

Program EVALPLOT  
(Version 2021-1)

by

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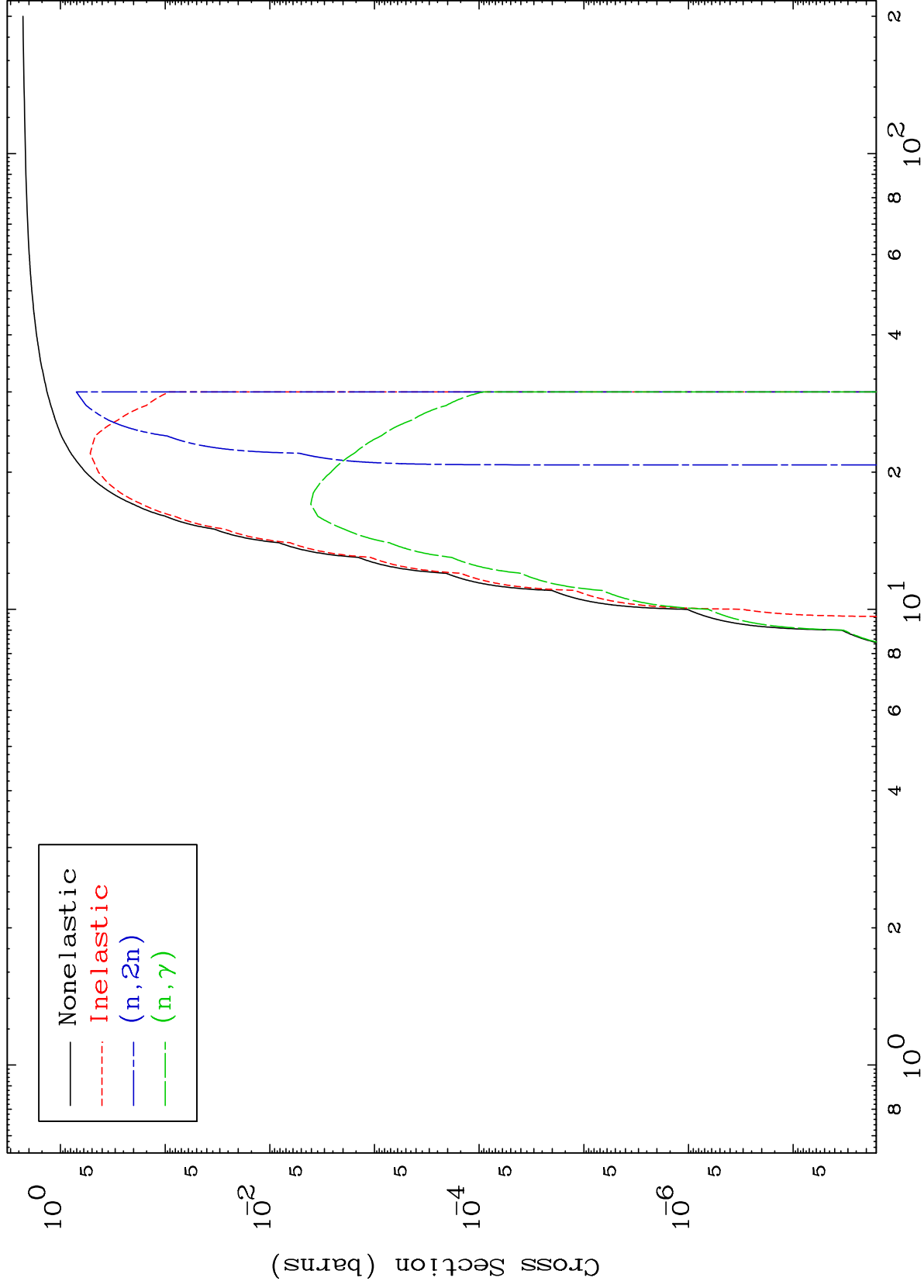
Press Mouse Button to Start

MAT 5701

$\alpha$  Major

57-La-130

0 Kelvin Cross Sections



1

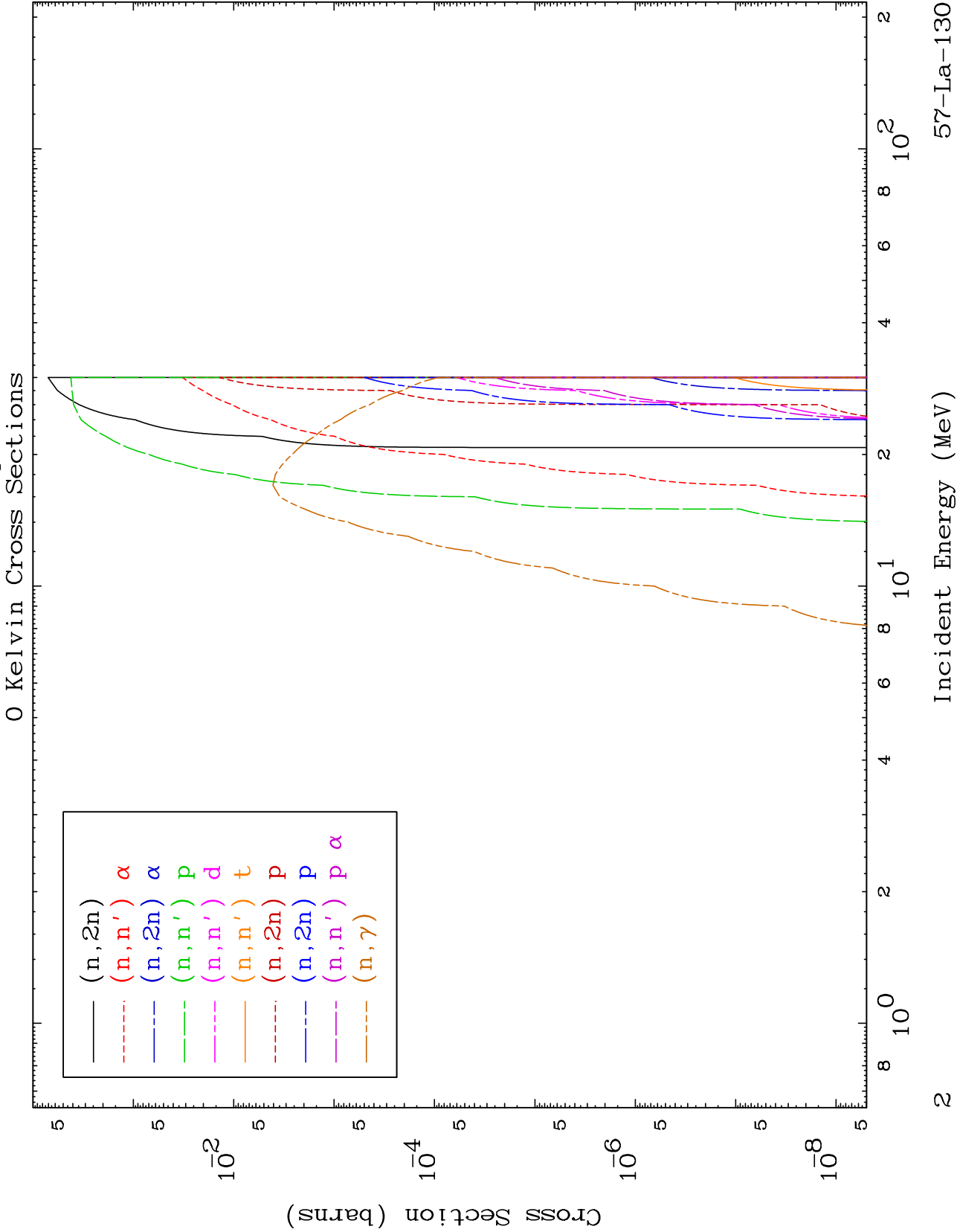
Incident Energy (MeV)

57-La-130

MAT 5701

$\alpha$  Neutron Absorption

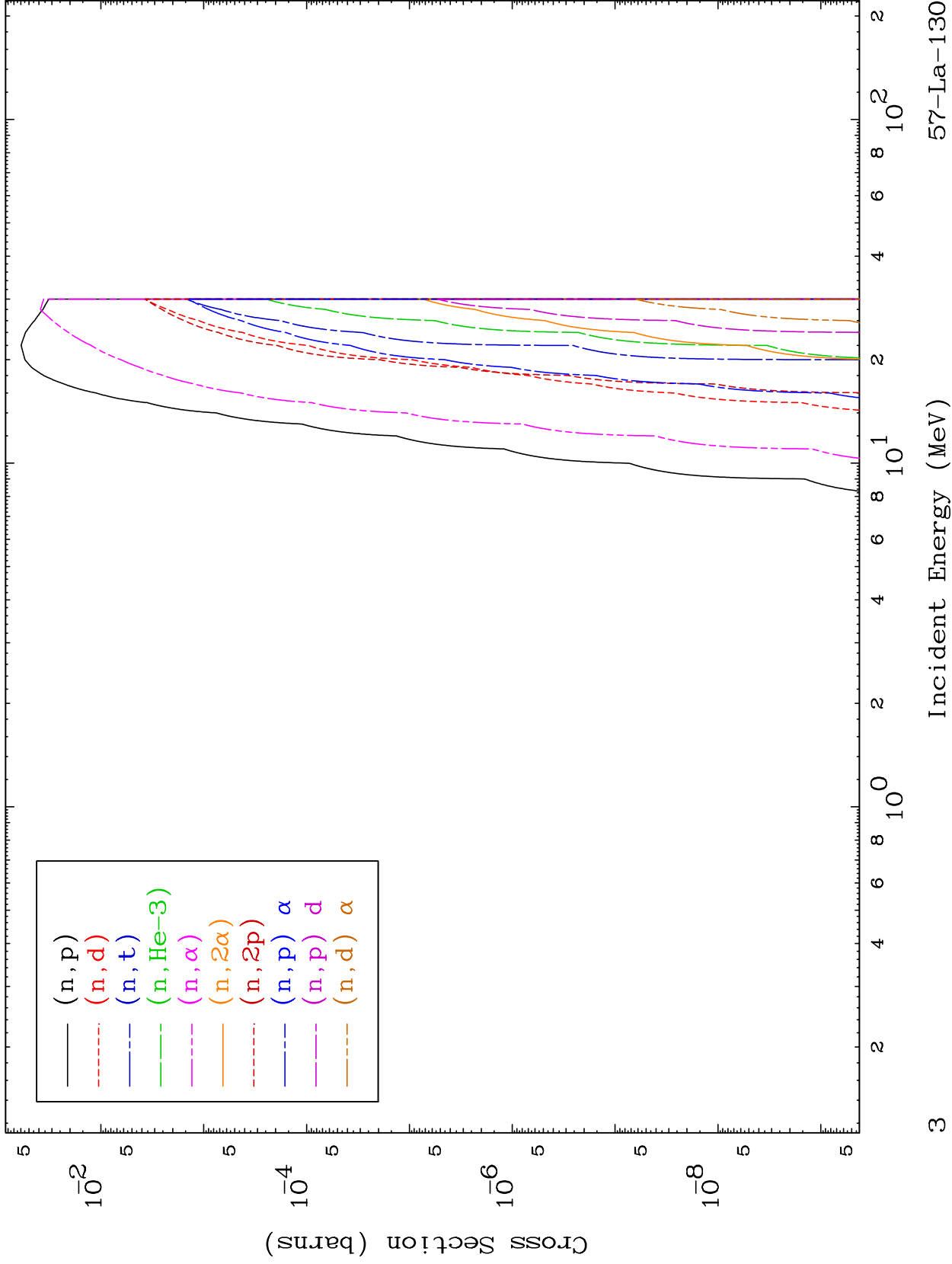
57-La-130



MAT 5701

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

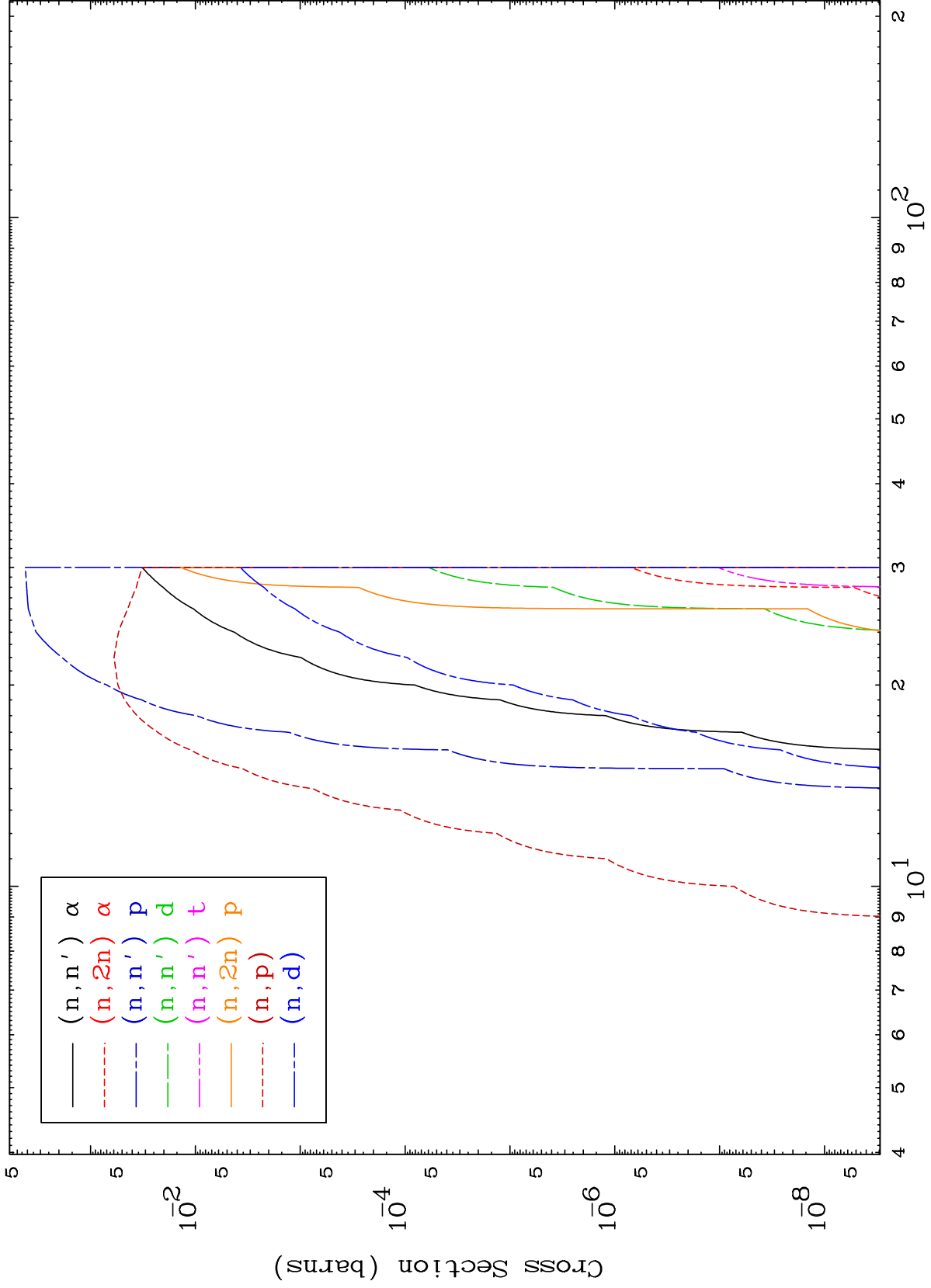
57-La-130



MAT 5701

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

57-La-130



4

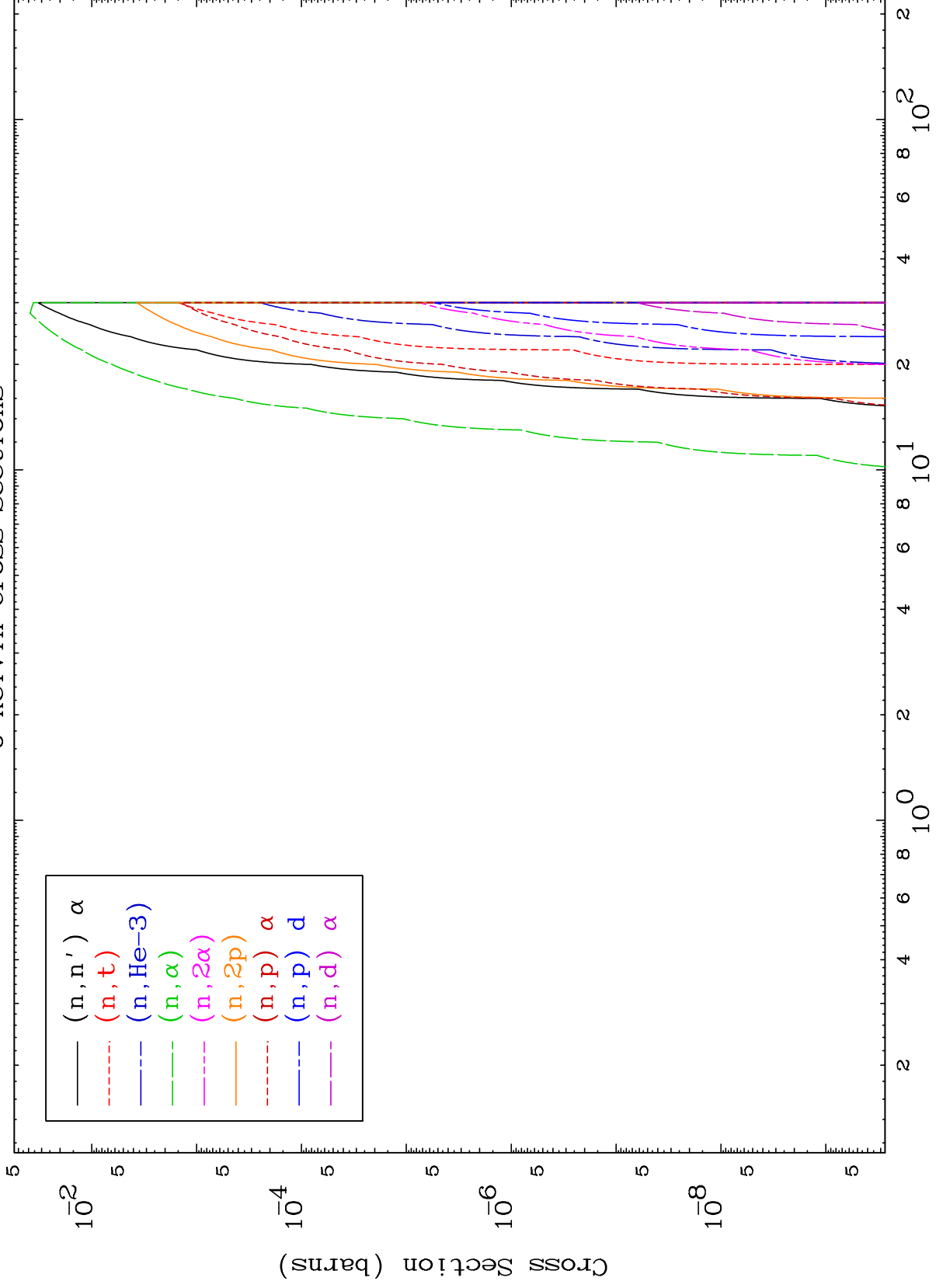
Incident Energy (MeV)

57-La-130

MAT 5701

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

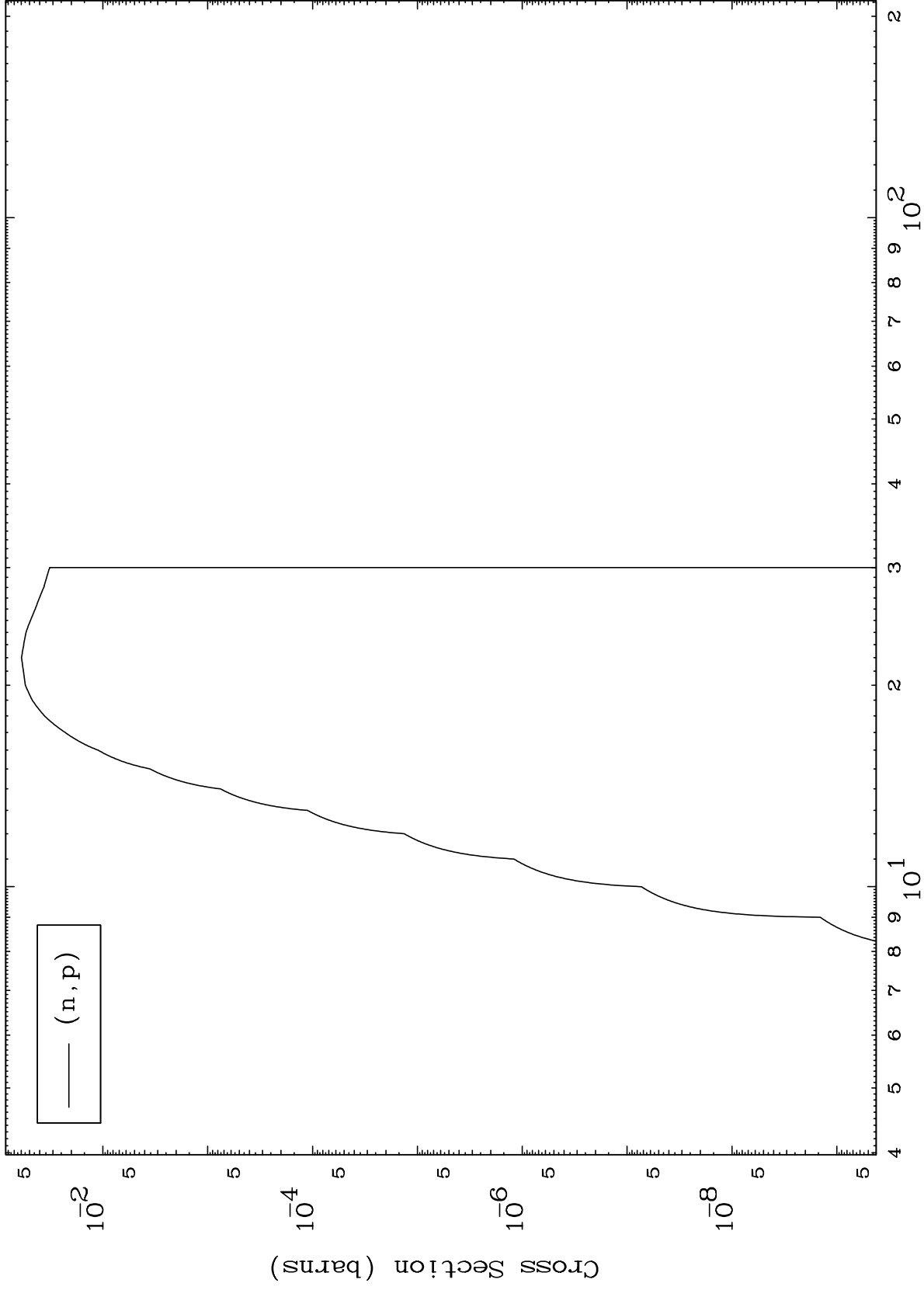
57-La-130



MAT 5701

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

57-La-130



6

Incident Energy (MeV)

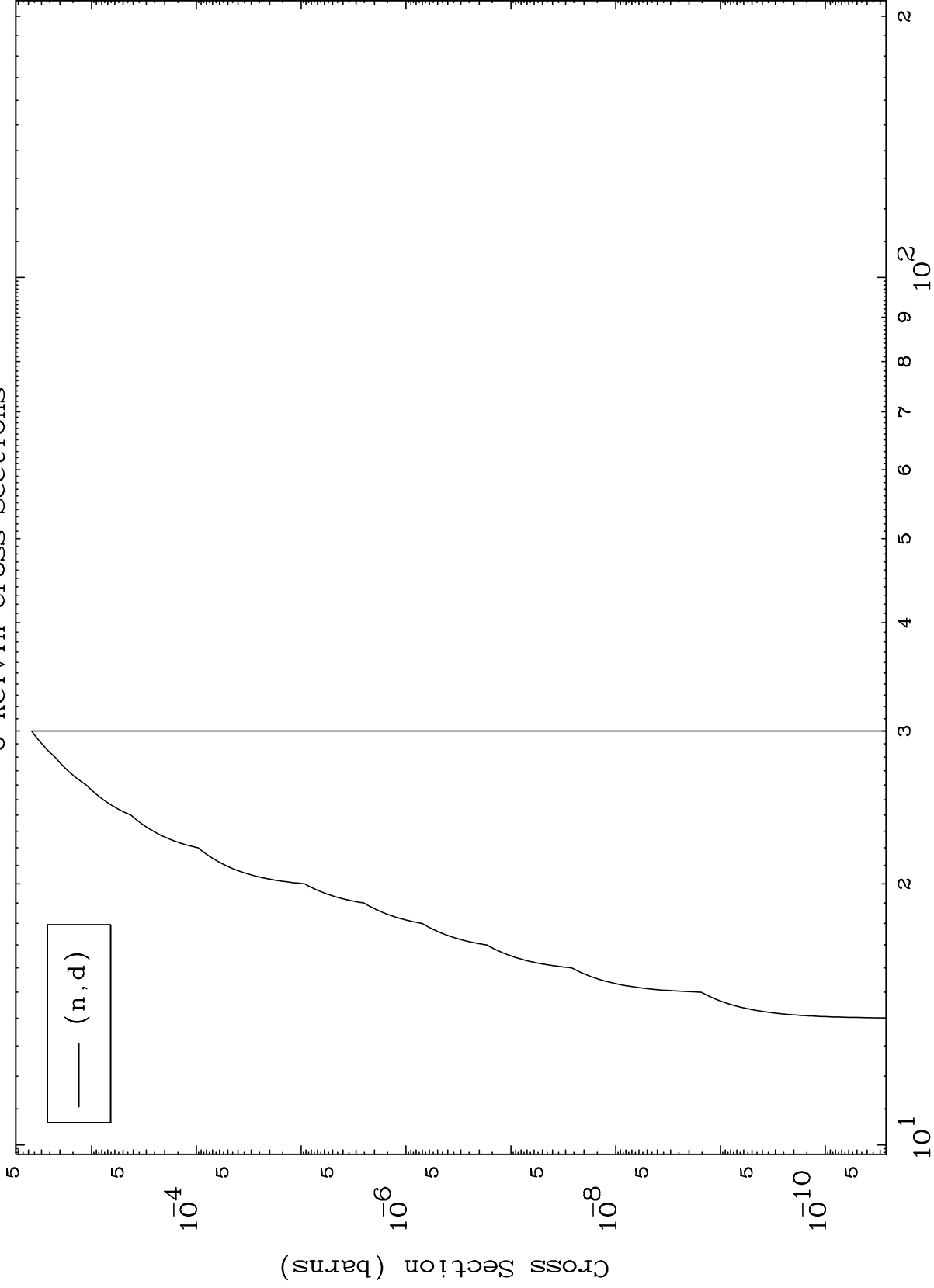
57-La-130

MAT 5701

( $\alpha, d$ ) Levels

57-La-130

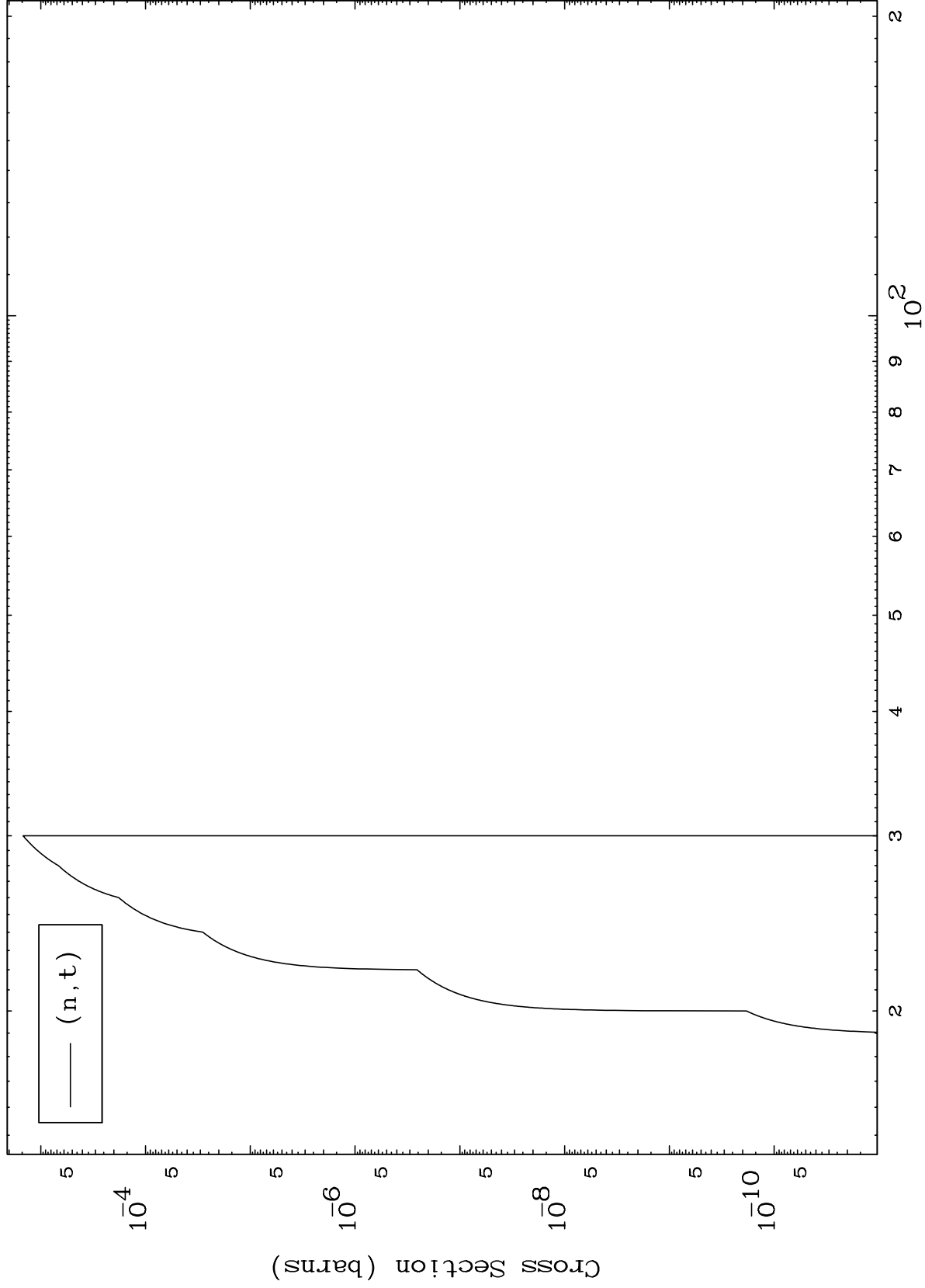
0 Kelvin Cross Sections



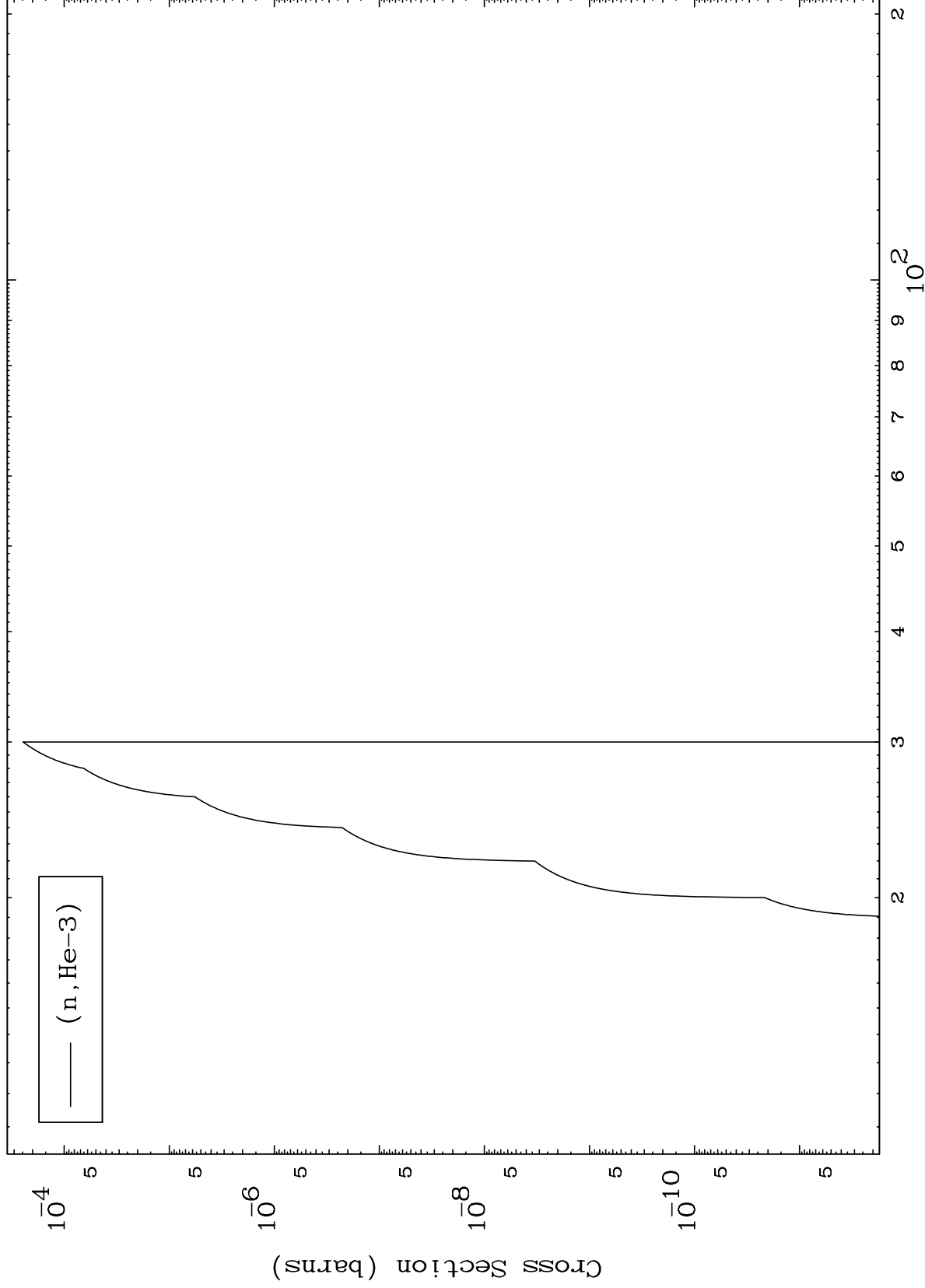
Incident Energy (MeV)

57-La-130

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

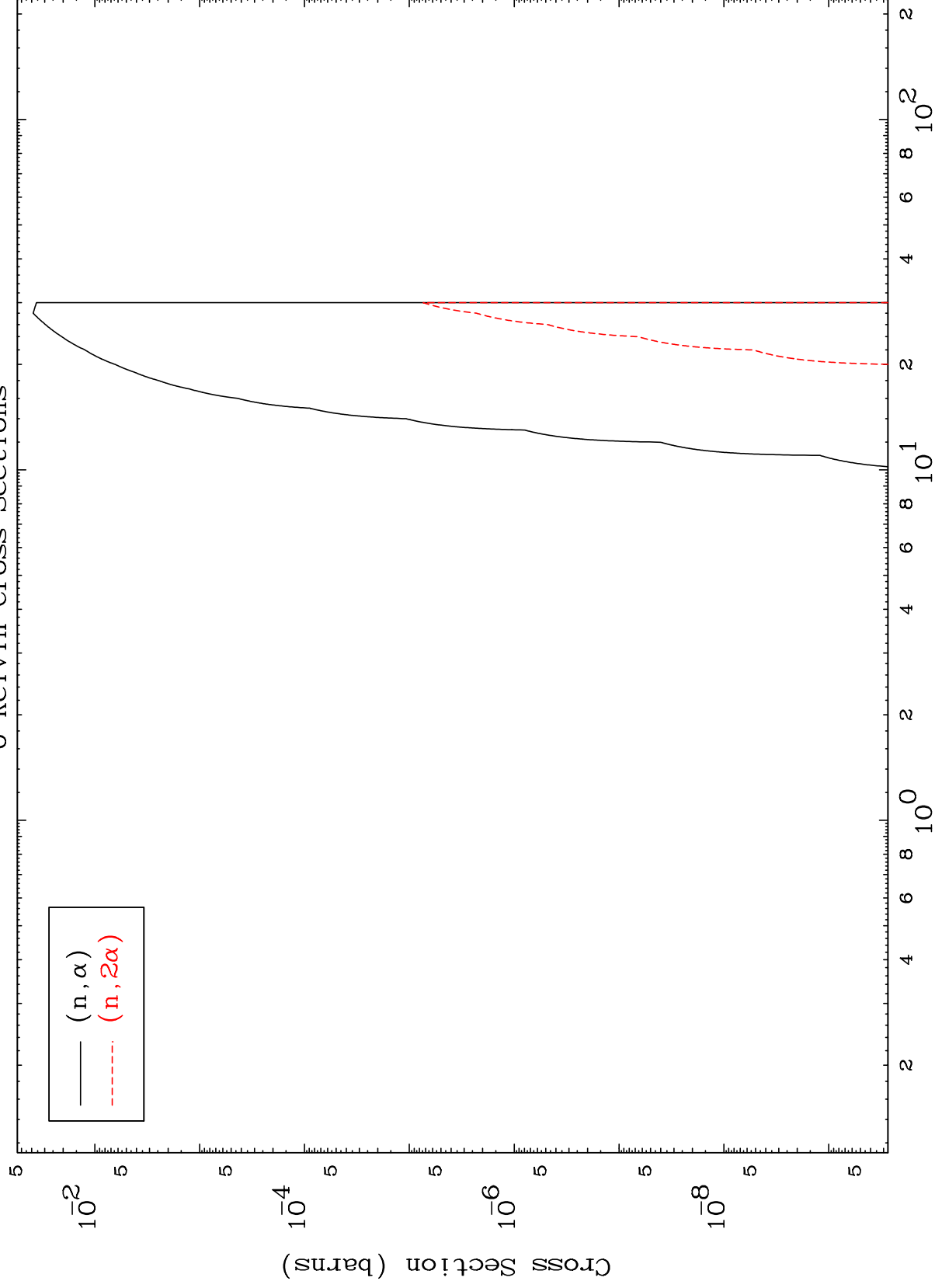


MAT 5701

( $\alpha, \alpha$ ) Levels

57-La-130

0 Kelvin Cross Sections



10

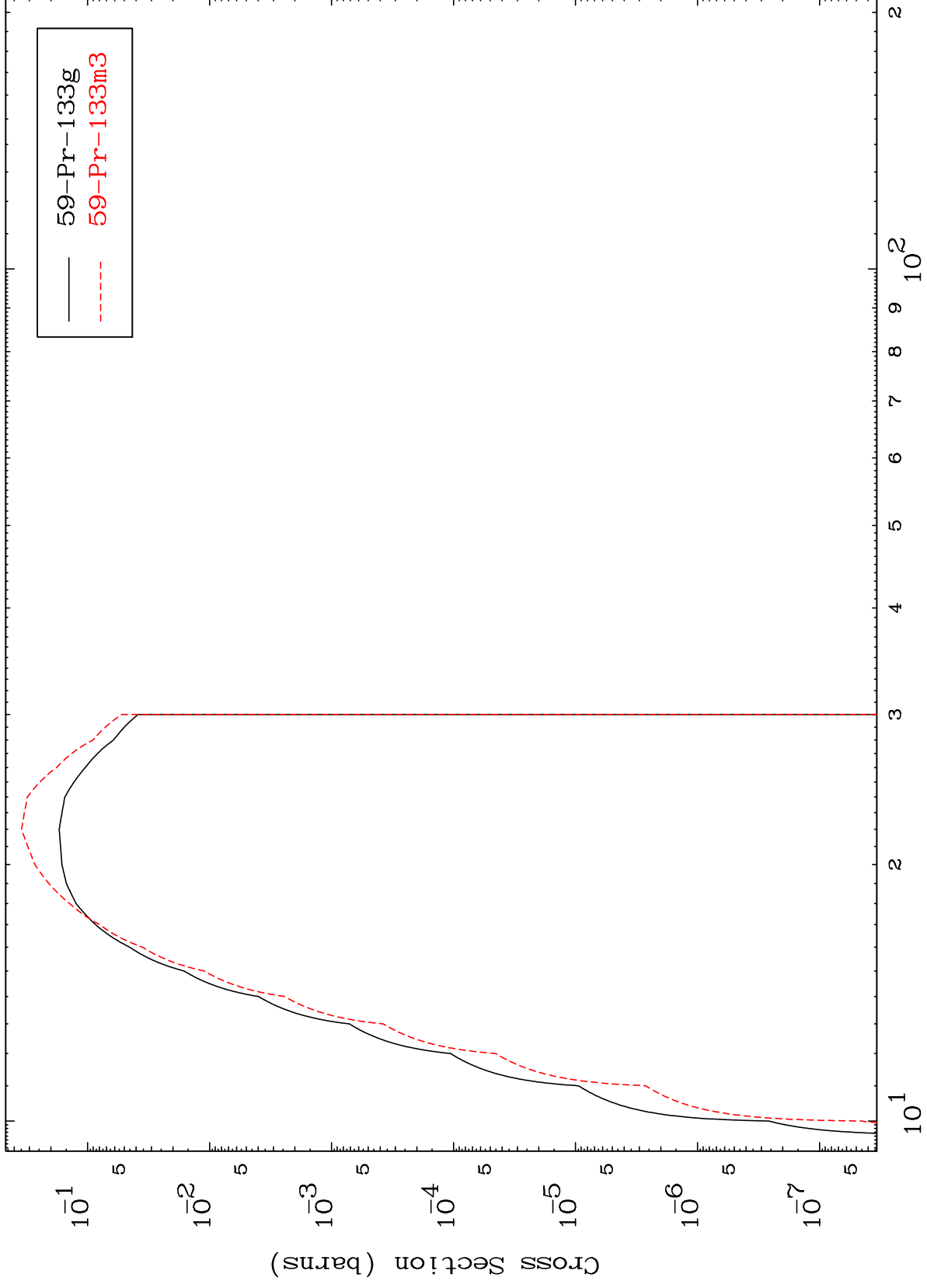
Incident Energy (MeV)

57-La-130

MAT 5701

Inelastic  
Radionuclide Production Cross Section

57-La-130



11

Incident Energy (MeV)

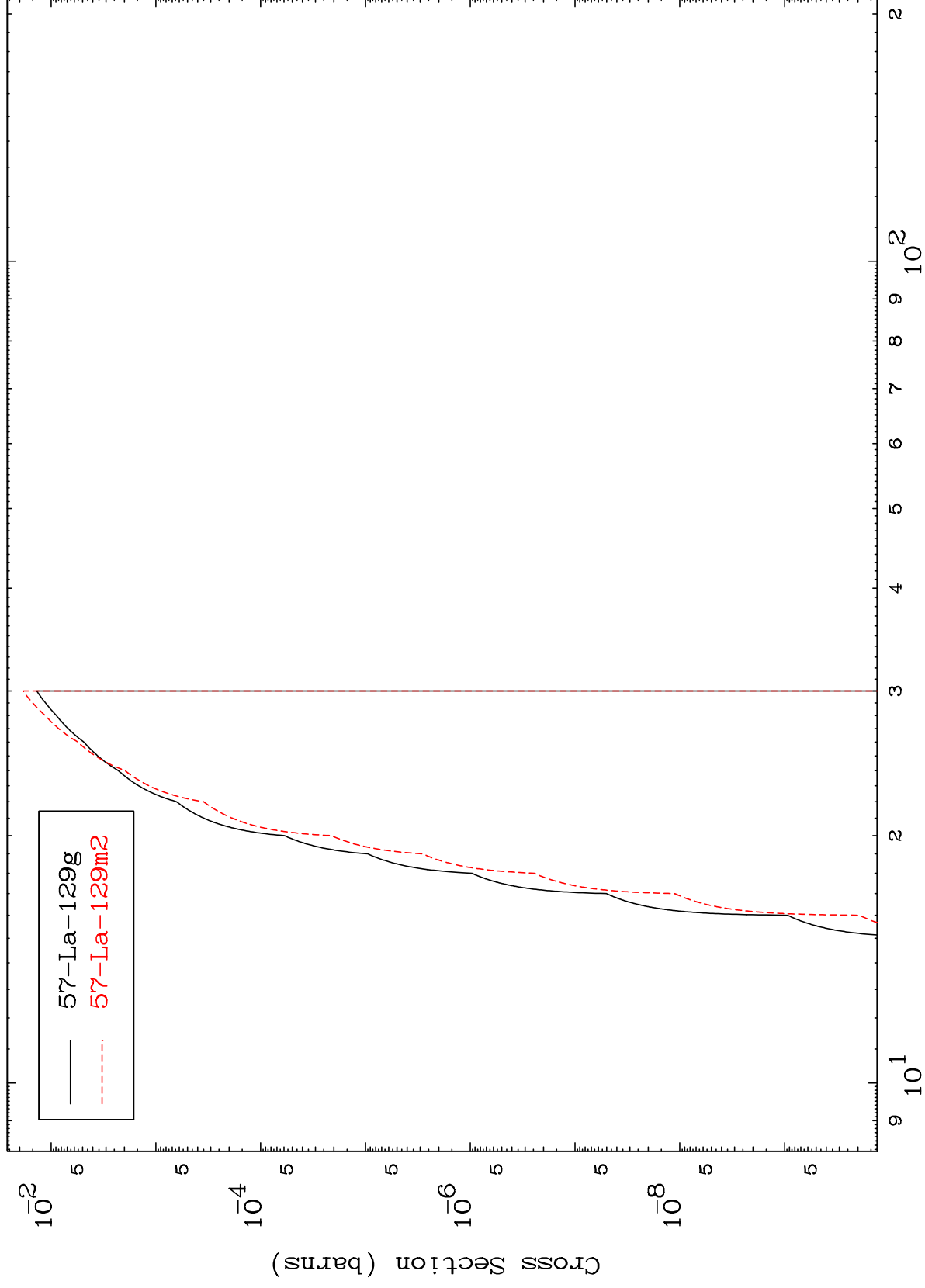
57-La-130

MAT 5701

57-La-130

(n,n')  $\alpha$

Radionuclide Production Cross Section



57-La-130

Incident Energy (MeV)

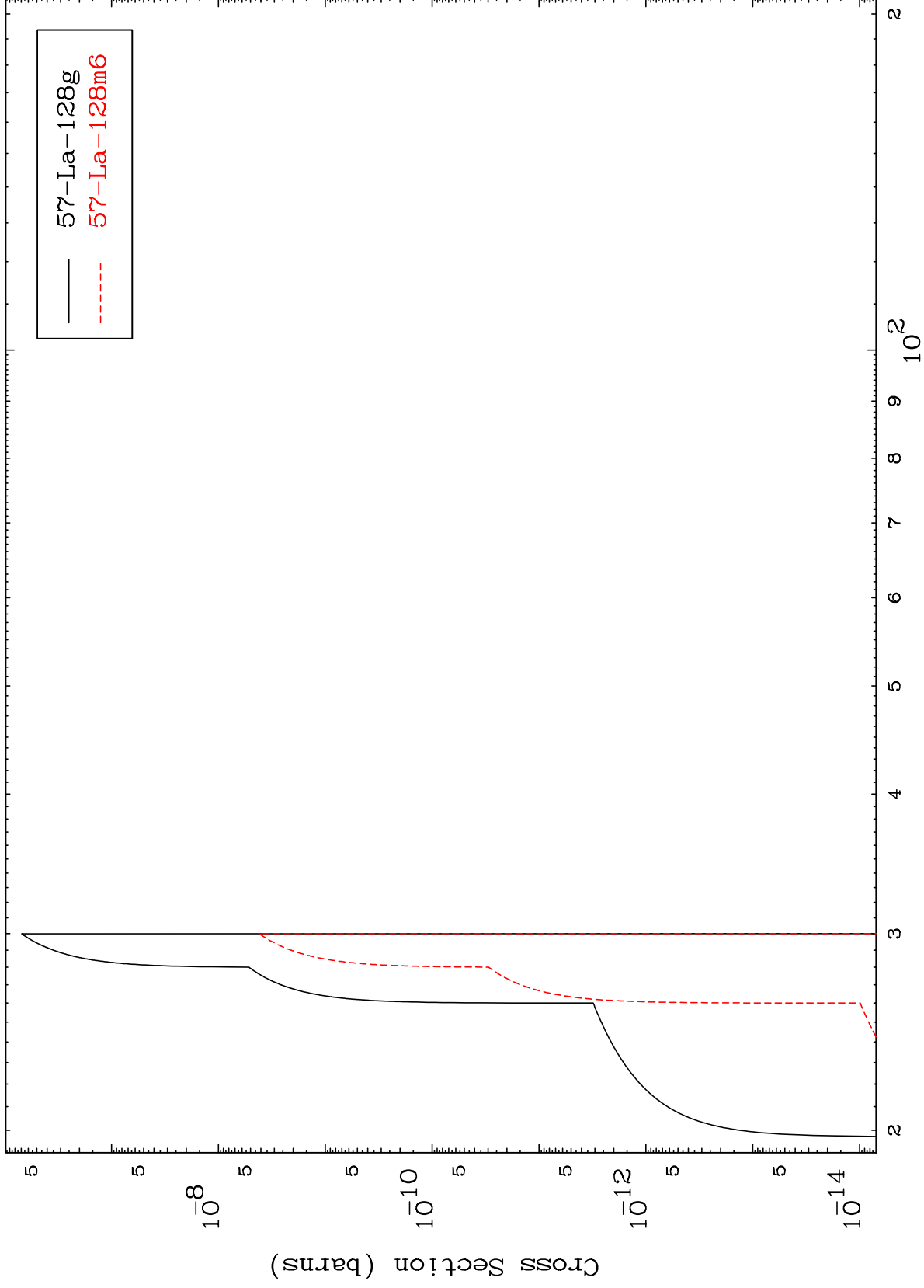
12

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$(n,2n) \alpha$

57-La-130

Radionuclide Production Cross Section



13

Incident Energy (MeV)

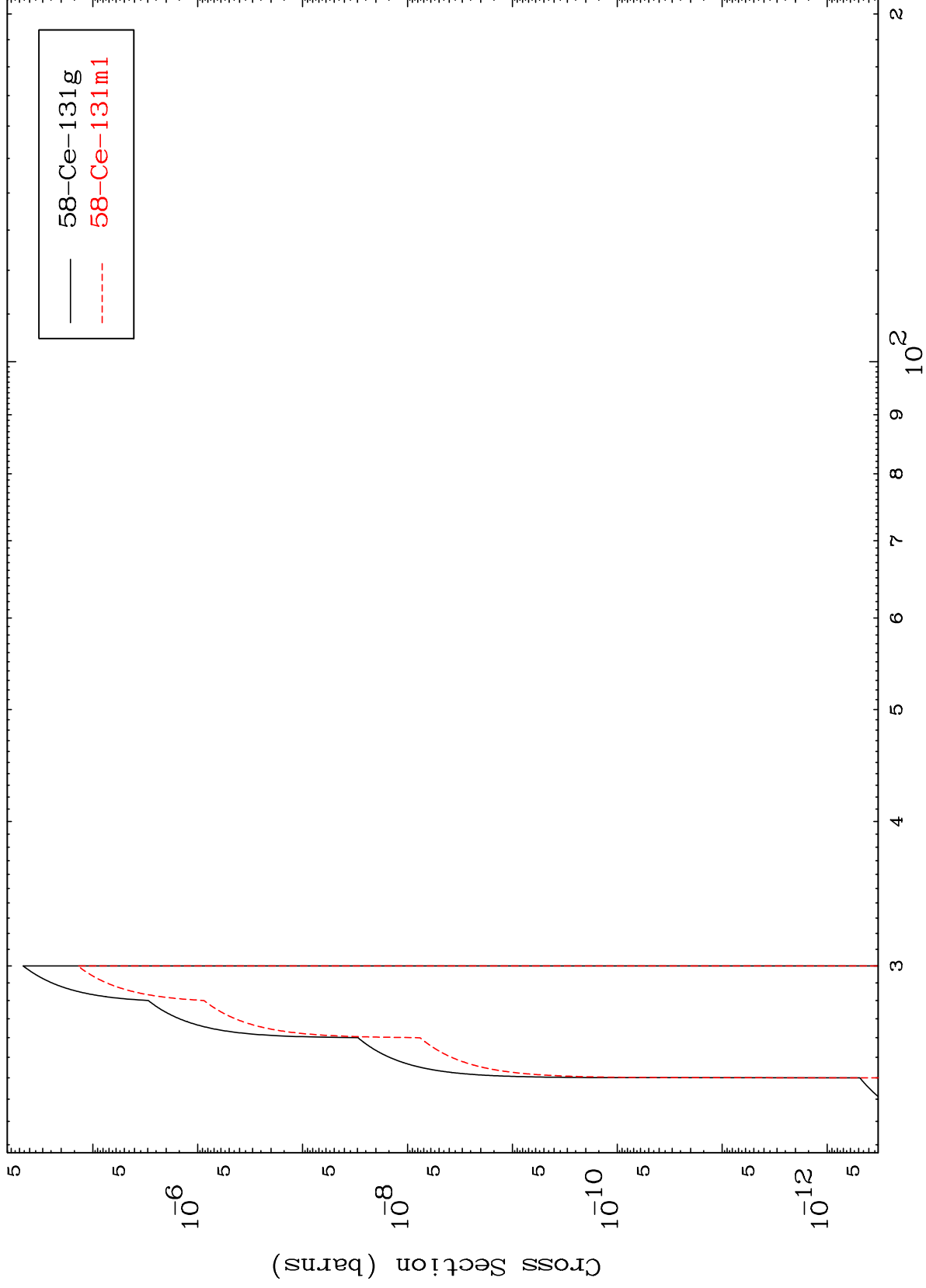
57-La-130

MAT 5701

(n,n') d

57-La-130

Radionuclide Production Cross Section



14

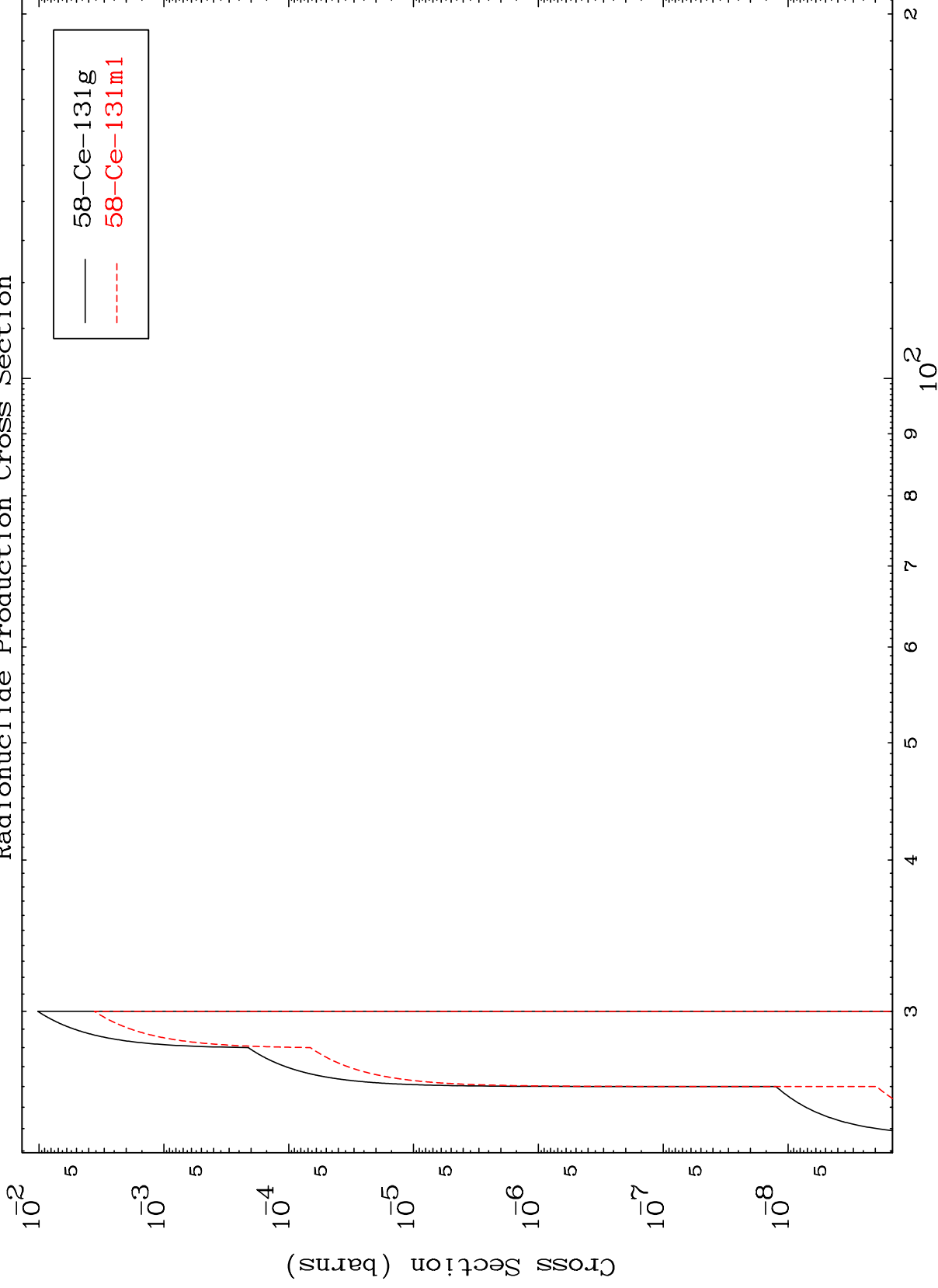
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,2n) p  
Radionuclide Production Cross Section



15

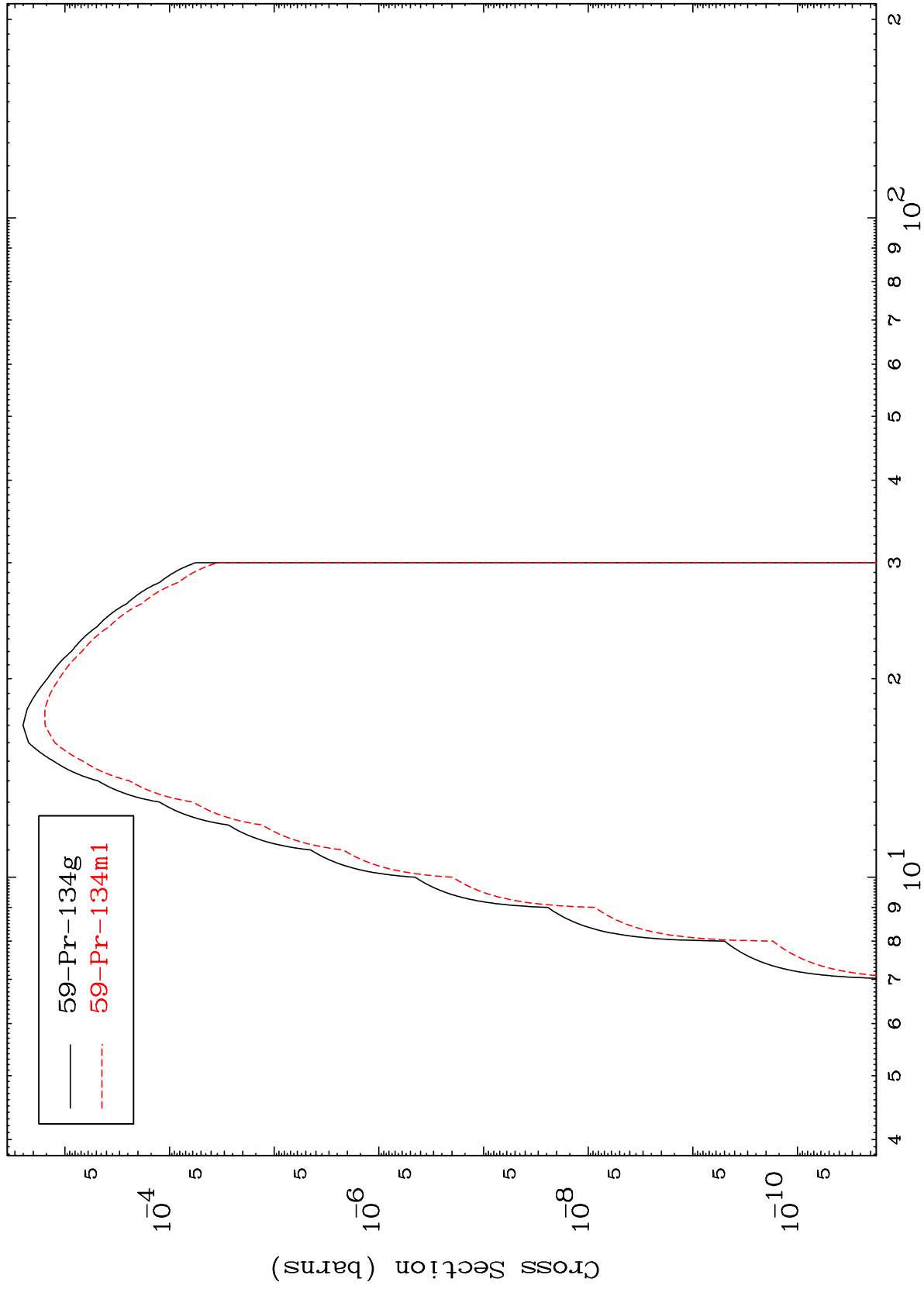
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,γ)  
Radionuclide Production Cross Section



57-La-130

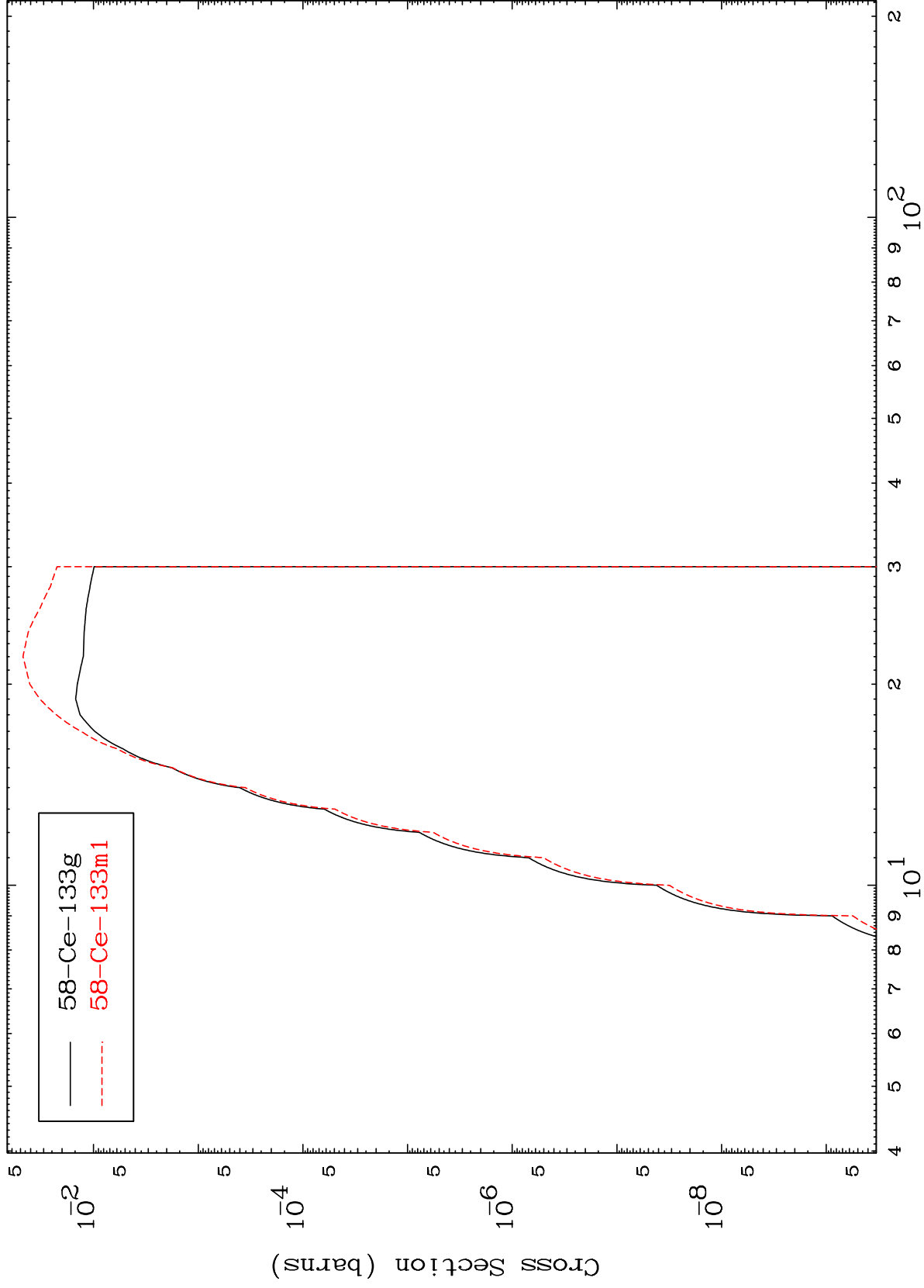
Incident Energy (MeV)

16

MAT 5701

57-La-130

(n,p)  
Radionuclide Production Cross Section



17

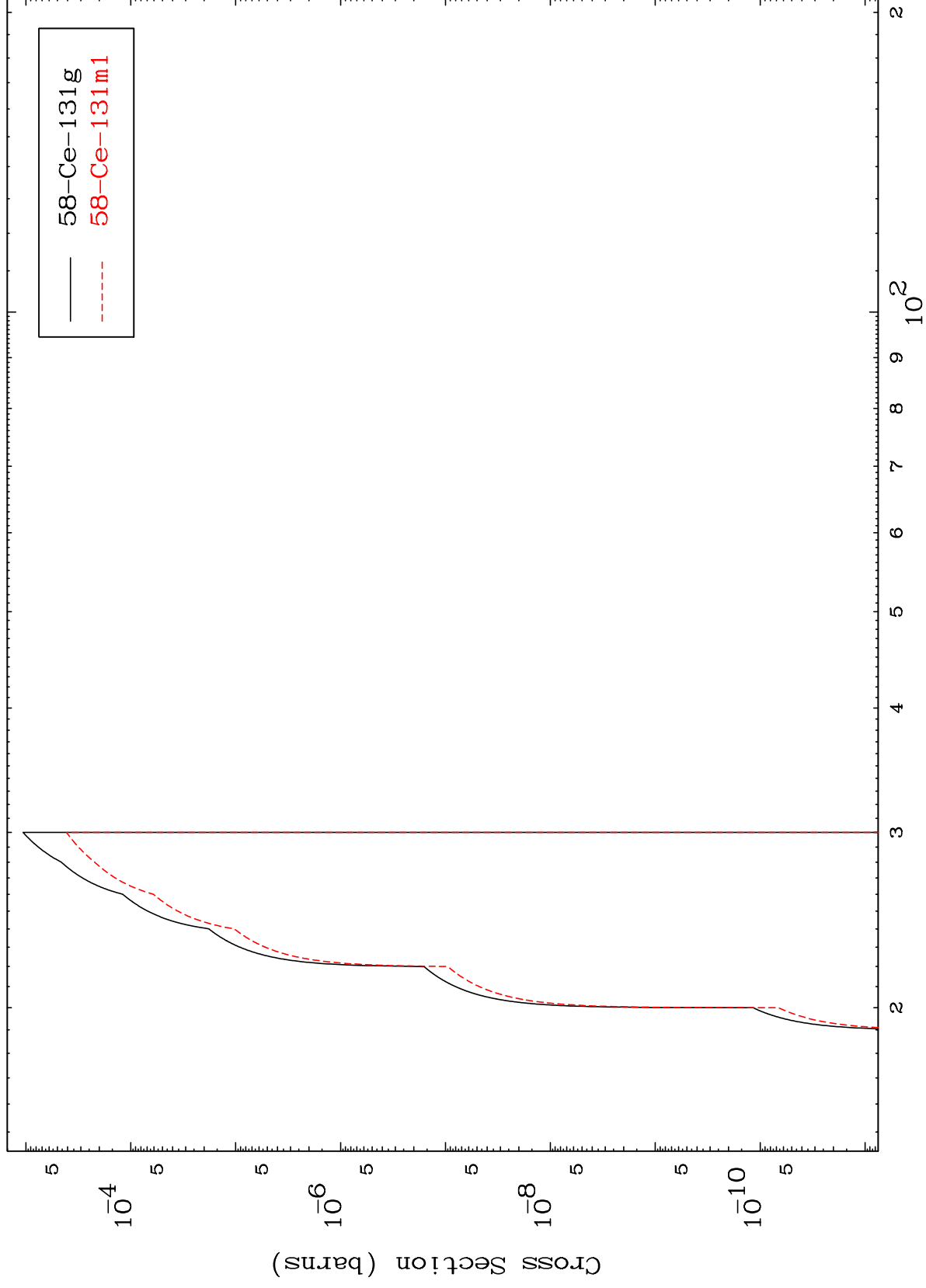
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,t)  
Radionuclide Production Cross Section



18

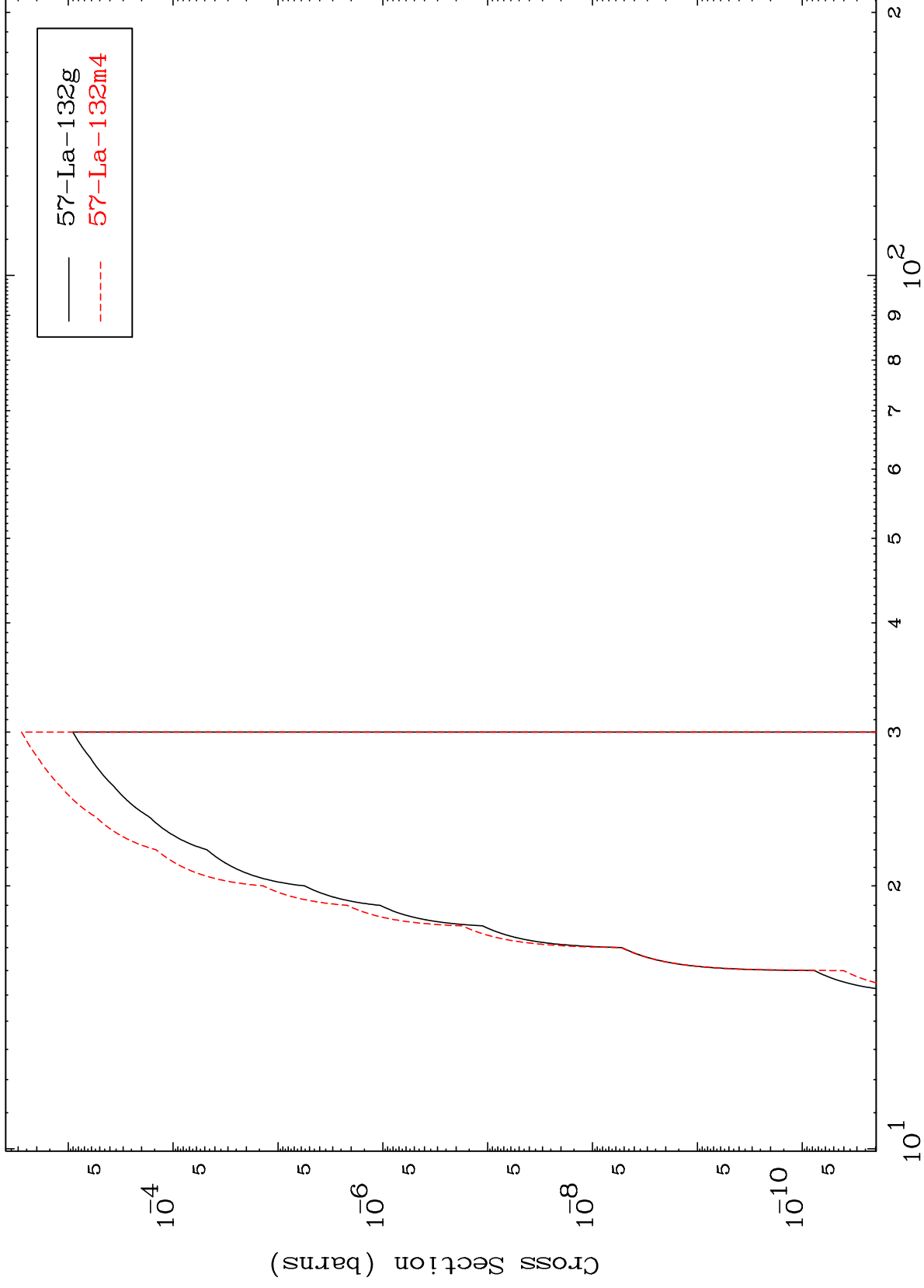
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,2p)  
Radionuclide Production Cross Section



57-La-130

Incident Energy (MeV)

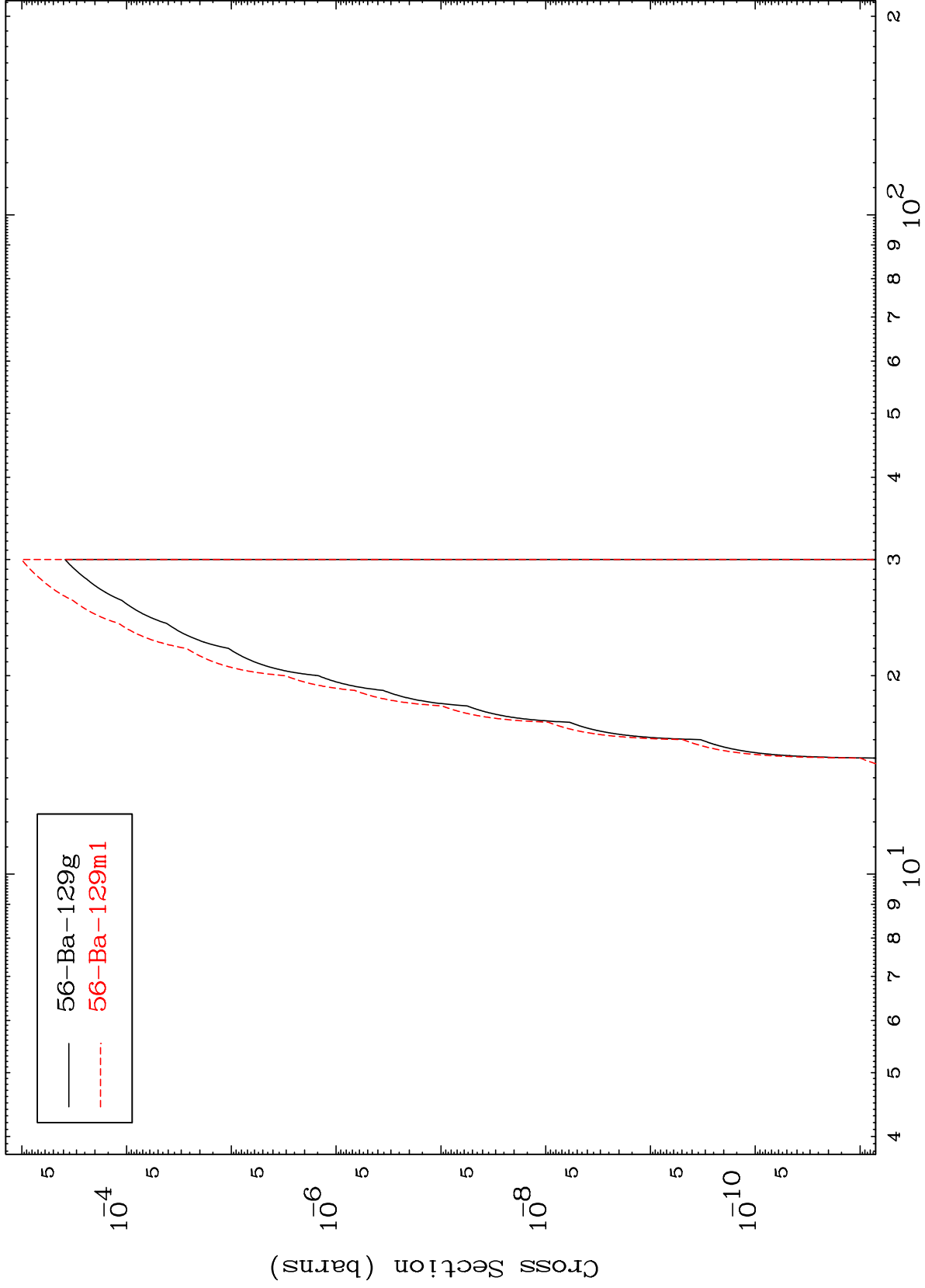
19

MAT 5701

(n,p)  $\alpha$

57-La-130

Radionuclide Production Cross Section



20

Incident Energy (MeV)

57-La-130