

Program EVALPLOT  
(Version 2021-1)

by

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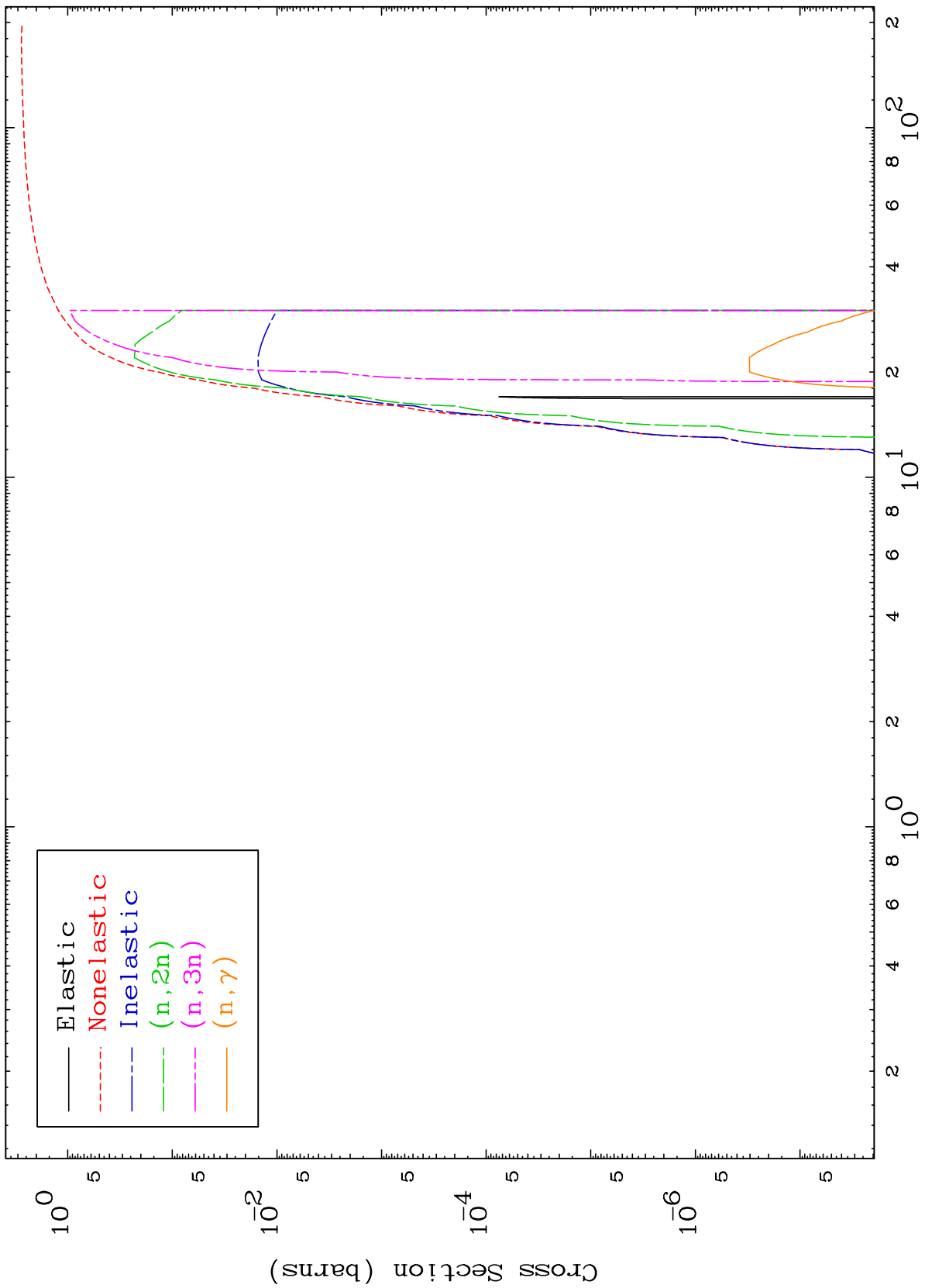
Press Mouse Button to Start

MAT 7746

0 Kelvin

Major Cross Sections

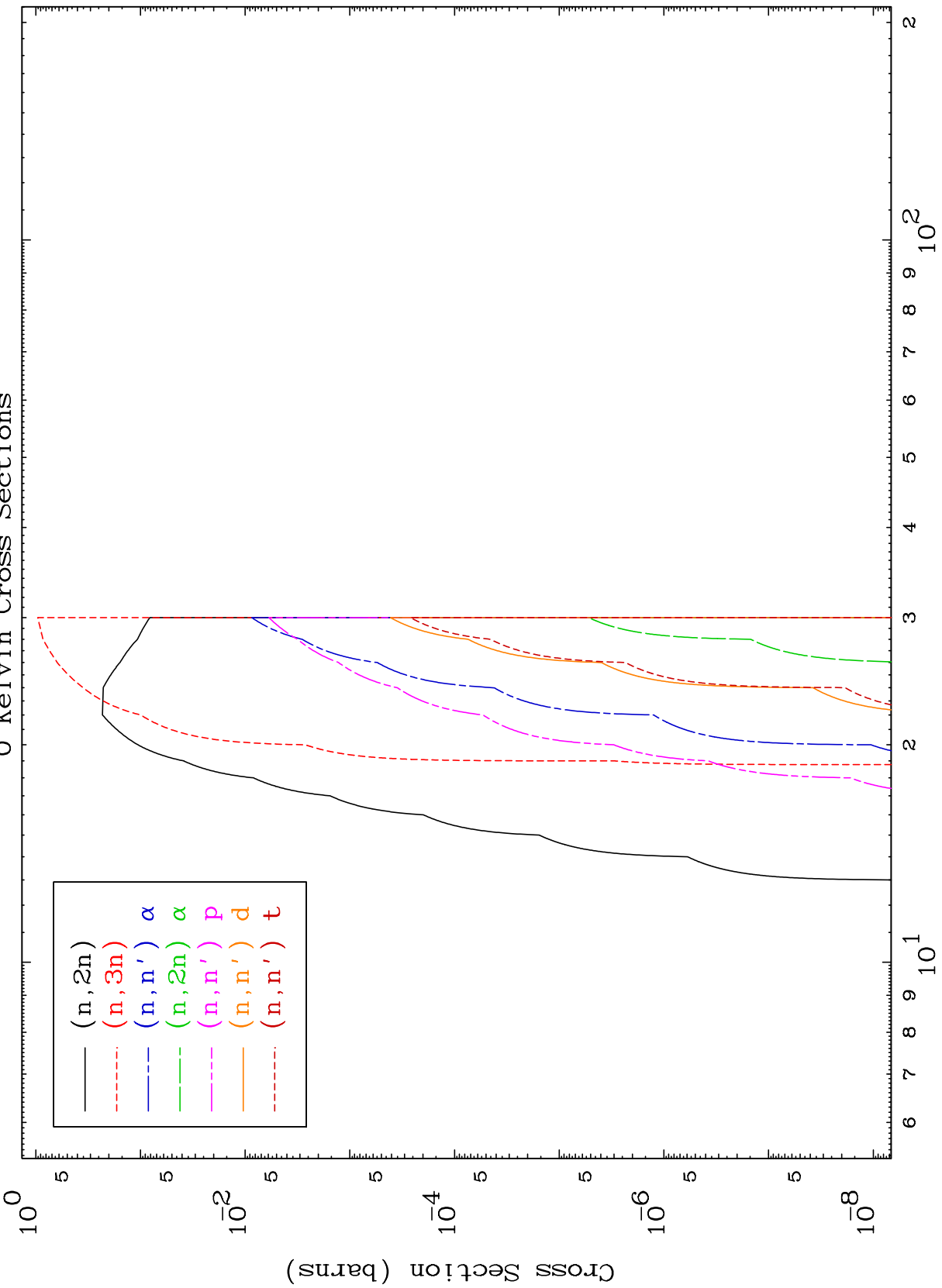
77-Ir-198

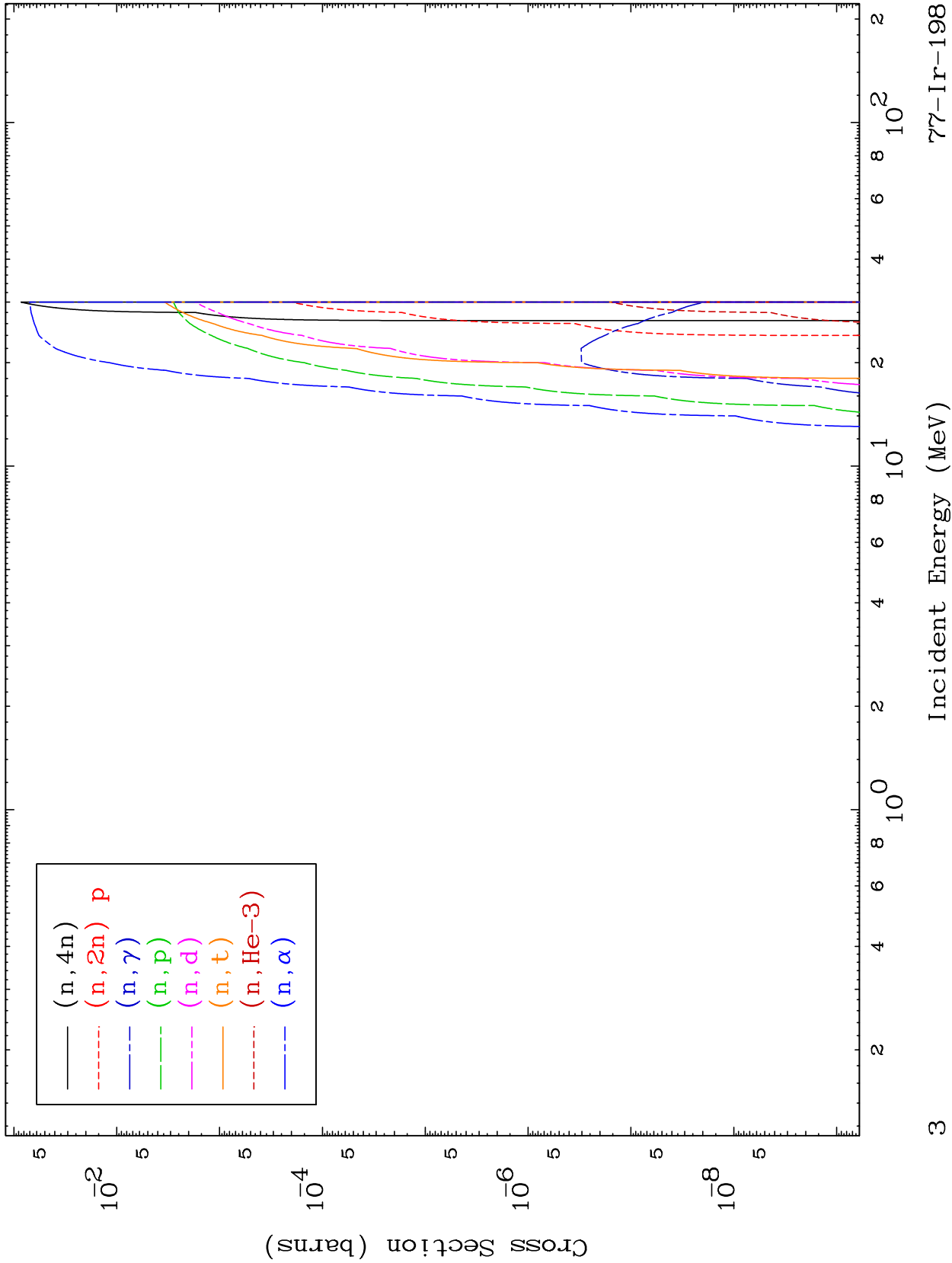


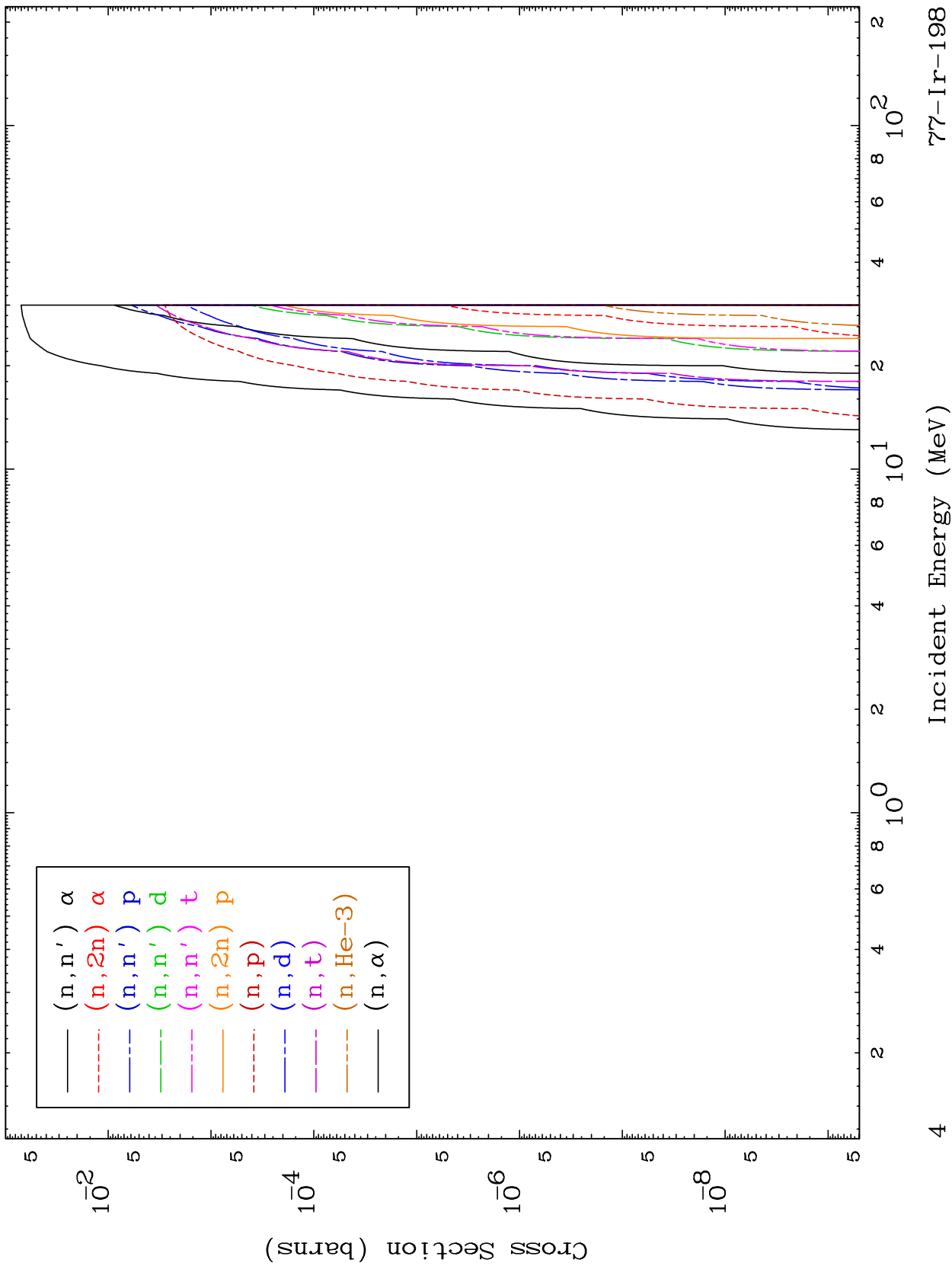
MAT 7746

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-198



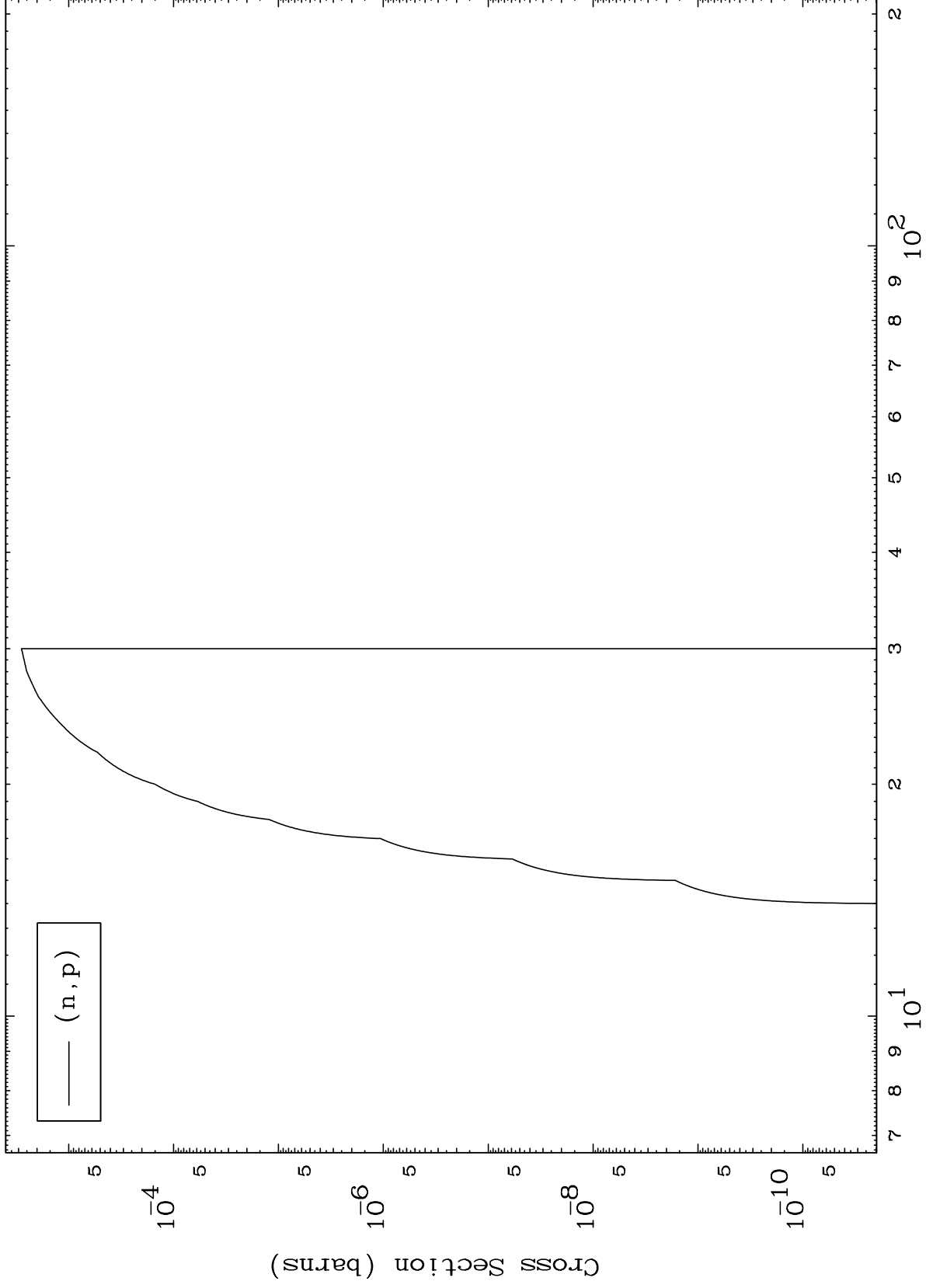




MAT 7746

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-198



(n, p)

5

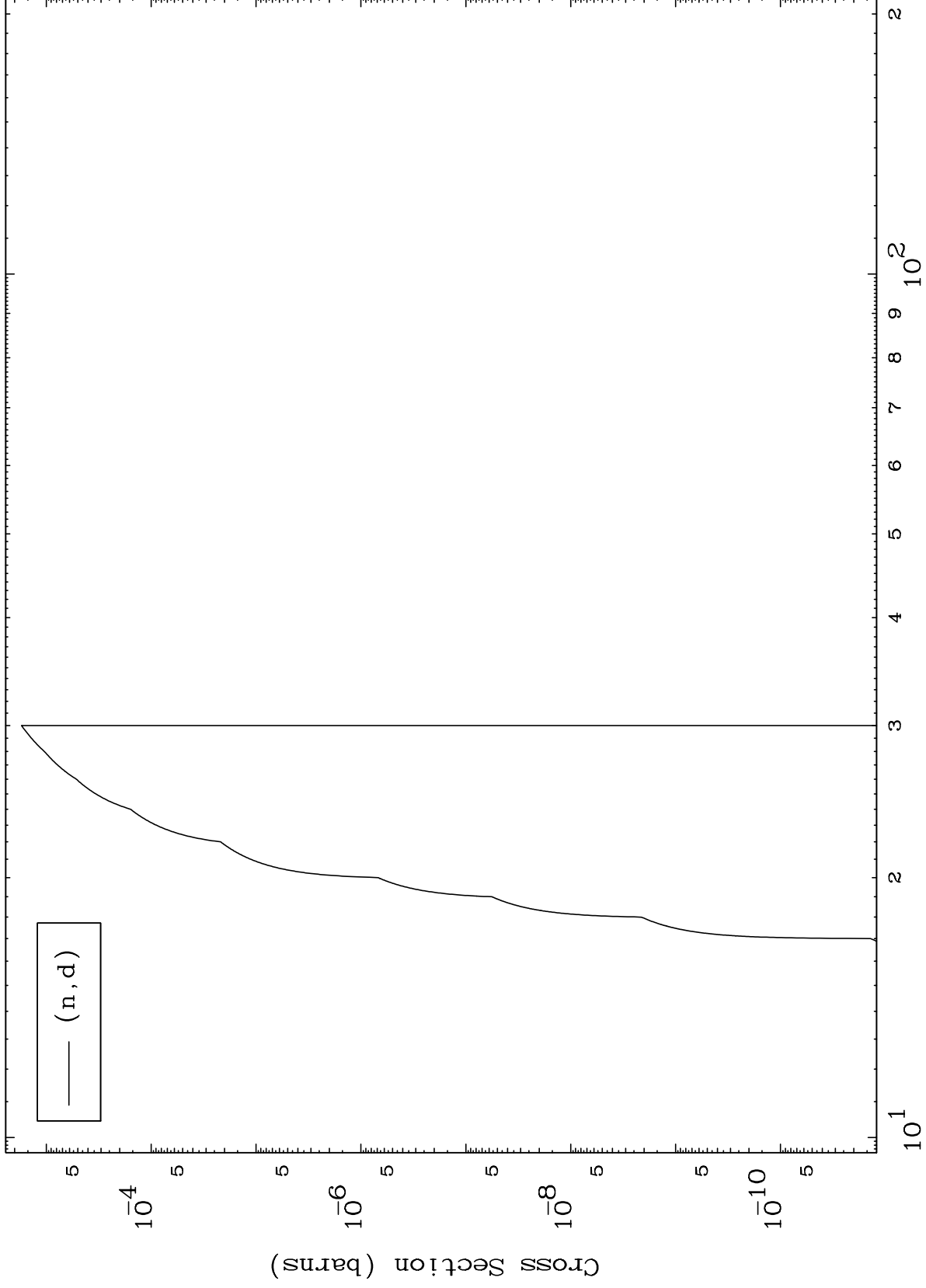
Incident Energy (MeV)

77-Ir-198

MAT 7746

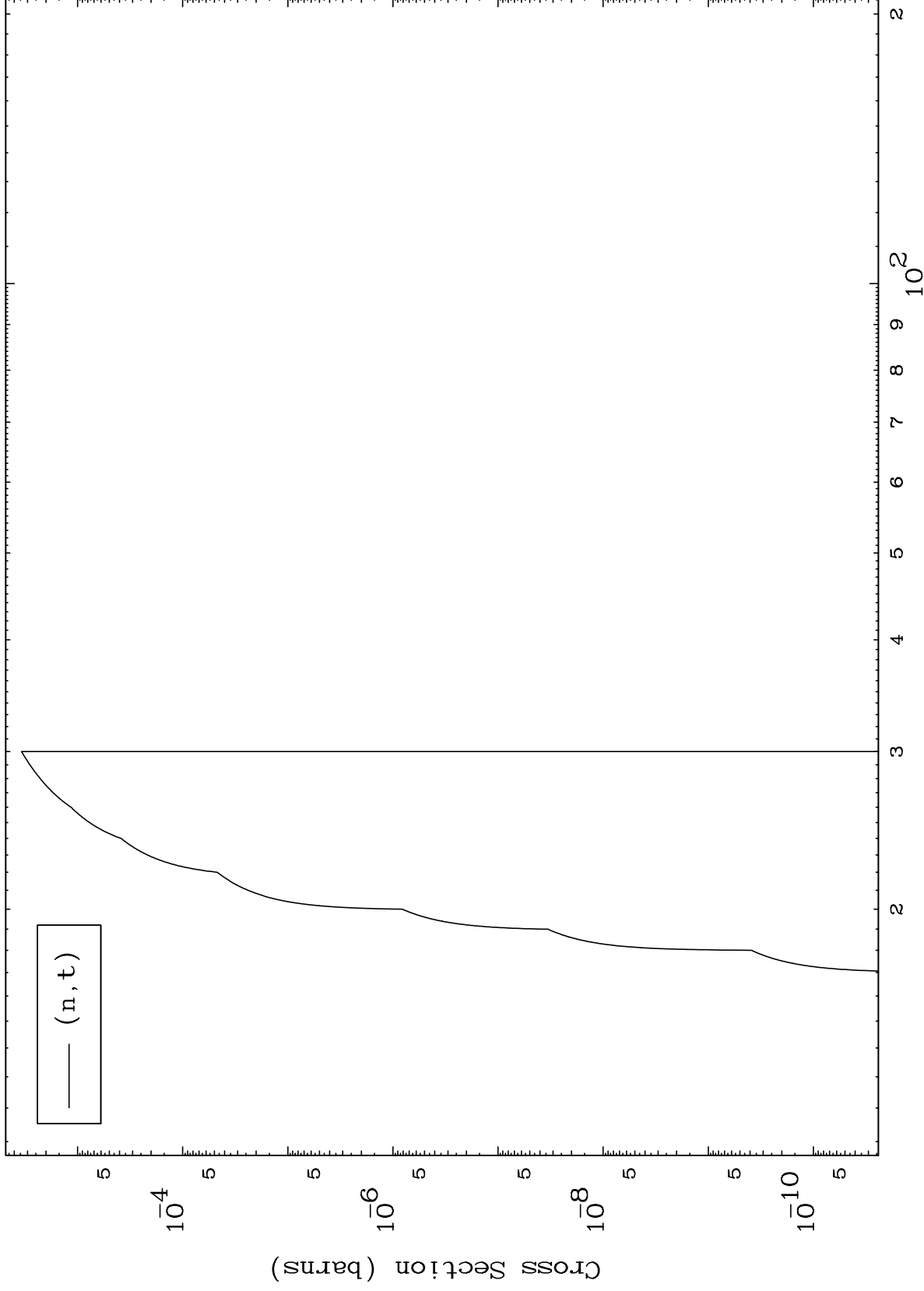
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-198

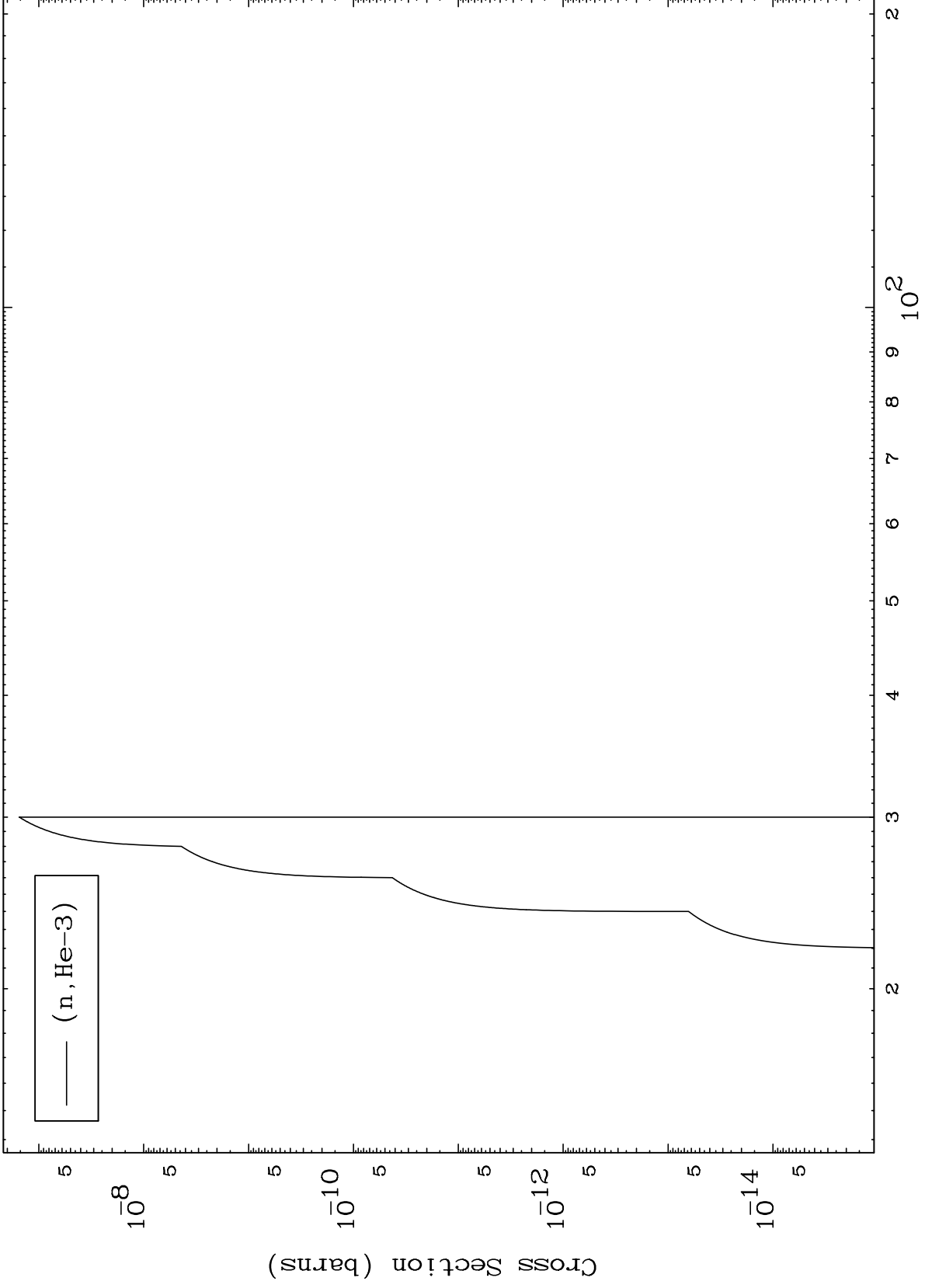


Incident Energy (MeV)

77-Ir-198



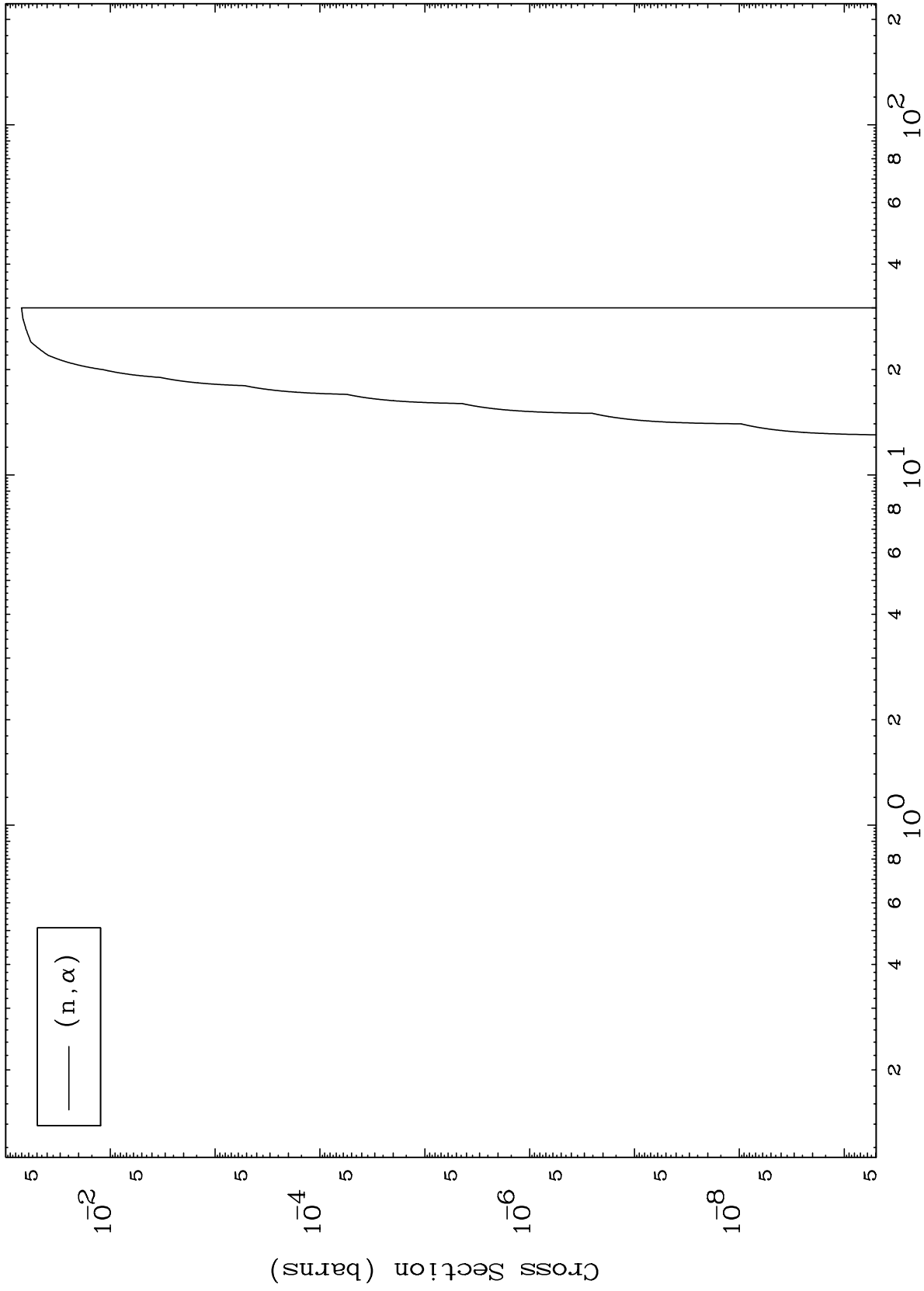
( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



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77-Ir-198

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

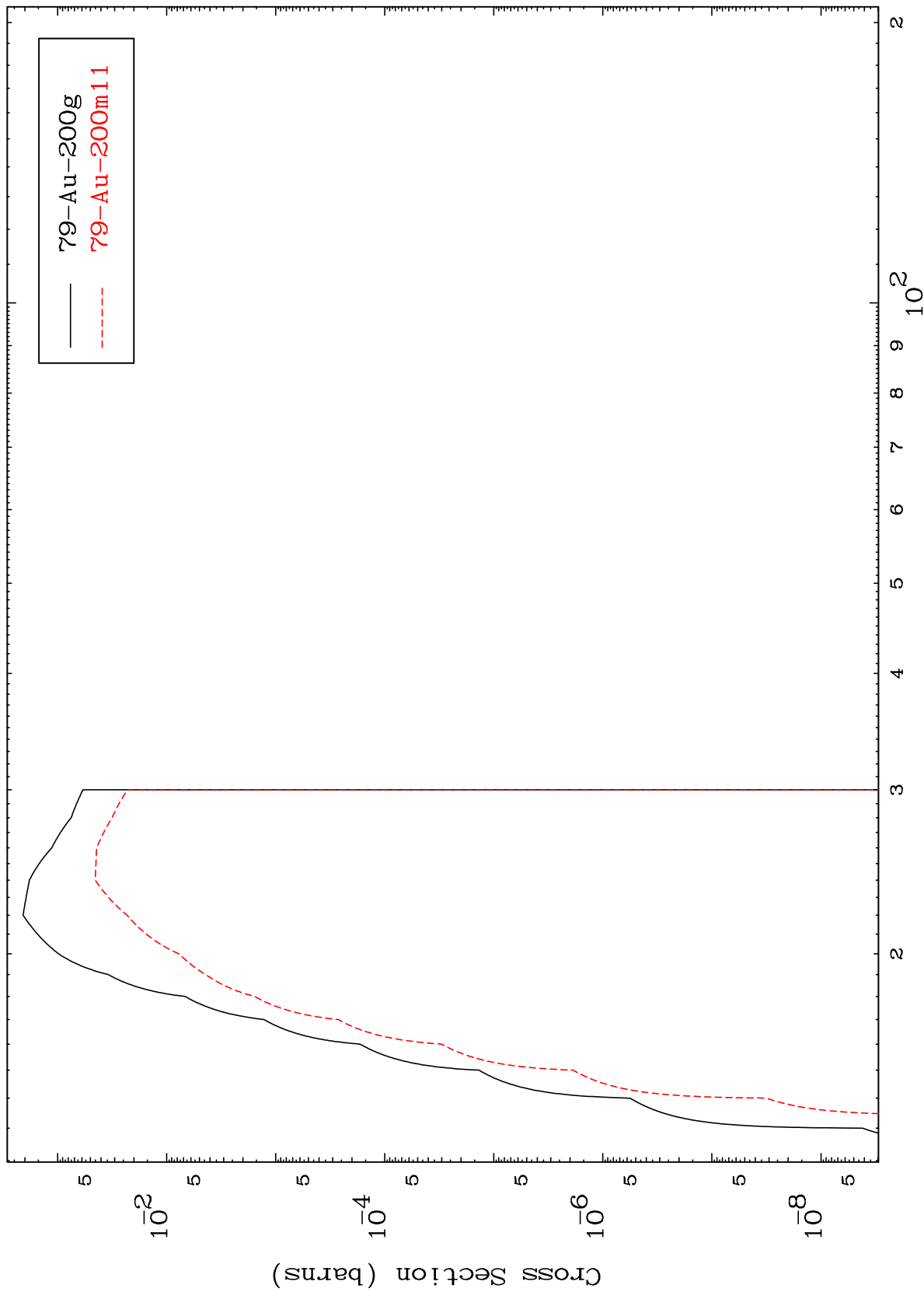


MAT 7746

77-Ir-198

(n,2n)

Radionuclide Production Cross Section



10

Incident Energy (MeV)

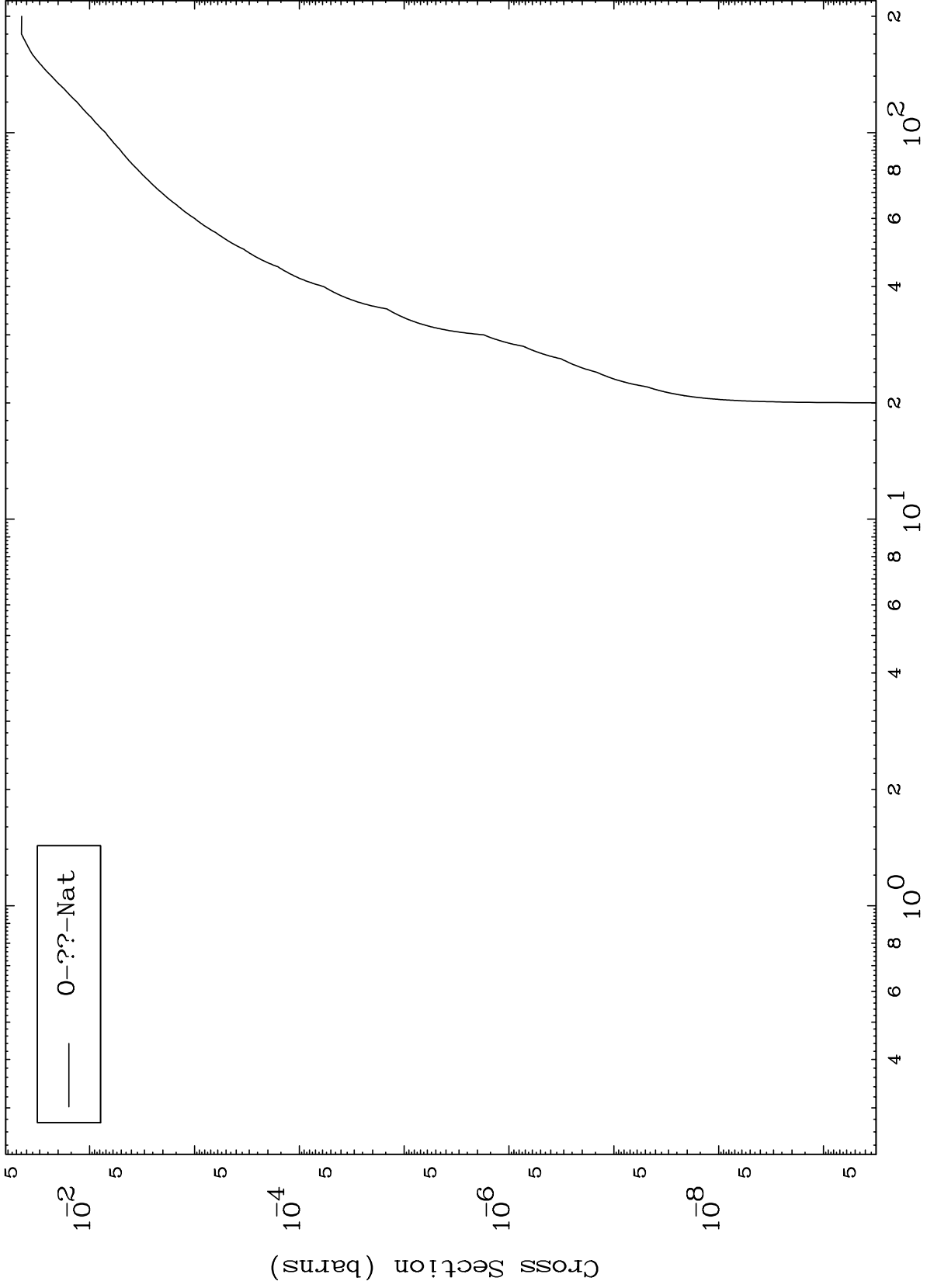
77-Ir-198

MAT 7746

Fission

77-Ir-198

Radionuclide Production Cross Section

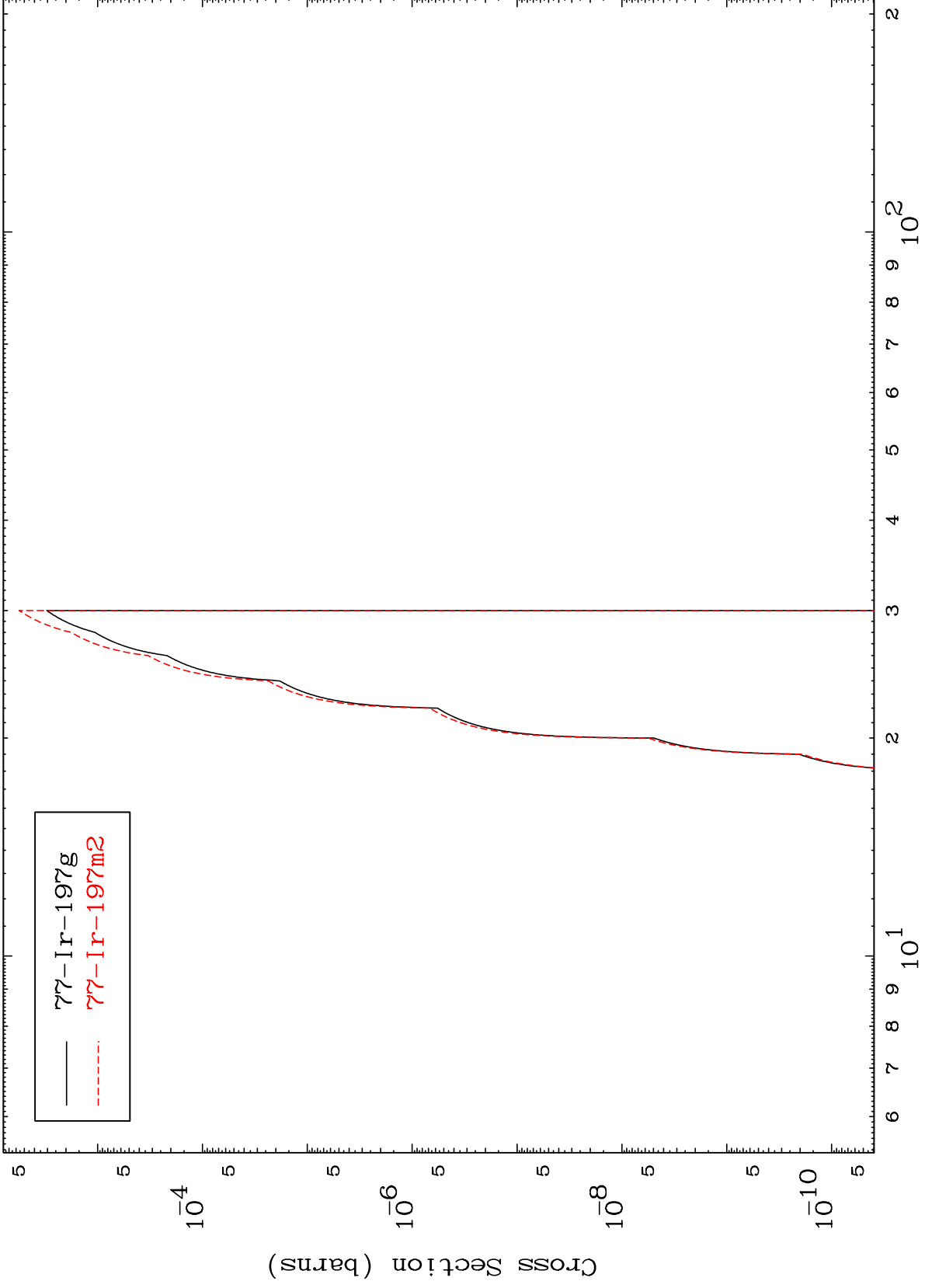


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$(n, n') \alpha$

$^{77}\text{Ir-198}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

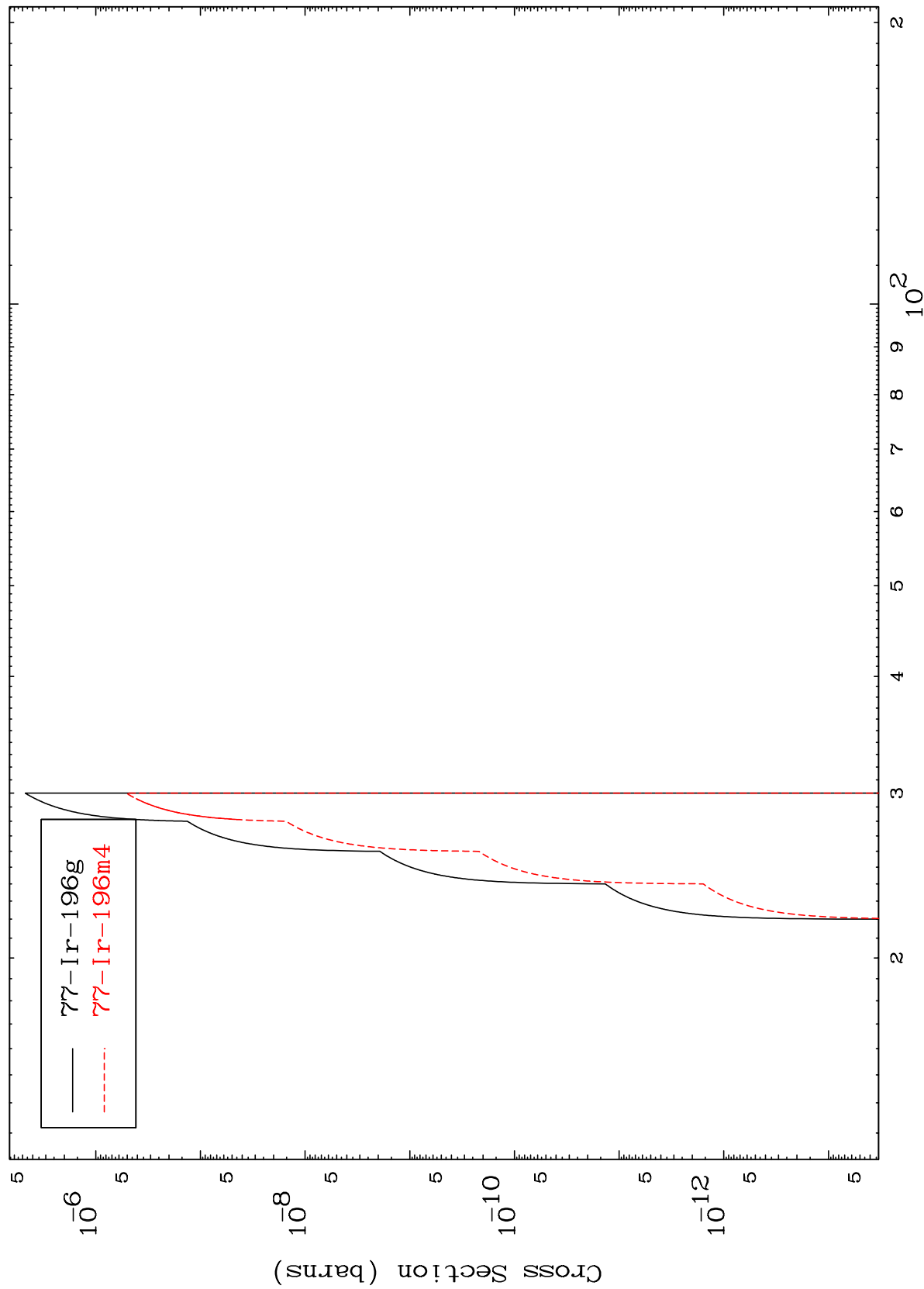
$^{77}\text{Ir-198}$

MAT 7746

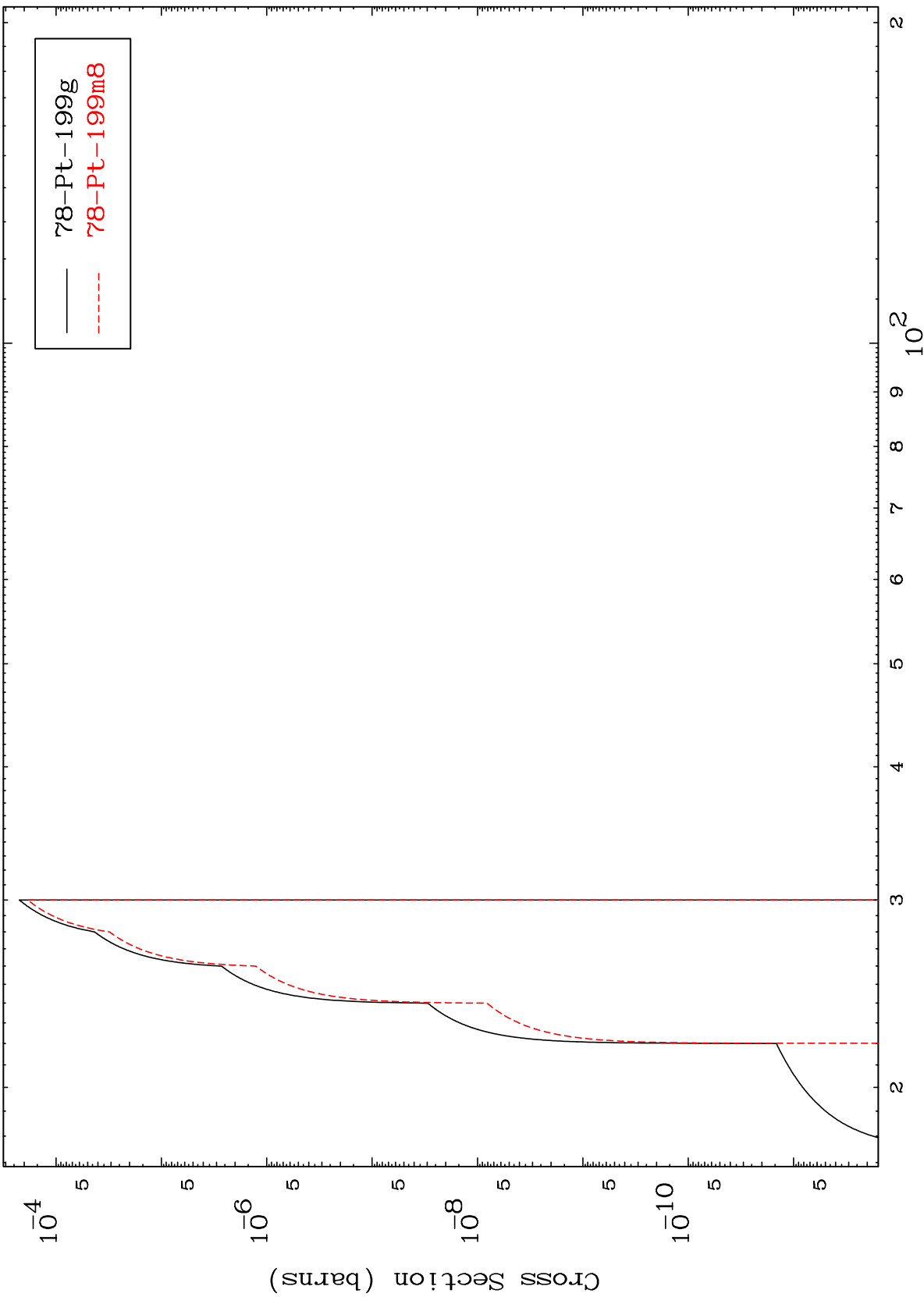
$^{77}\text{Ir}$   $\alpha$

$^{77}\text{Ir}$ -198

Radionuclide Production Cross Section



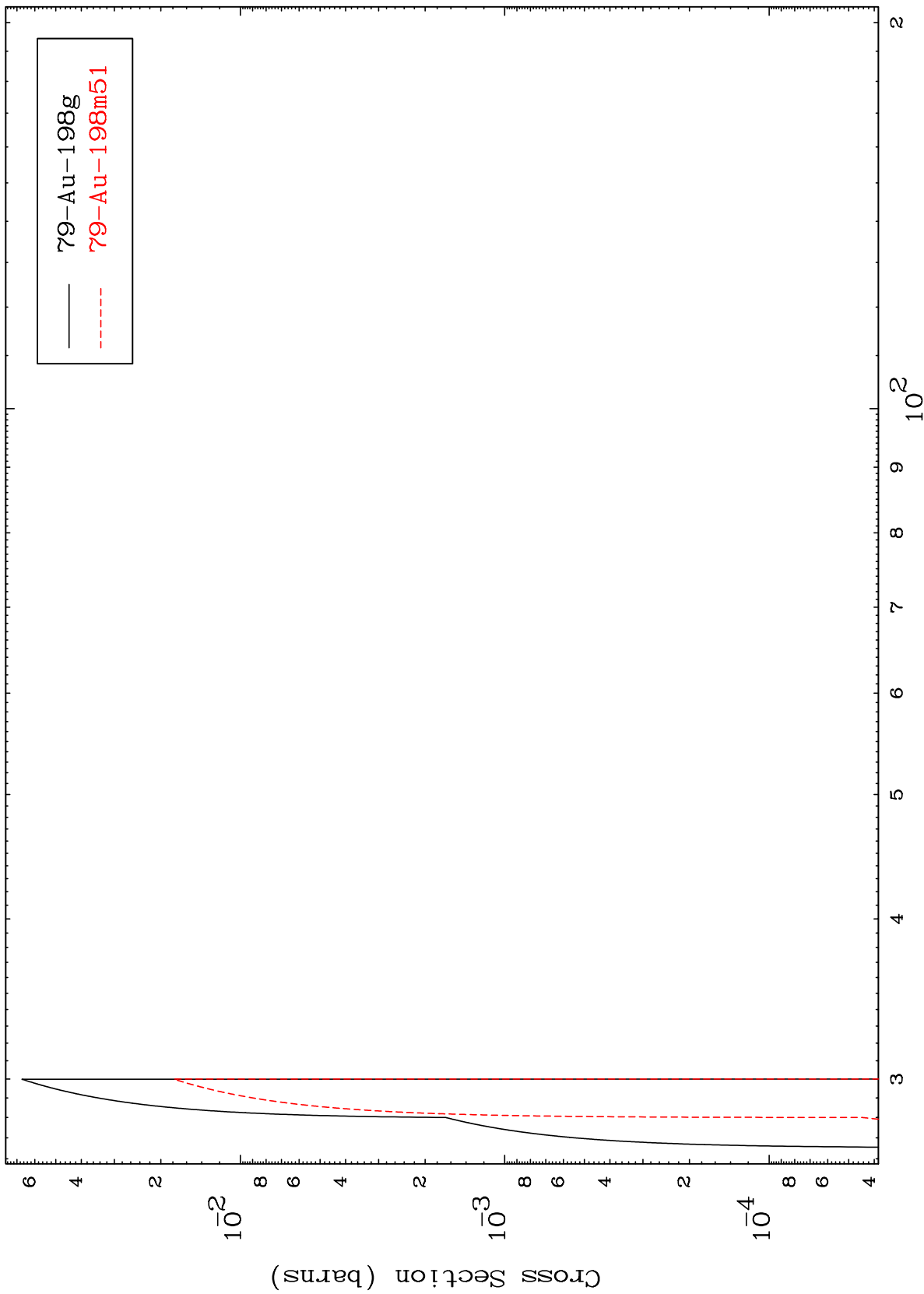
Radionuclide Production Cross Section



MAT 7746

77-Ir-198

(n,4n)  
Radionuclide Production Cross Section



79-Au-198g  
79-Au-198m51

15

77-Ir-198

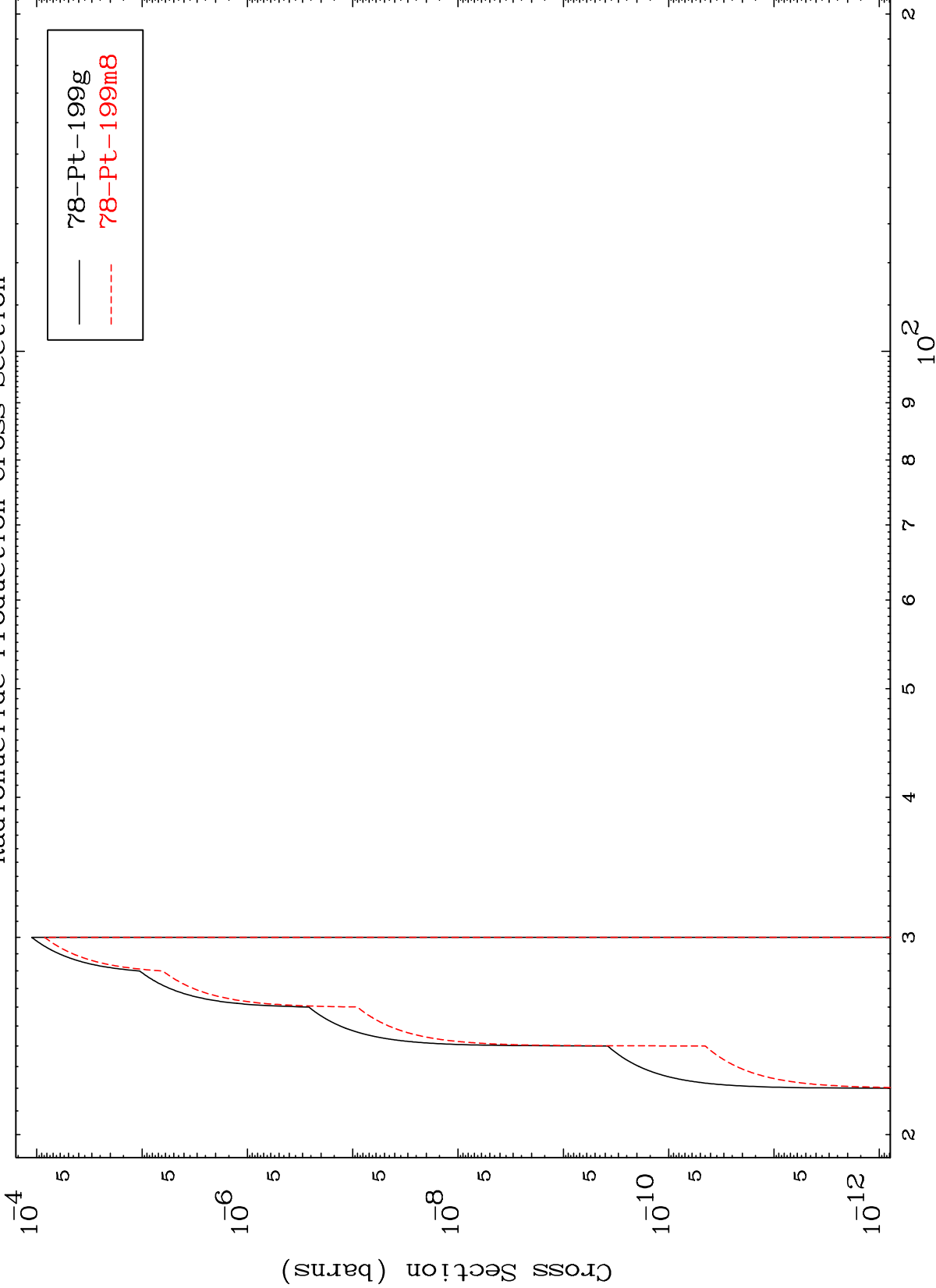
Incident Energy (MeV)

MAT 7746

(n,2n) p

77-Ir-198

Radionuclide Production Cross Section



16

Incident Energy (MeV)

77-Ir-198

(n, t)  
Radionuclide Production Cross Section

