

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

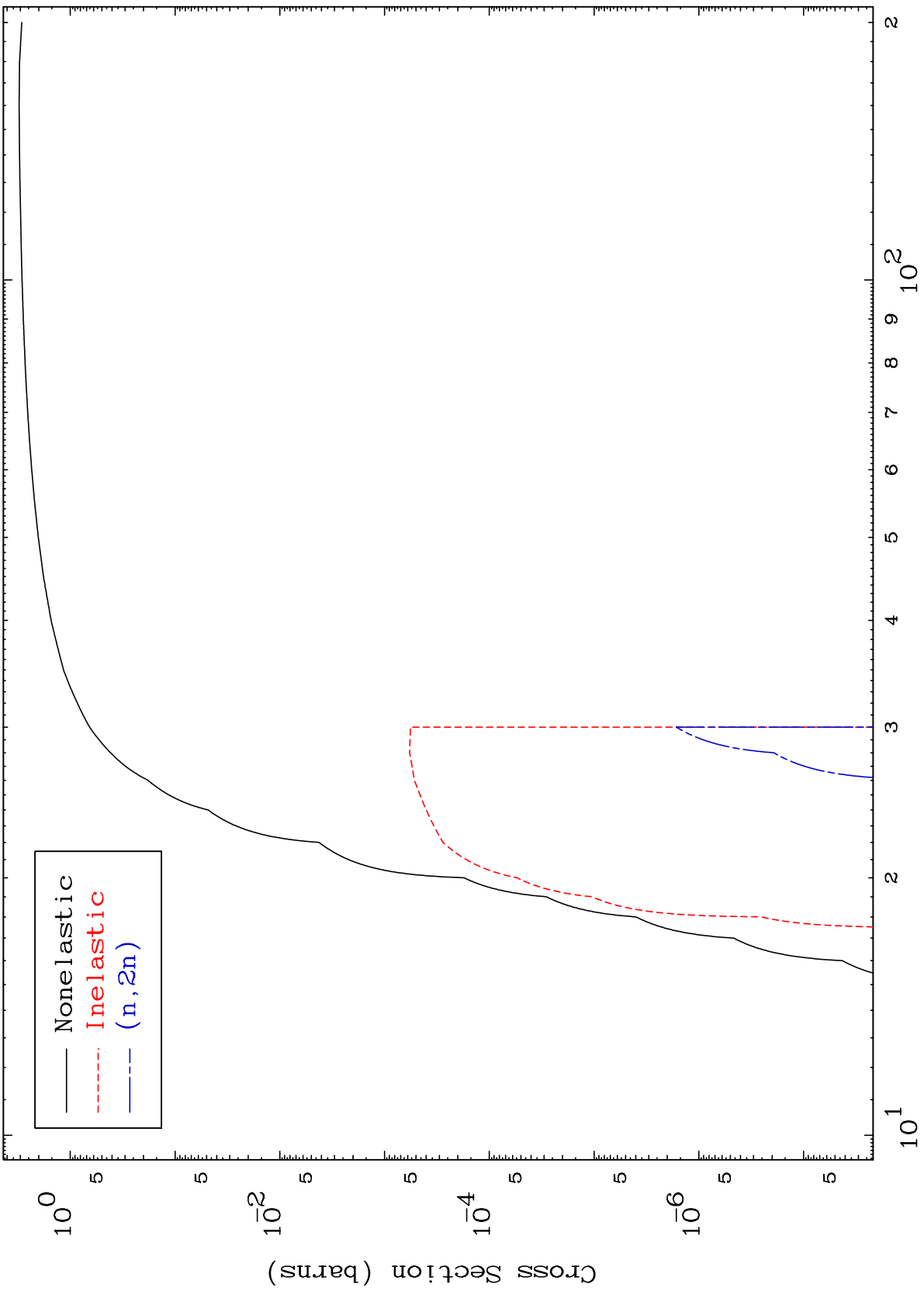
Press Mouse Button to Start

MAT 9984

$\alpha$  Major

103-Lr-255

0 Kelvin Cross Sections



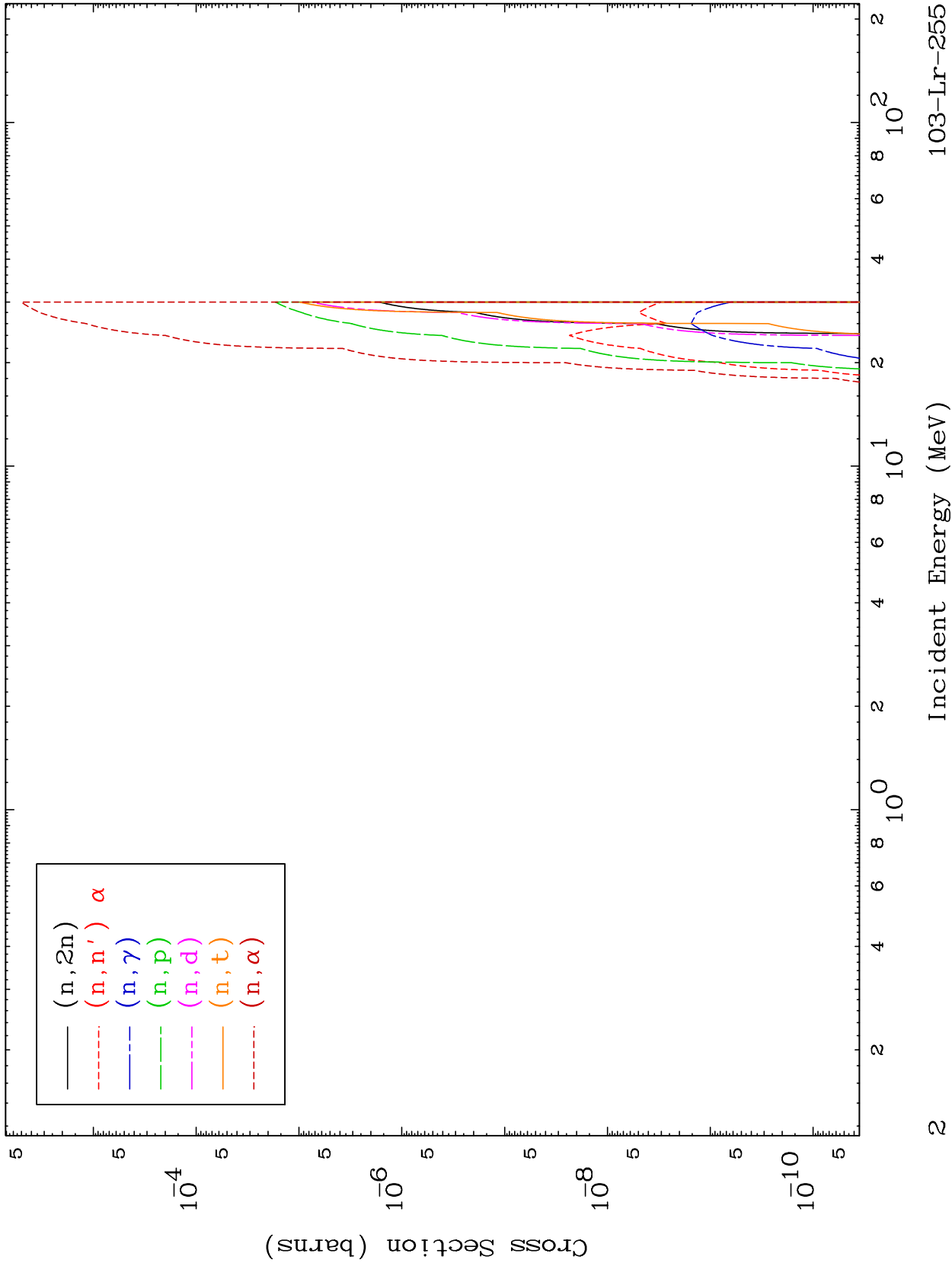
Incident Energy (MeV)

103-Lr-255

MAT 9984

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

103-Lr-255

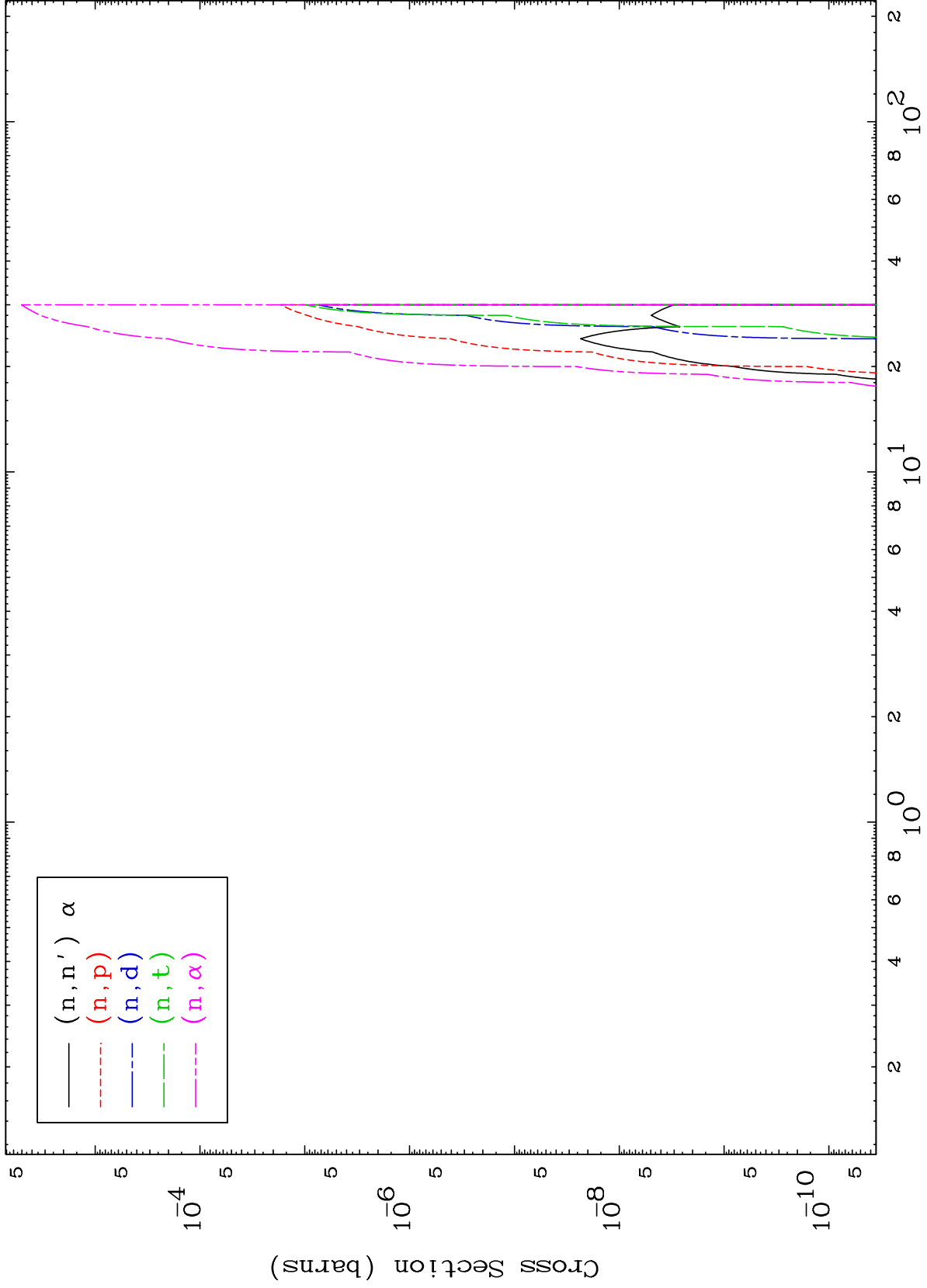


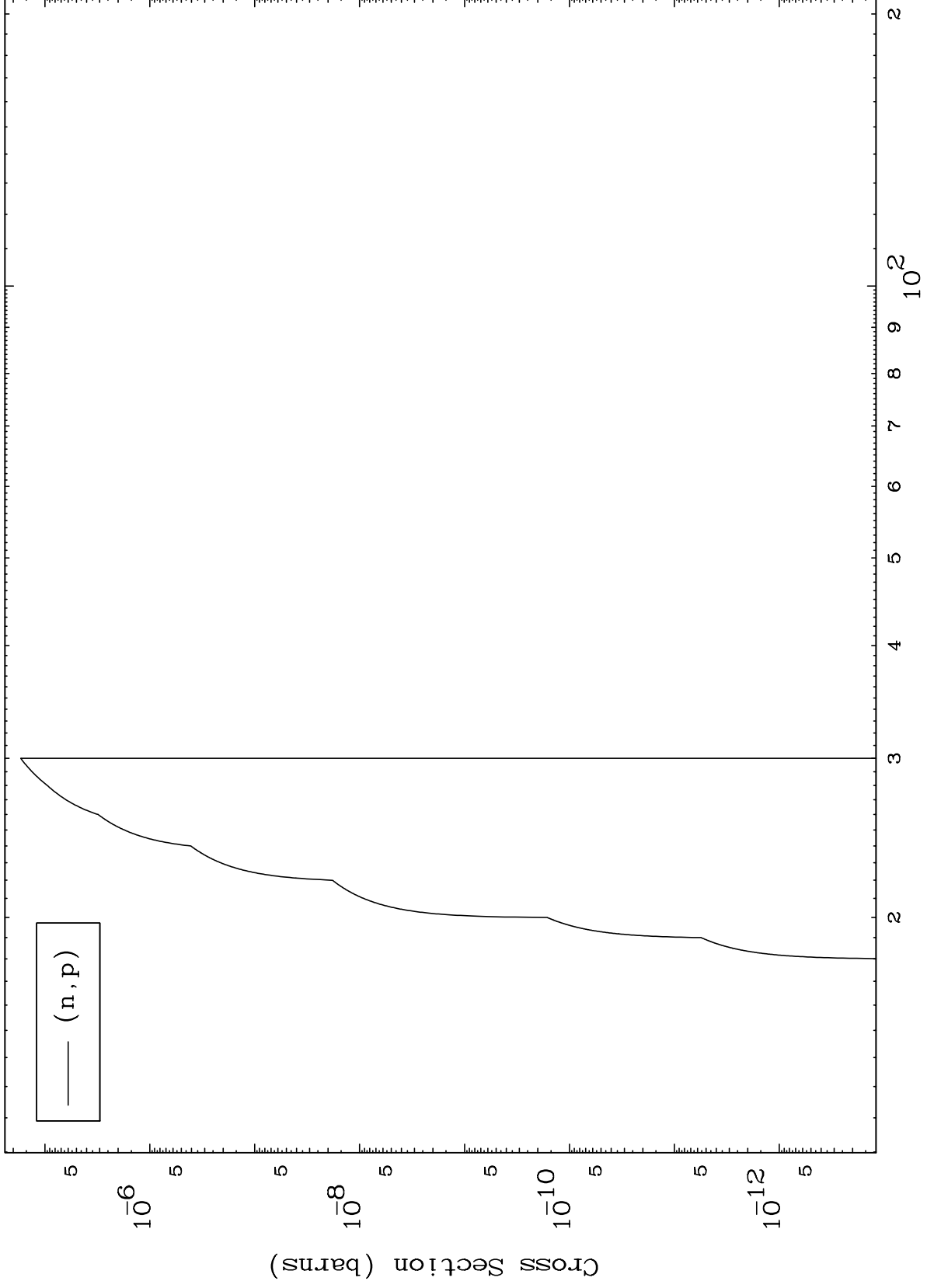
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$\alpha$  Charged Particle

103-Lr-255

0 Kelvin Cross Sections

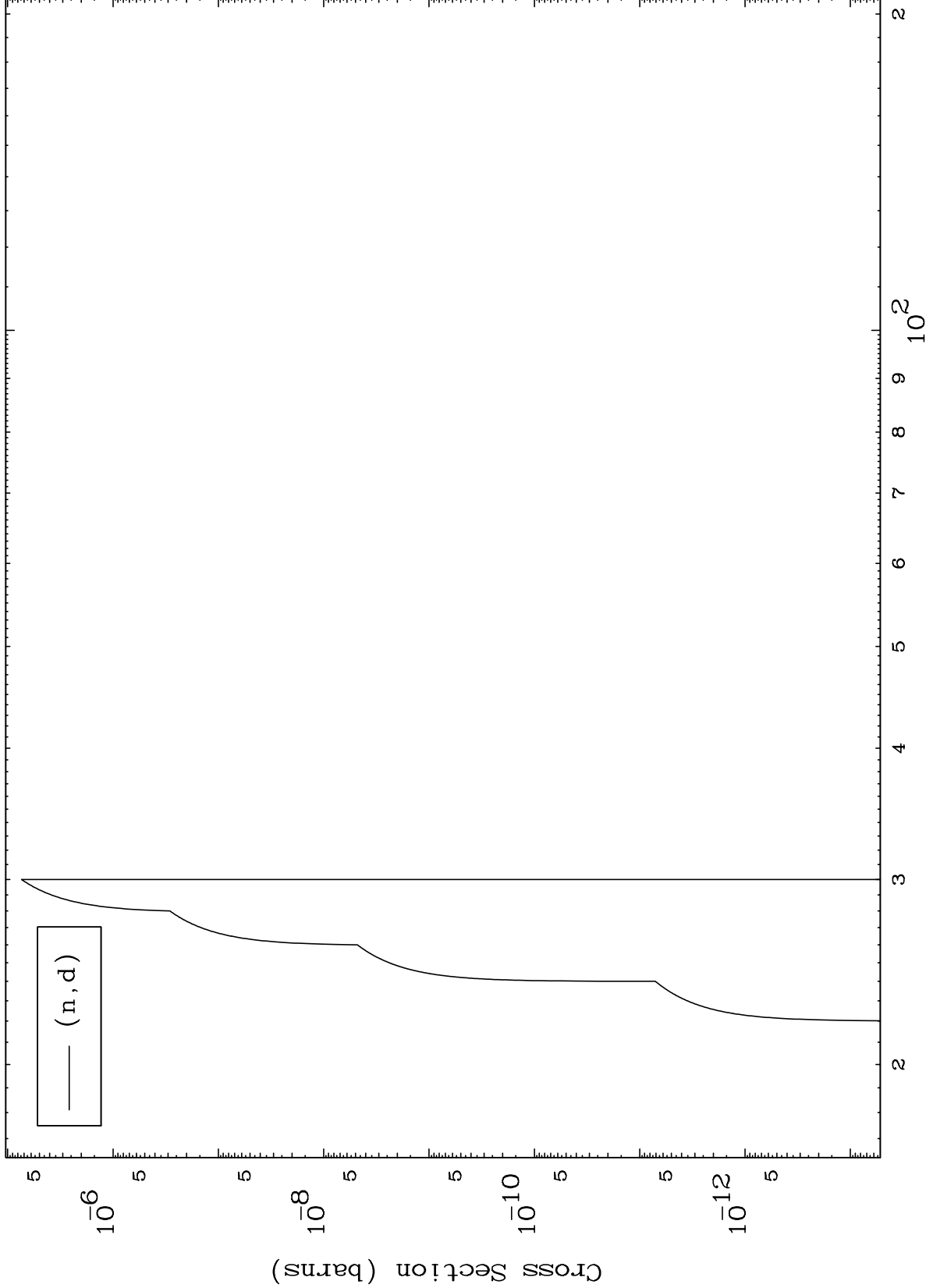




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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

103-Lr-255



5

Incident Energy (MeV)

103-Lr-255

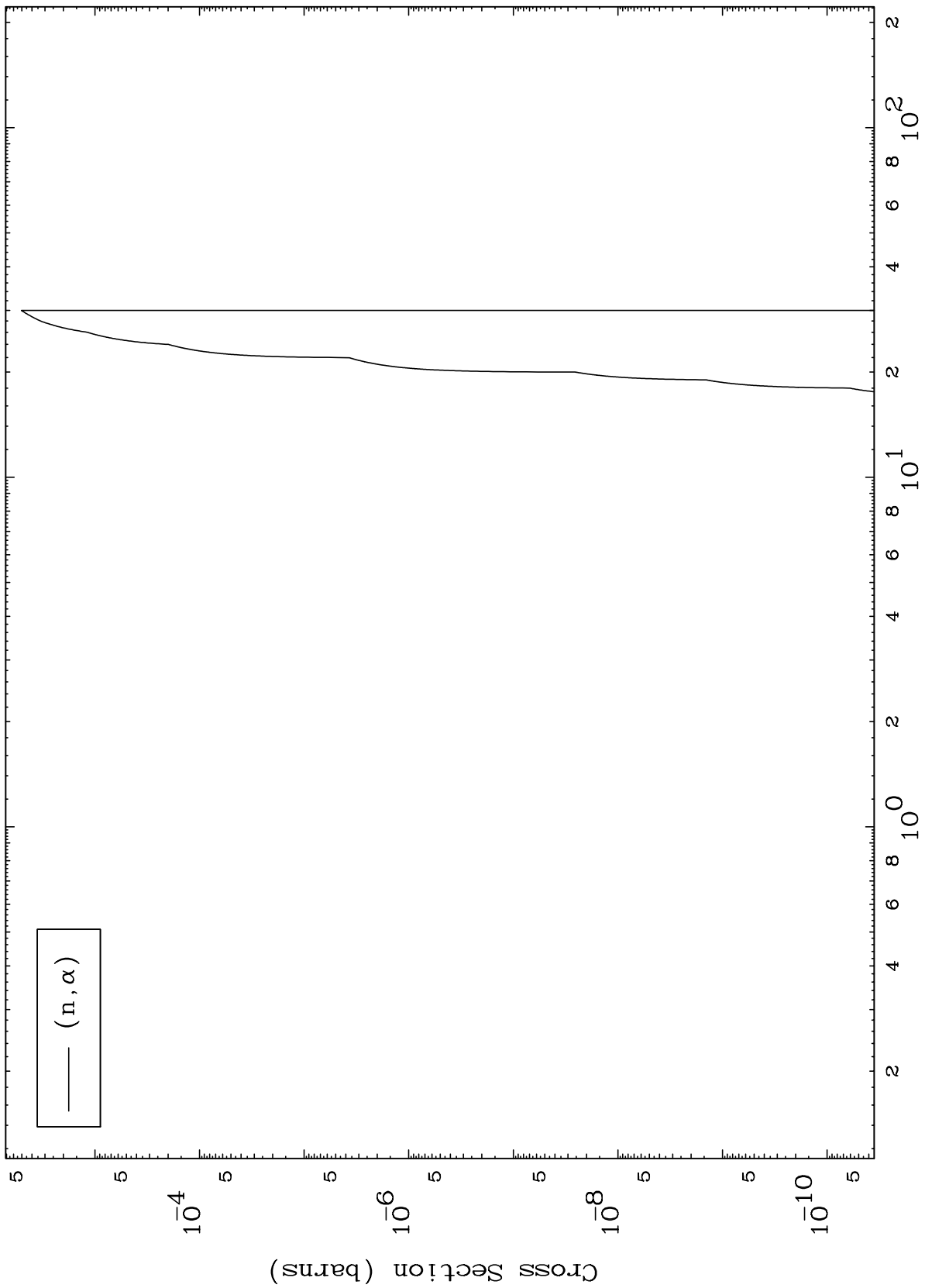


MAT 9984

( $\alpha, \alpha$ ) Levels

103-Lr-255

0 Kelvin Cross Sections

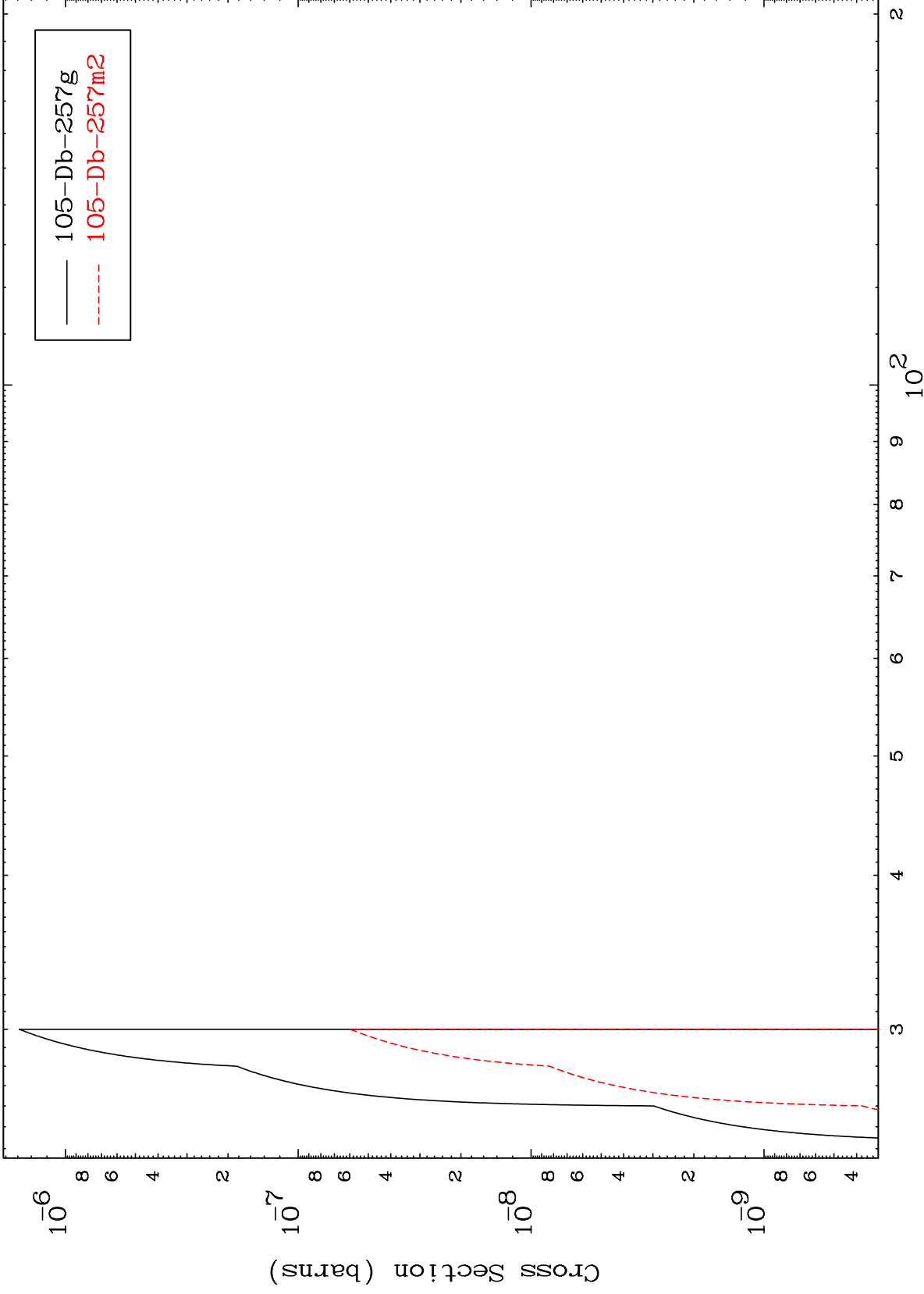


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(n,2n)

103-Lr-255

Radionuclide Production Cross Section



105-Db-257g  
105-Db-257m2

8

Incident Energy (MeV)

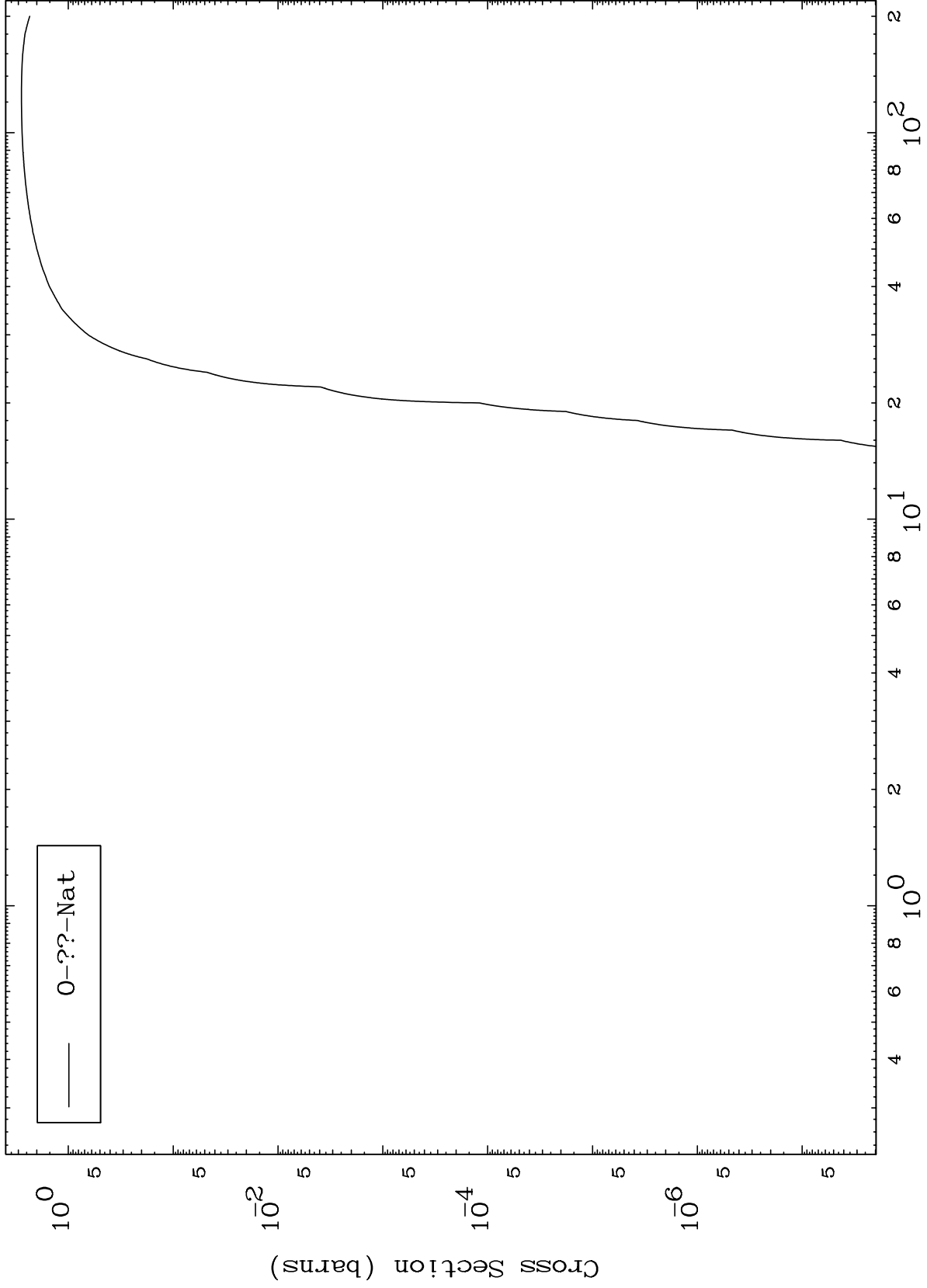
103-Lr-255

MAT 9984

Fission

103-Lr-255

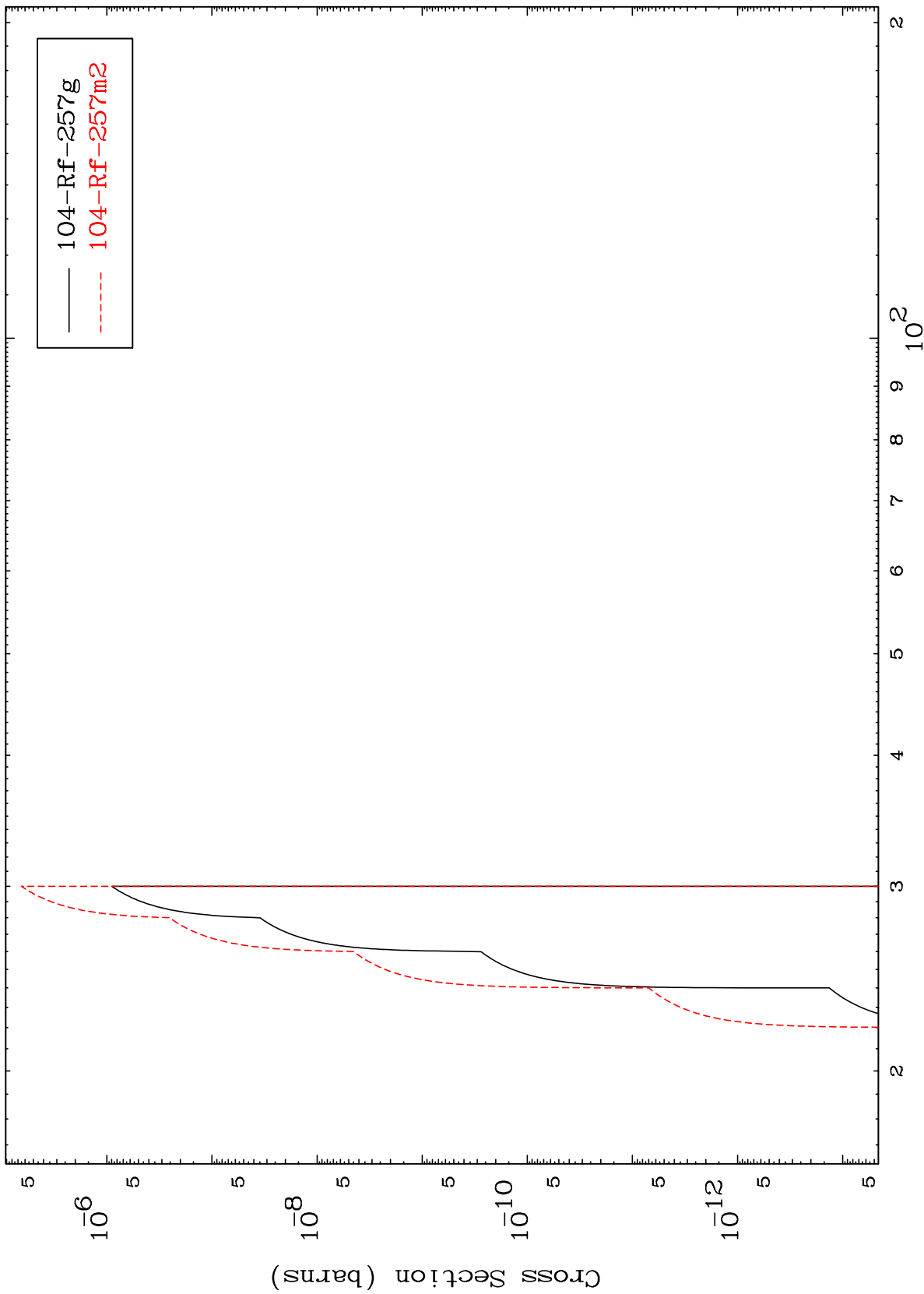
Radionuclide Production Cross Section



MAT 9984

103-Lr-255

(n,d)  
Radionuclide Production Cross Section



103-Lr-255

Incident Energy (MeV)

10

MAT 9984

103-Lr-255

(n,  $\alpha$ )  
Radionuclide Production Cross Section

