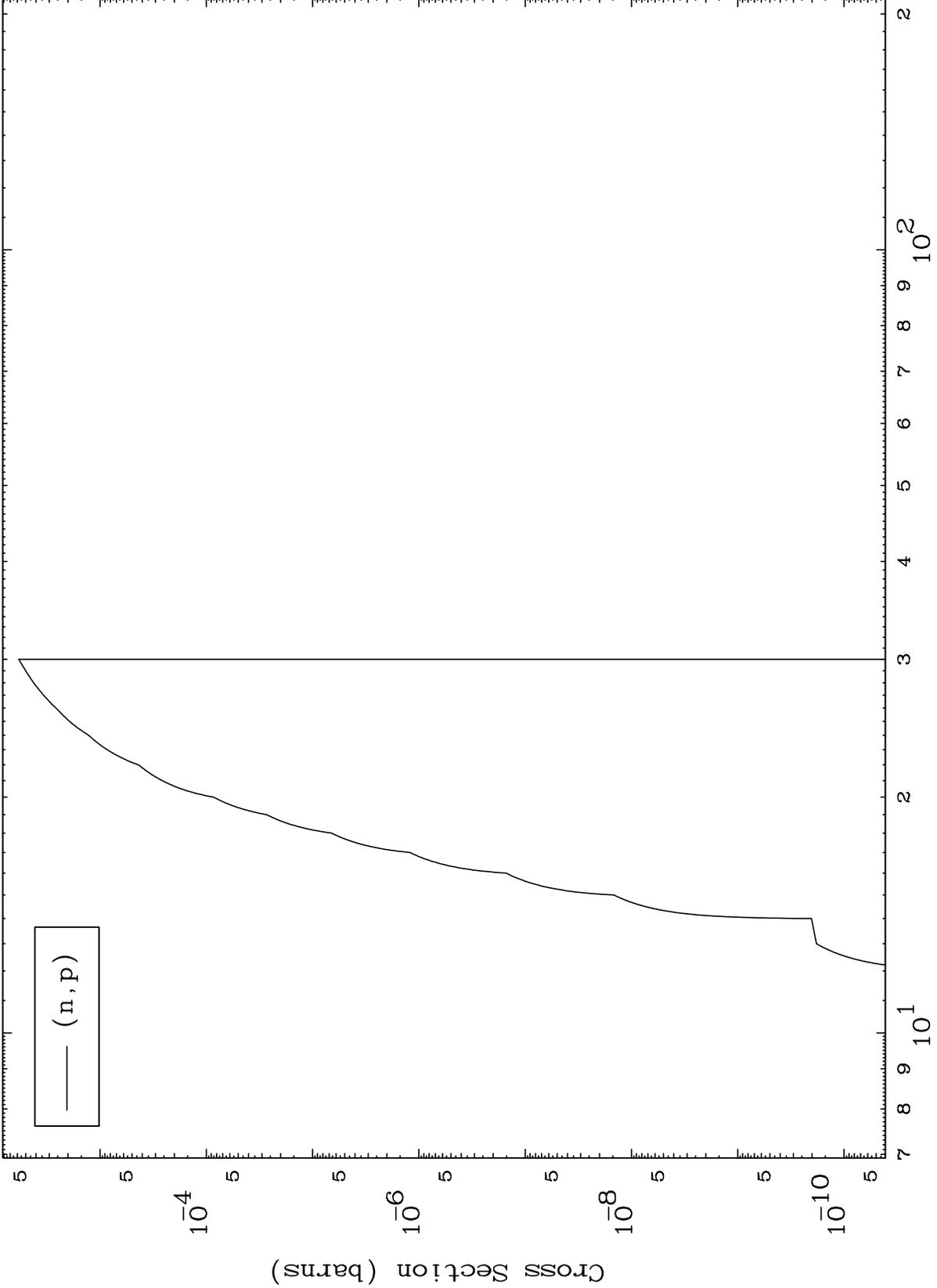




MAT 7713

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-187



6

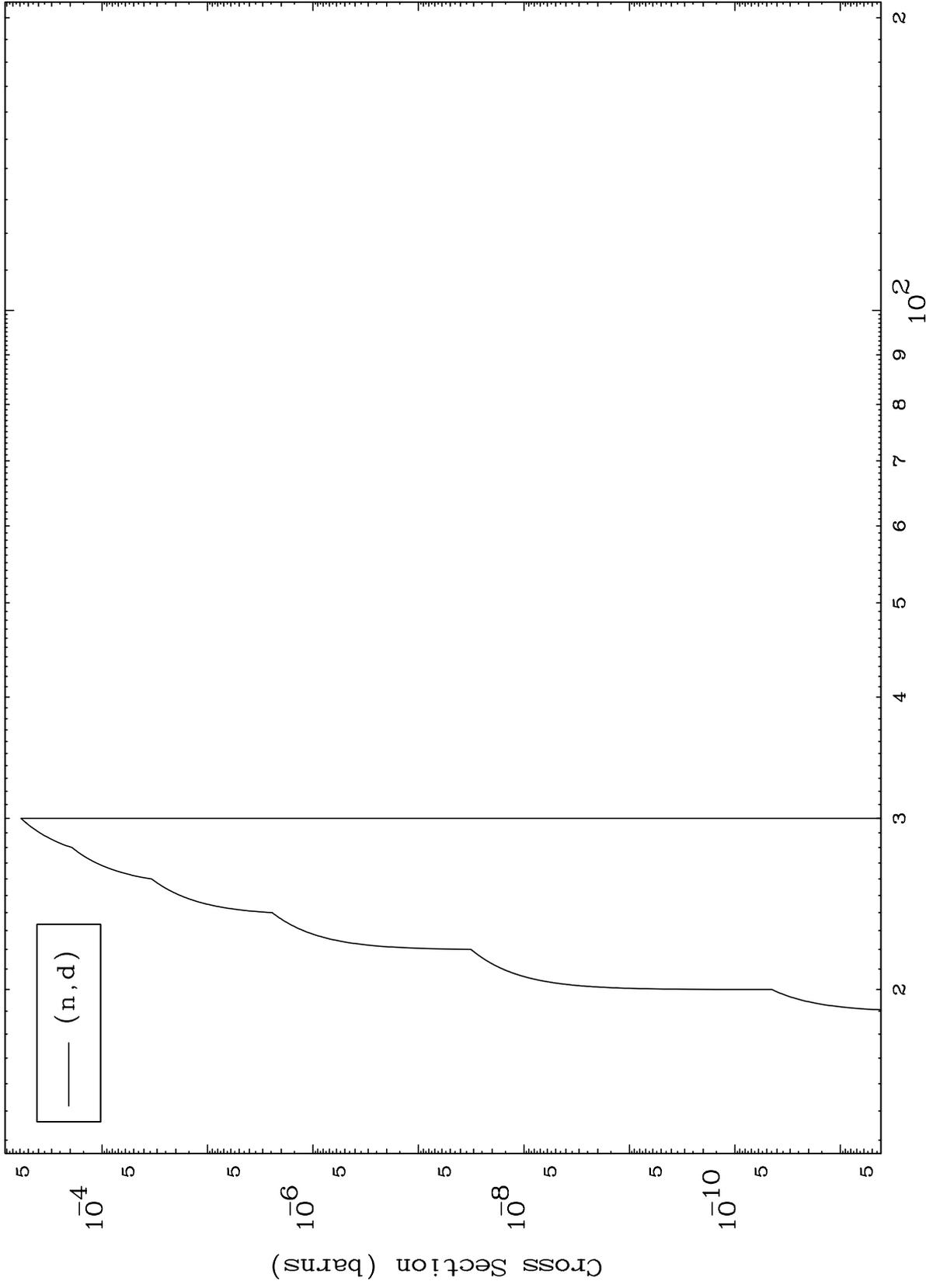
Incident Energy (MeV)

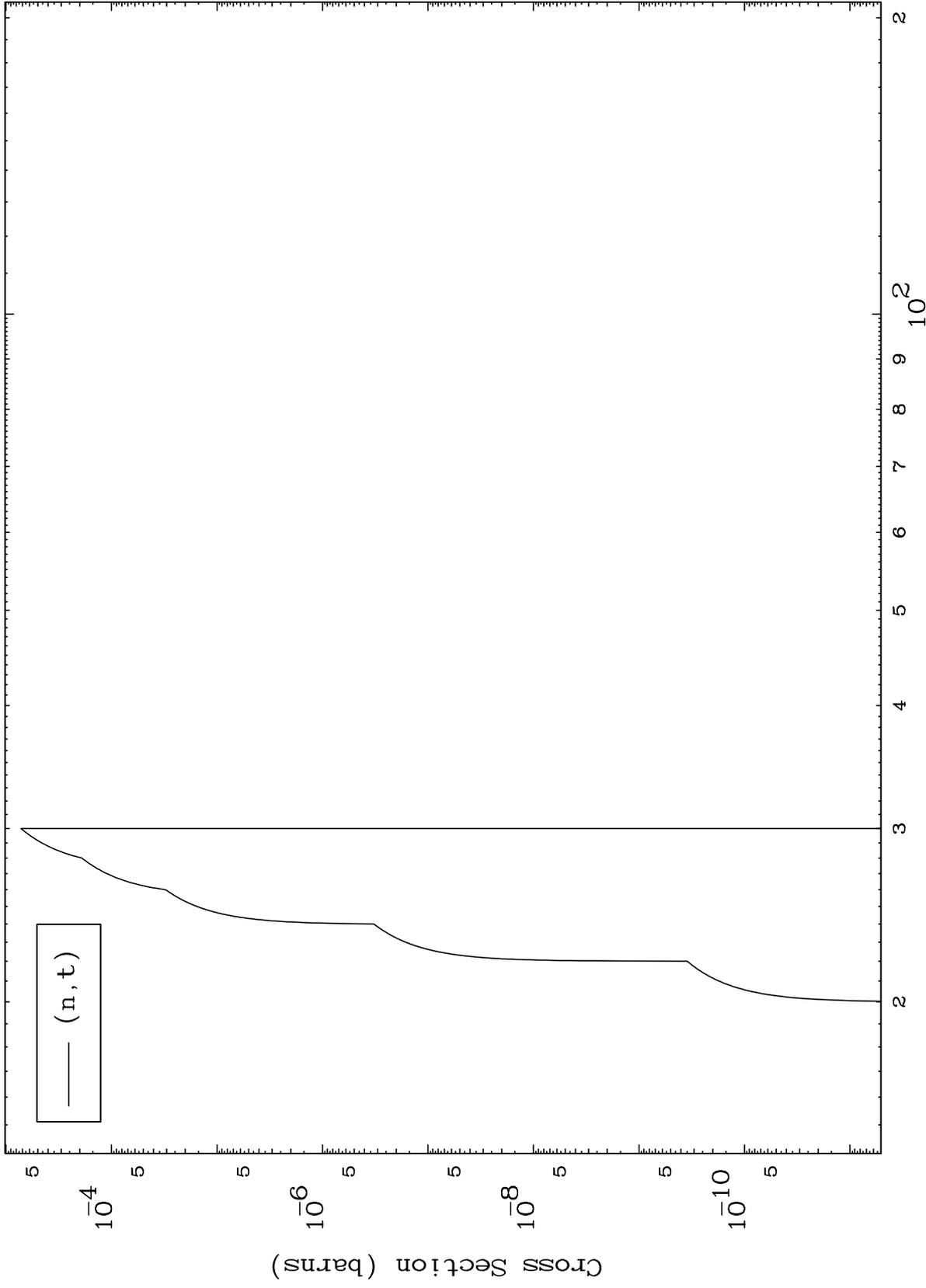
77-Ir-187

MAT 7713

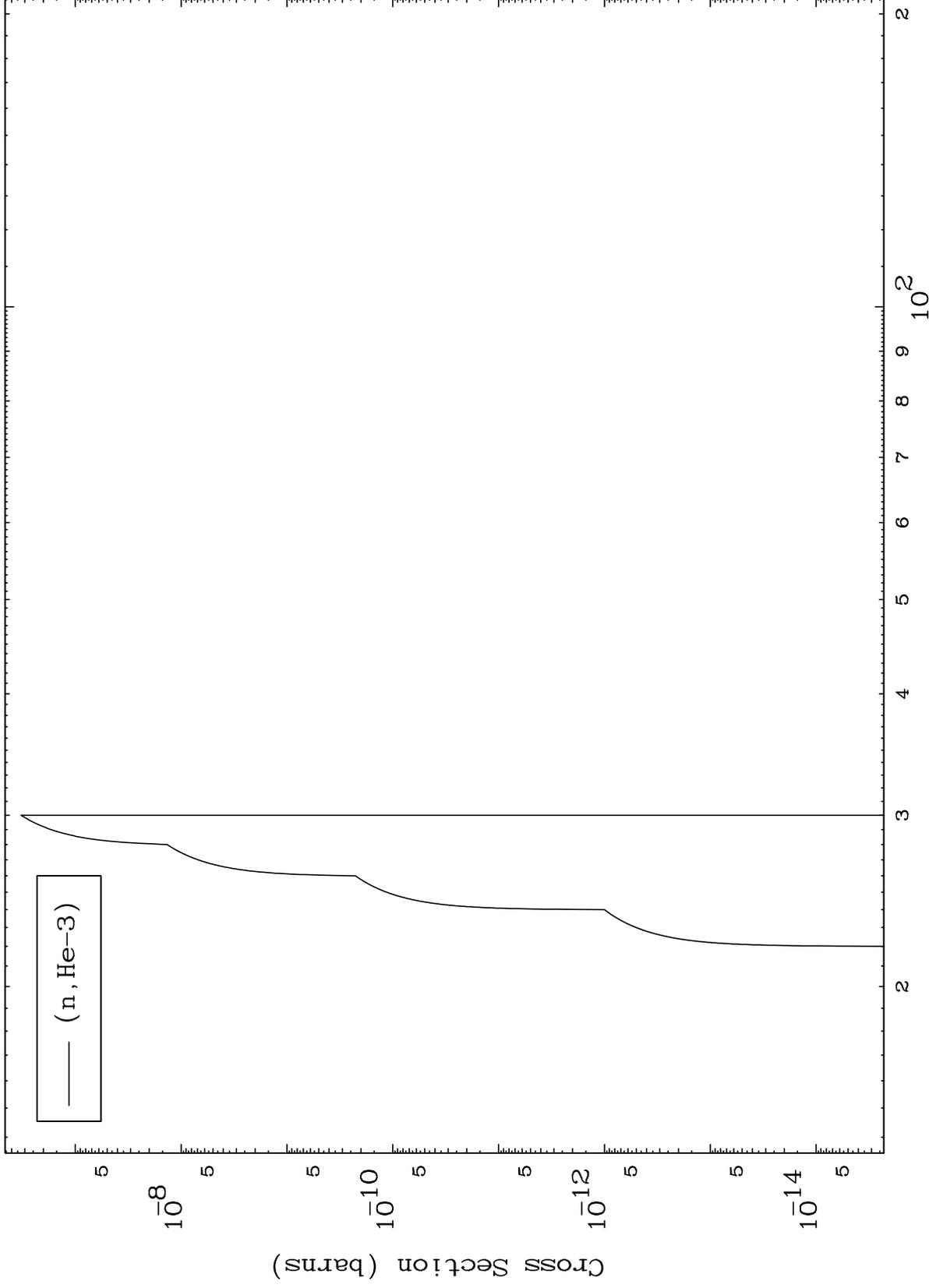
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-187





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

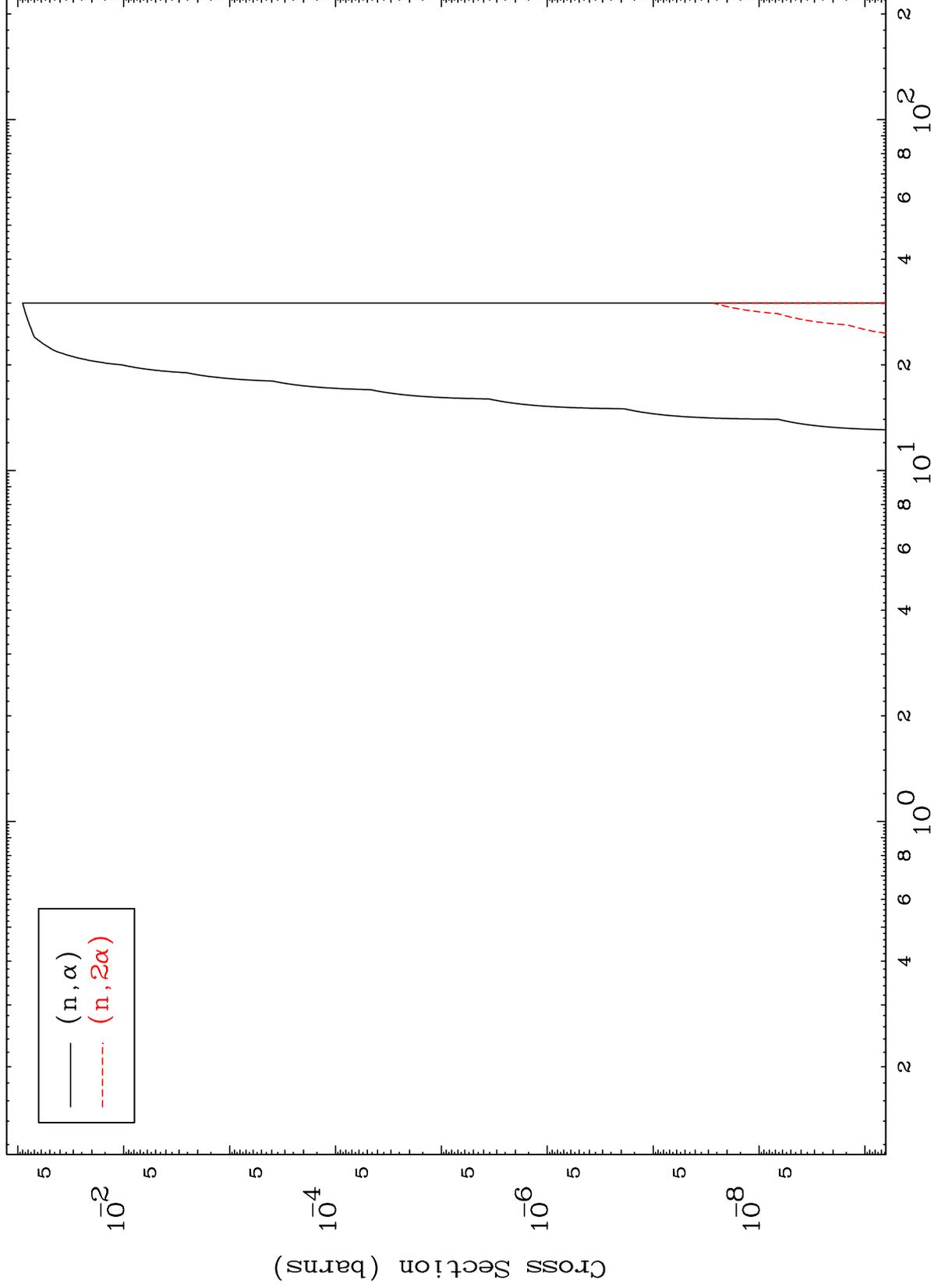


MAT 7713

( $\alpha, \alpha$ ) Levels

77-Ir-187

0 Kelvin Cross Sections



10

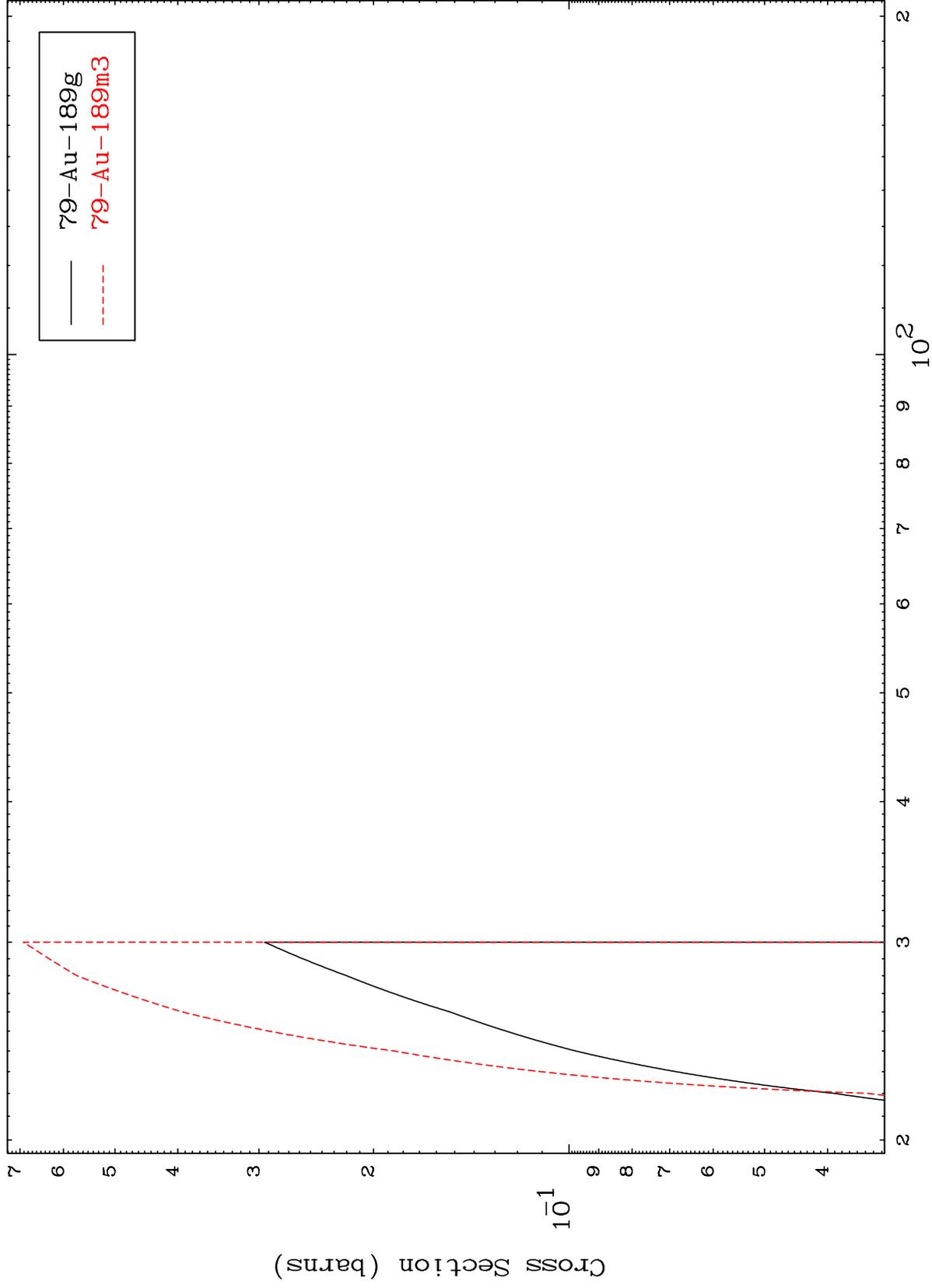
Incident Energy (MeV)

77-Ir-187

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77-Ir-187

(n,2n)  
Radionuclide Production Cross Section



77-Ir-187

Incident Energy (MeV)

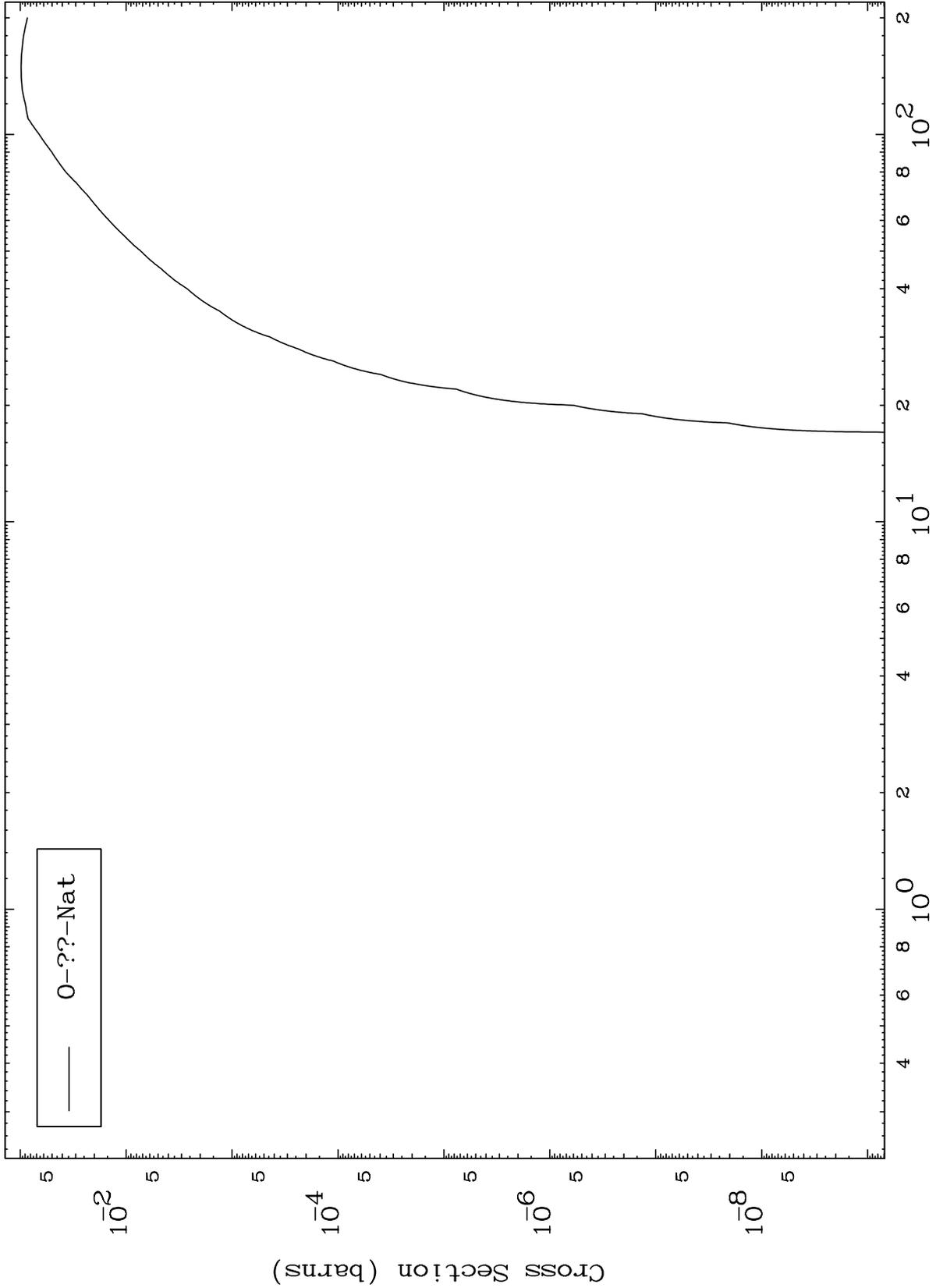
11

MAT 7713

Fission

<sup>77</sup>Ir-187

Radionuclide Production Cross Section



12

Incident Energy (MeV)

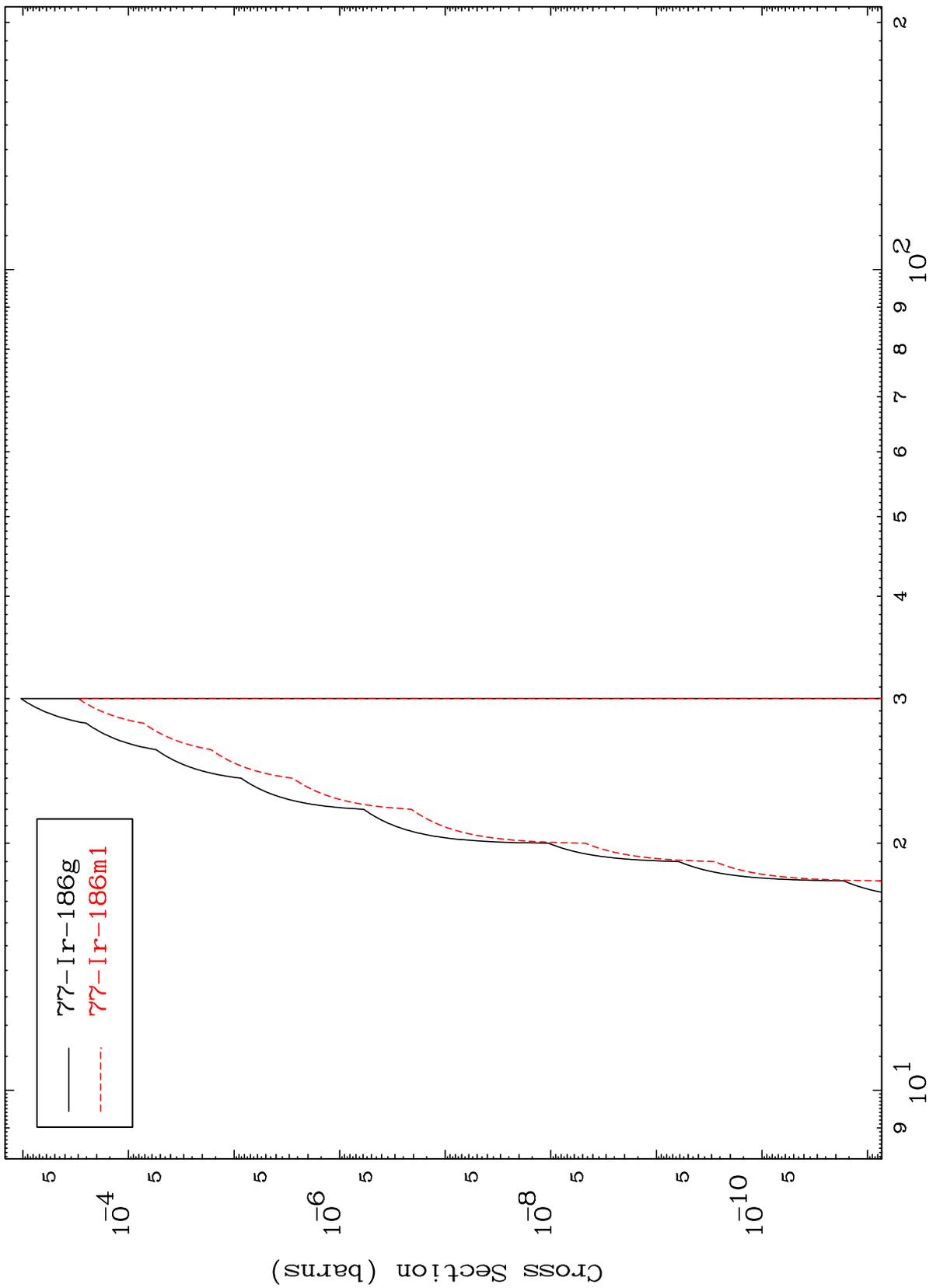
<sup>77</sup>Ir-187

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$(n, n') \alpha$

$^{77}\text{Ir}-187$

Radionuclide Production Cross Section



13

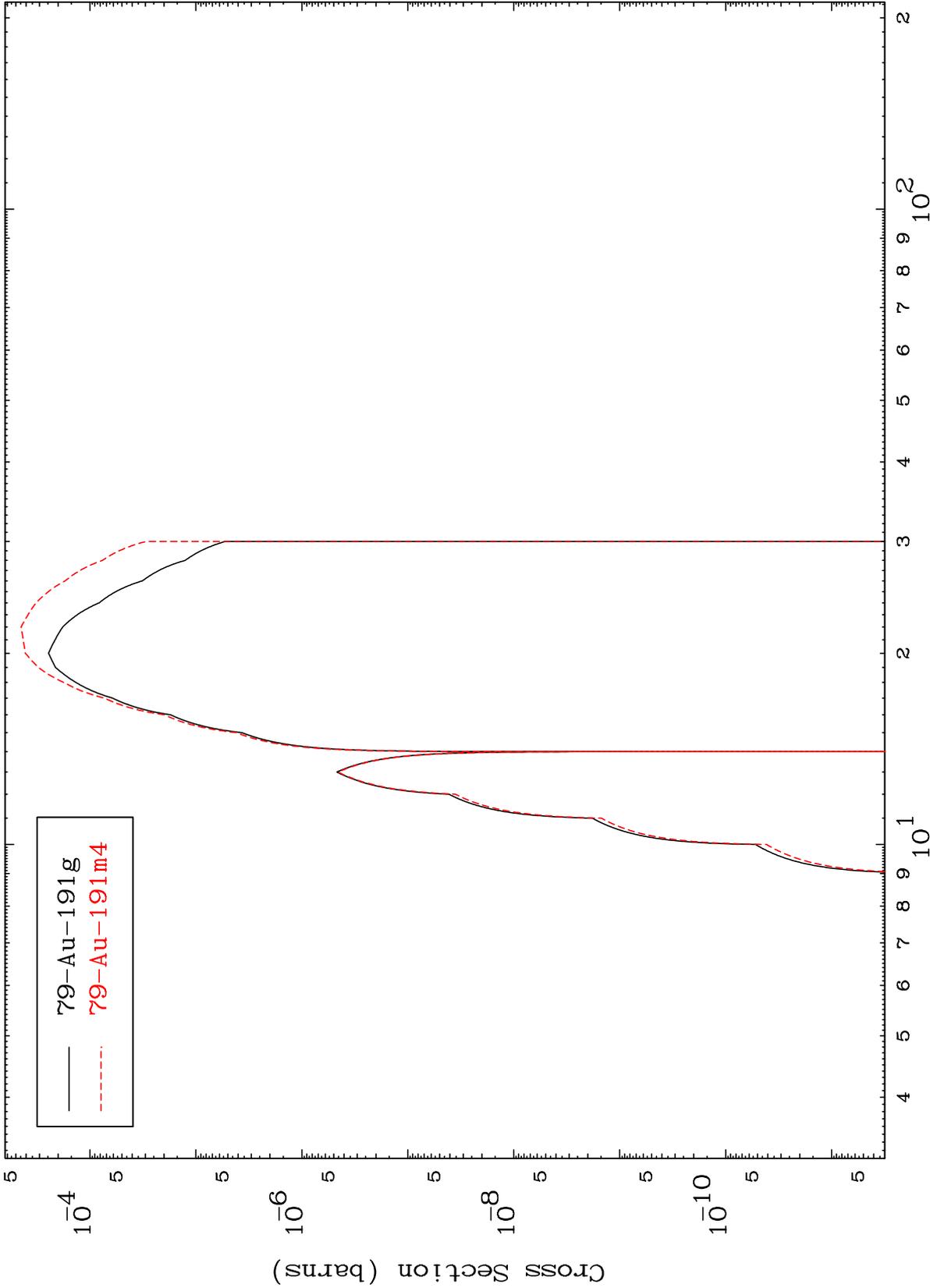
Incident Energy (MeV)

$^{77}\text{Ir}-187$

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77-Ir-187

(n,γ)  
Radionuclide Production Cross Section



Incident Energy (MeV)

77-Ir-187

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