

Program EVALPLOT  
(Version 2021-1)

by

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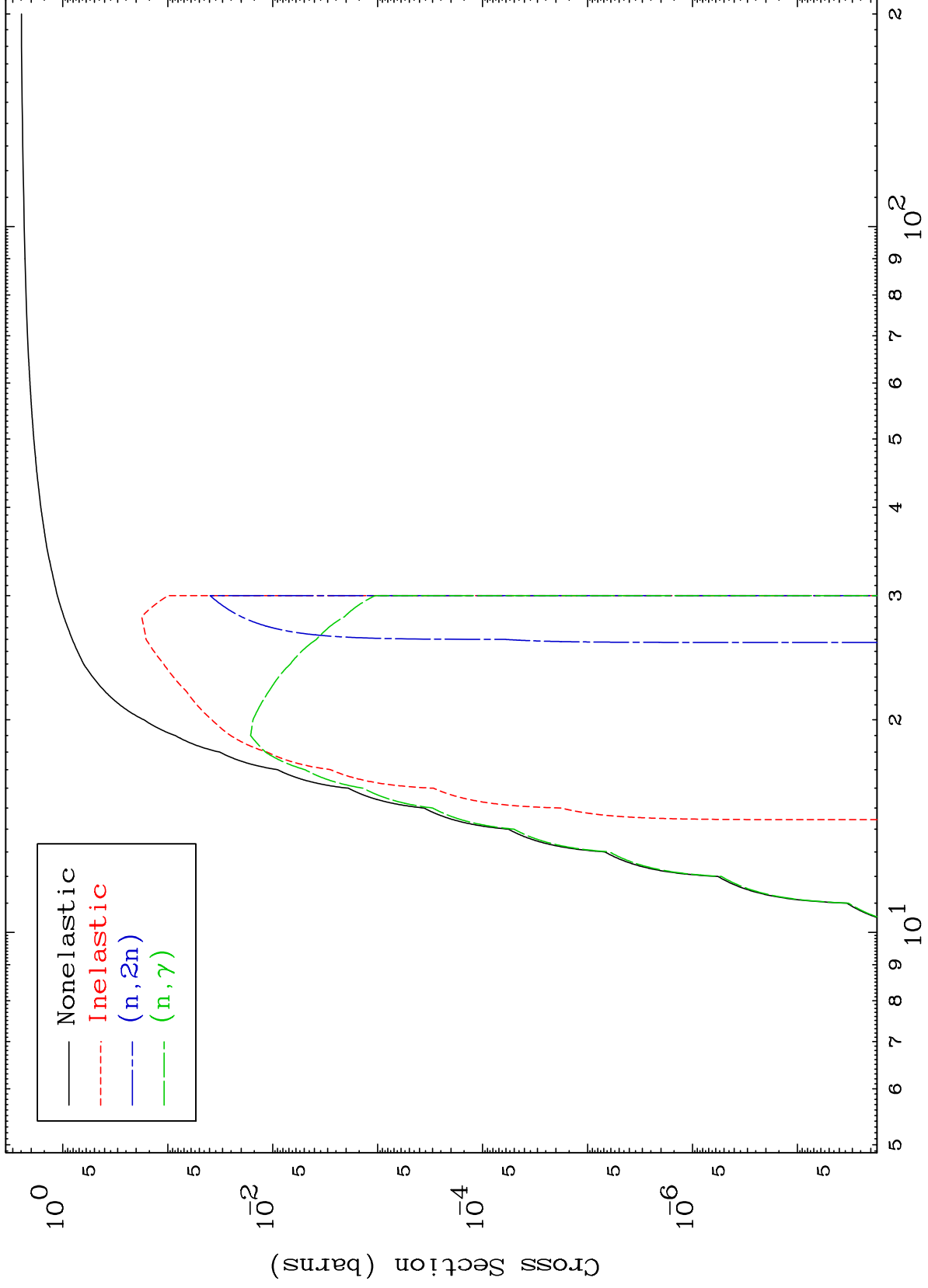
Press Mouse Button to Start

MAT 7074

0 Kelvin Cross Sections

$\alpha$  Major

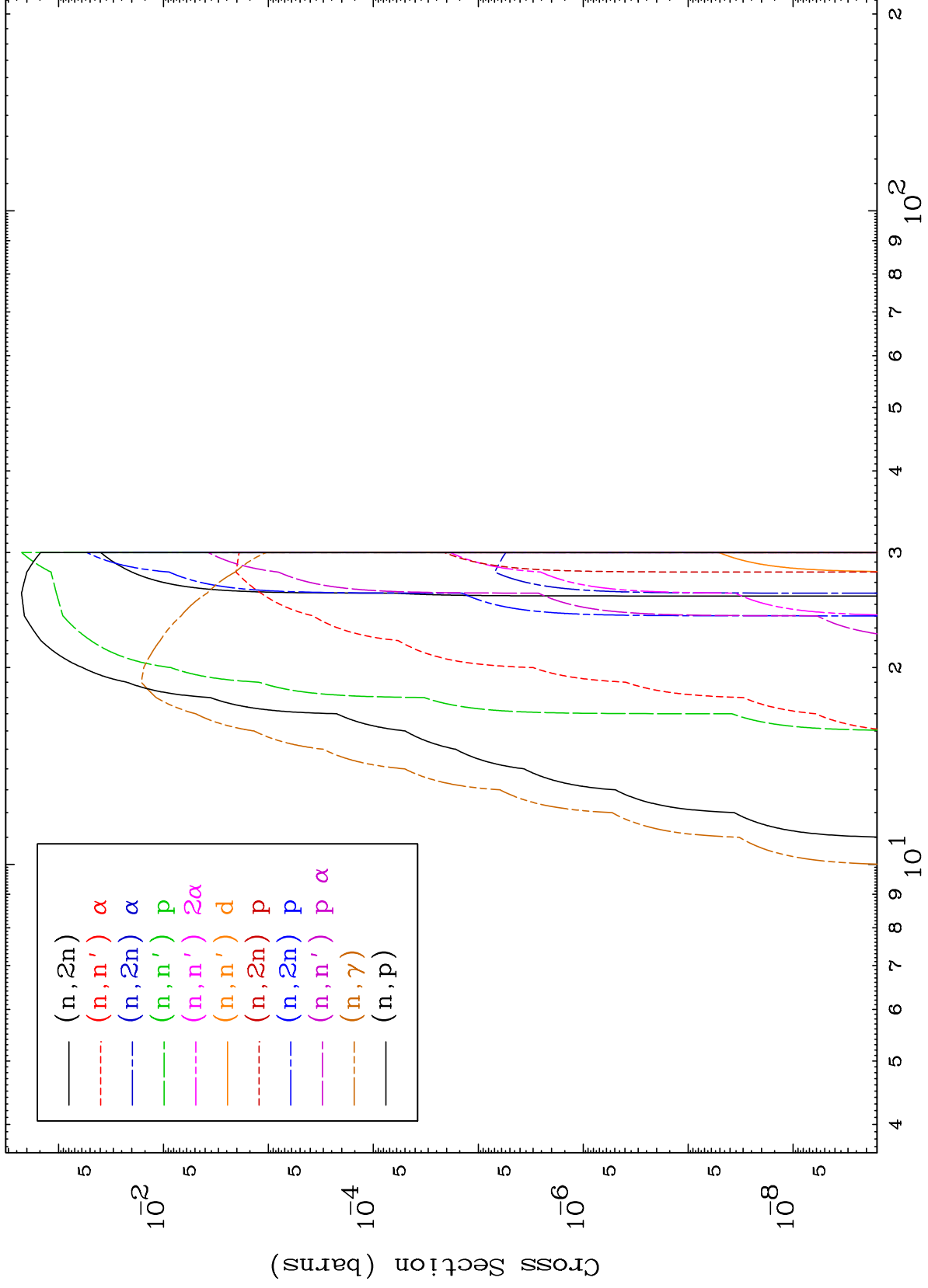
71-Lu-158



1

Incident Energy (MeV)

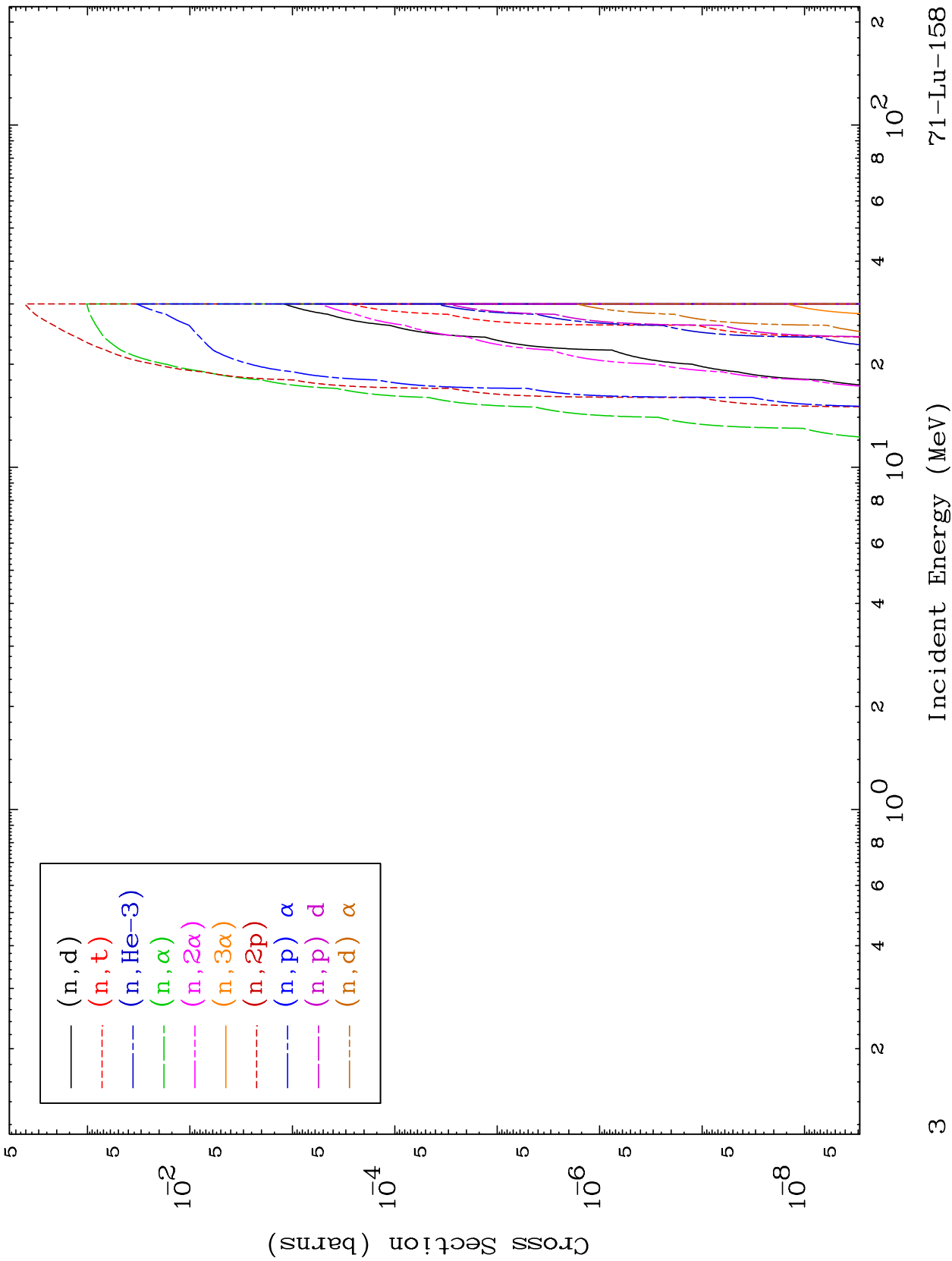
71-Lu-158

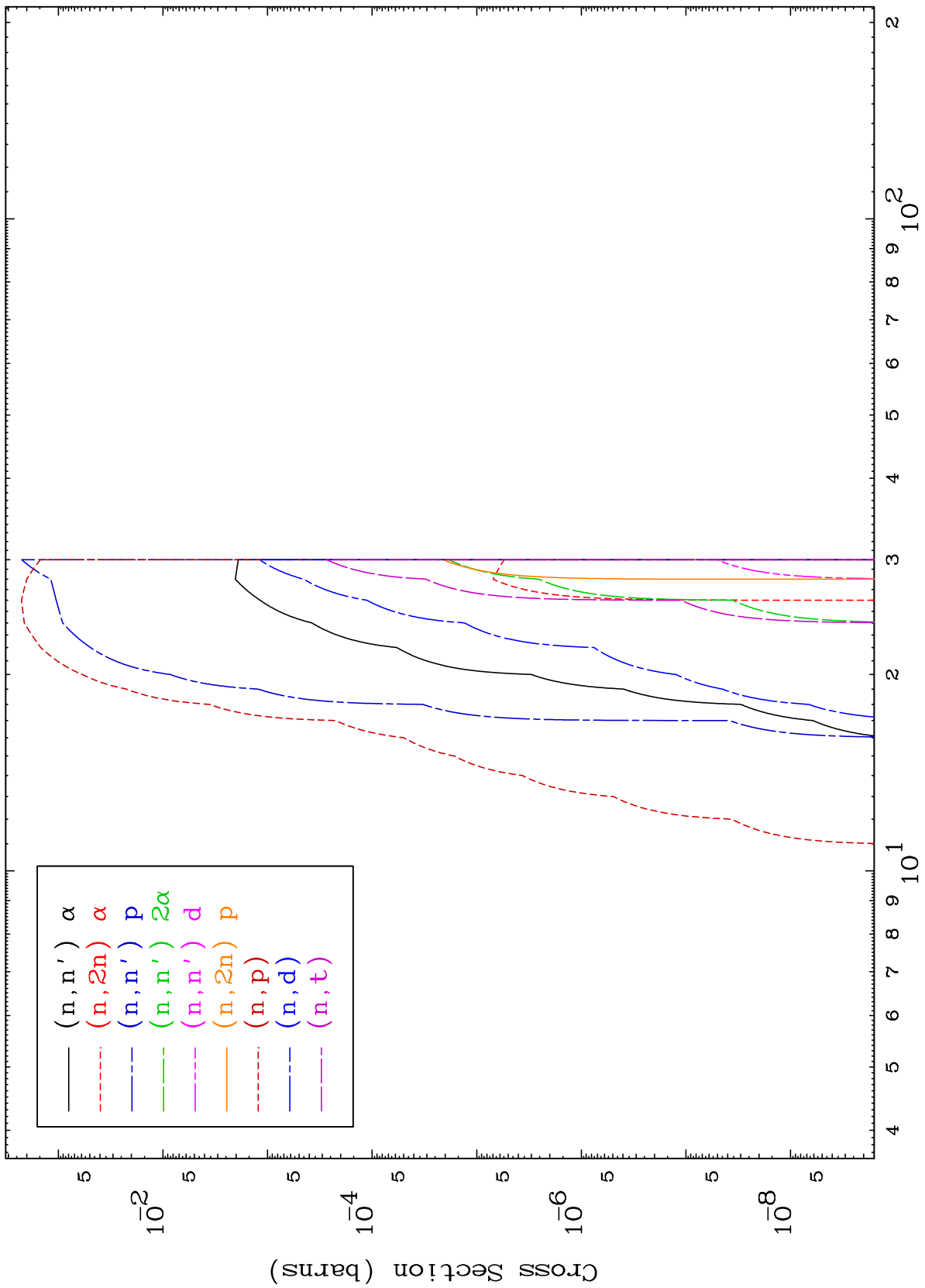


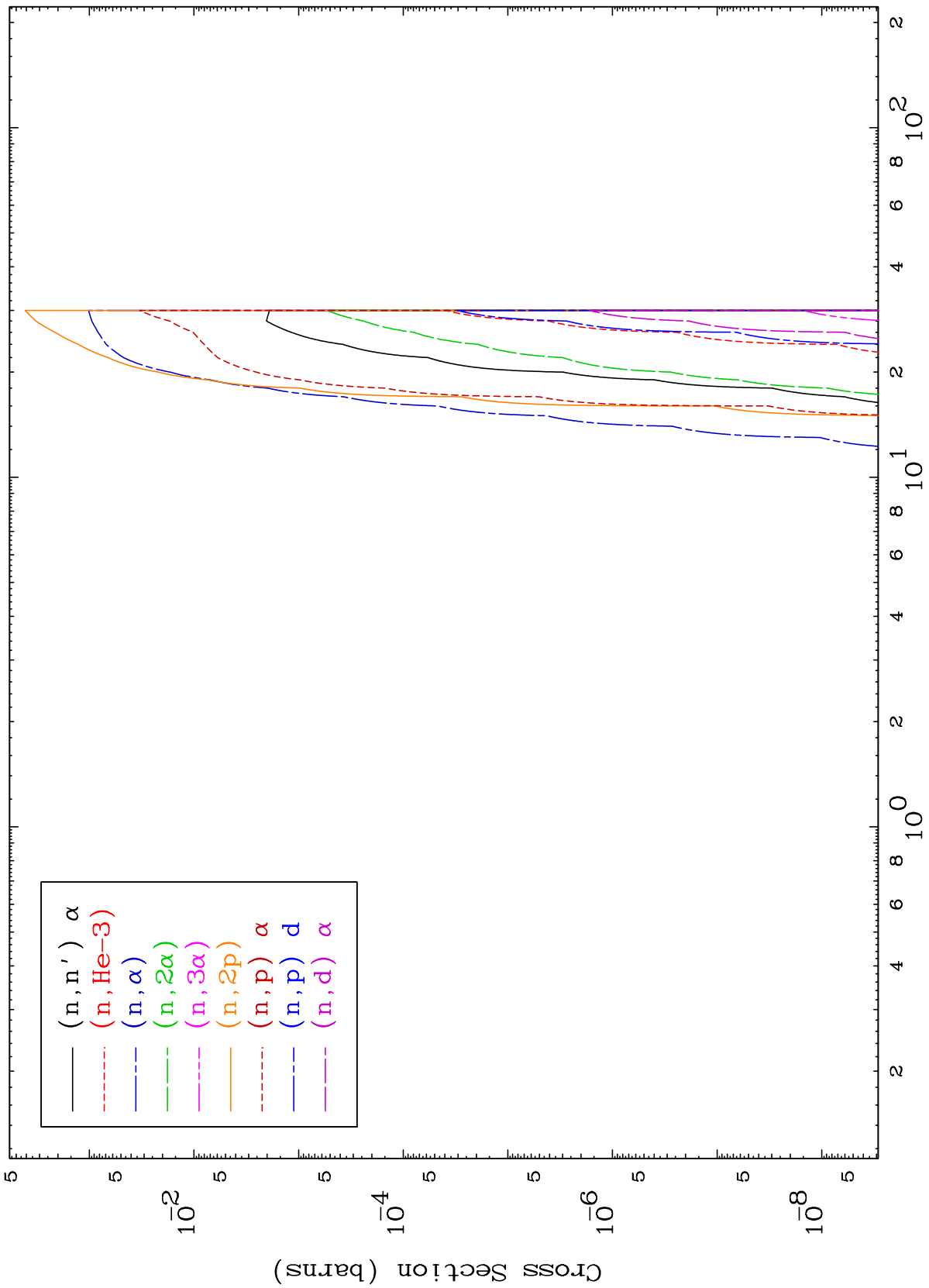
MAT 7074

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-158





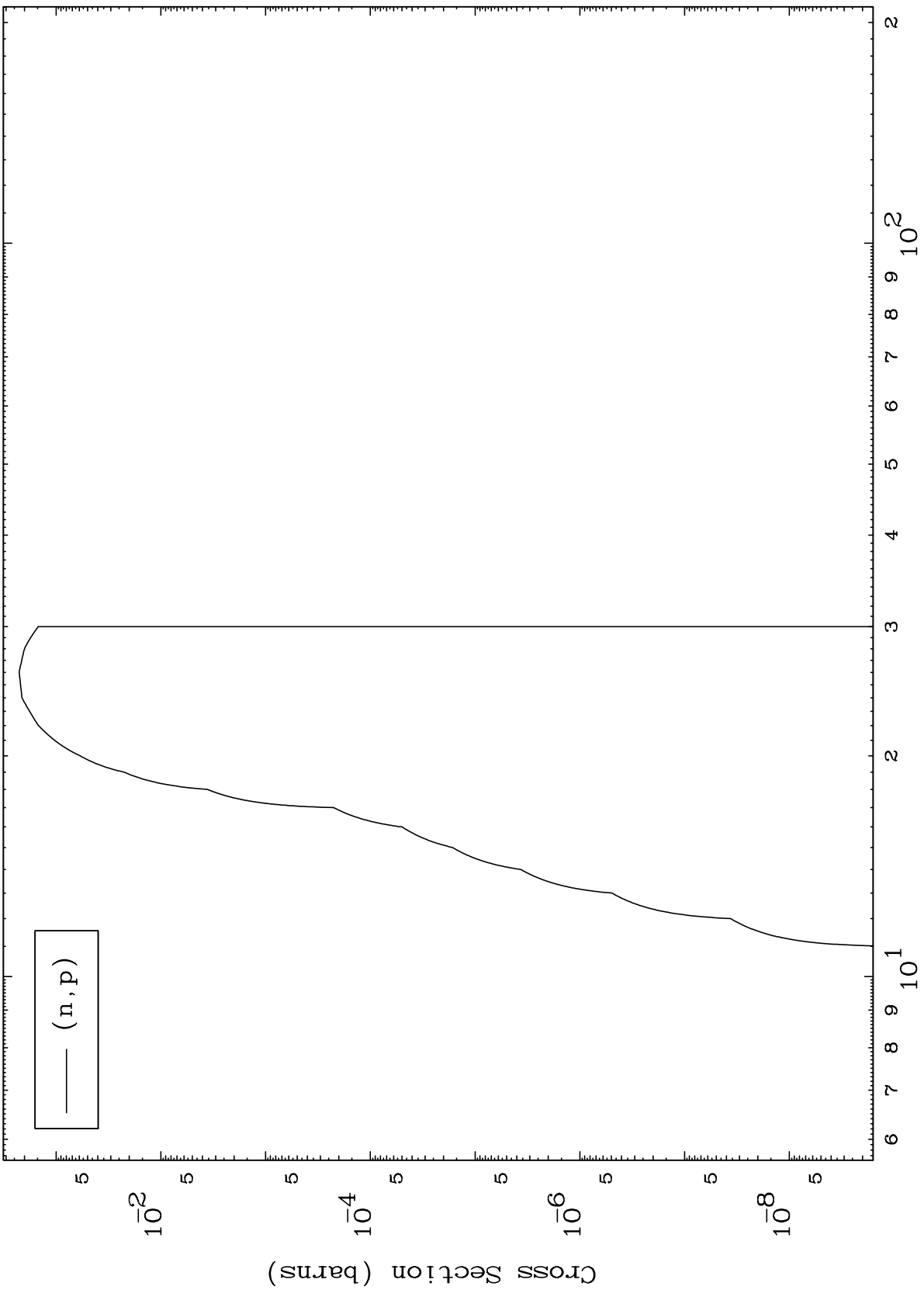


MAT 7074

( $\alpha, p$ ) Levels

71-Lu-158

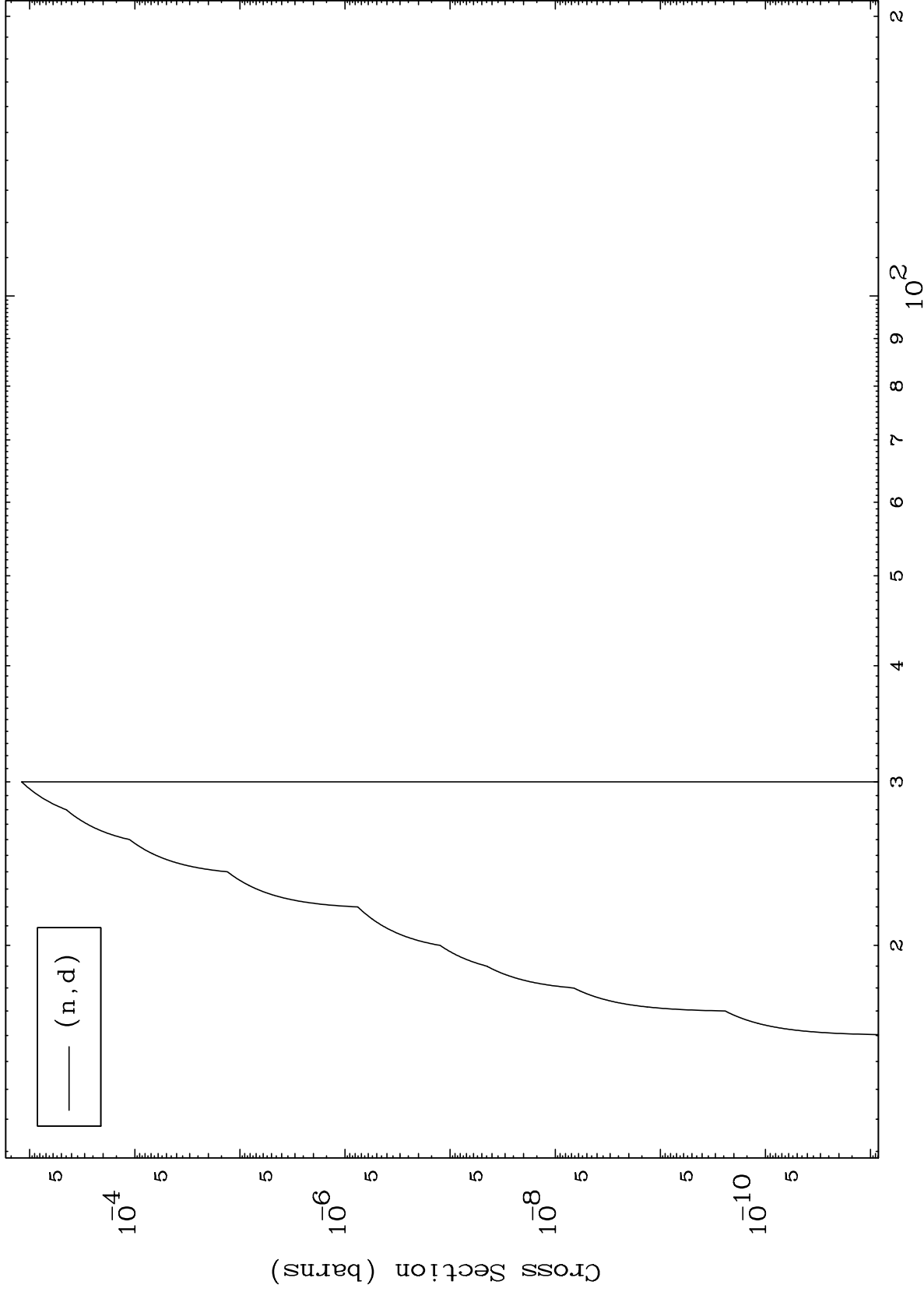
0 Kelvin Cross Sections



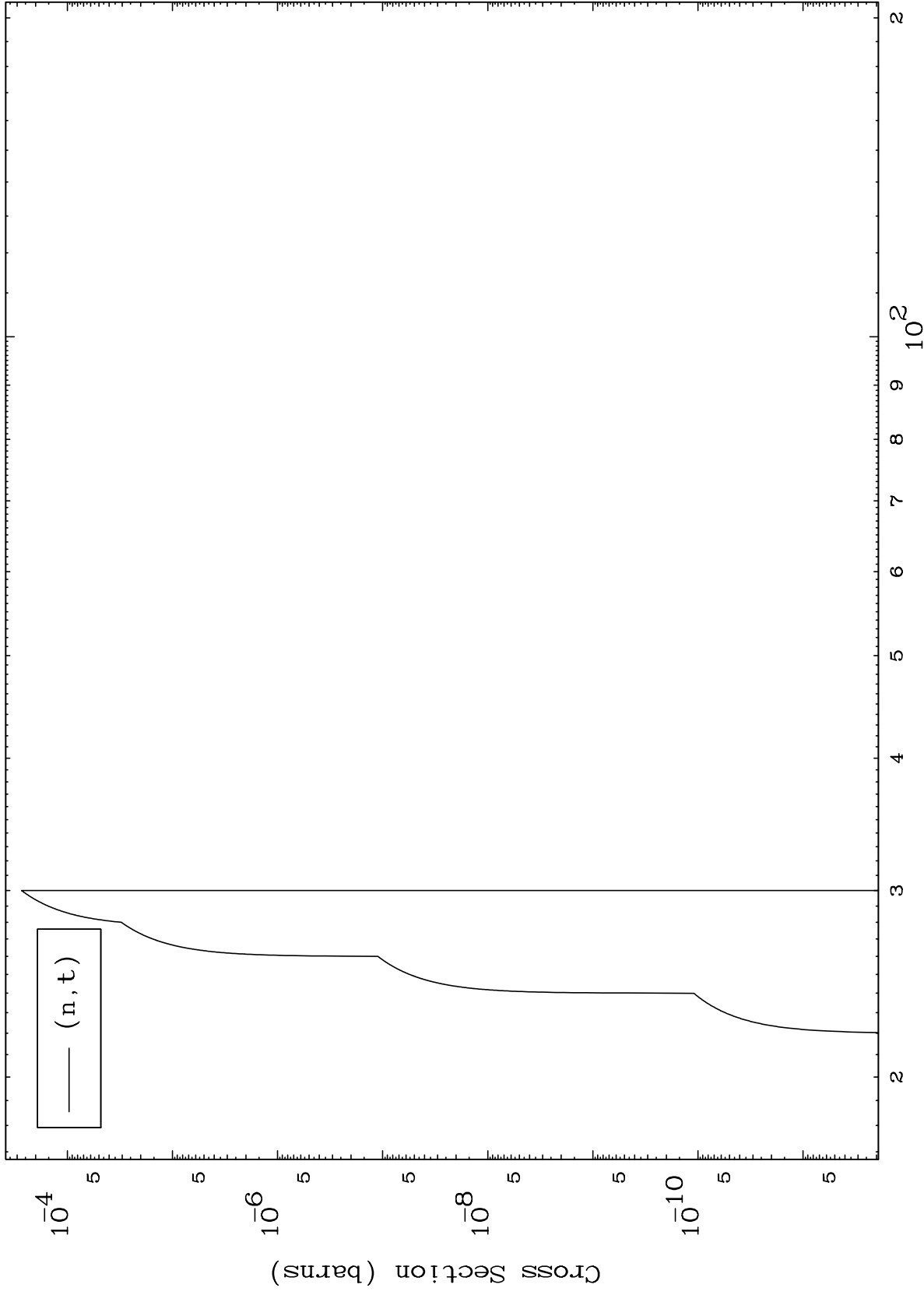
6

Incident Energy (MeV)

71-Lu-158



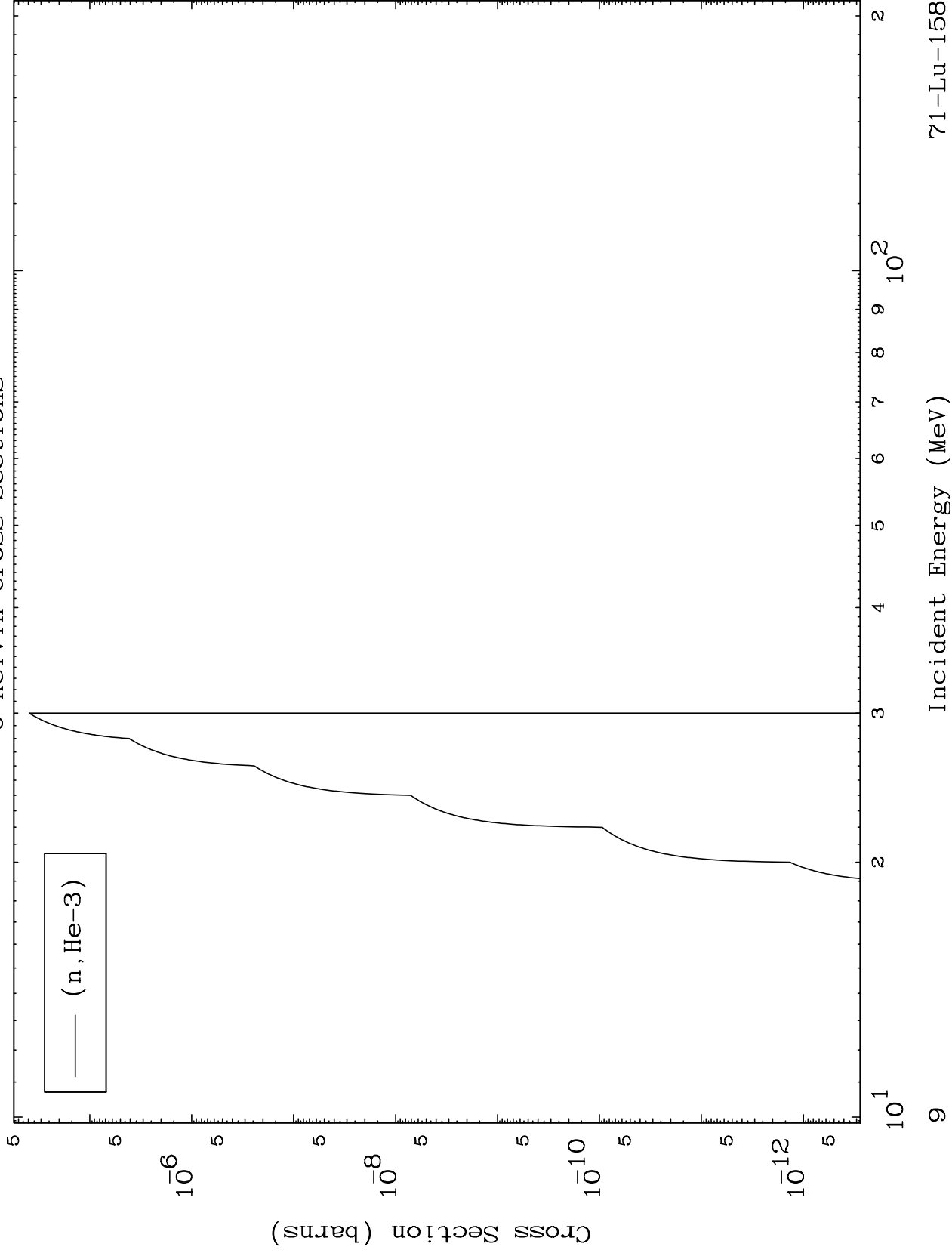




MAT 7074

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

71-Lu-158

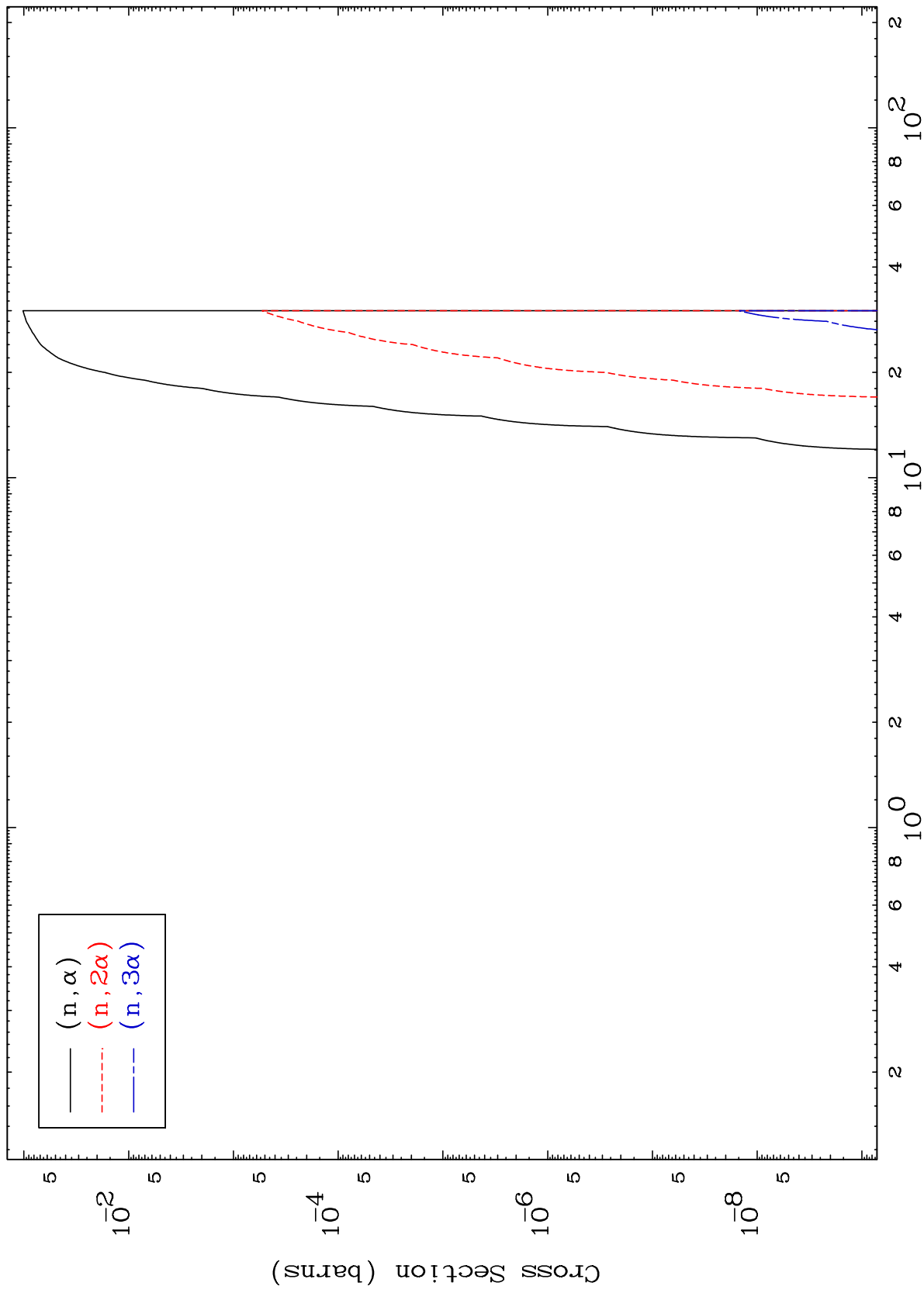


71-Lu-158

MAT 7074

71-Lu-158

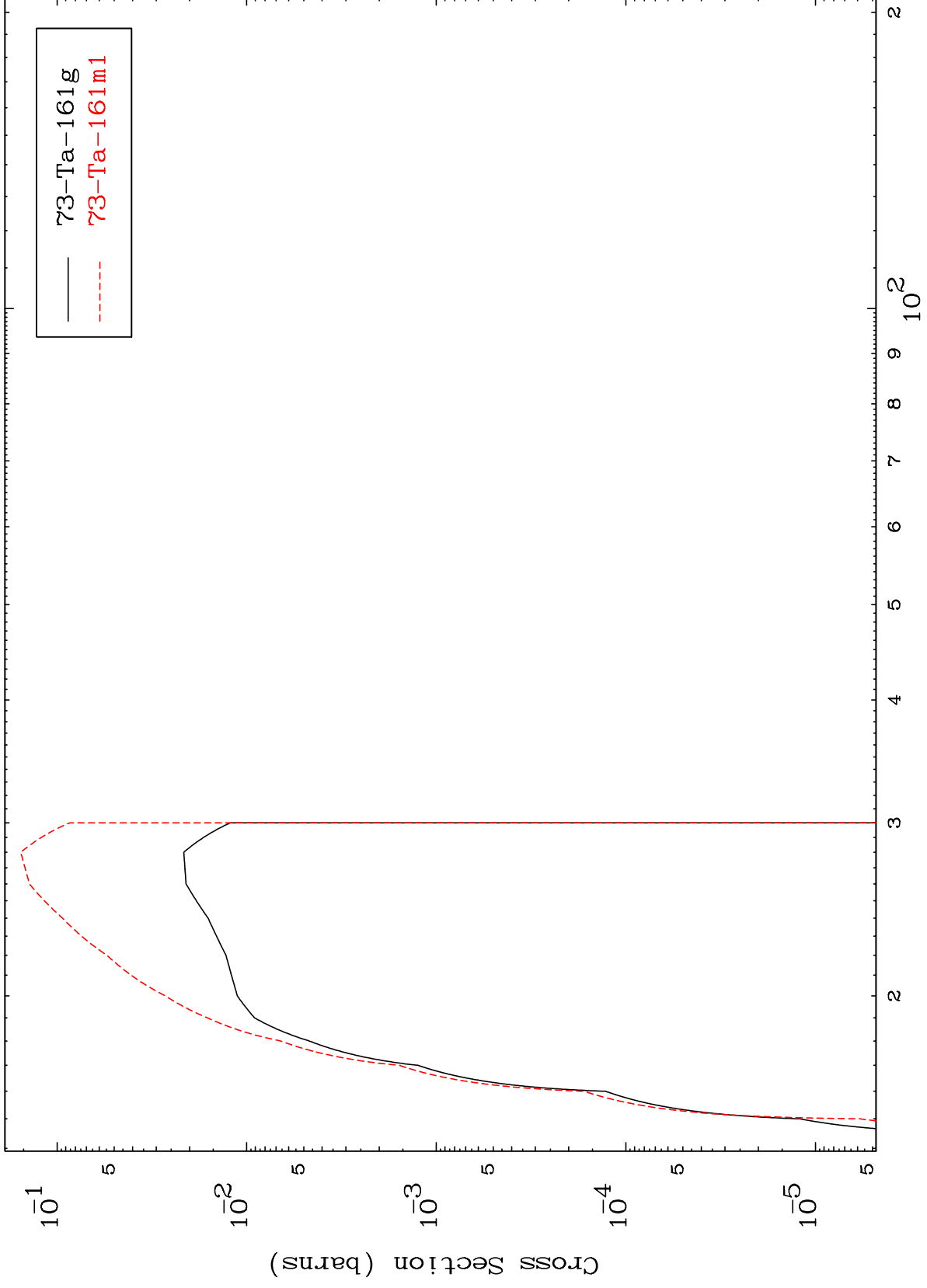
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



10

Incident Energy (MeV)

71-Lu-158

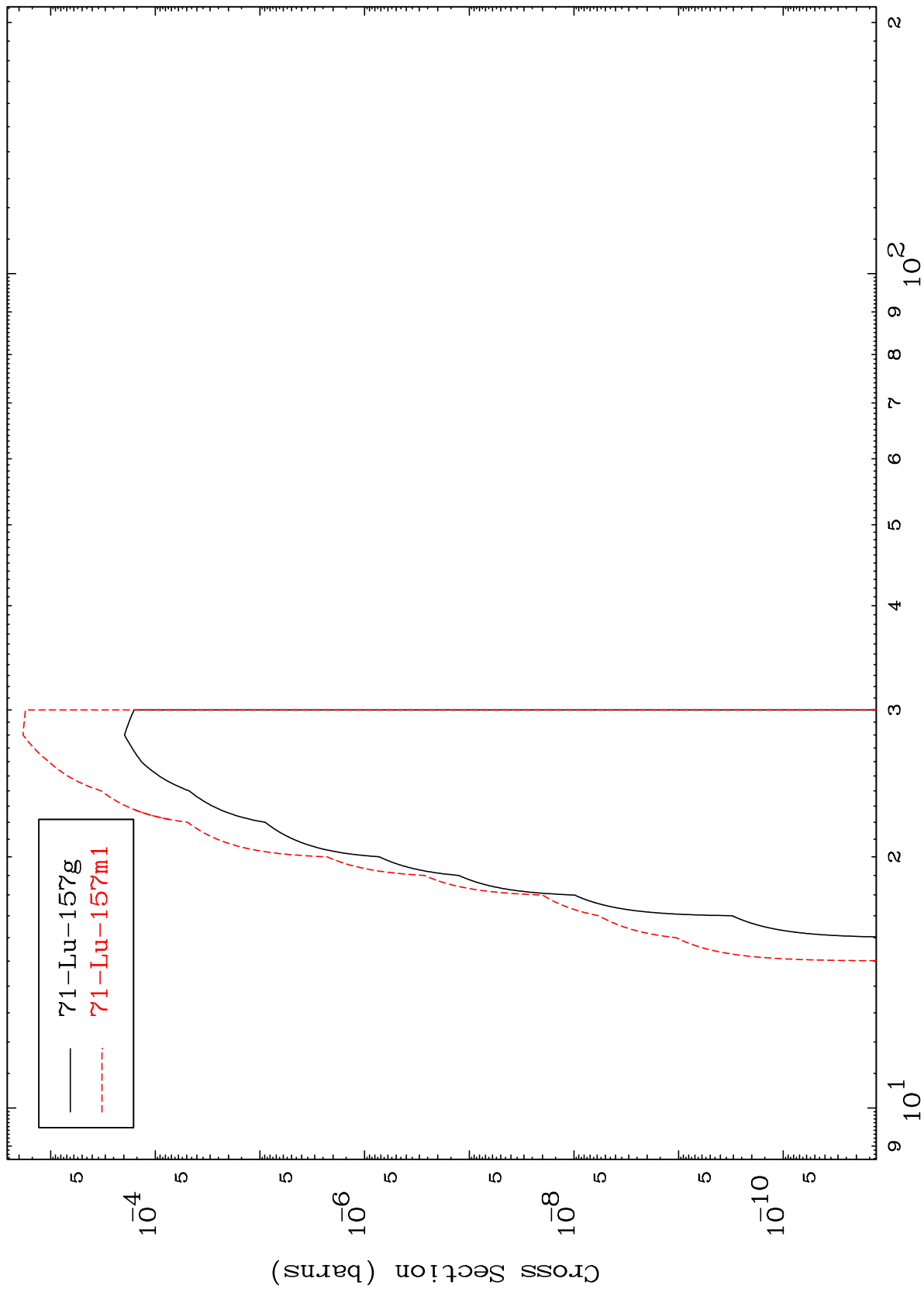


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$(n, n') \alpha$

$^{71}\text{Lu-158}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

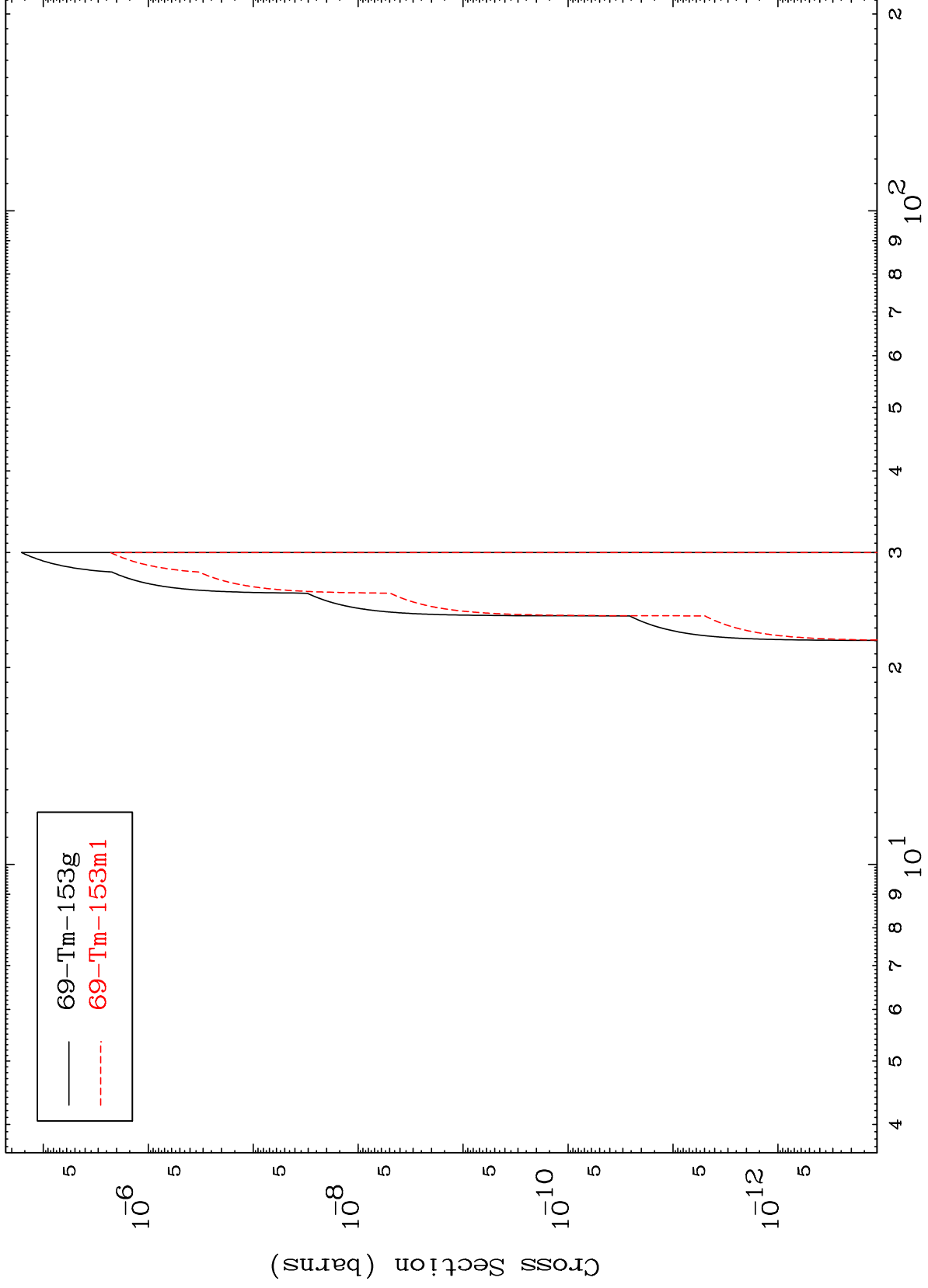
$^{71}\text{Lu-158}$

MAT 7074

$(n, n')$   $2\alpha$

$^{71}\text{Lu-158}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

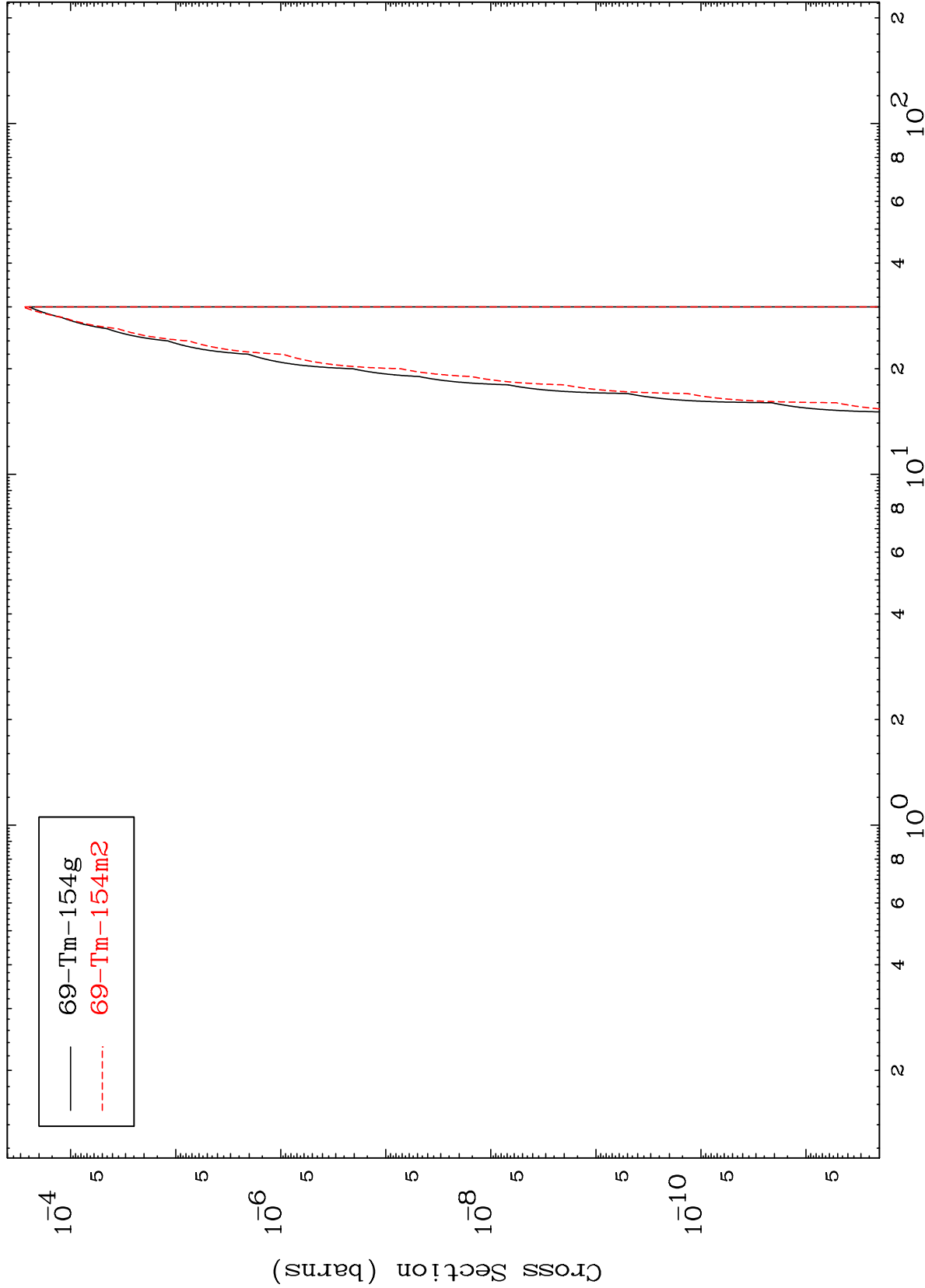
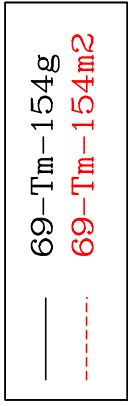
$^{71}\text{Lu-158}$

MAT 7074

(n,2α)

71-Lu-158

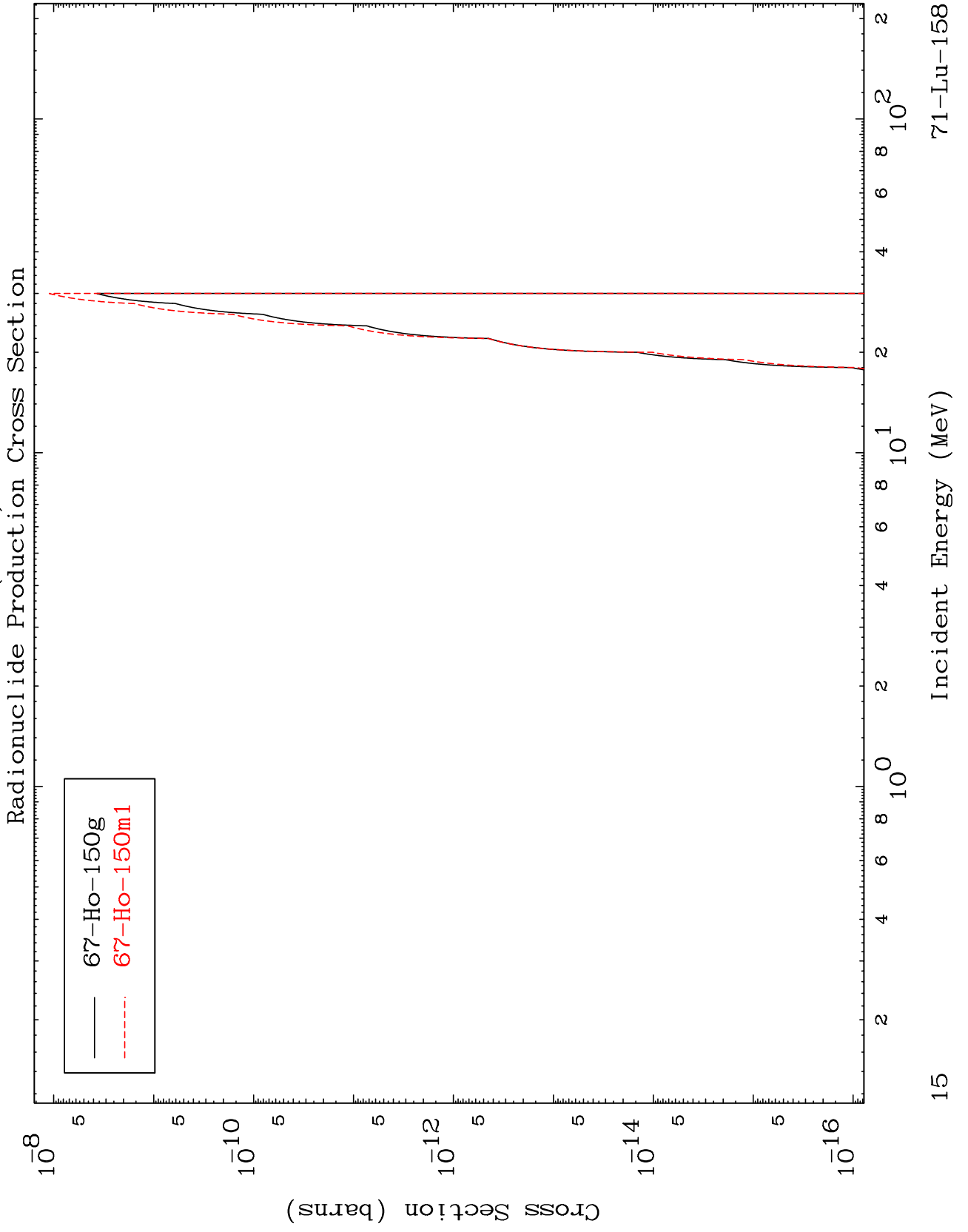
Radionuclide Production Cross Section



MAT 7074

(n, 3α)

71-Lu-158

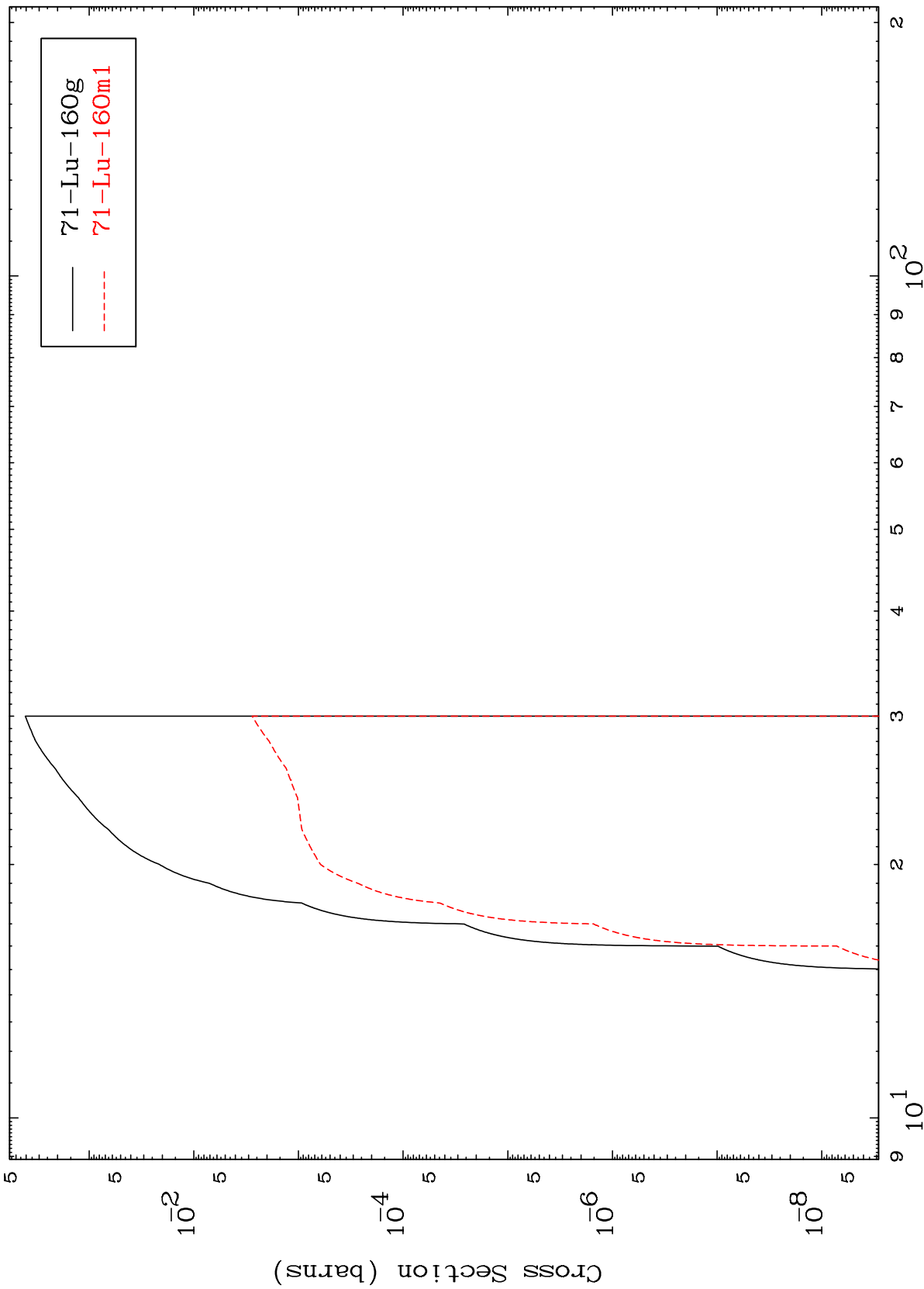




MAT 7074

71-Lu-158

(n,2p)  
Radionuclide Production Cross Section



71-Lu-158

Incident Energy (MeV)

16