

Program EVALPLOT
(Version 2021-1)

by

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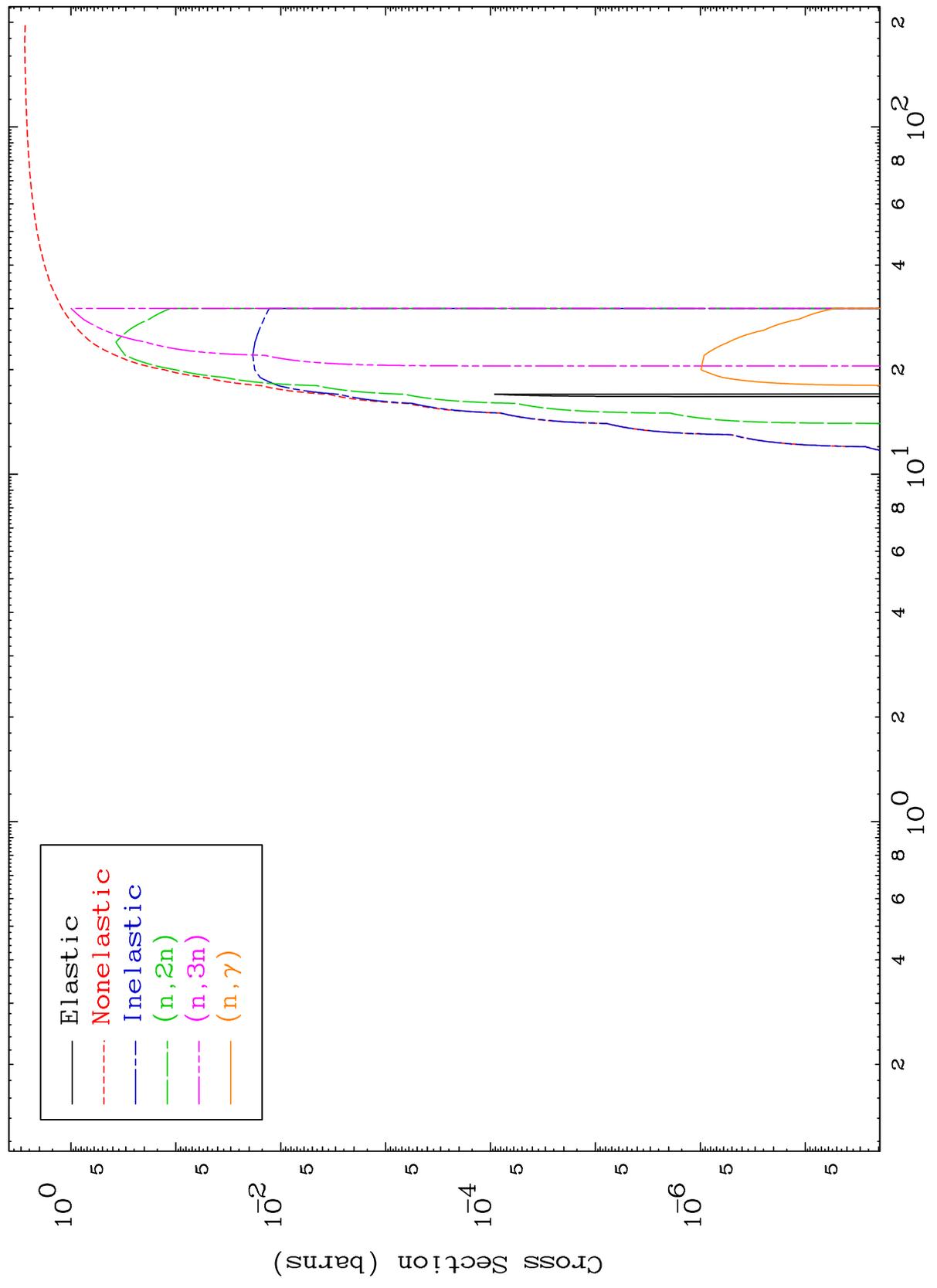
Press Mouse Button to Start

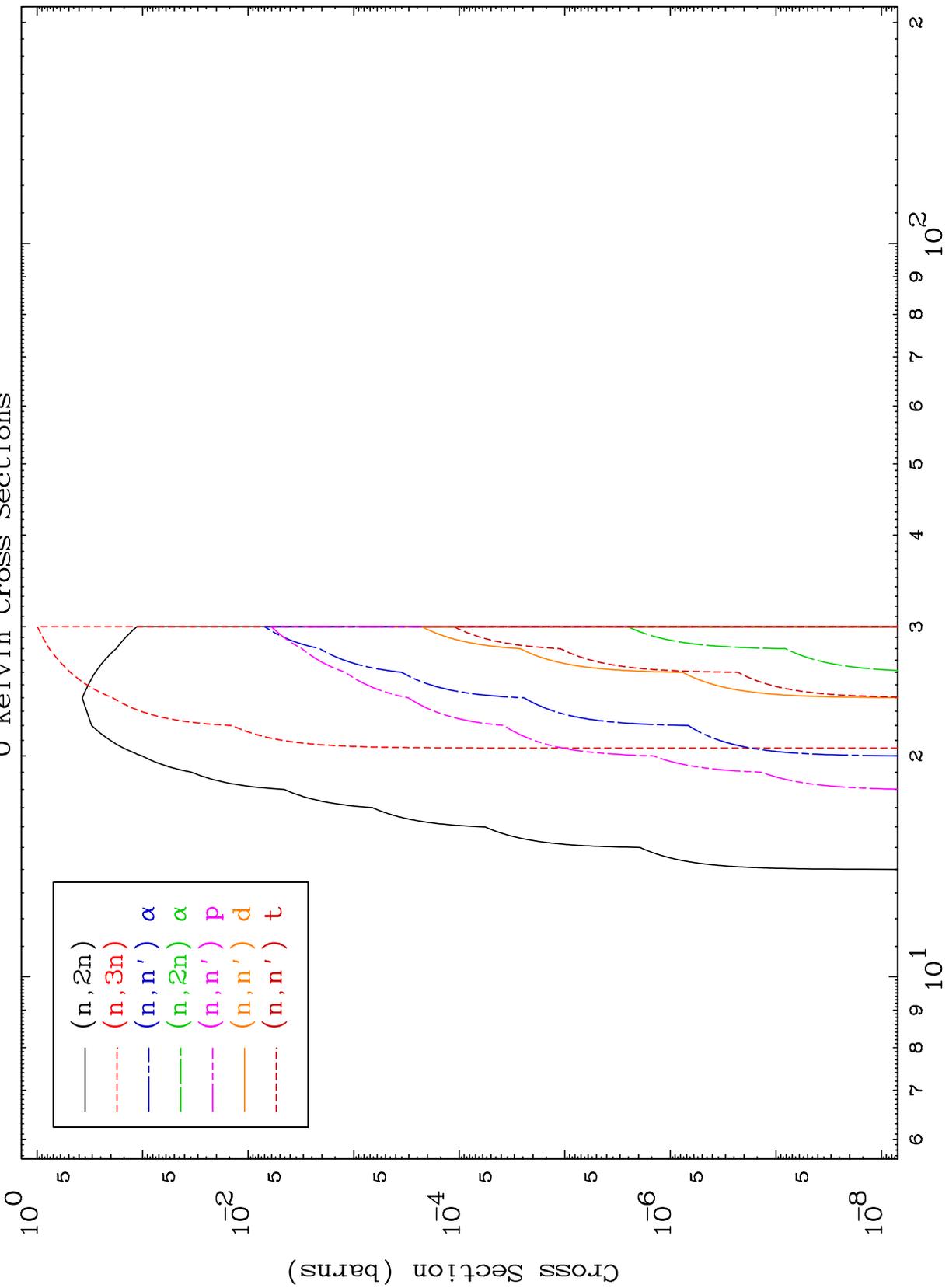
MAT 7740

0 Kelvin

Major Cross Sections

77-Ir-196

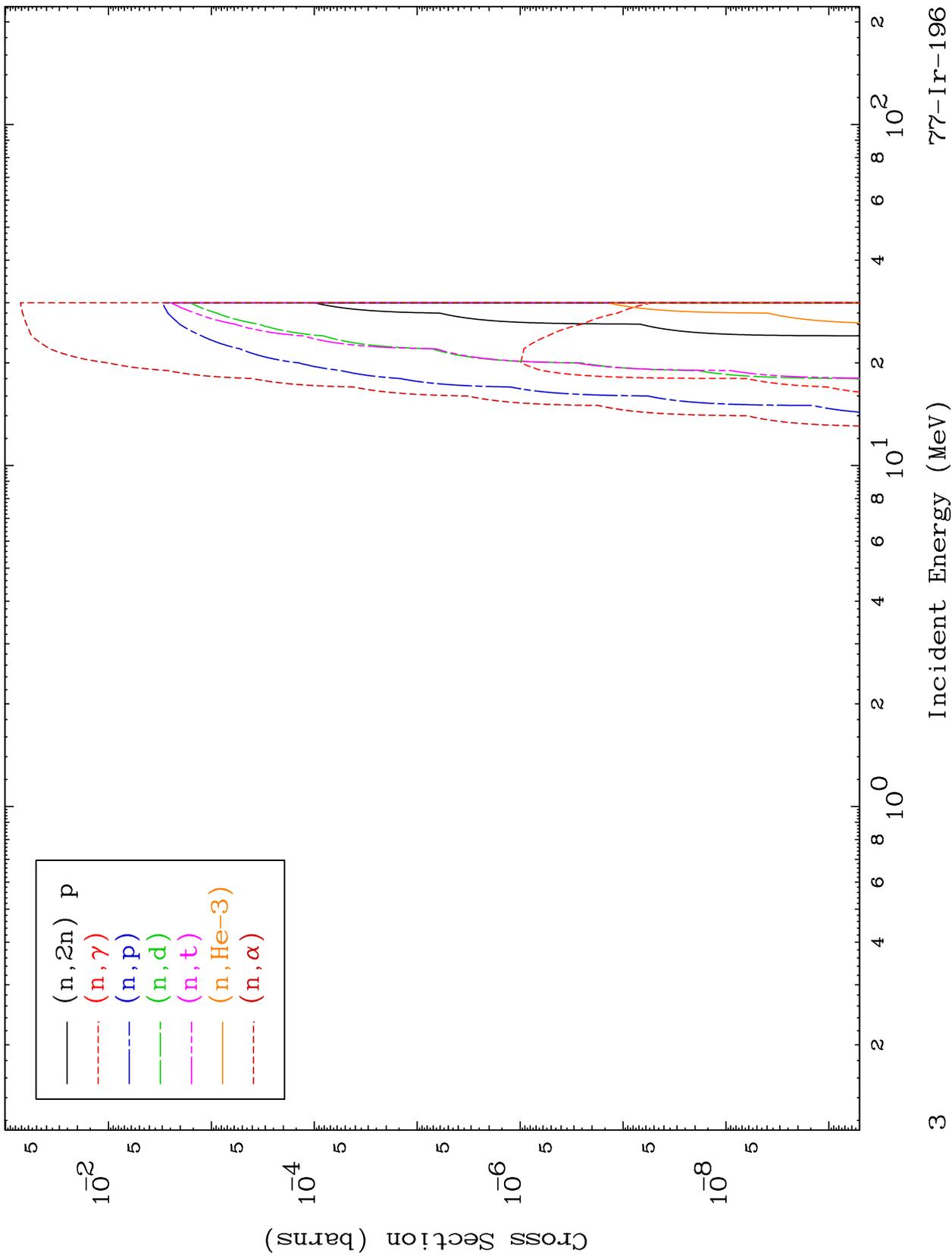




MAT 7740

α Neutron Absorption
0 Kelvin Cross Sections

77-Ir-196



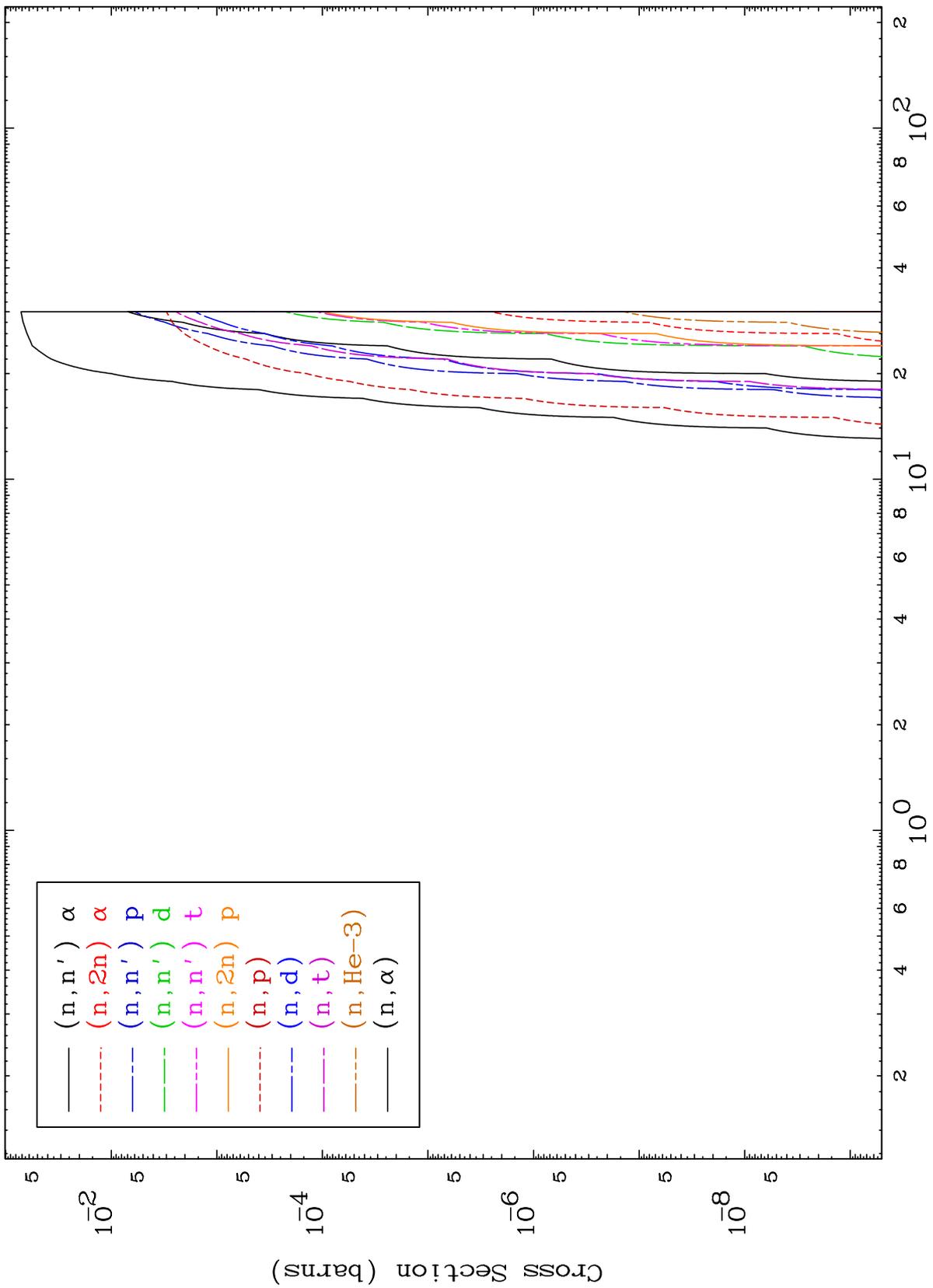
77-Ir-196

Incident Energy (MeV)

MAT 7740

α Charged Particle
0 Kelvin Cross Sections

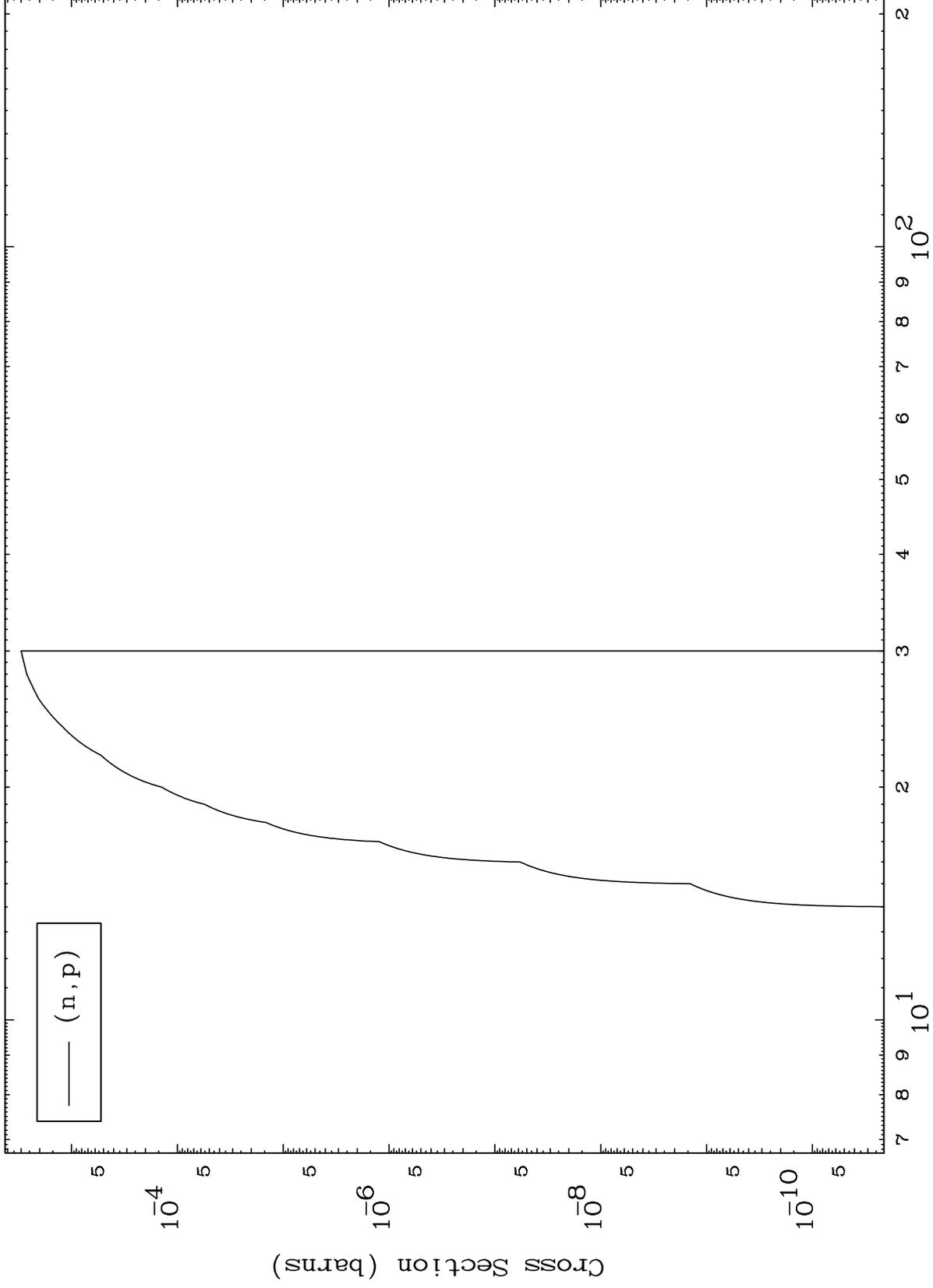
77-Ir-196



MAT 7740

(α, p) Levels
0 Kelvin Cross Sections

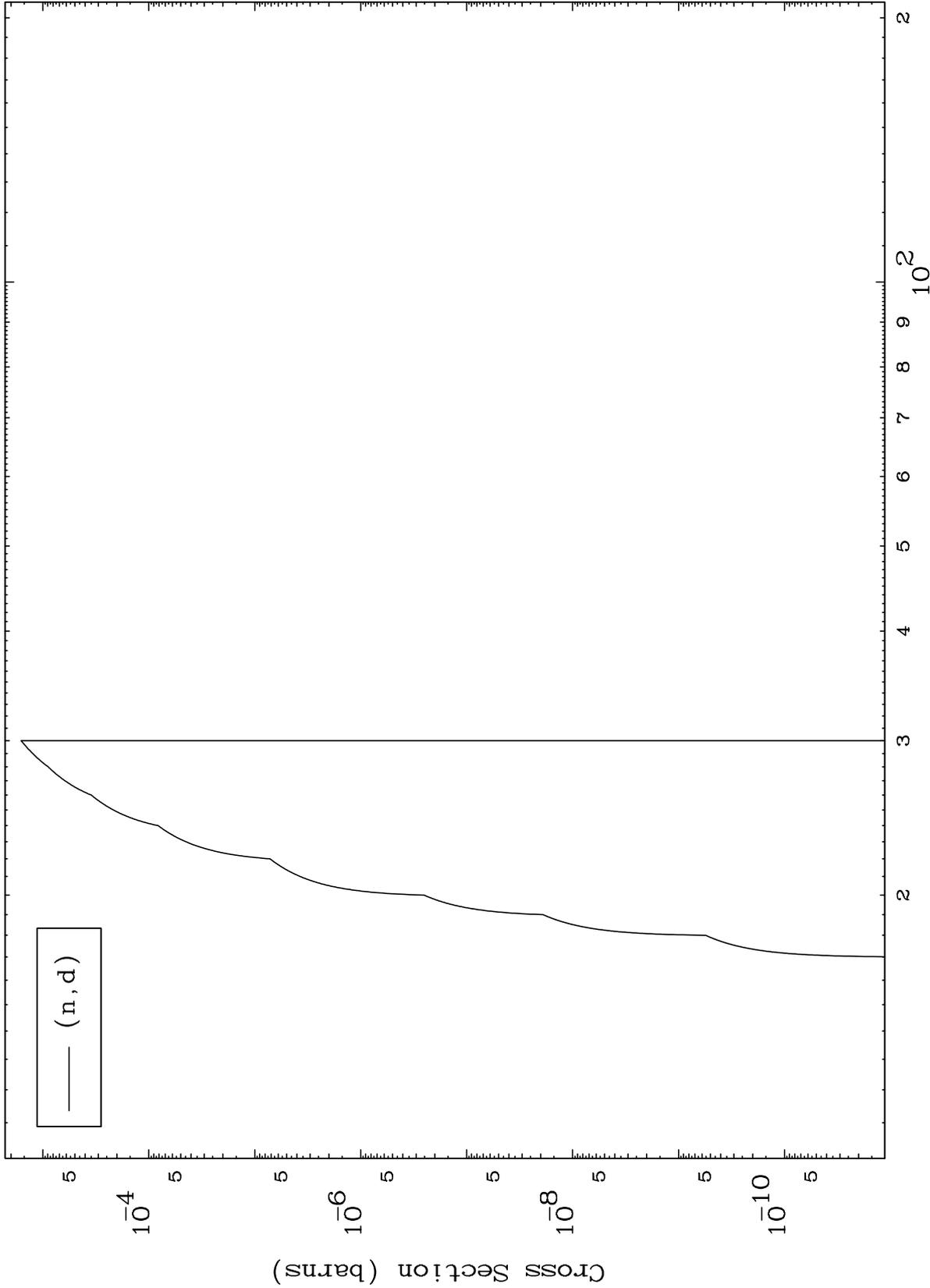
77-Ir-196

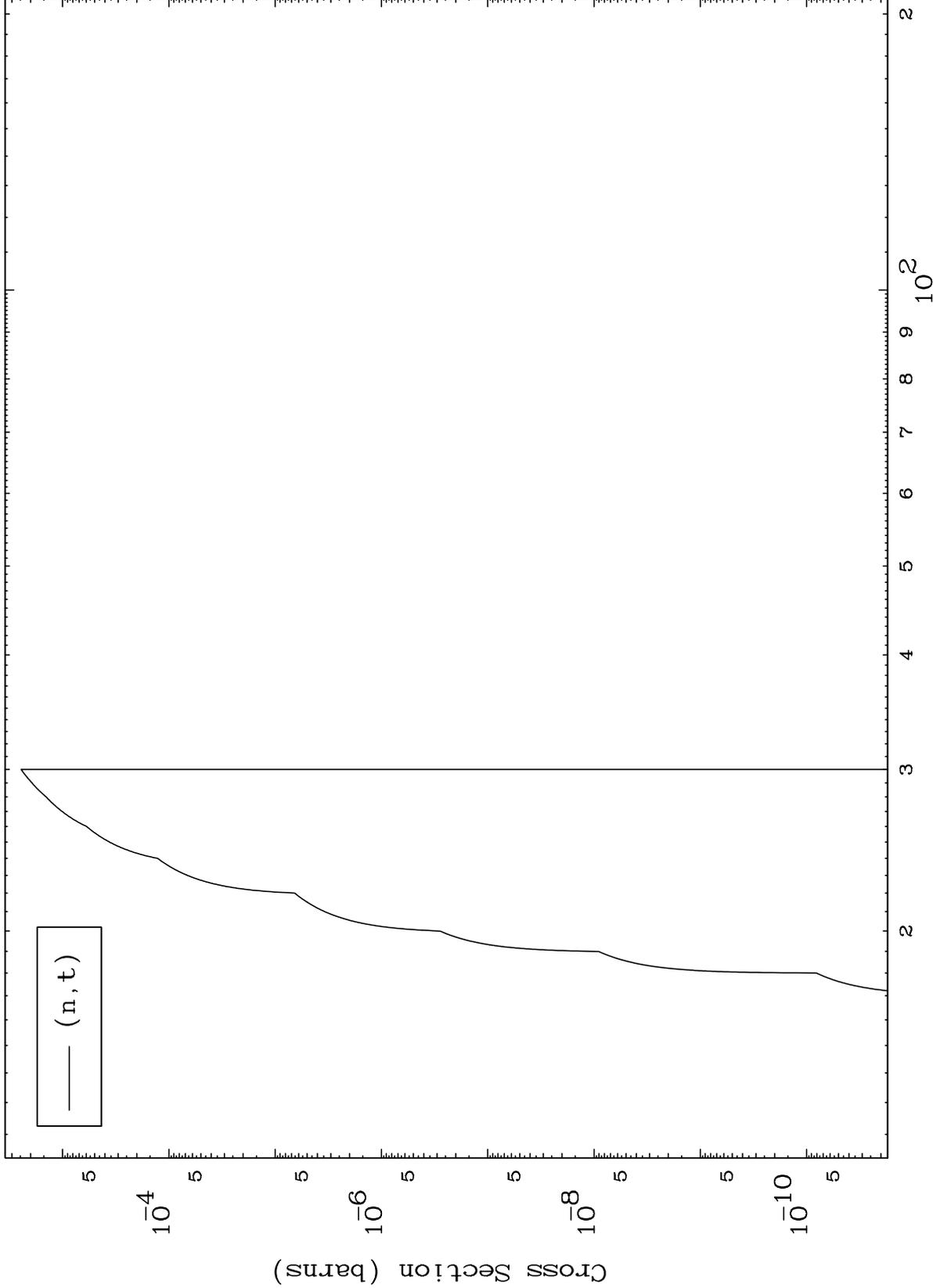


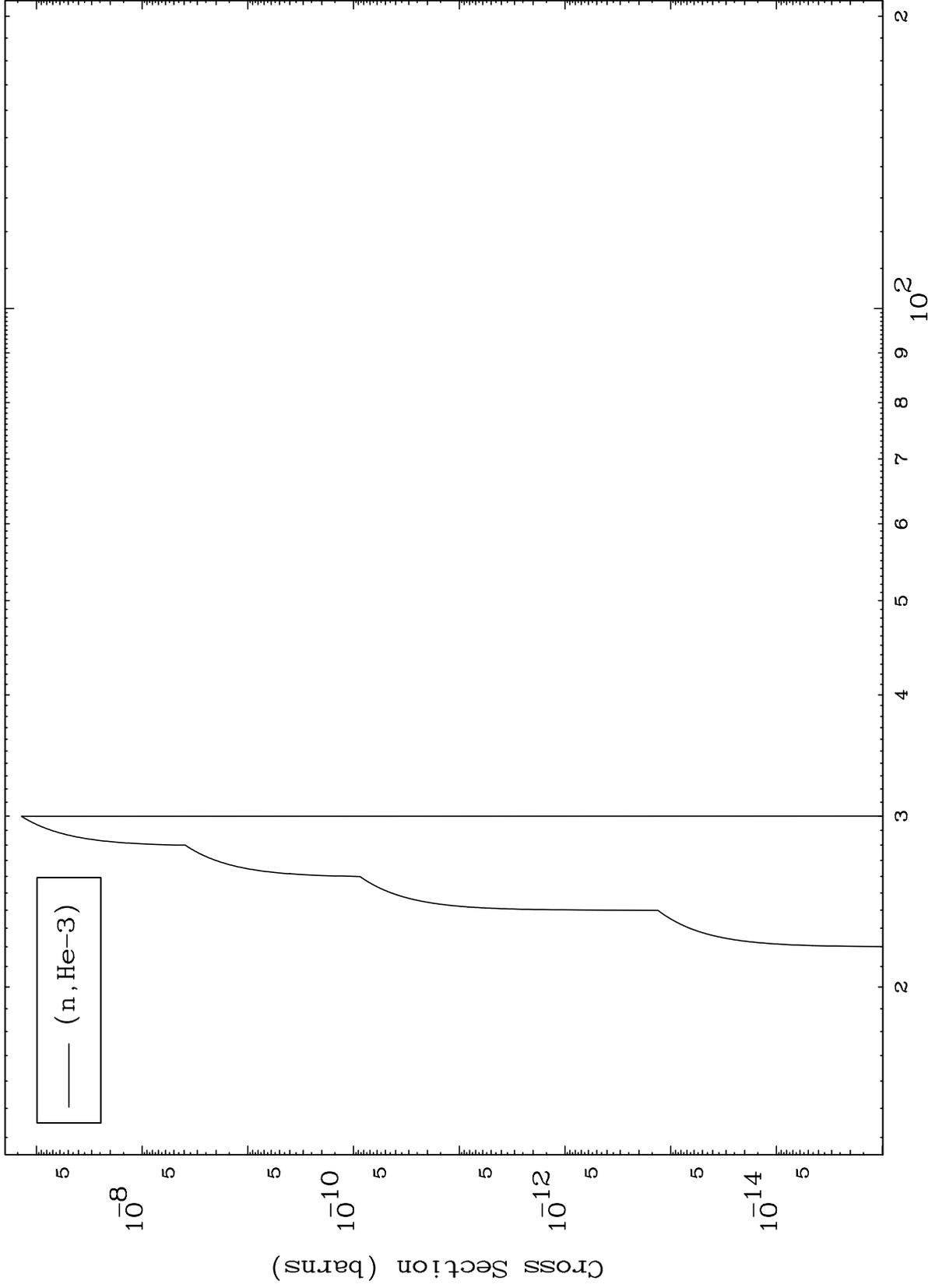
5

Incident Energy (MeV)

77-Ir-196





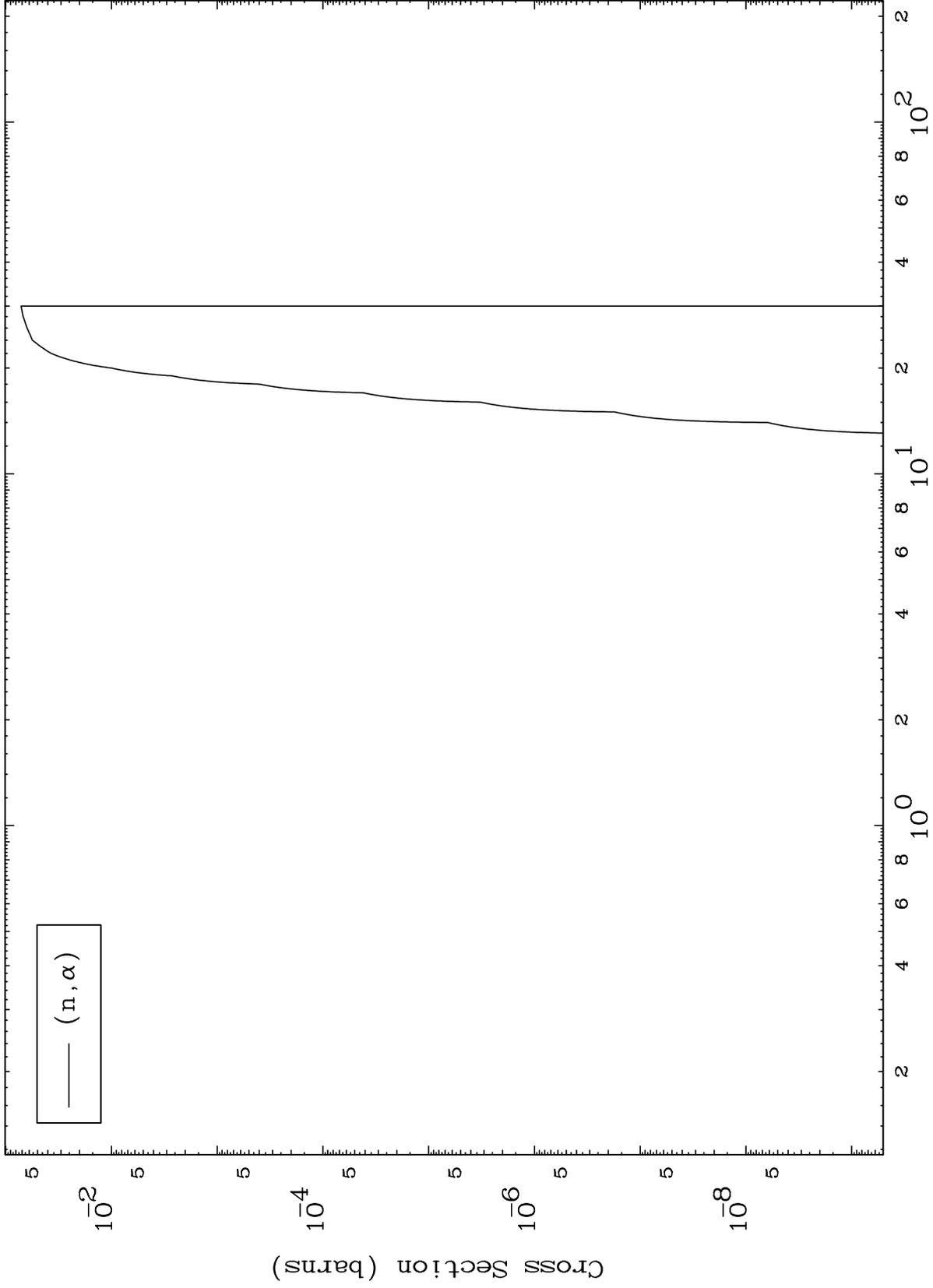


MAT 7740

(α, α) Levels

77-Ir-196

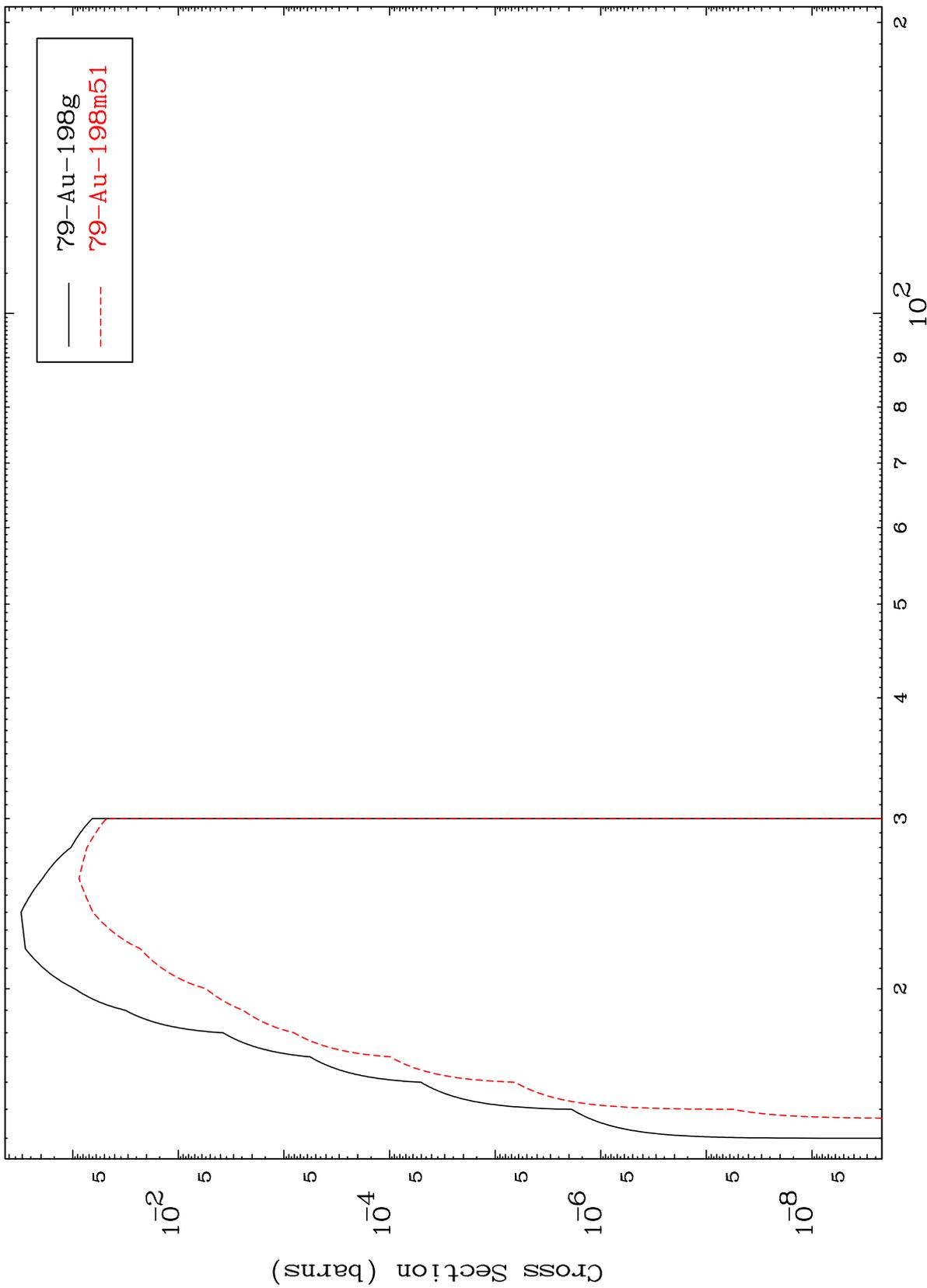
0 Kelvin Cross Sections



MAT 7740

77-Ir-196

(n,2n)
Radionuclide Production Cross Section



10

Incident Energy (MeV)

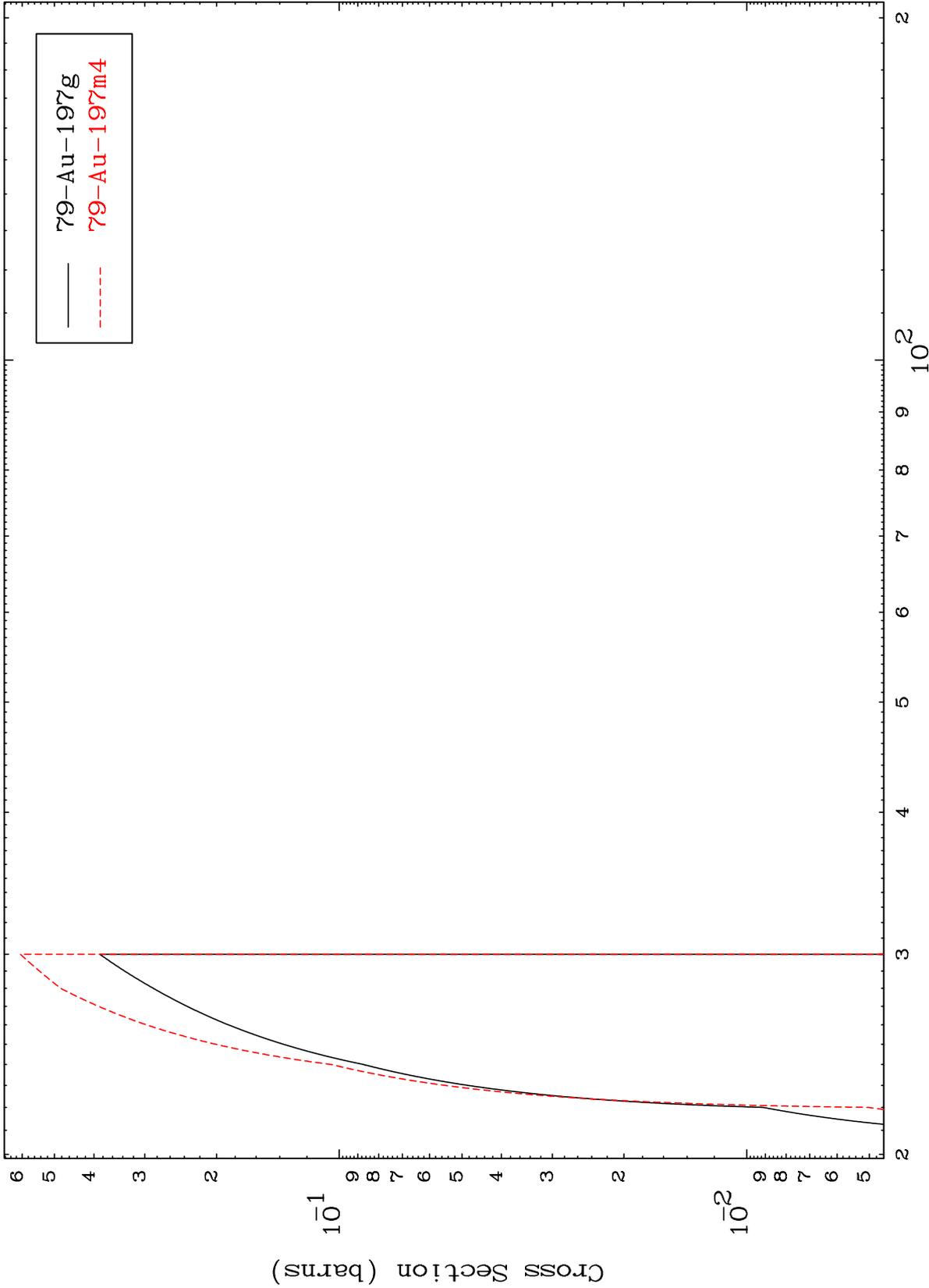
77-Ir-196

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(n,3n)

77-Ir-196

Radionuclide Production Cross Section



11

Incident Energy (MeV)

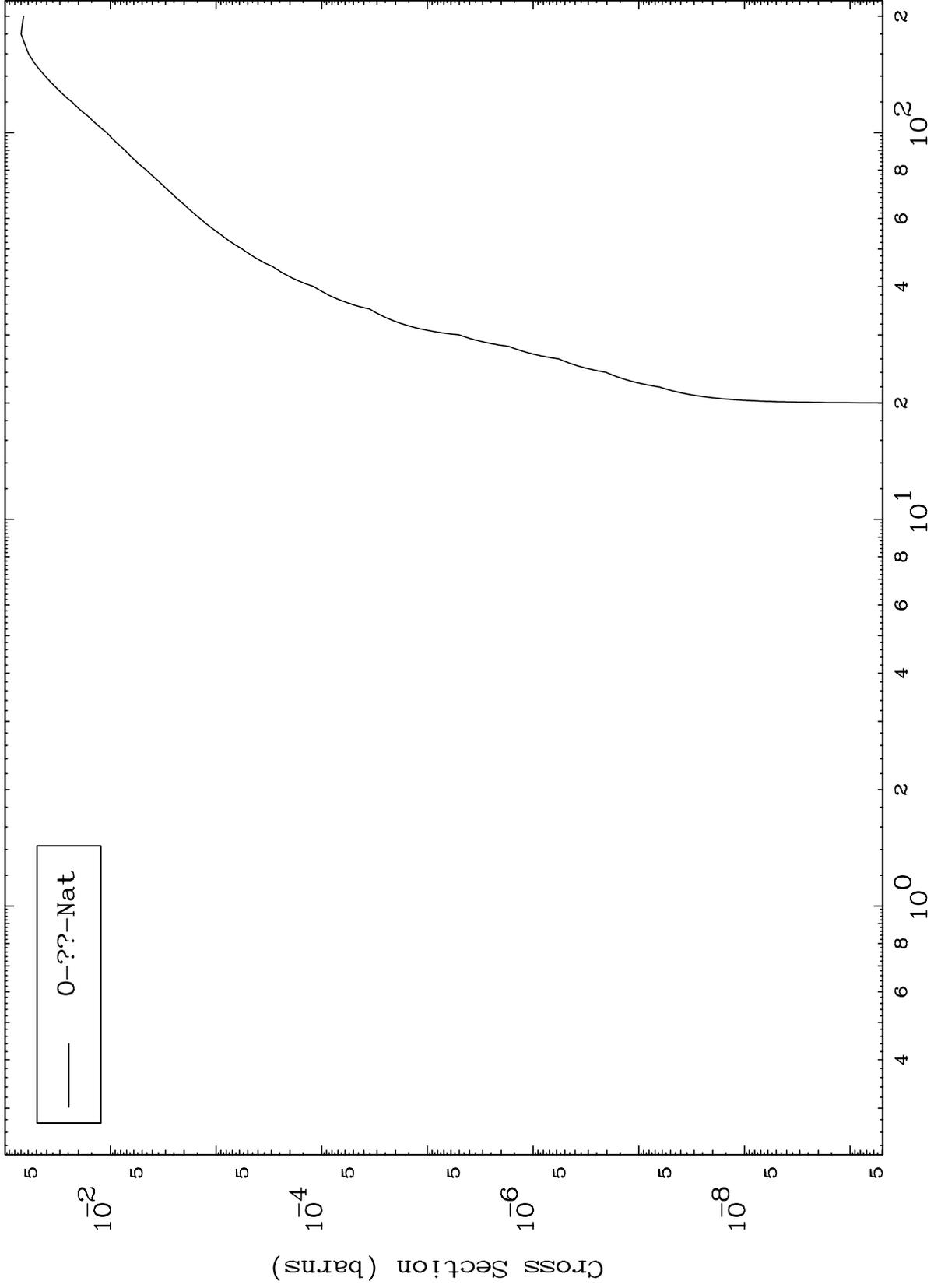
77-Ir-196

MAT 7740

Fission

⁷⁷Ir-196

Radionuclide Production Cross Section



12

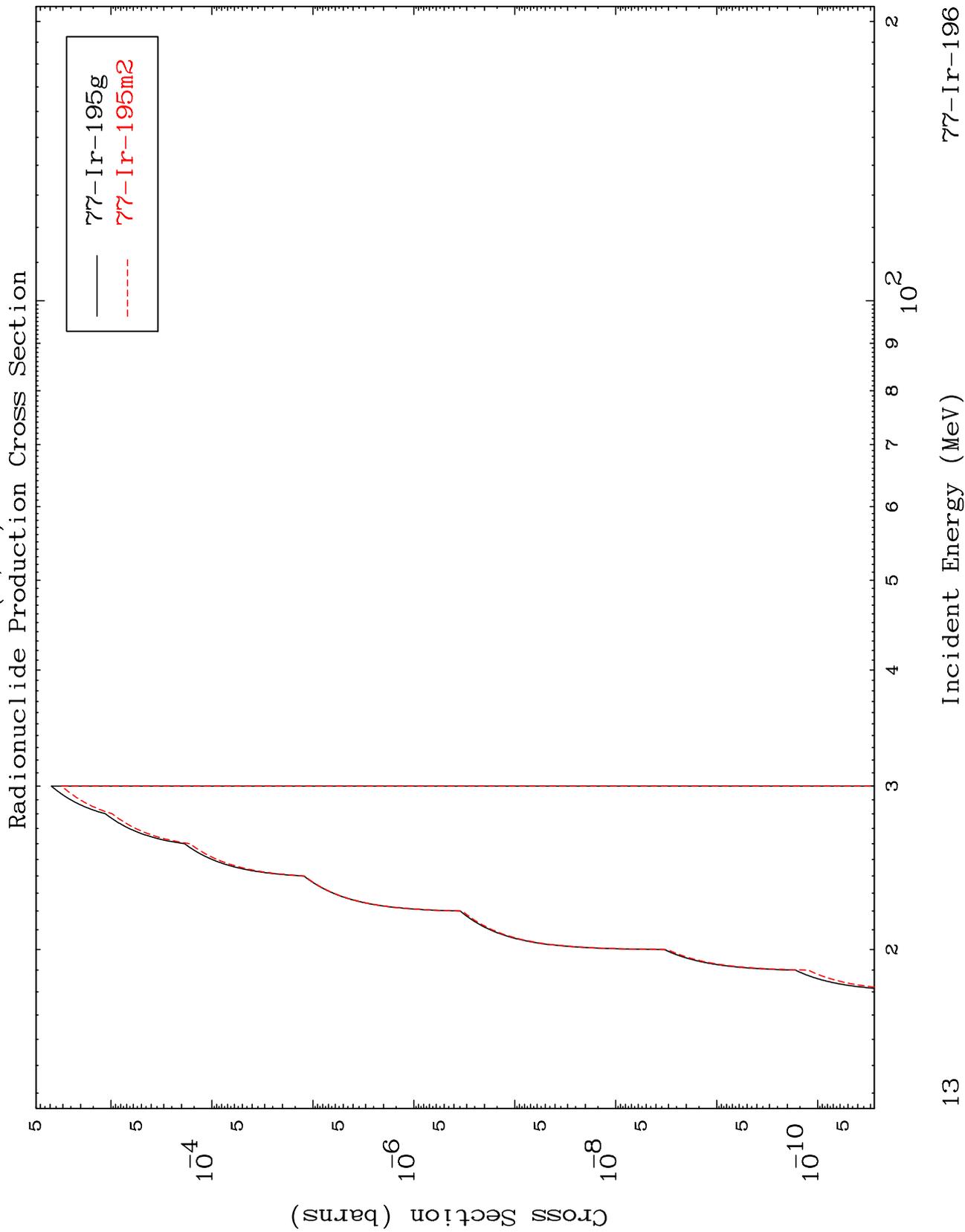
Incident Energy (MeV)

⁷⁷Ir-196

MAT 7740

$(n, n') \alpha$

77-Ir-196



13

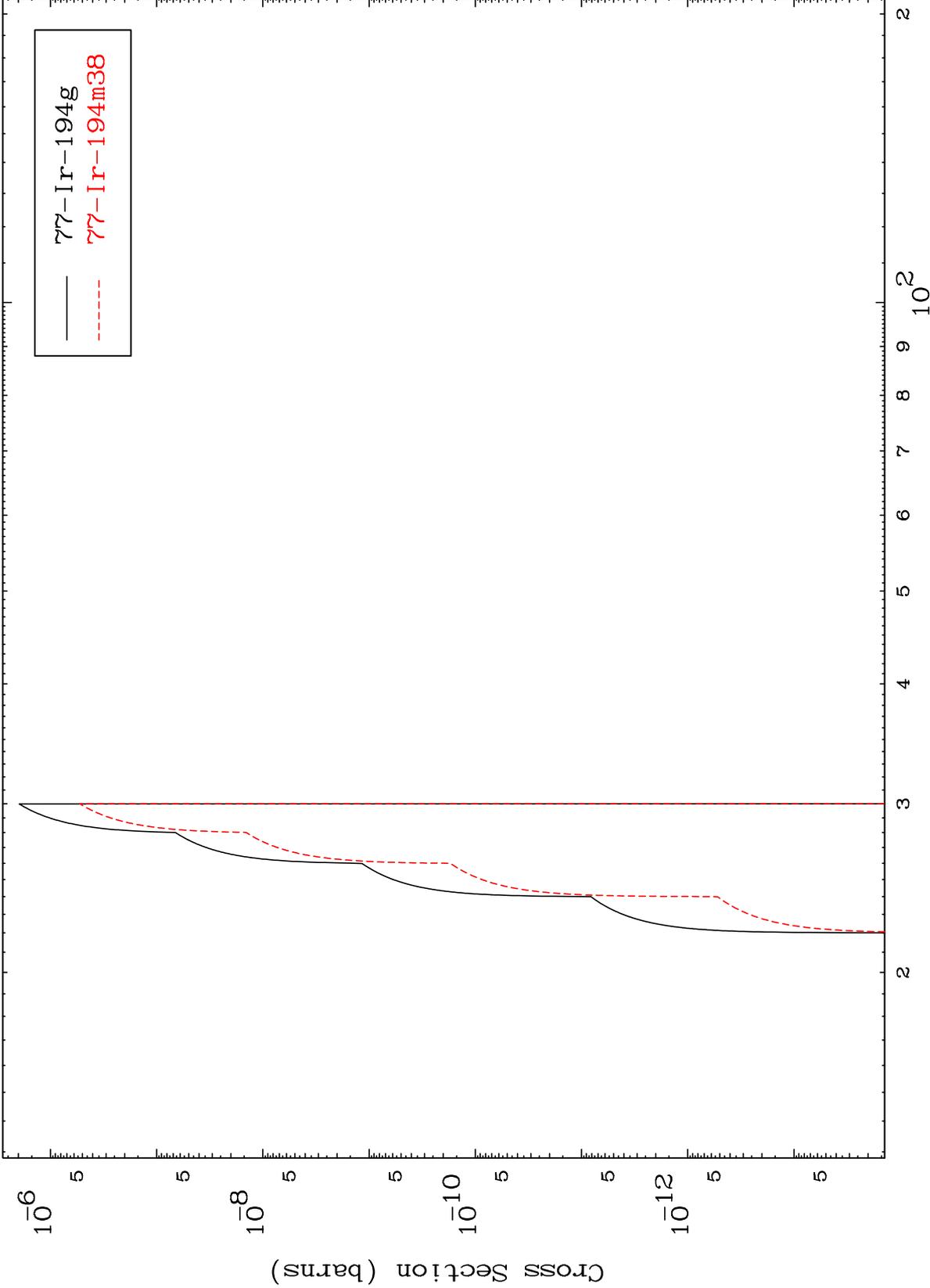
77-Ir-196

MAT 7740

(n,2n) α

77-Ir-196

Radionuclide Production Cross Section



14

Incident Energy (MeV)

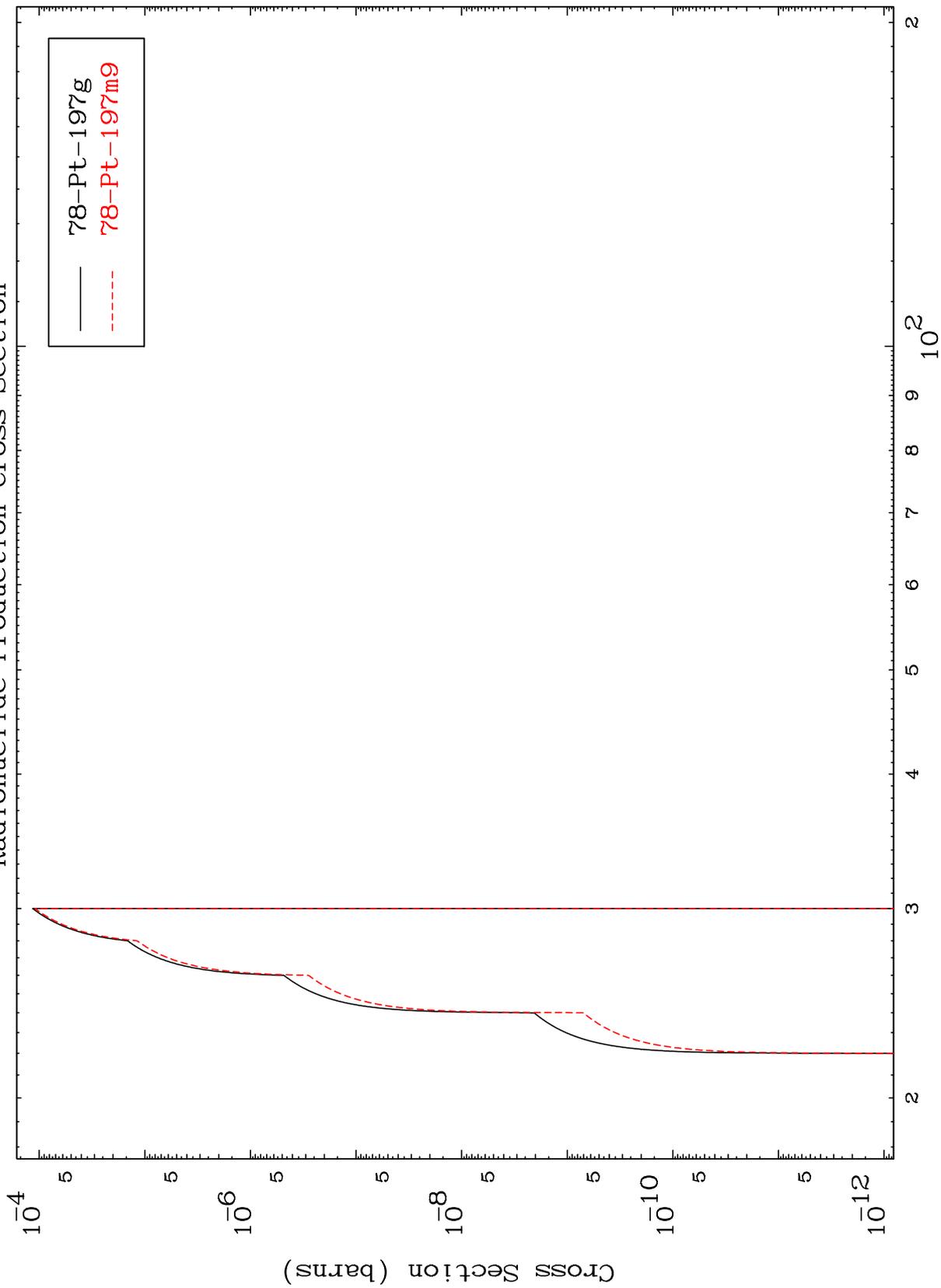
77-Ir-196

MAT 7740

(n,n') d

77-Ir-196

Radionuclide Production Cross Section



15

Incident Energy (MeV)

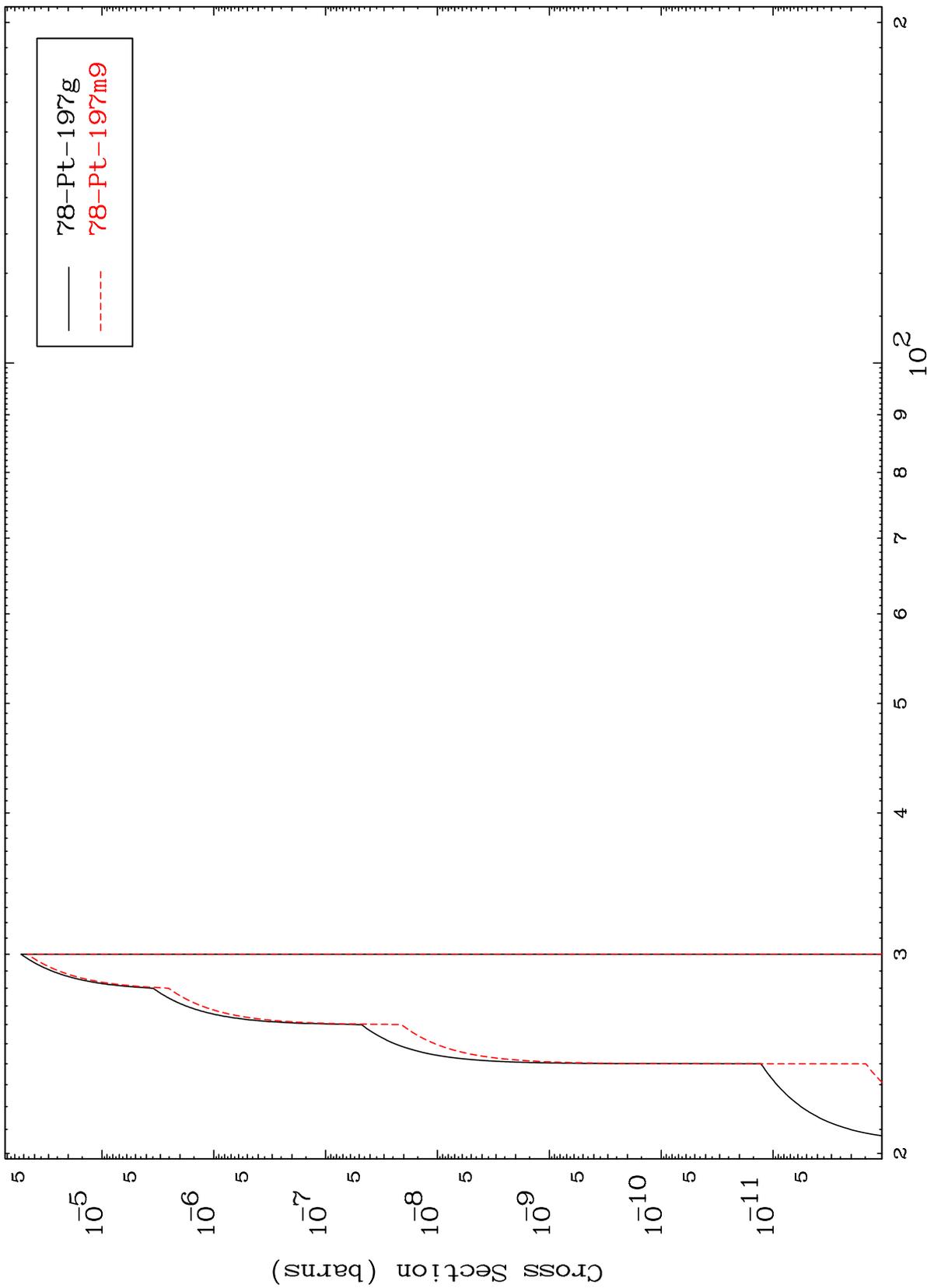
77-Ir-196

MAT 7740

(n,2n) p

77-Ir-196

Radionuclide Production Cross Section



16

Incident Energy (MeV)

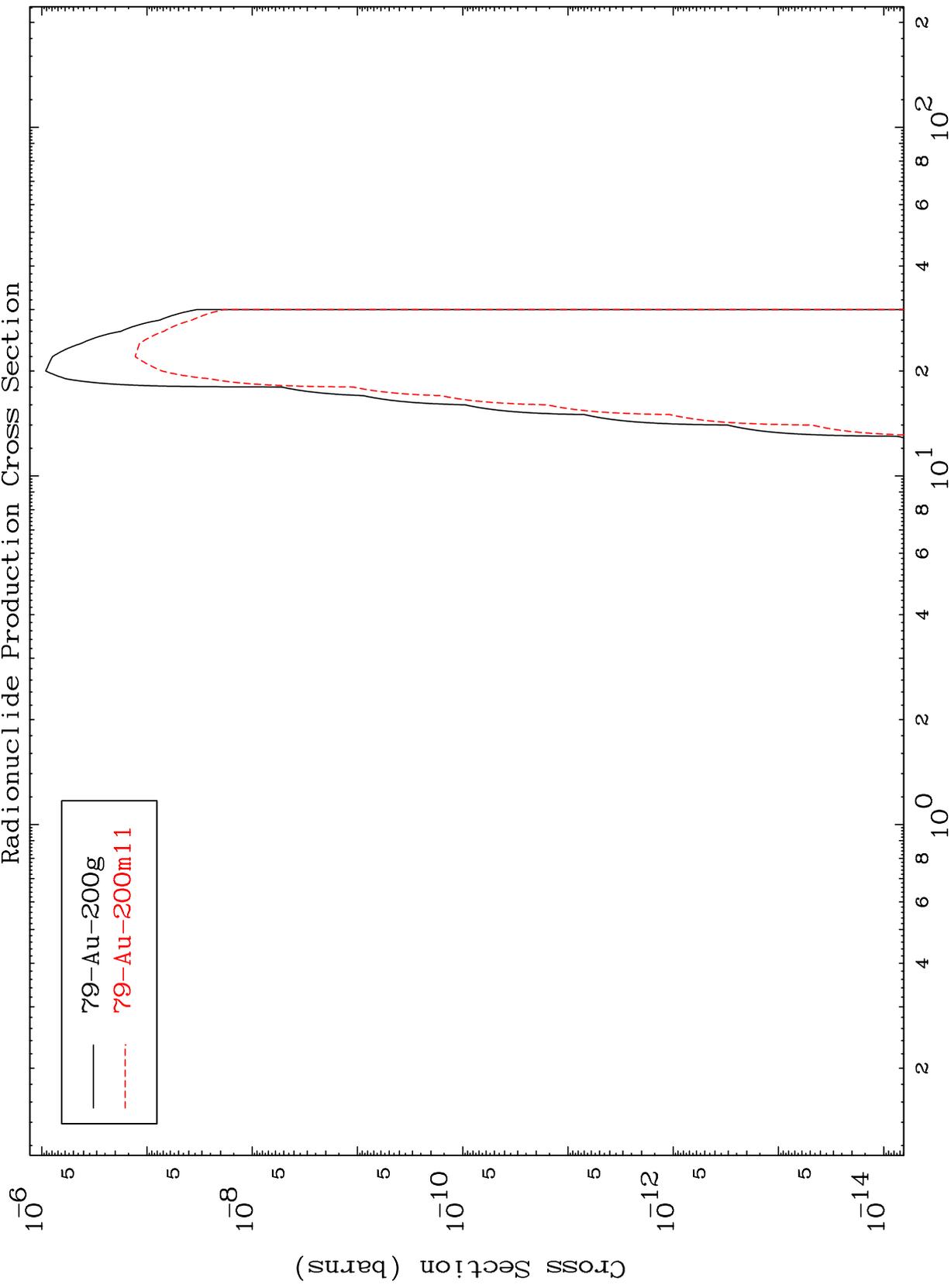
77-Ir-196

MAT 7740

(n, γ)

⁷⁷Ir-196

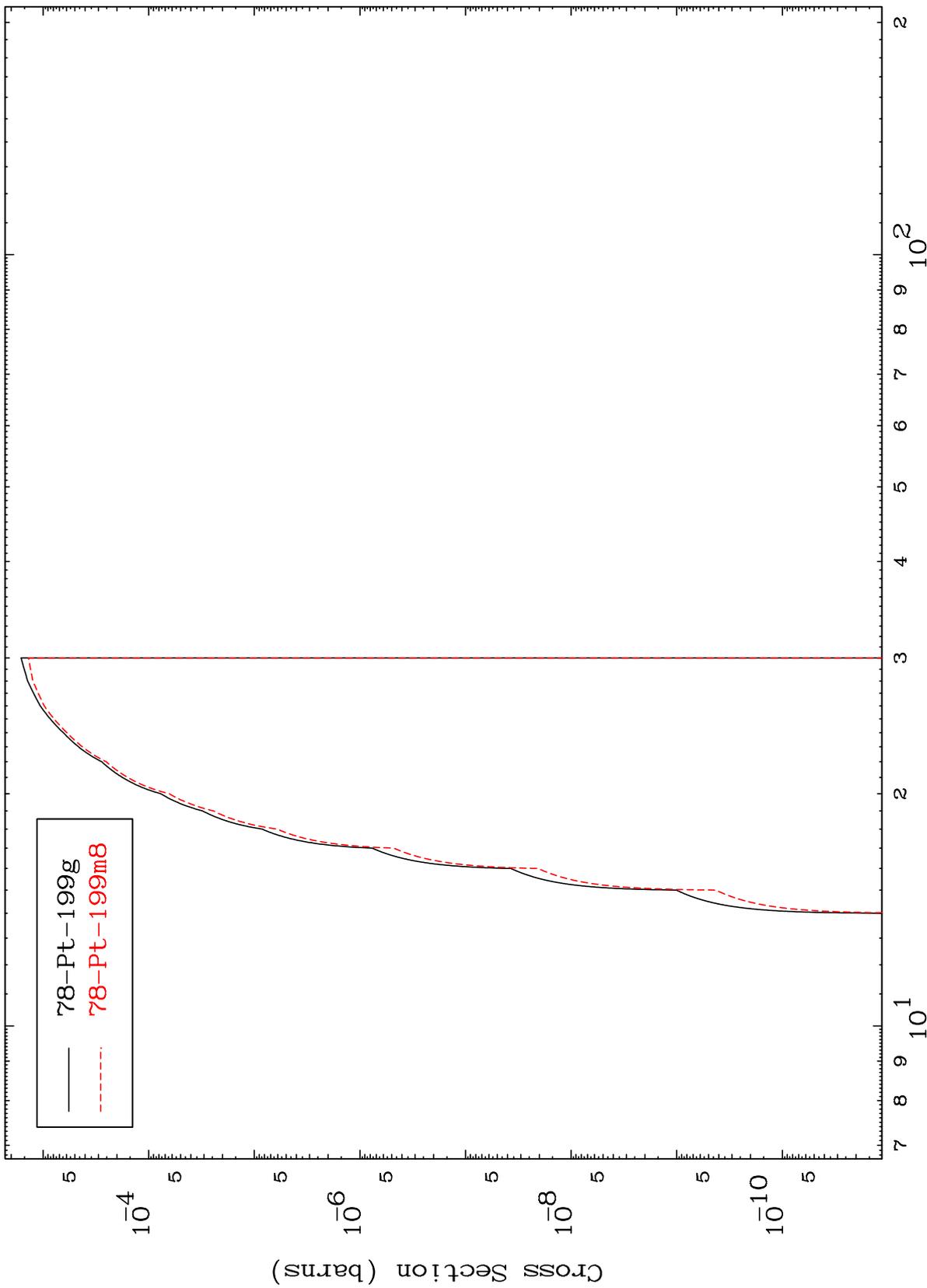
Radionuclide Production Cross Section



MAT 7740

77-Ir-196

(n,p)
Radionuclide Production Cross Section

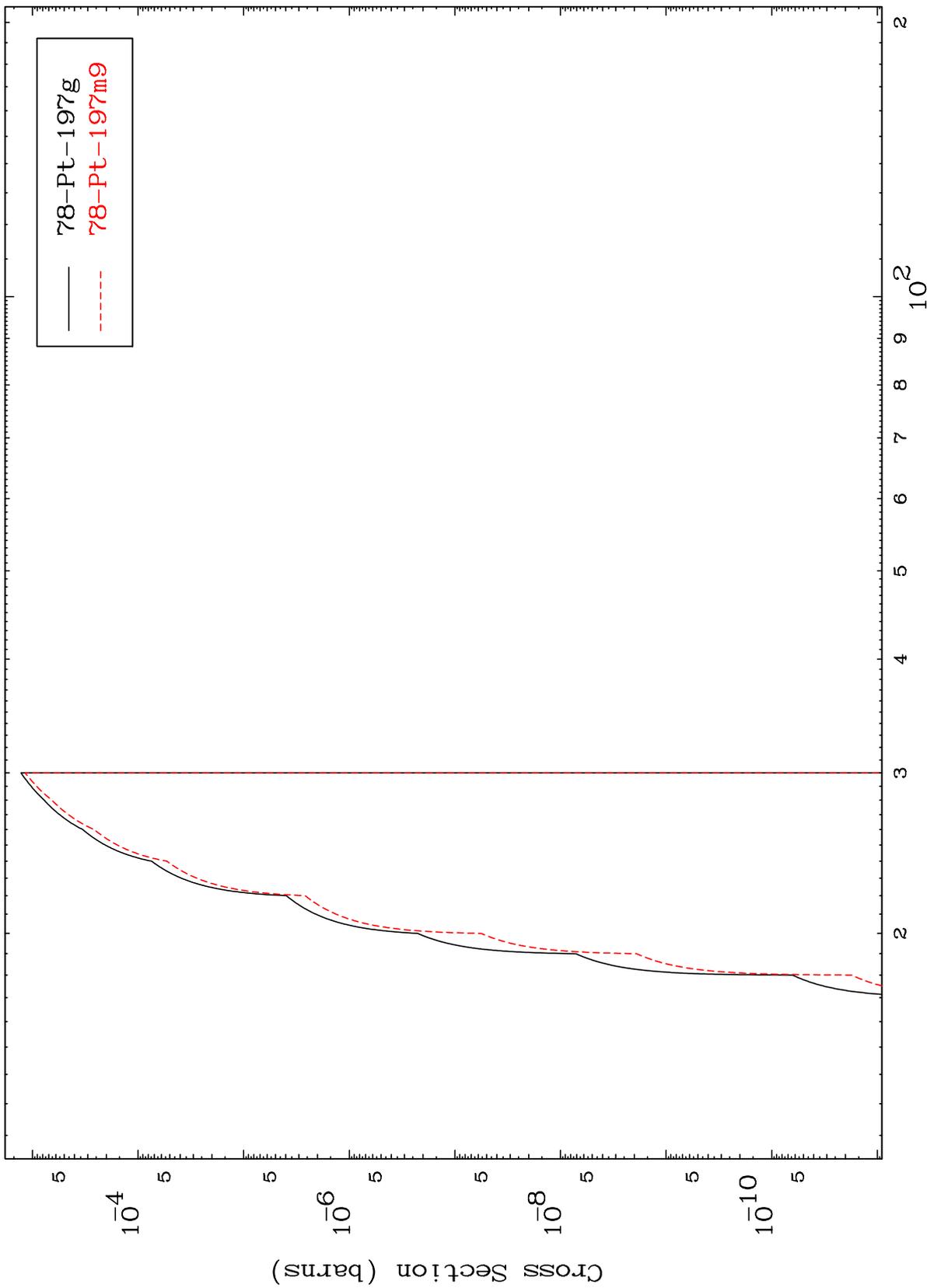


18

Incident Energy (MeV)

77-Ir-196

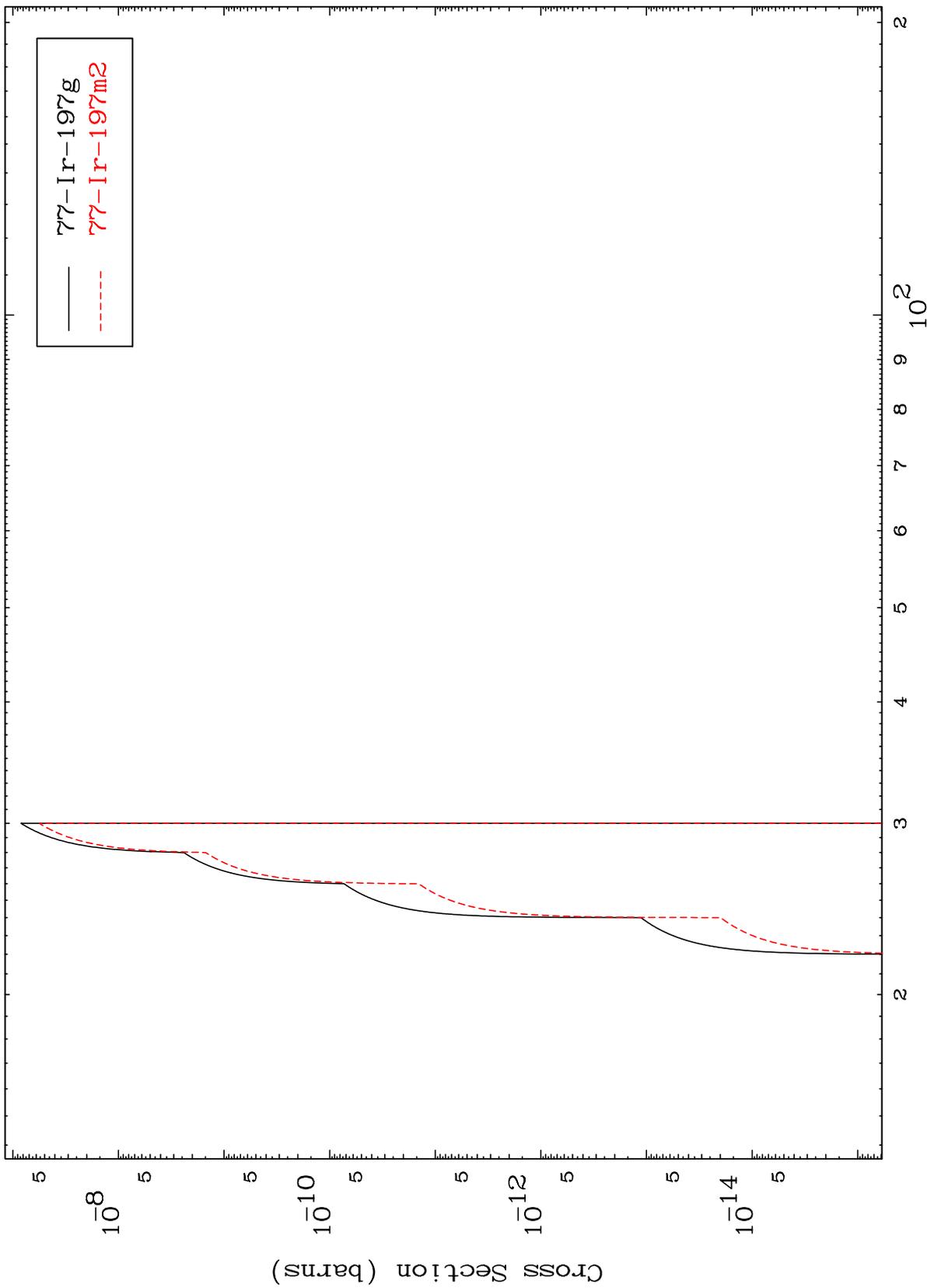
(n,t)
Radionuclide Production Cross Section



MAT 7740

77-Ir-196

(n,He-3)
Radionuclide Production Cross Section



20

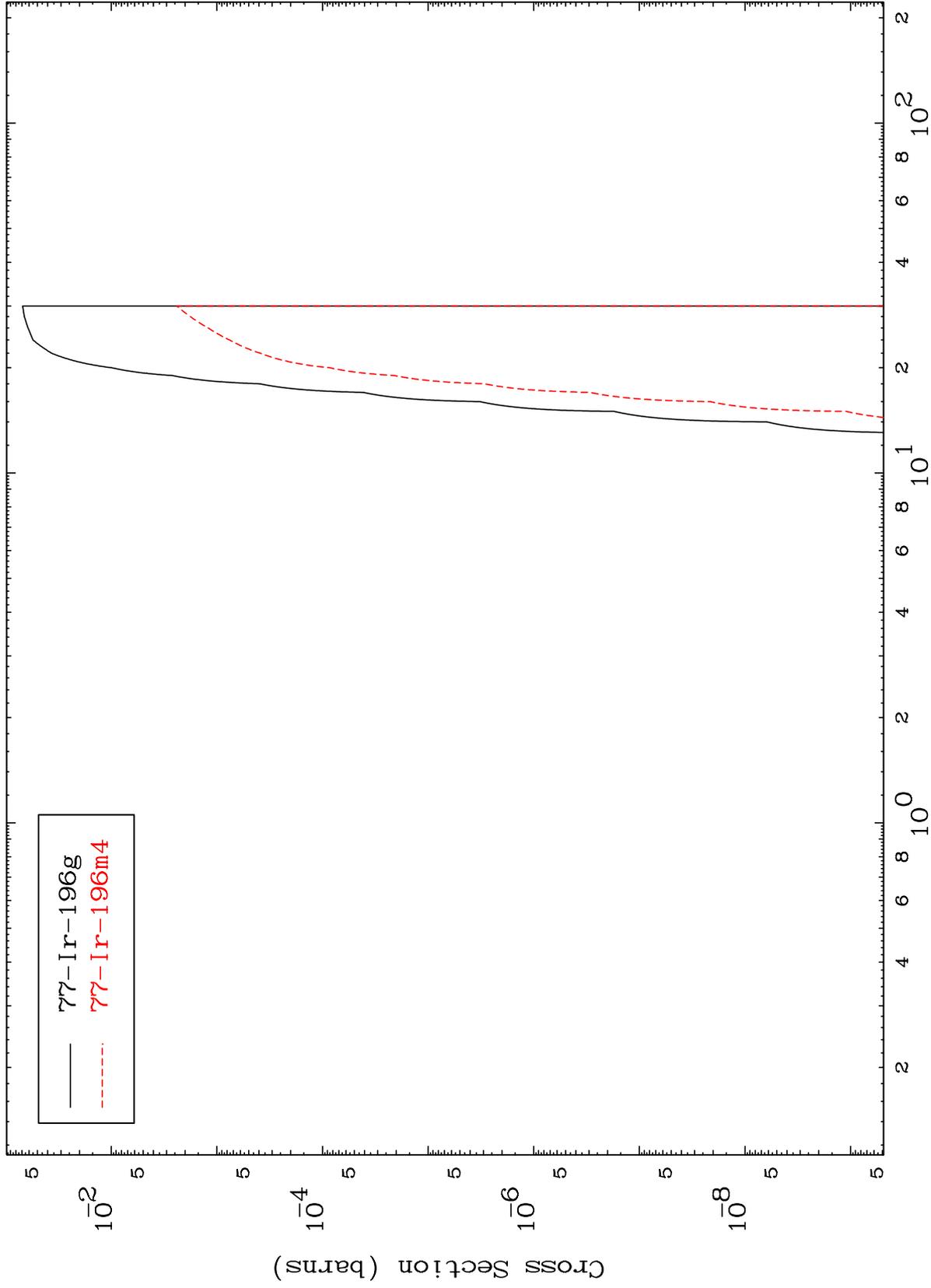
Incident Energy (MeV)

77-Ir-196

MAT 7740

⁷⁷Ir-196

Radionuclide Production Cross Section



Incident Energy (MeV)

⁷⁷Ir-196