

Program EVALPLOT  
(Version 2021-1)

by

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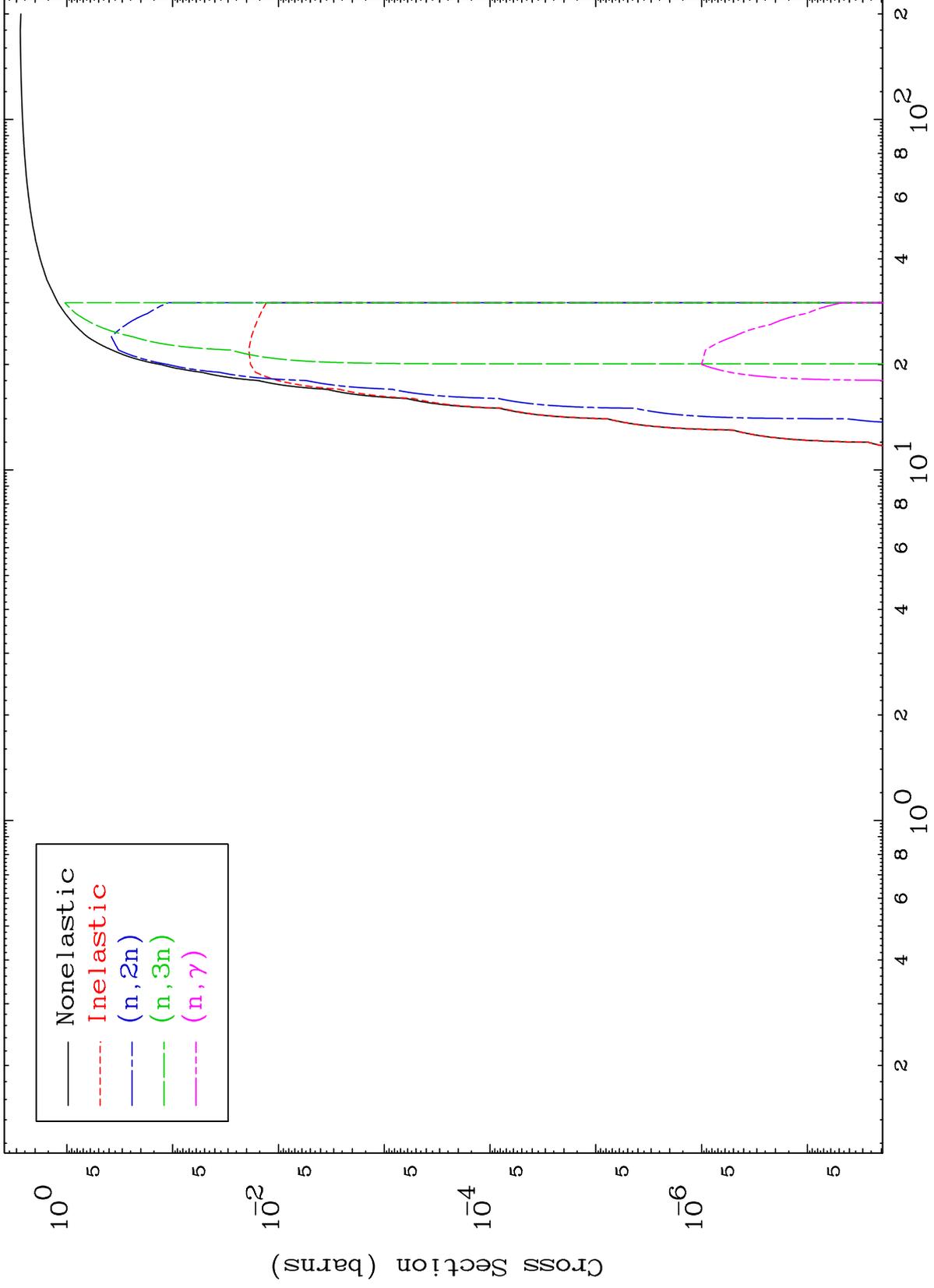
Press Mouse Button to Start

MAT 7741

0 Kelvin

$\alpha$  Major Cross Sections

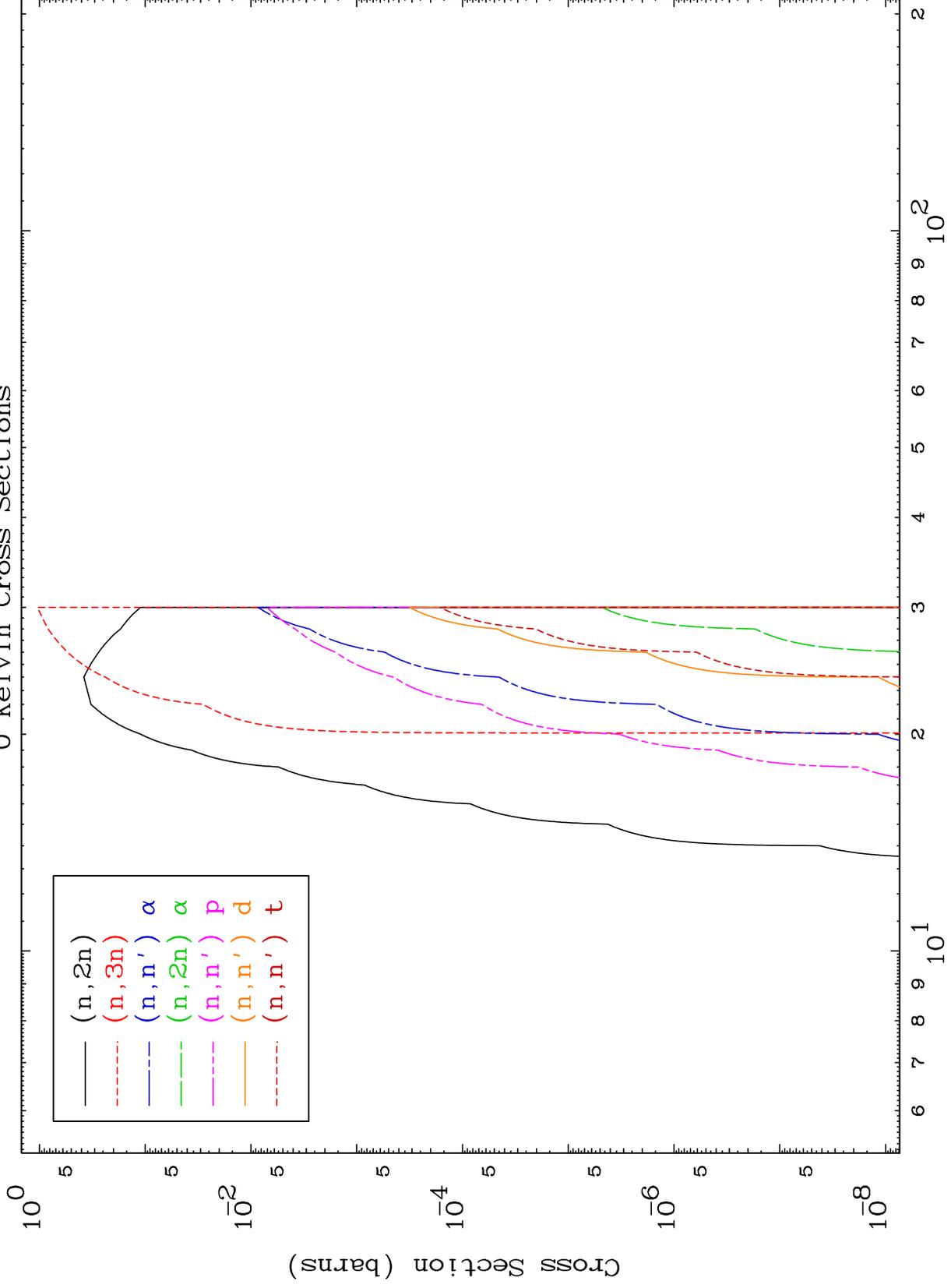
77-Ir-196m



MAT 7741

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

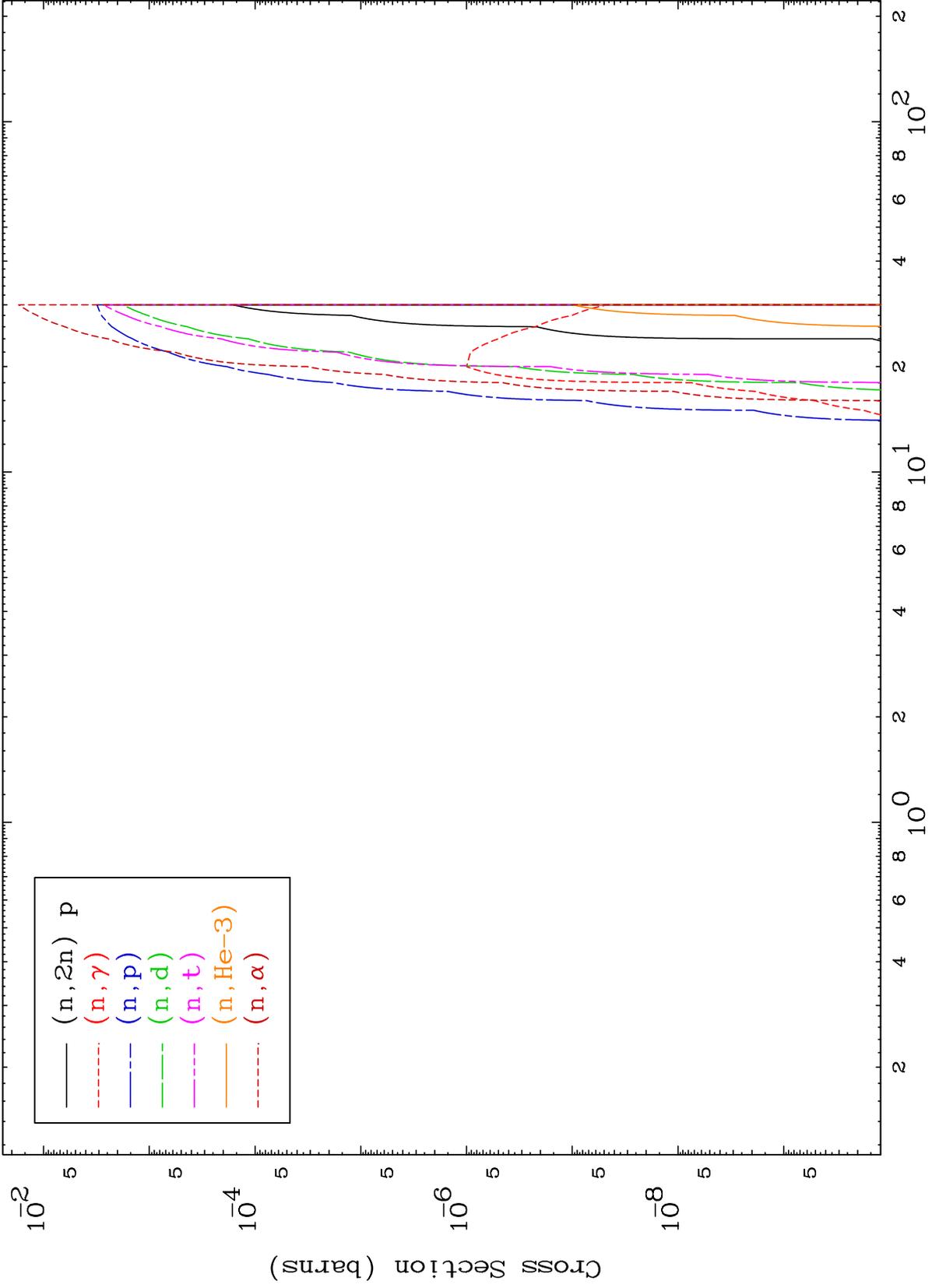
77-Ir-196m



MAT 7741

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

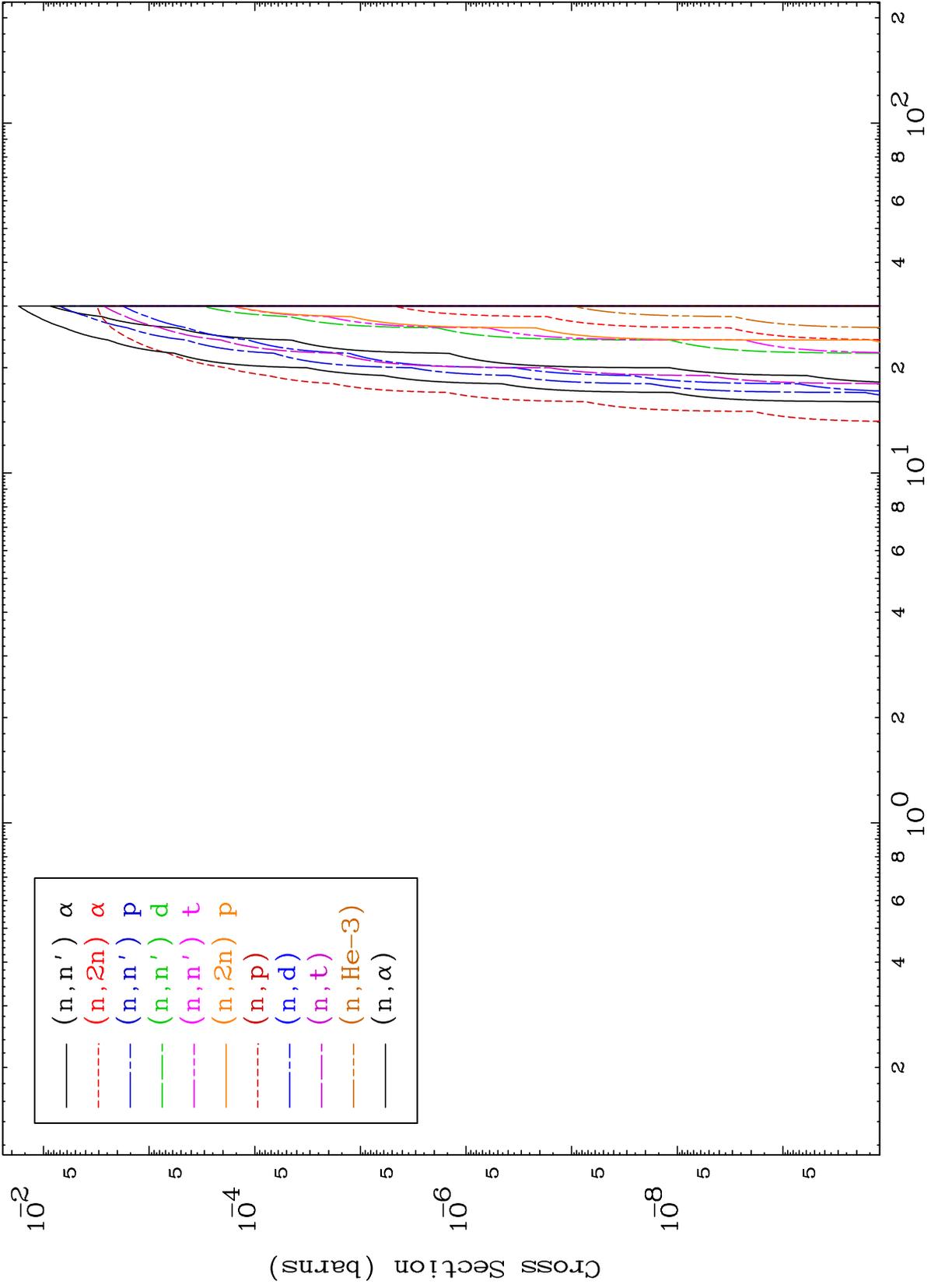
77-Ir-196m



MAT 7741

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

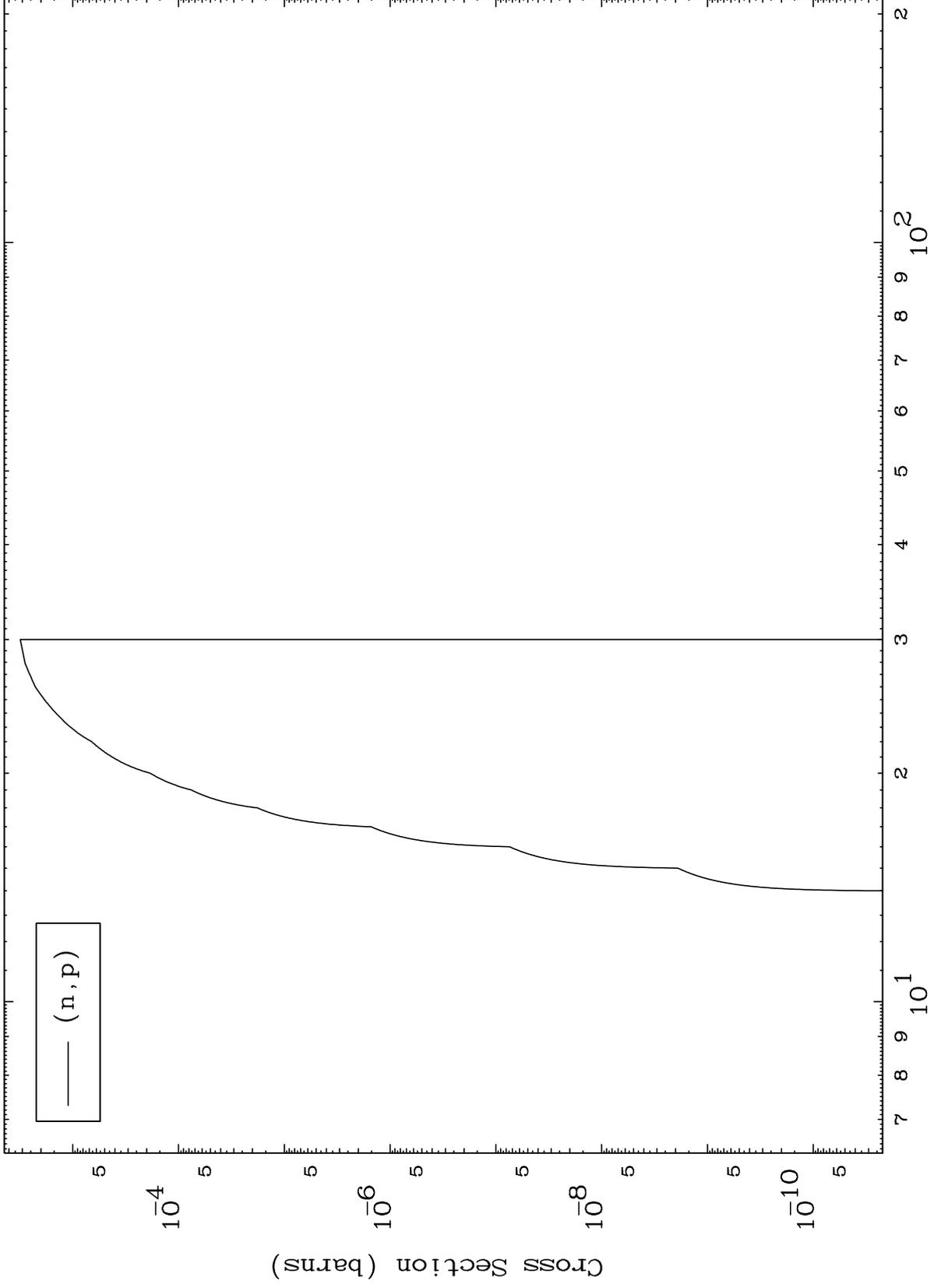
77-Ir-196m



MAT 7741

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196m



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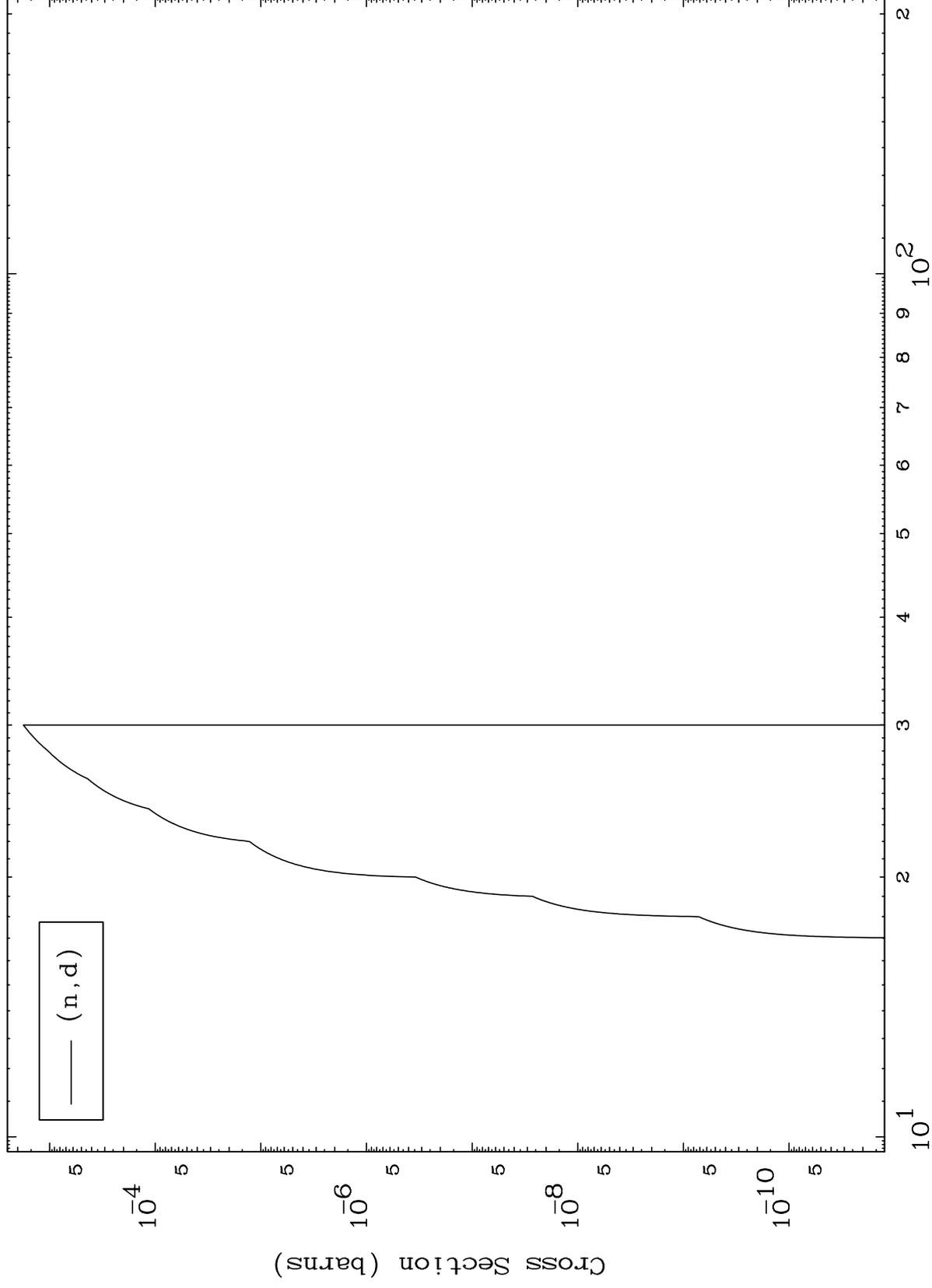
Incident Energy (MeV)

77-Ir-196m

MAT 7741

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-196m



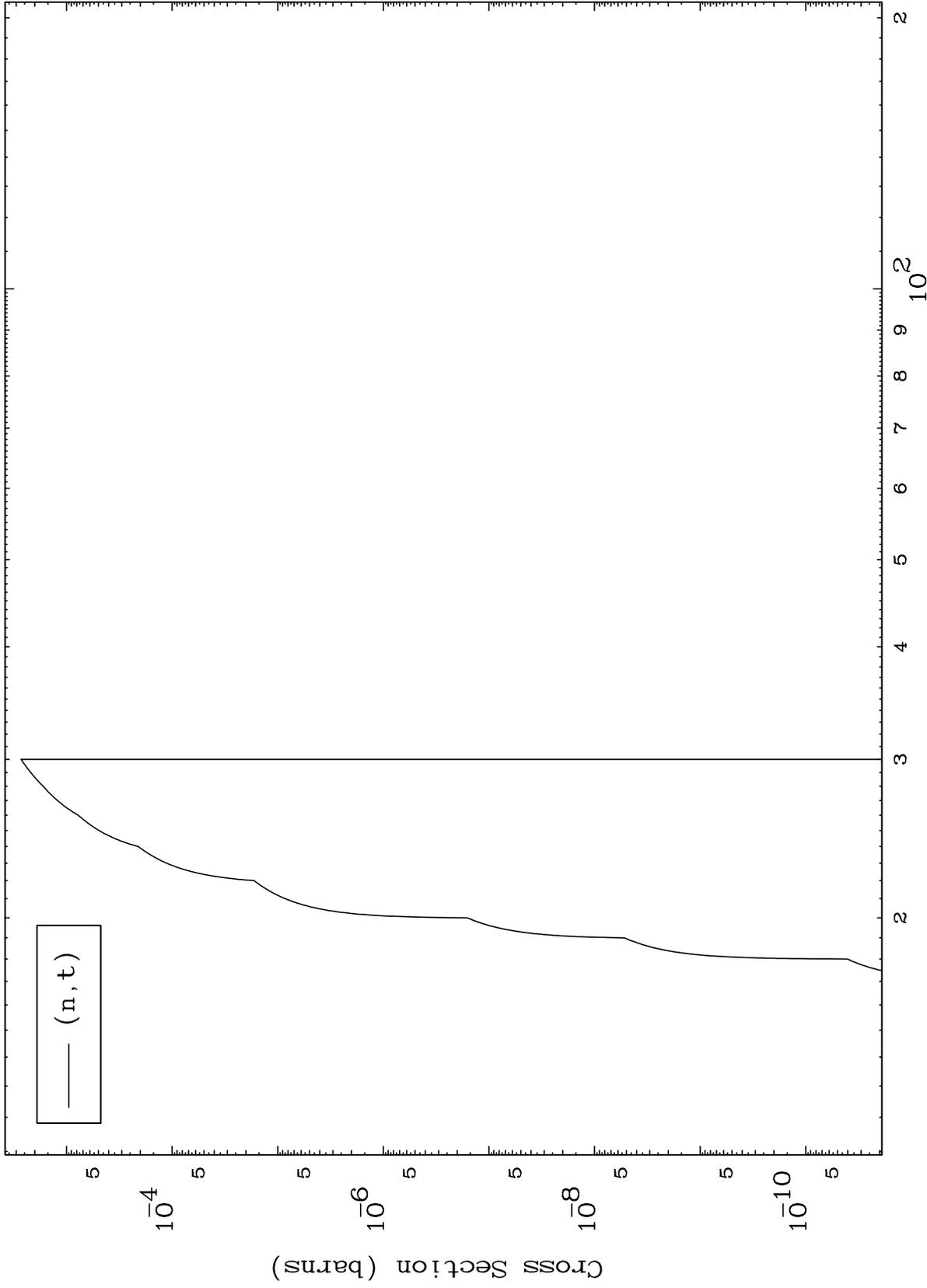
Incident Energy (MeV)

77-Ir-196m

MAT 7741

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

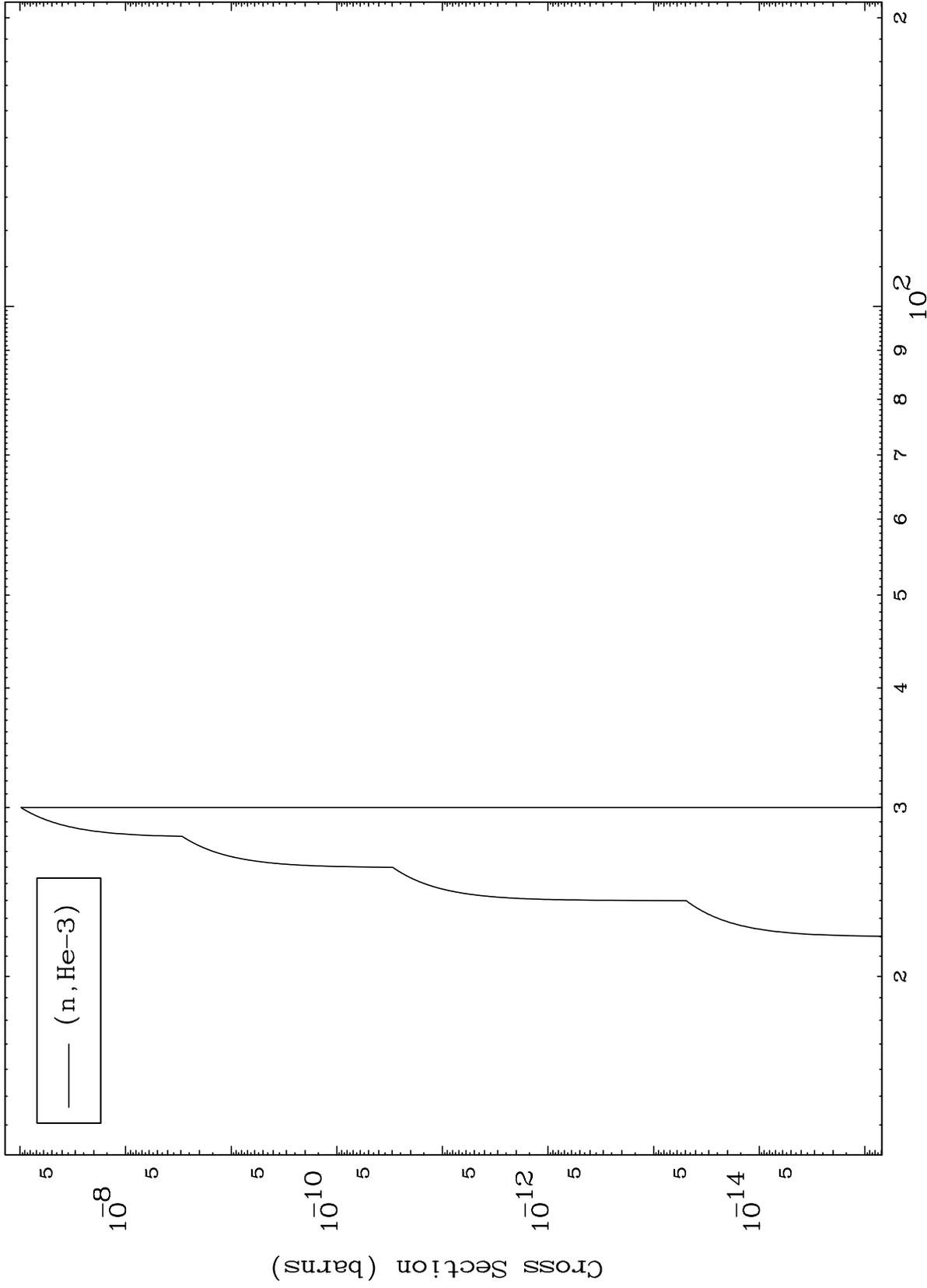
77-Ir-196m



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Incident Energy (MeV)

77-Ir-196m

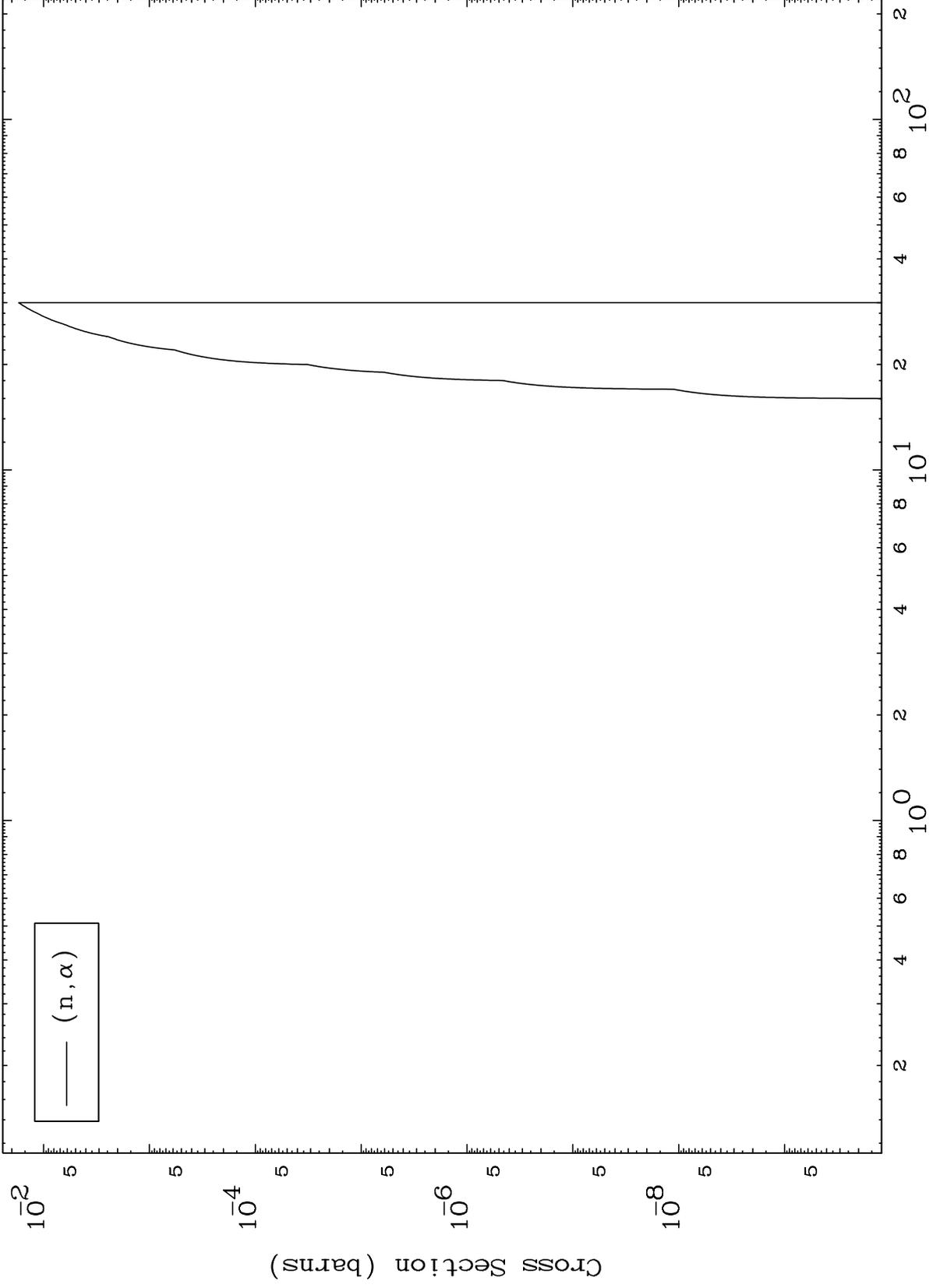


MAT 7741

( $\alpha, \alpha$ ) Levels

77-Ir-196m

0 Kelvin Cross Sections

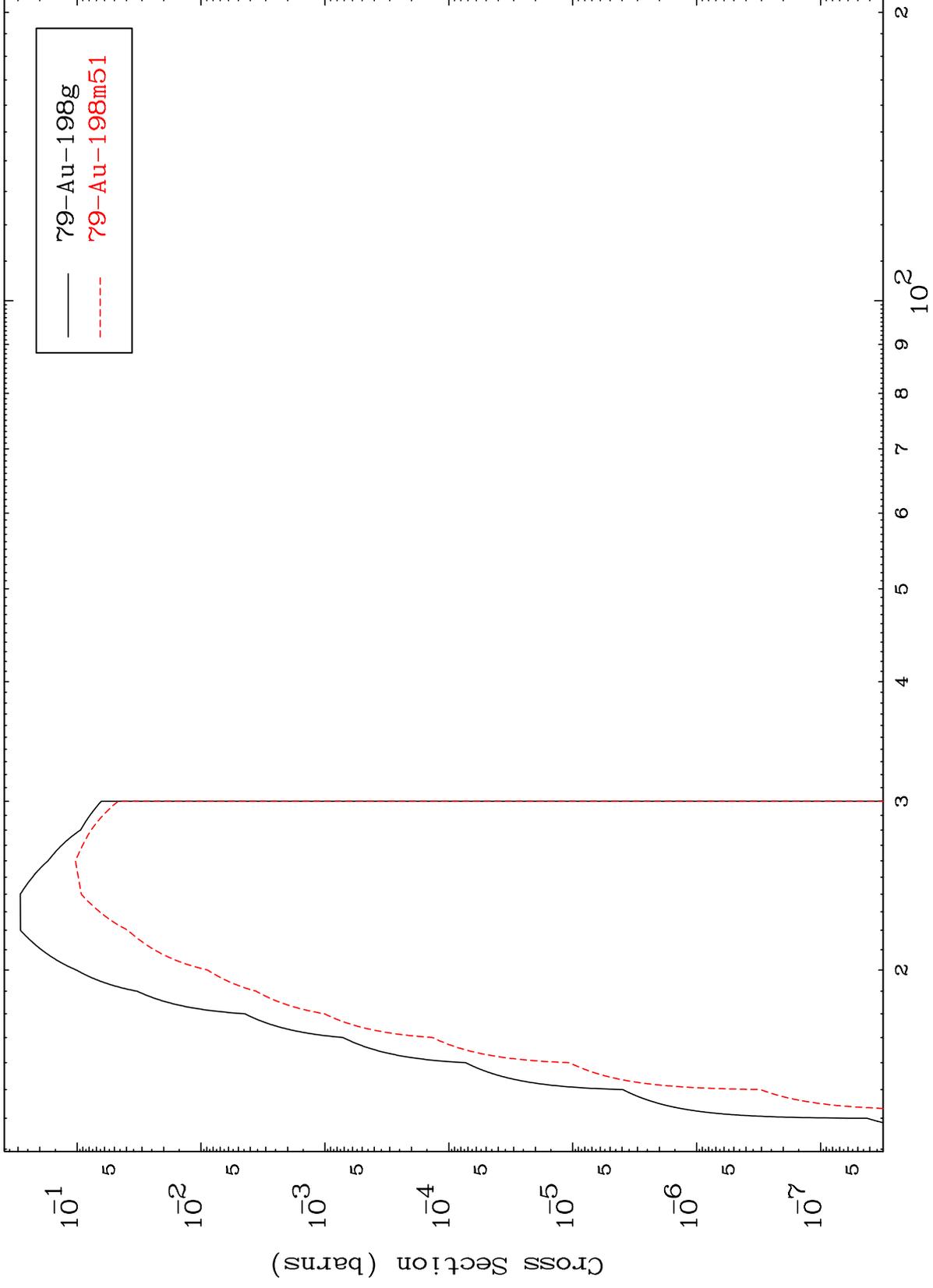


MAT 7741

(n,2n)

<sup>77</sup>Ir-196m

Radionuclide Production Cross Section



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Incident Energy (MeV)

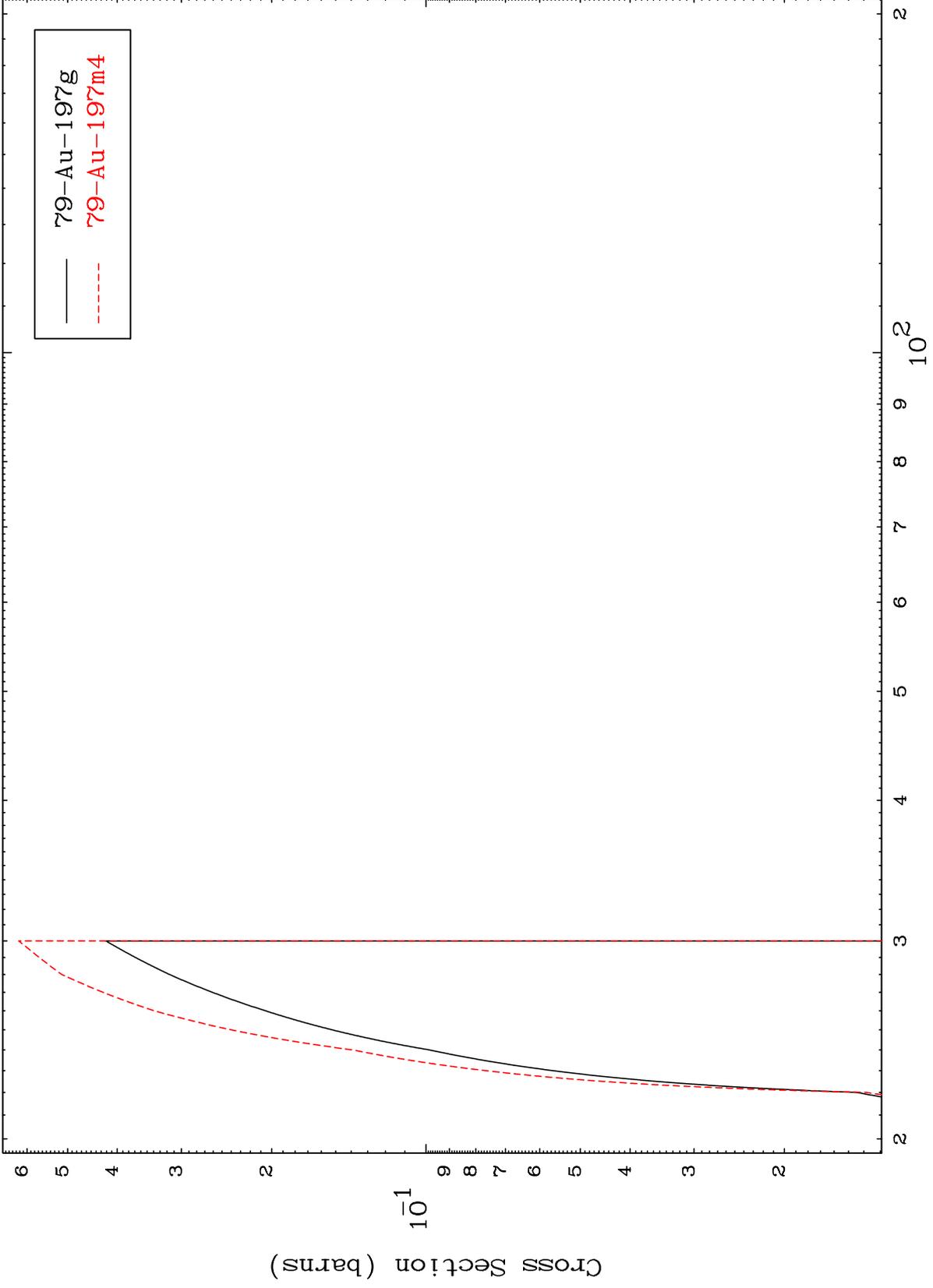
<sup>77</sup>Ir-196m

MAT 7741

(n,3n)

<sup>77</sup>Ir-196m

Radionuclide Production Cross Section



<sup>79</sup>Au-197g  
<sup>79</sup>Au-197m4

11

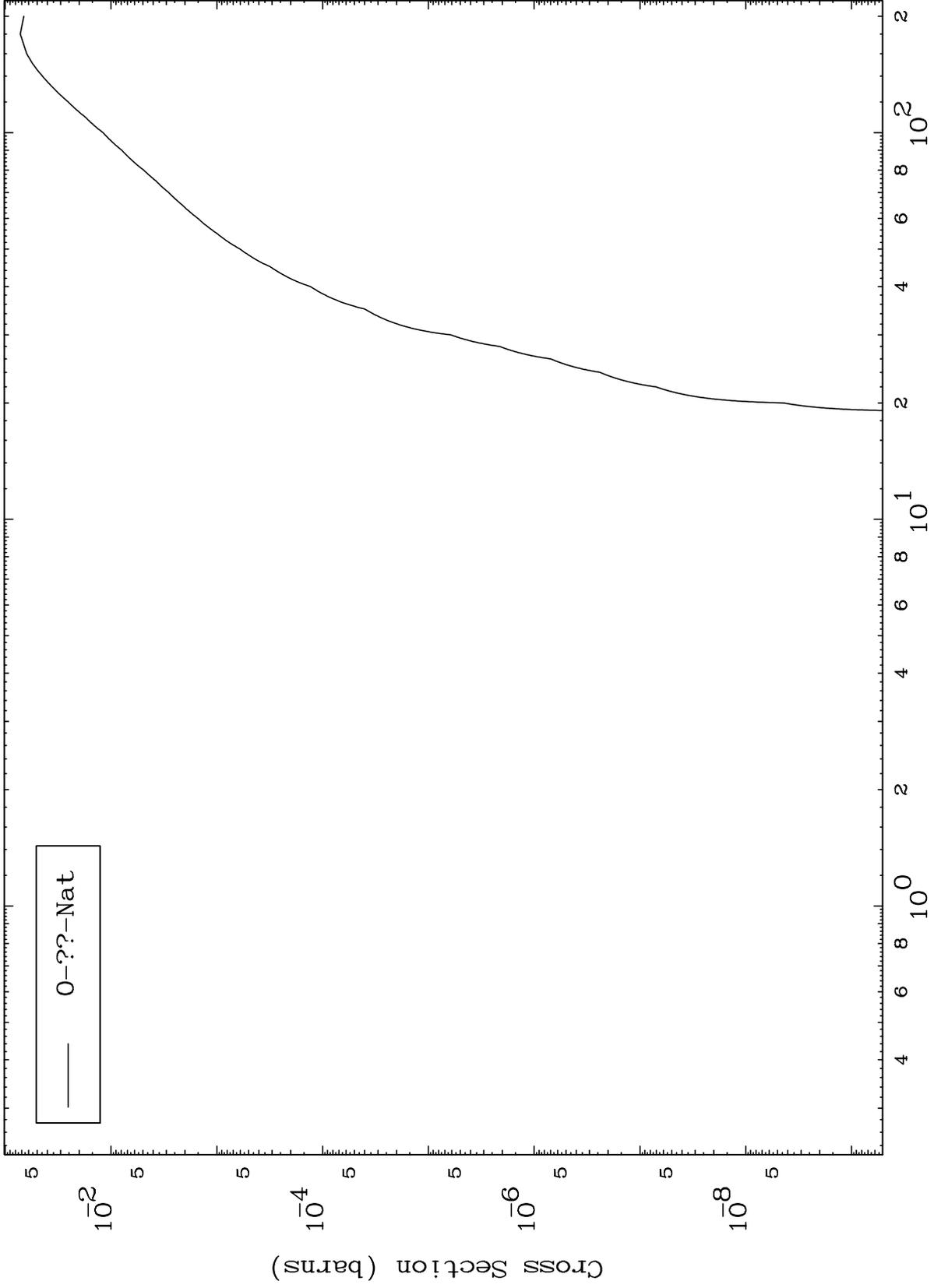
Incident Energy (MeV)

<sup>77</sup>Ir-196m

MAT 7741

Fission  
Radionuclide Production Cross Section

77-Ir-196m



12

Incident Energy (MeV)

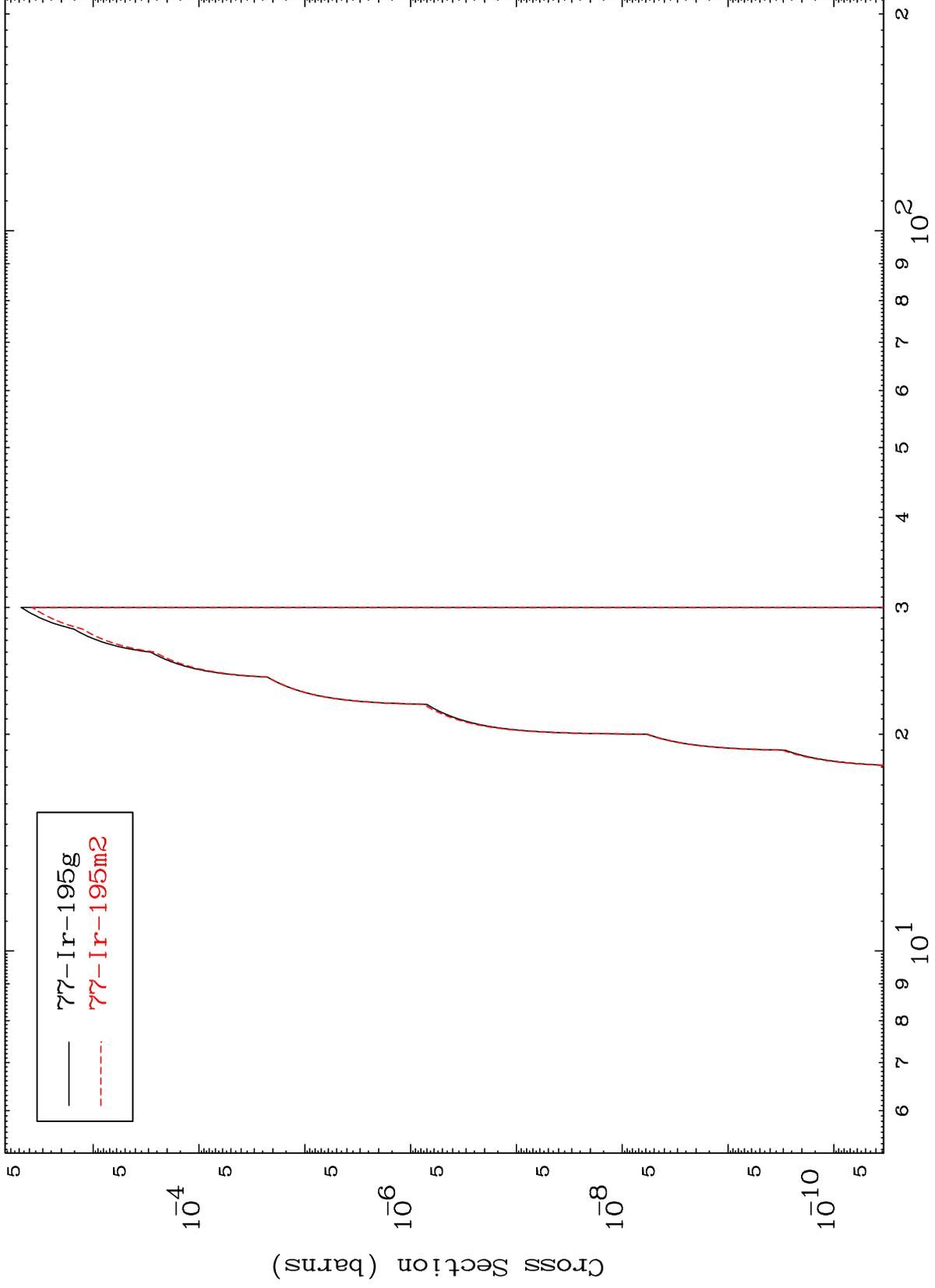
77-Ir-196m

MAT 7741

$(n, n') \alpha$

$^{77}\text{Ir}-196\text{m}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

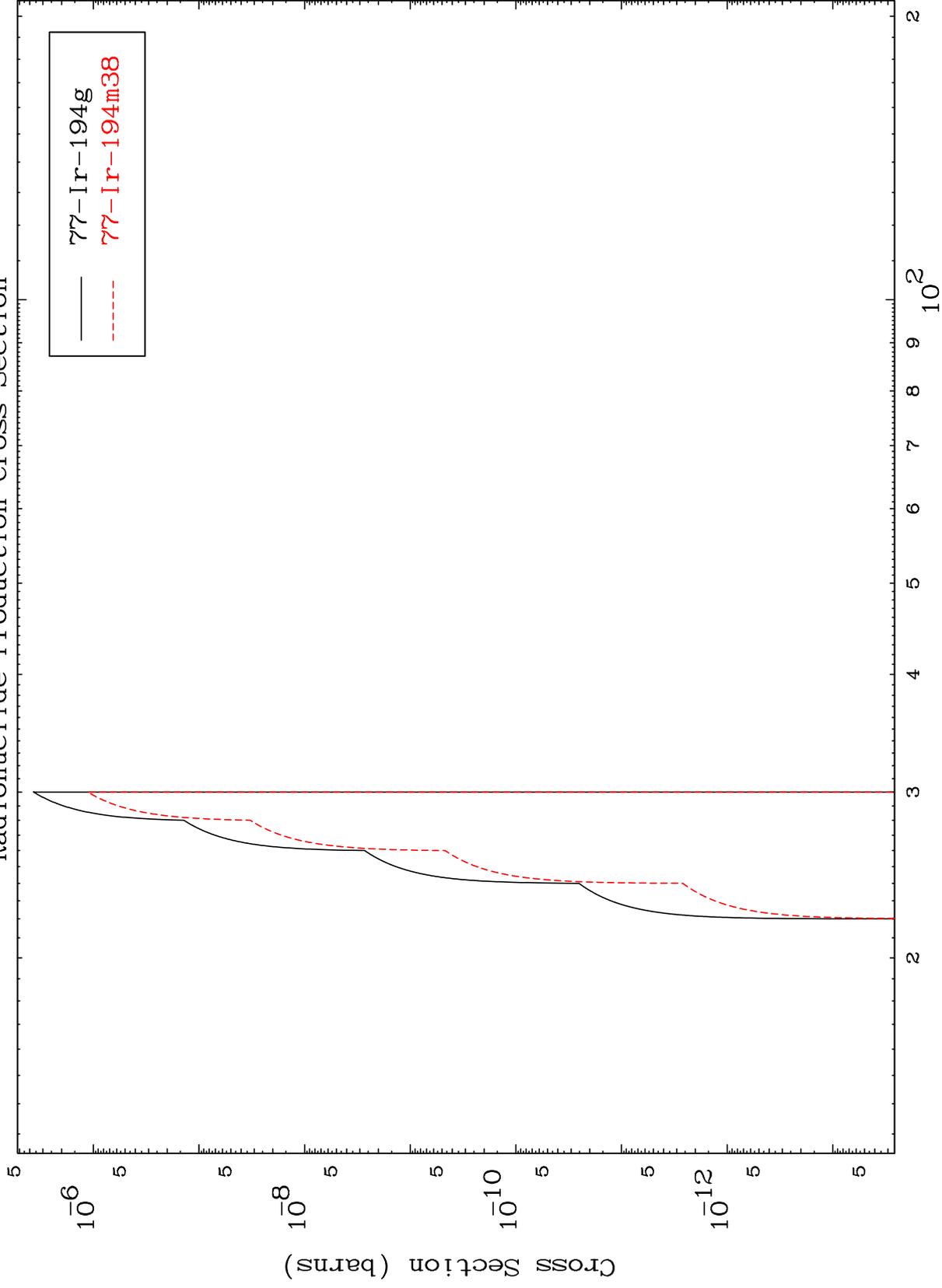
$^{77}\text{Ir}-196\text{m}$

MAT 7741

77-Ir-196m

(n,2n)  $\alpha$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

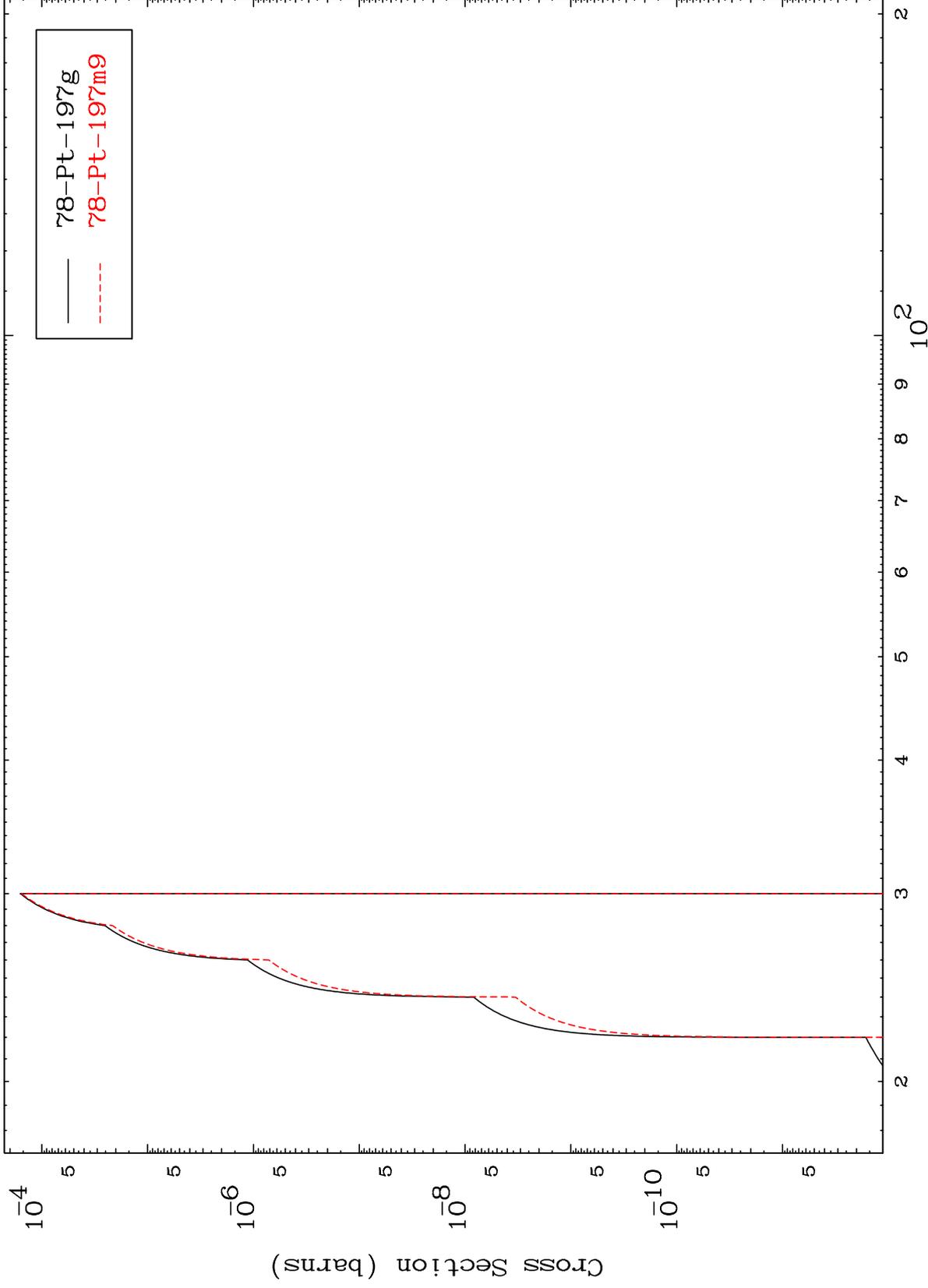
77-Ir-196m

MAT 7741

(n,n') d

77-Ir-196m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

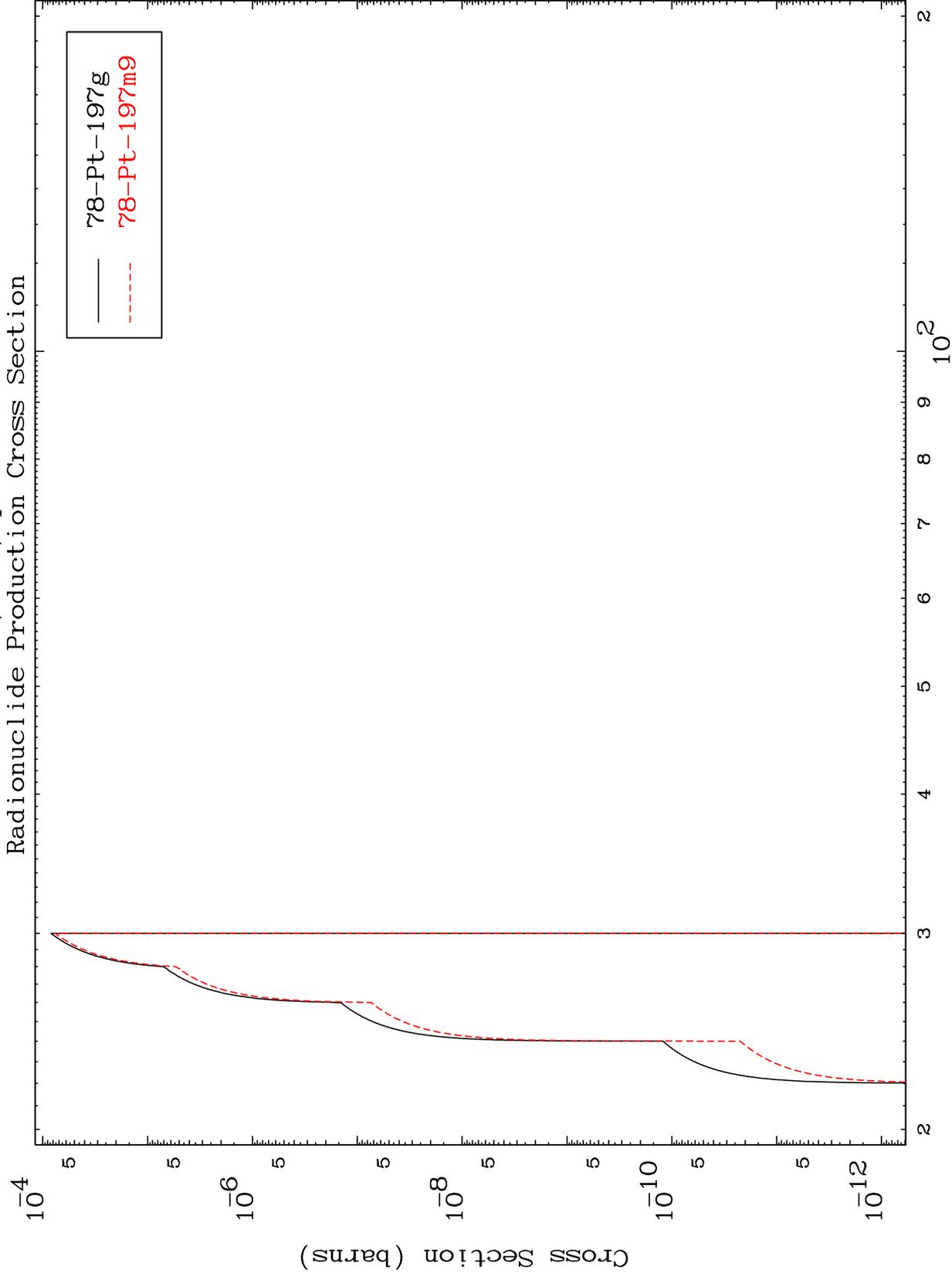
77-Ir-196m

MAT 7741

(n,2n) p

77-Ir-196m

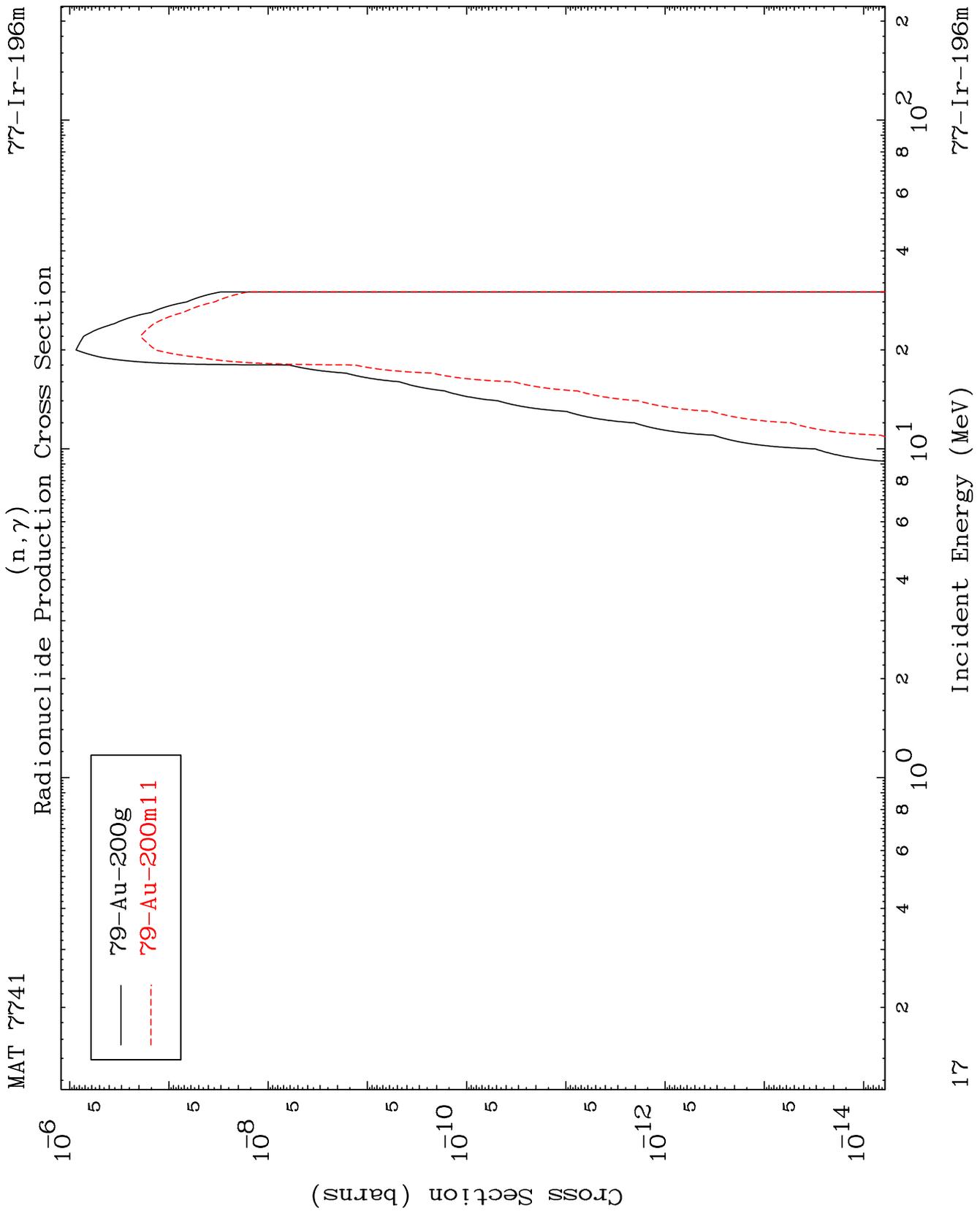
Radionuclide Production Cross Section



16

Incident Energy (MeV)

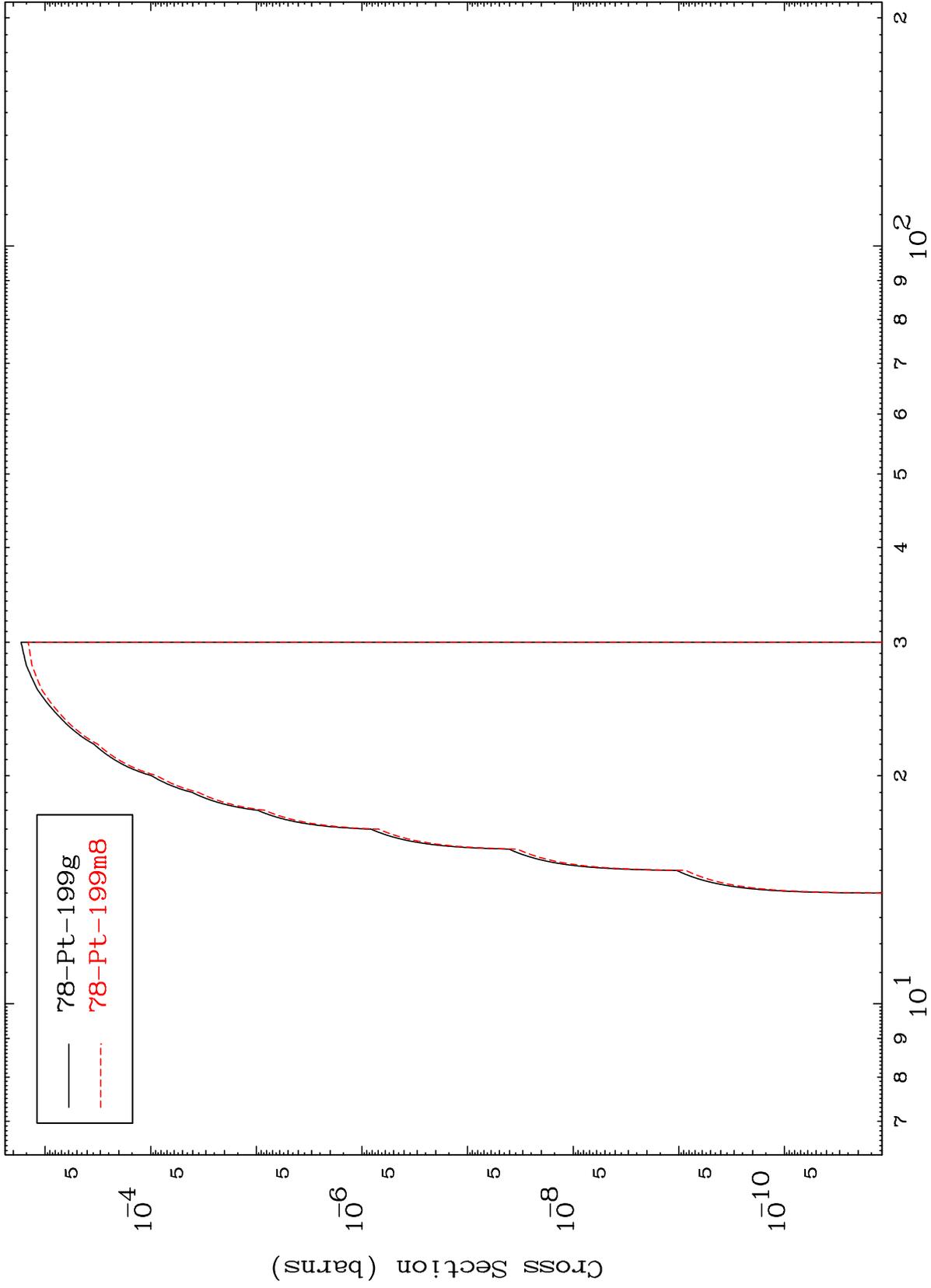
77-Ir-196m



MAT 7741

77-Ir-196m

(n,p)  
Radionuclide Production Cross Section



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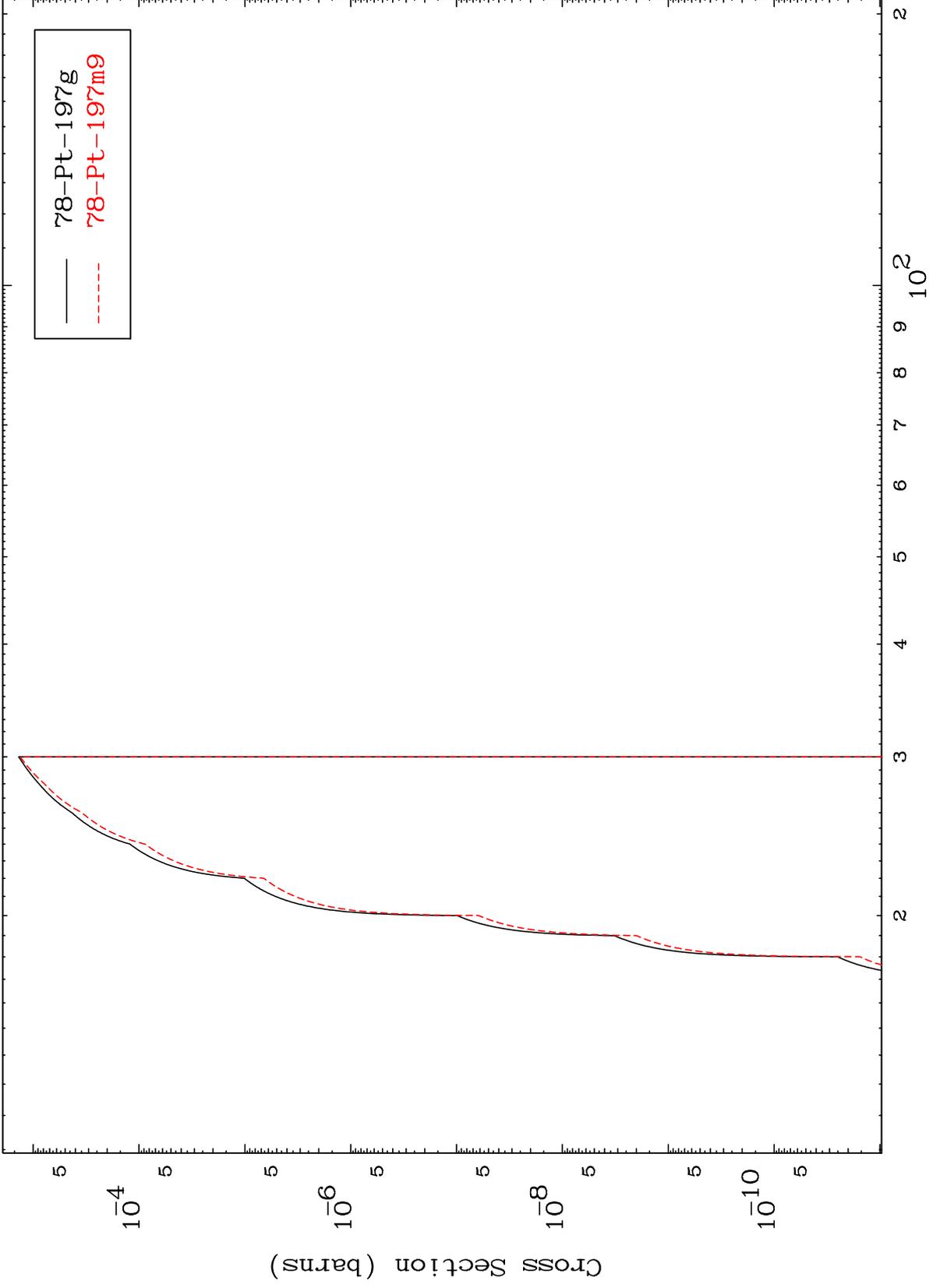
Incident Energy (MeV)

77-Ir-196m

MAT 7741

77-Ir-196m

(n, t)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

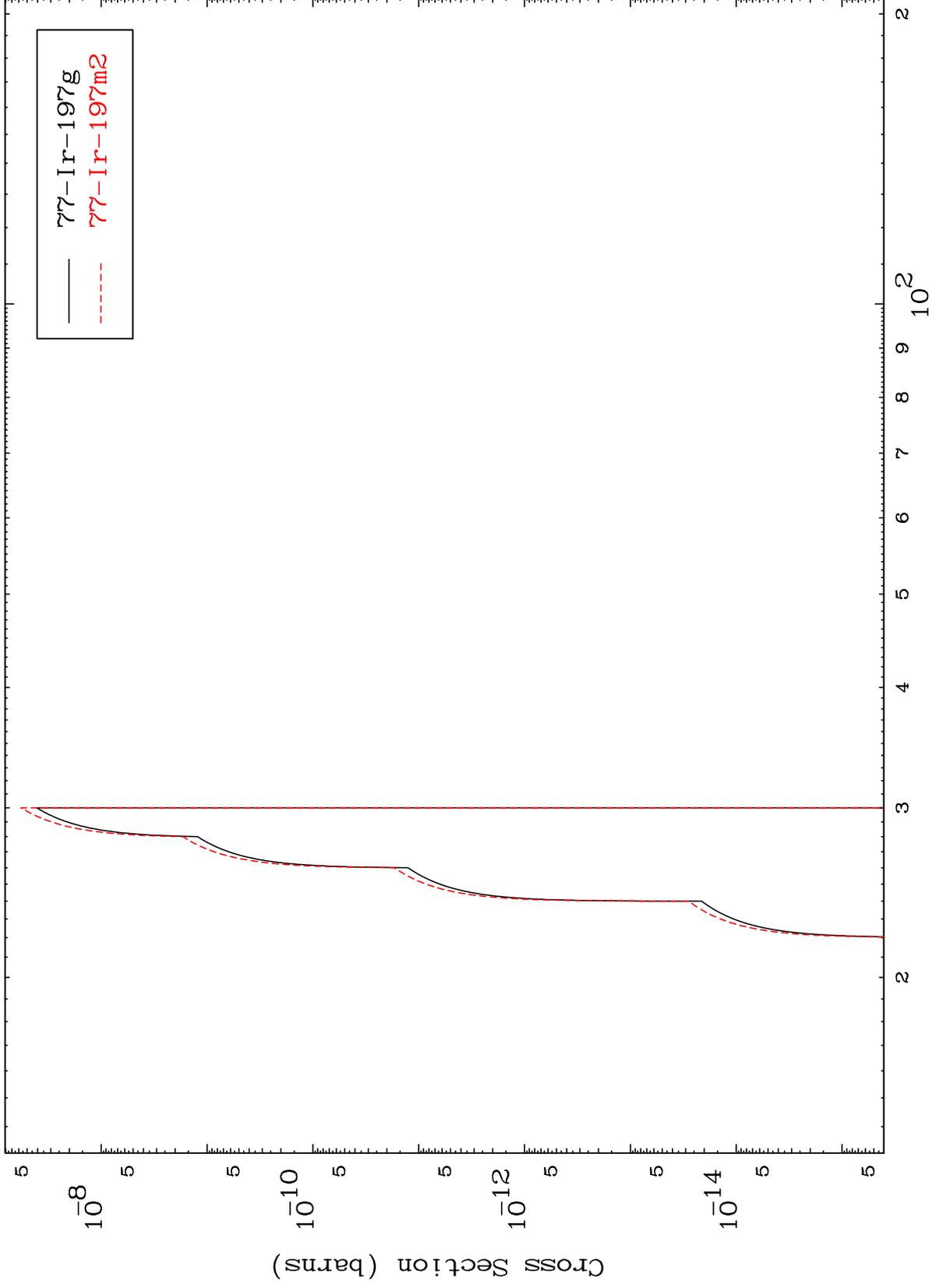
77-Ir-196m

MAT 7741

(n,He-3)

77-Ir-196m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

77-Ir-196m

MAT 7741

(n,  $\alpha$ )

<sup>77</sup>Ir-196m

Radionuclide Production Cross Section

