

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

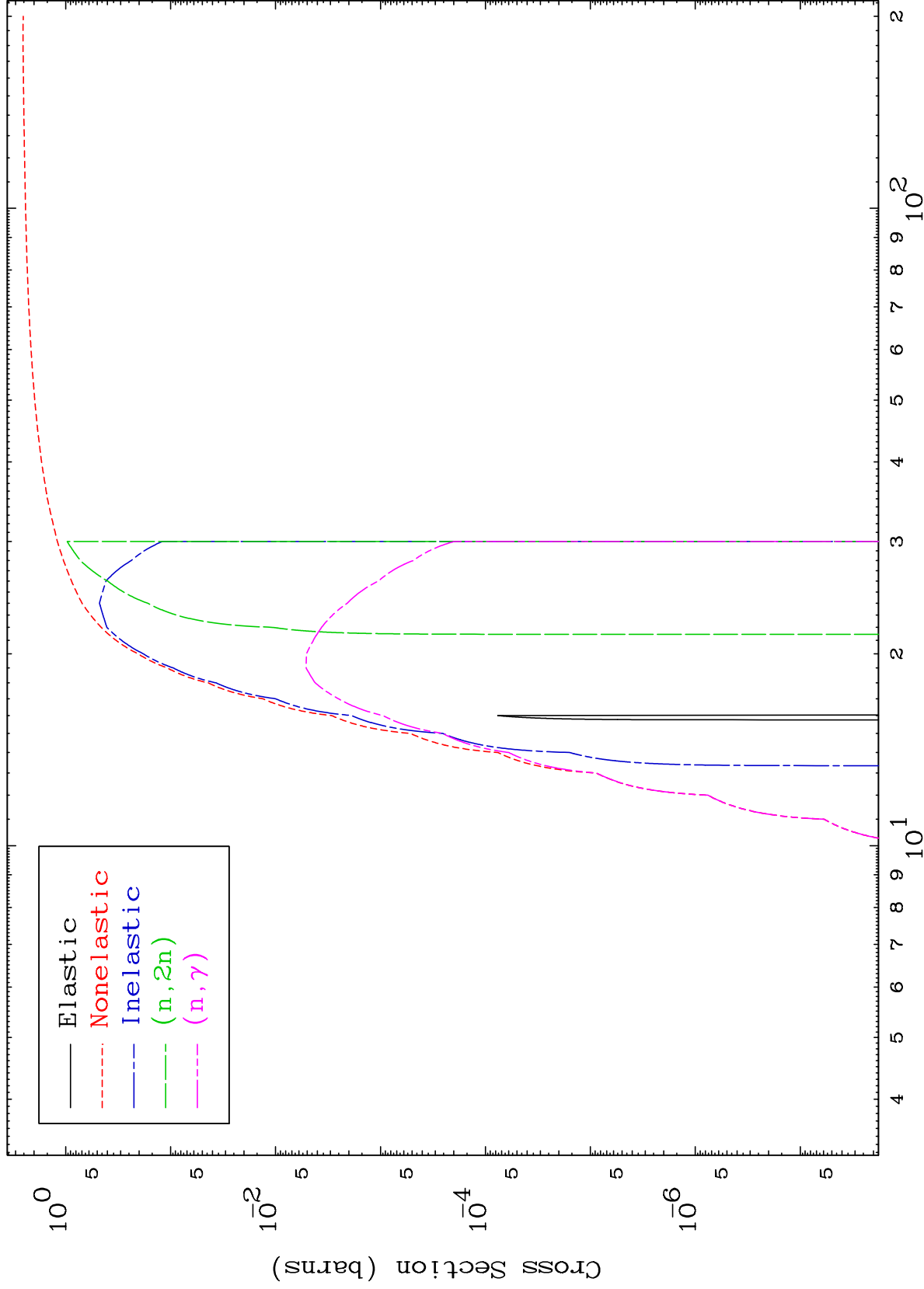
Press Mouse Button to Start

MAT 7101

0 Kelvin

$\alpha$  Major Cross Sections

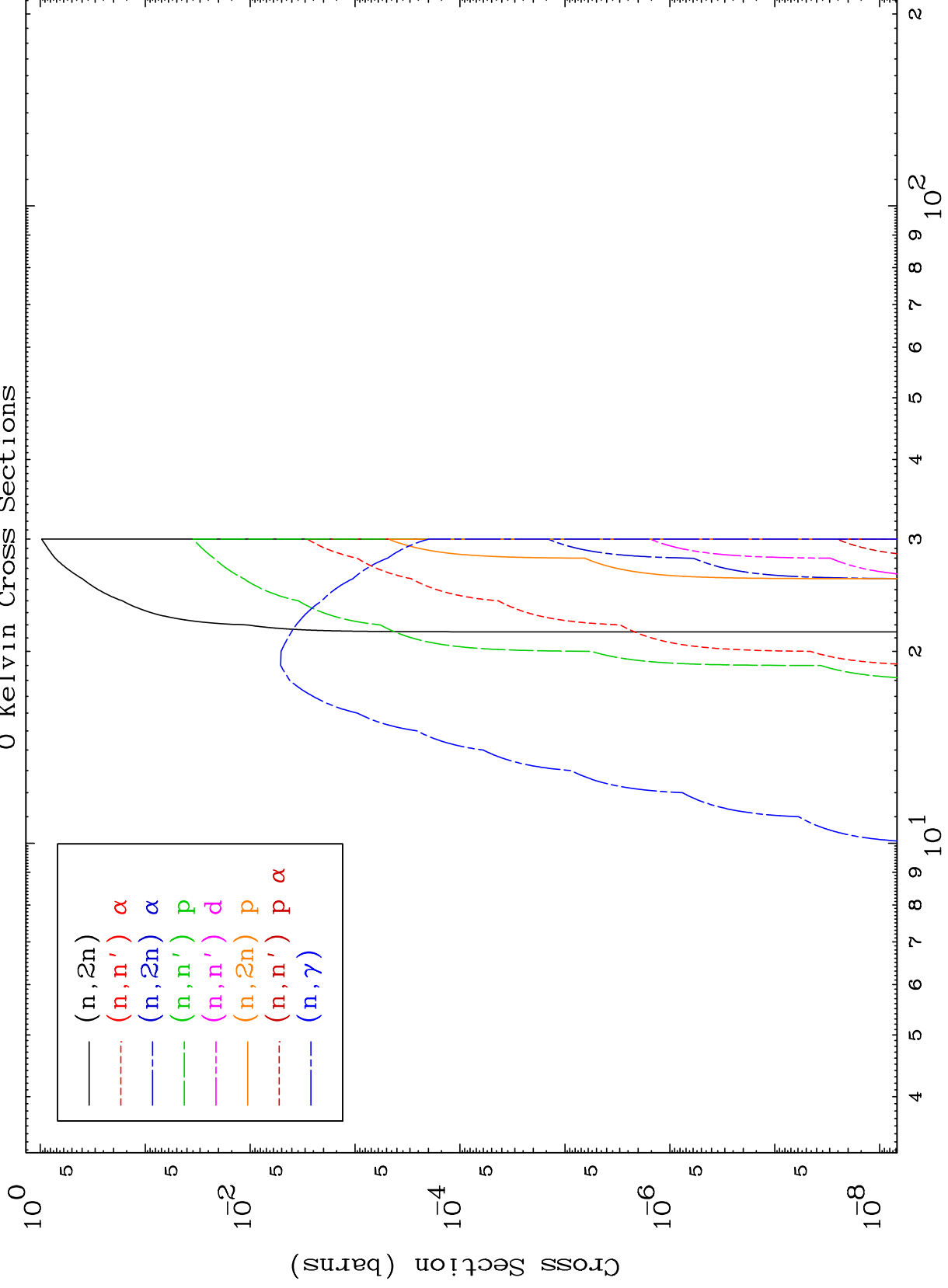
71-Lu-167



1

Incident Energy (MeV)

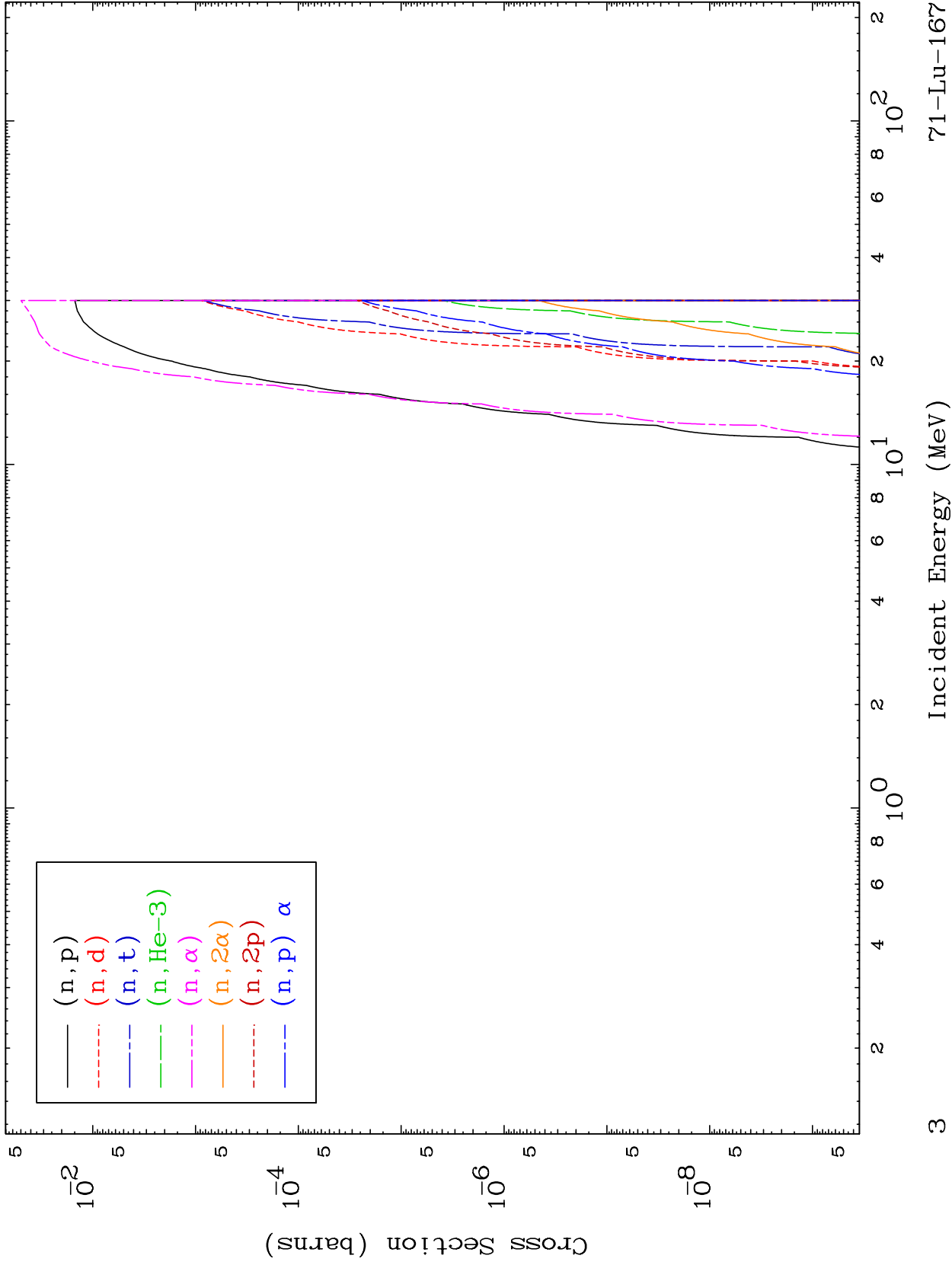
71-Lu-167

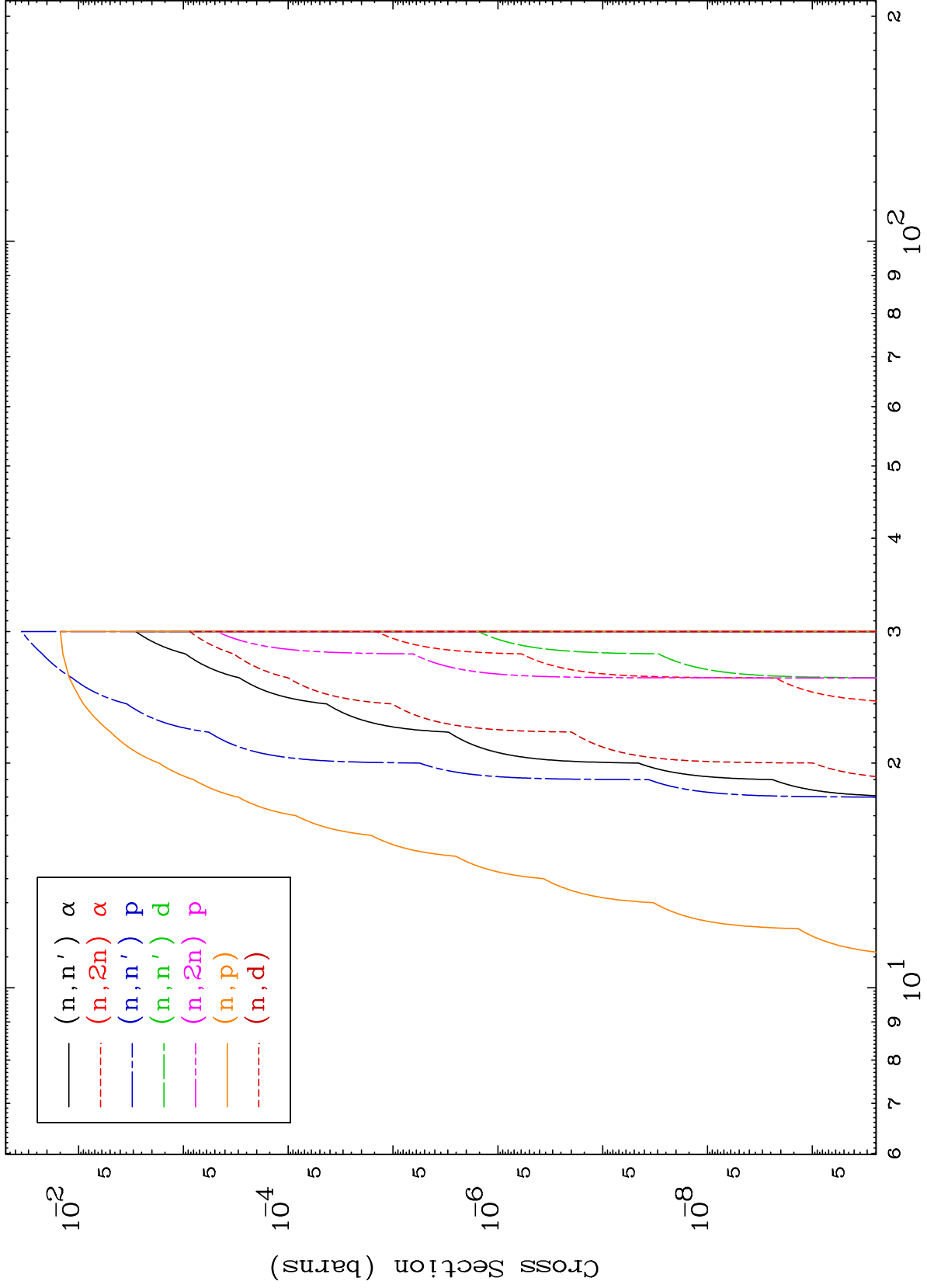


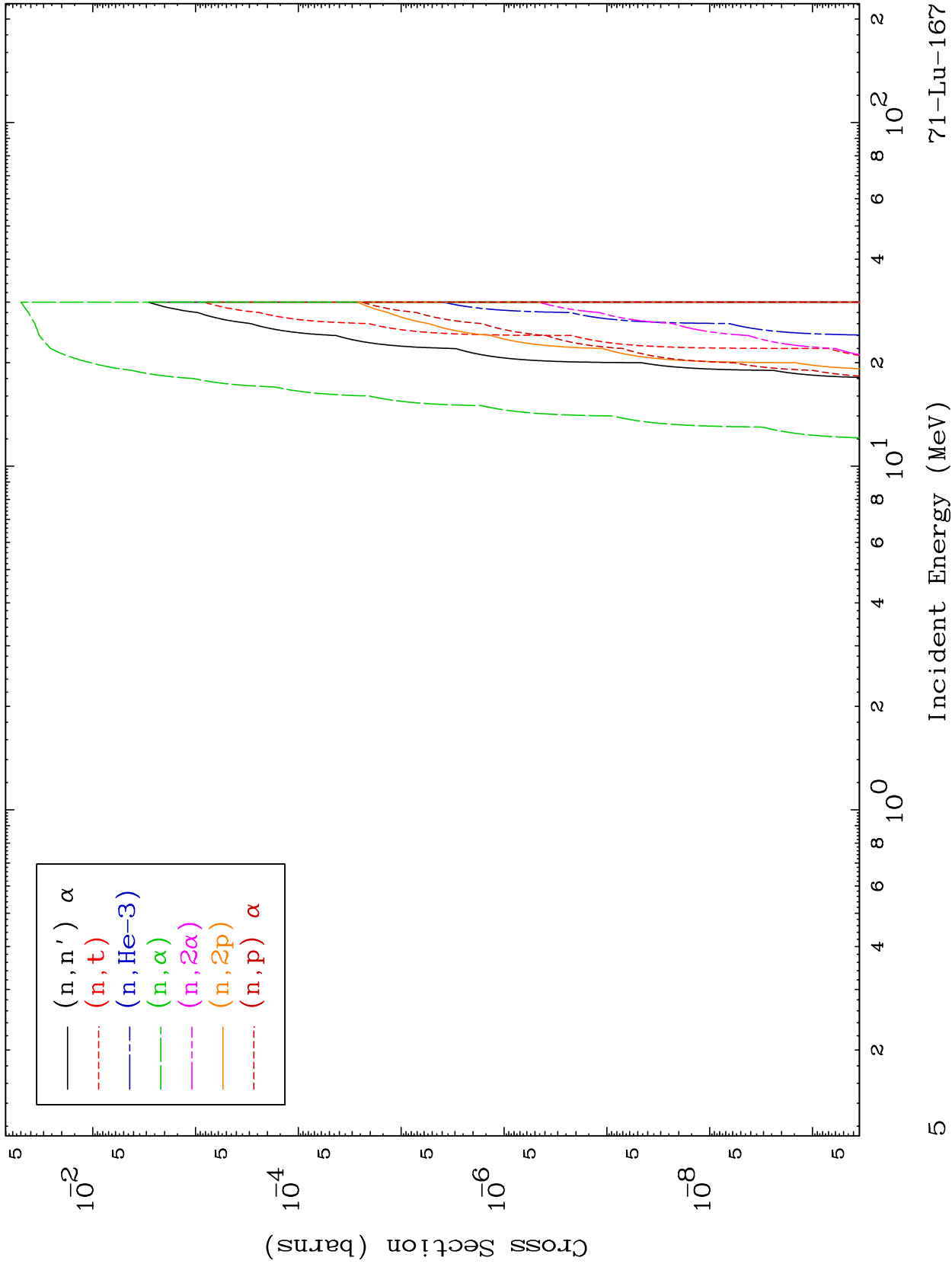
MAT 7101

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-167



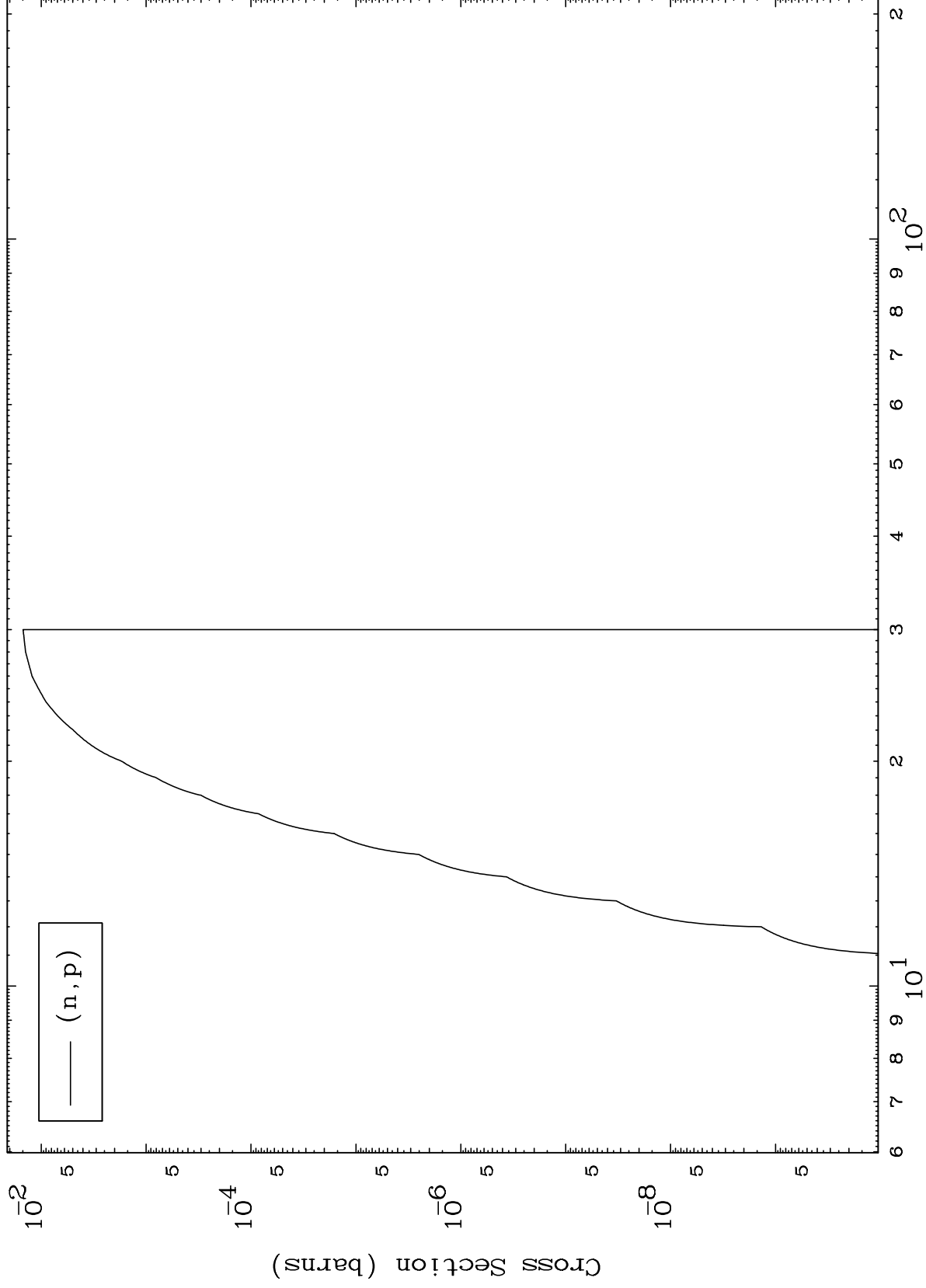




MAT 7101

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

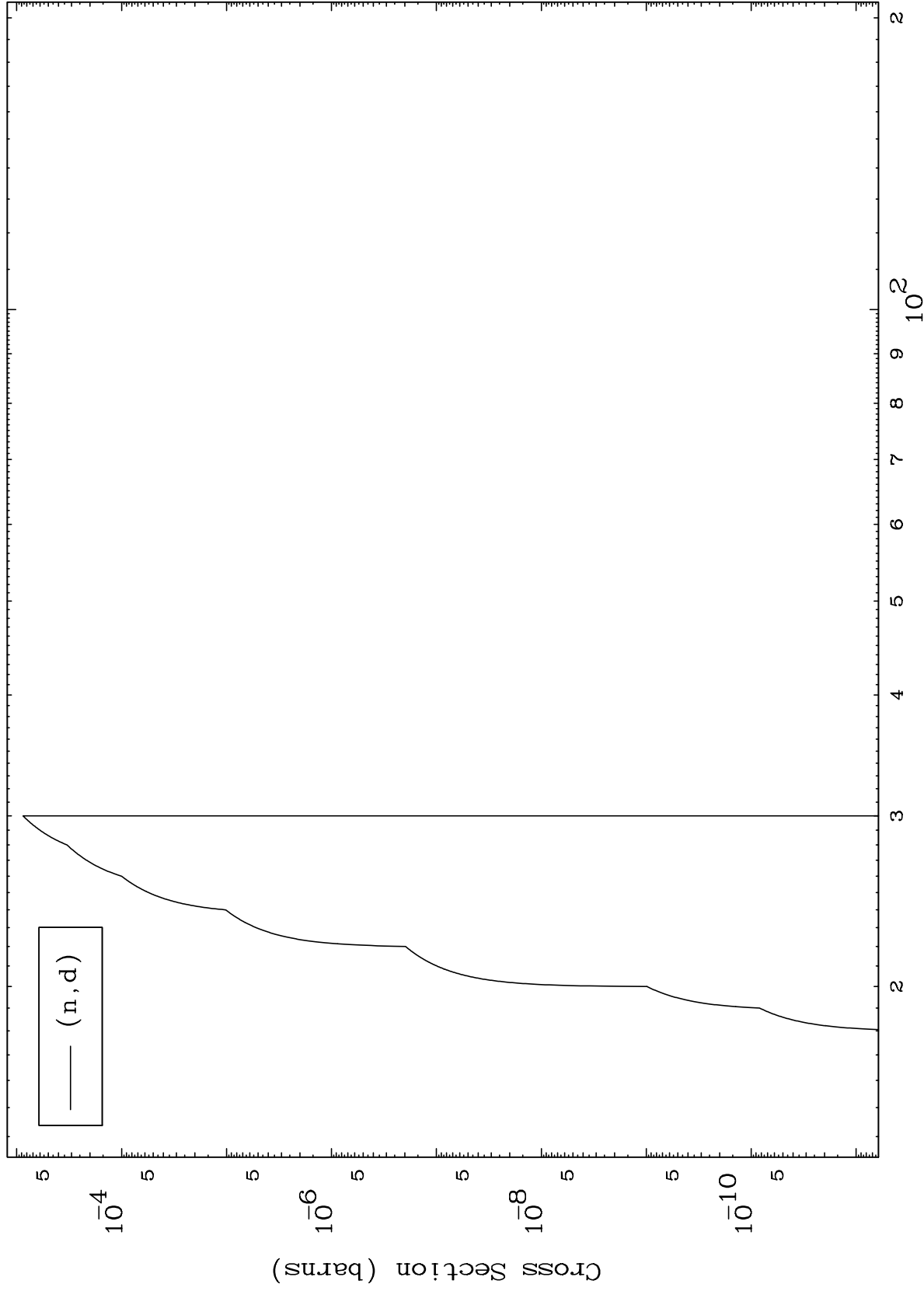
71-Lu-167



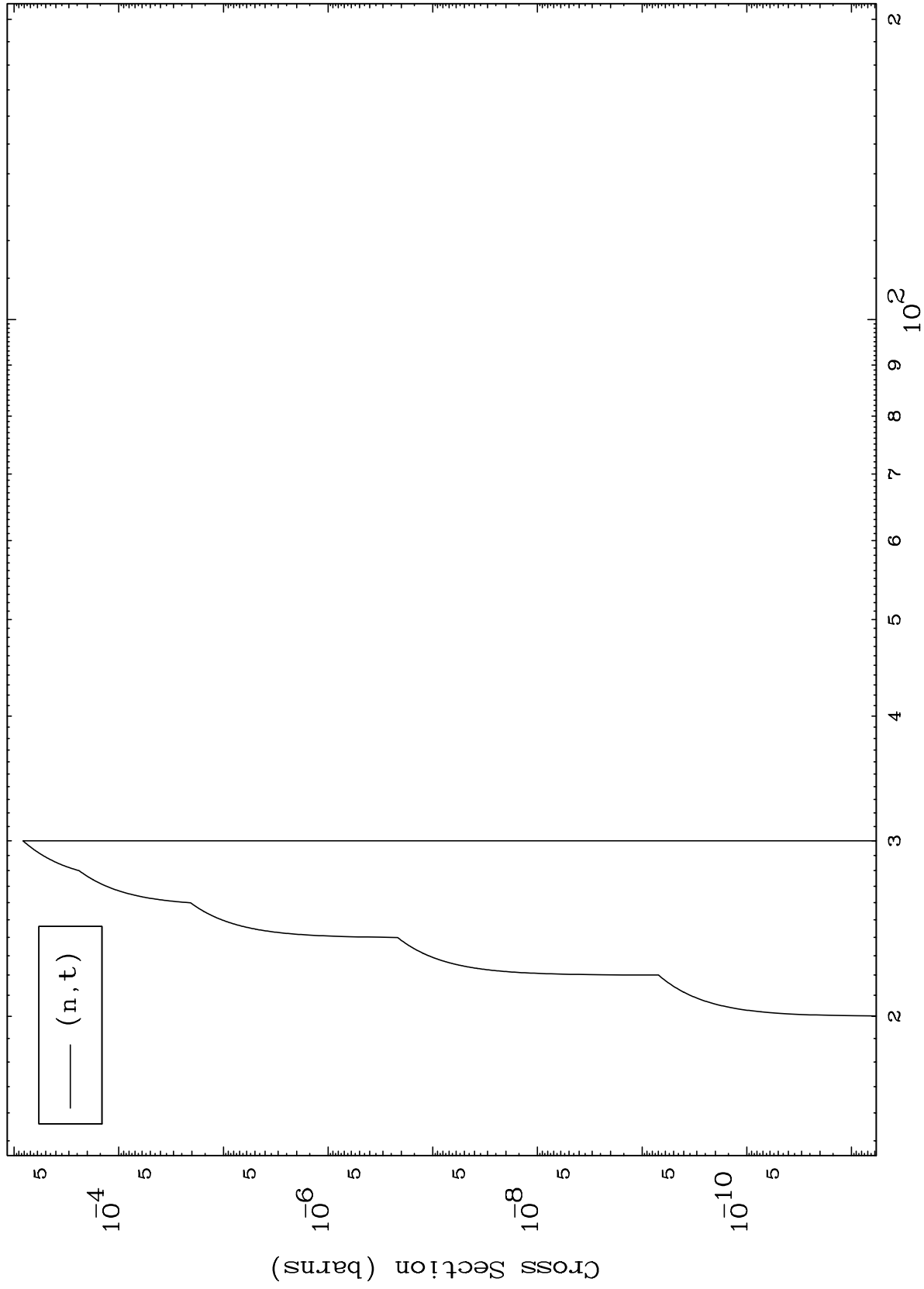
6

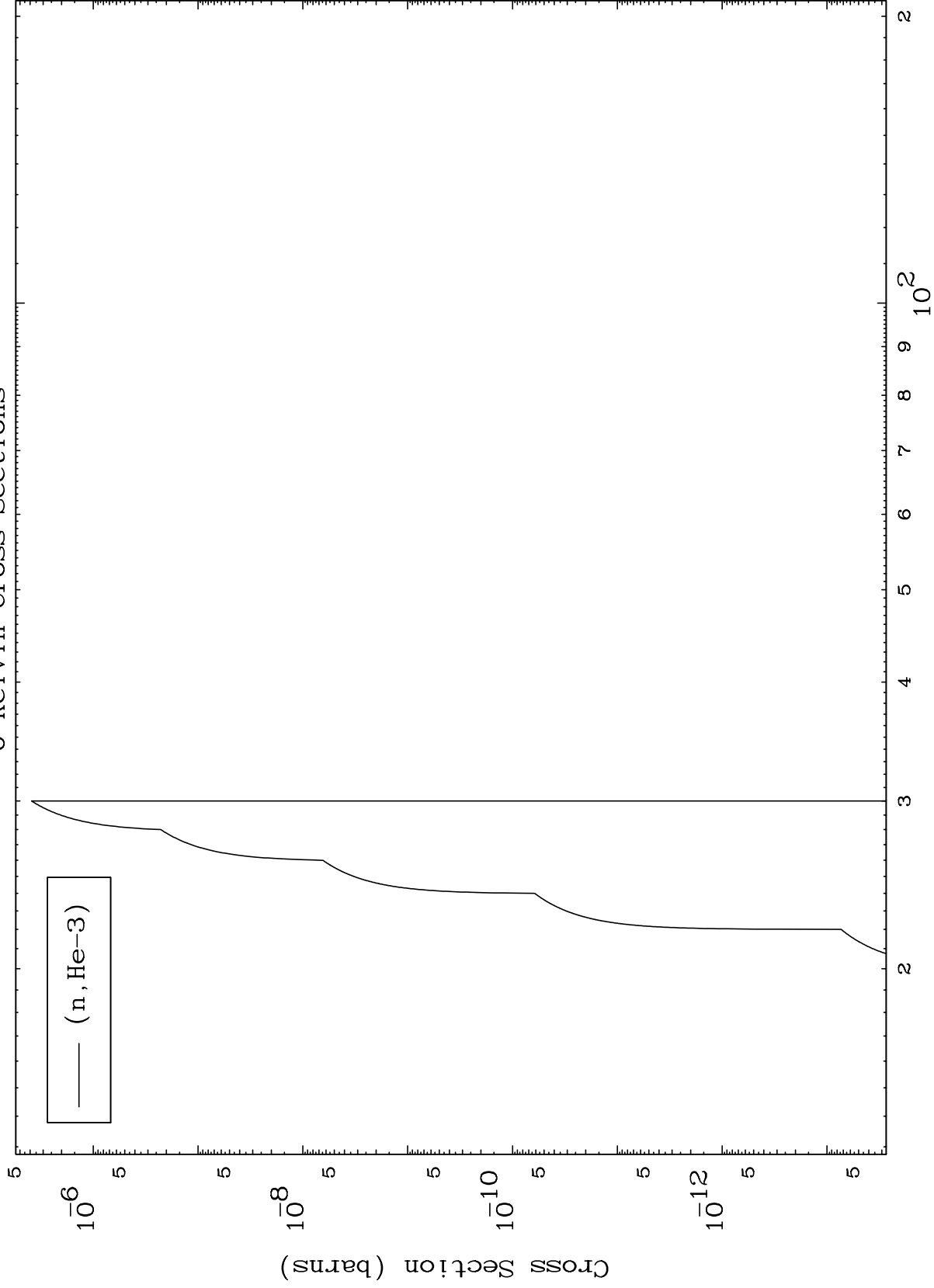
Incident Energy (MeV)

71-Lu-167



( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



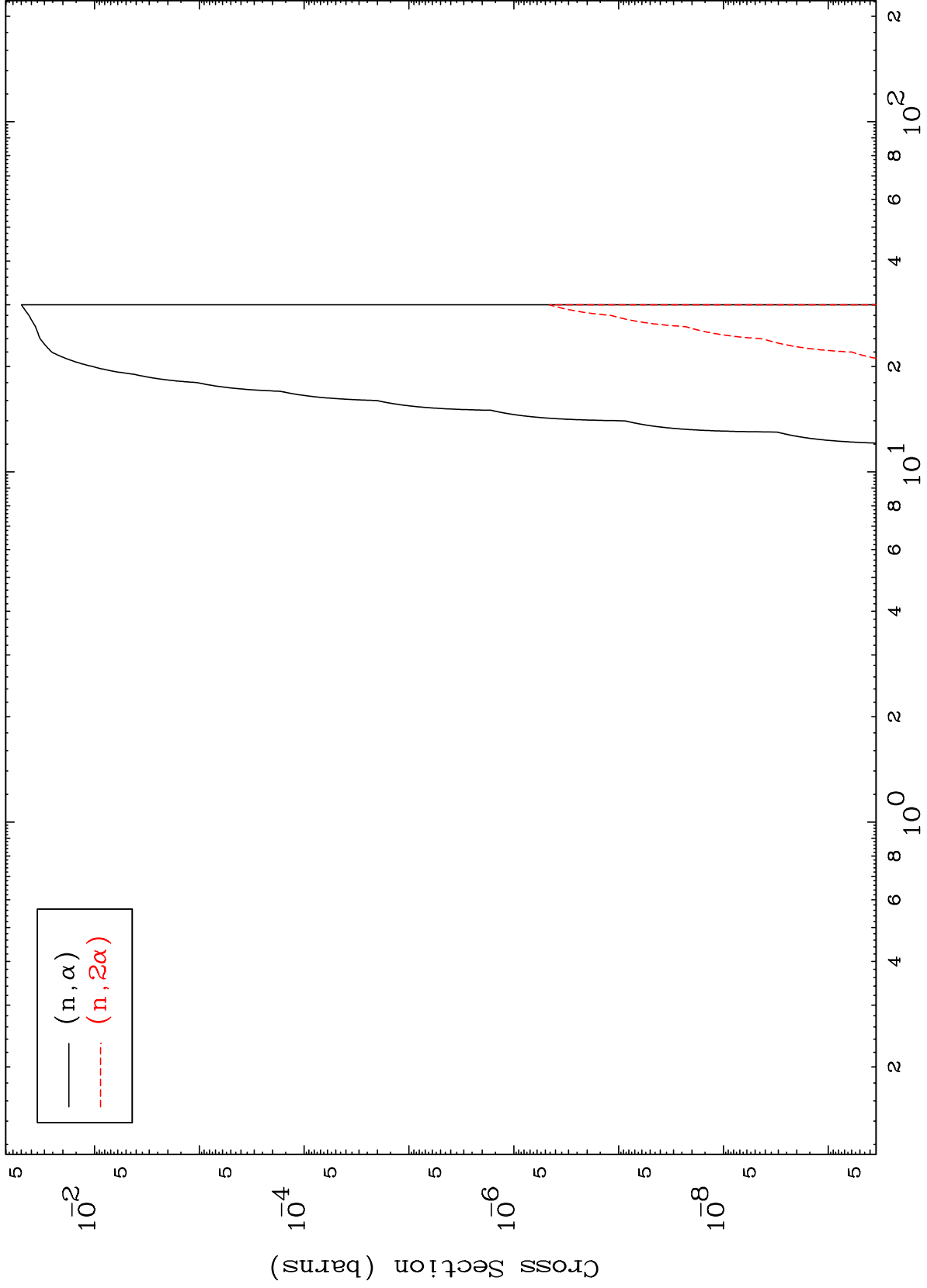


MAT 7101

( $\alpha, \alpha$ ) Levels

71-Lu-167

0 Kelvin Cross Sections



10

Incident Energy (MeV)

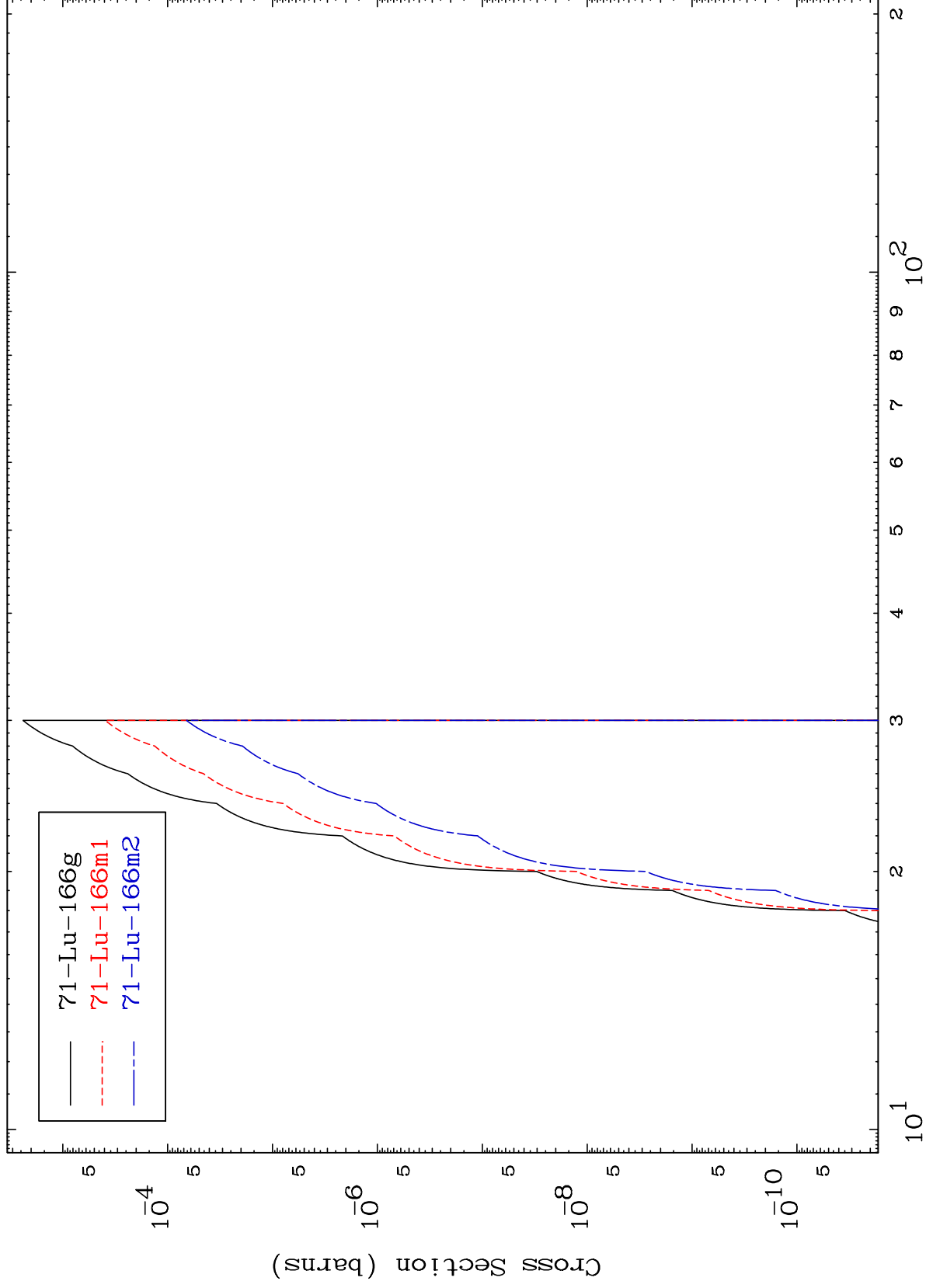
71-Lu-167

MAT 7101

$^{71}\text{Lu-167}$

$(n, n') \alpha$

Radionuclide Production Cross Section



$^{71}\text{Lu-167}$

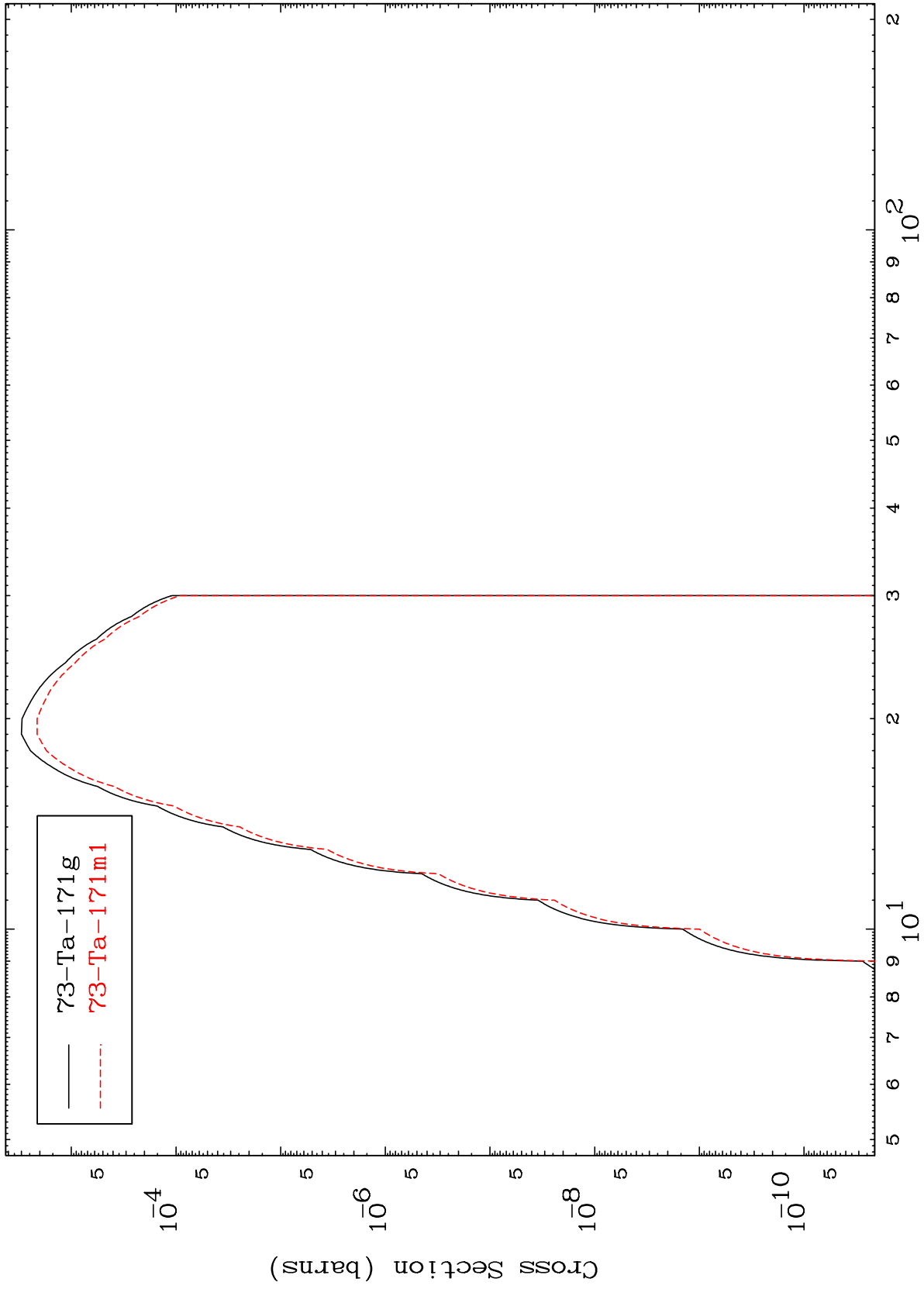
Incident Energy (MeV)

11

MAT 7101

<sup>71</sup>Lu-167

Radionuclide Production Cross Section (n,γ)



<sup>71</sup>Lu-167

Incident Energy (MeV)

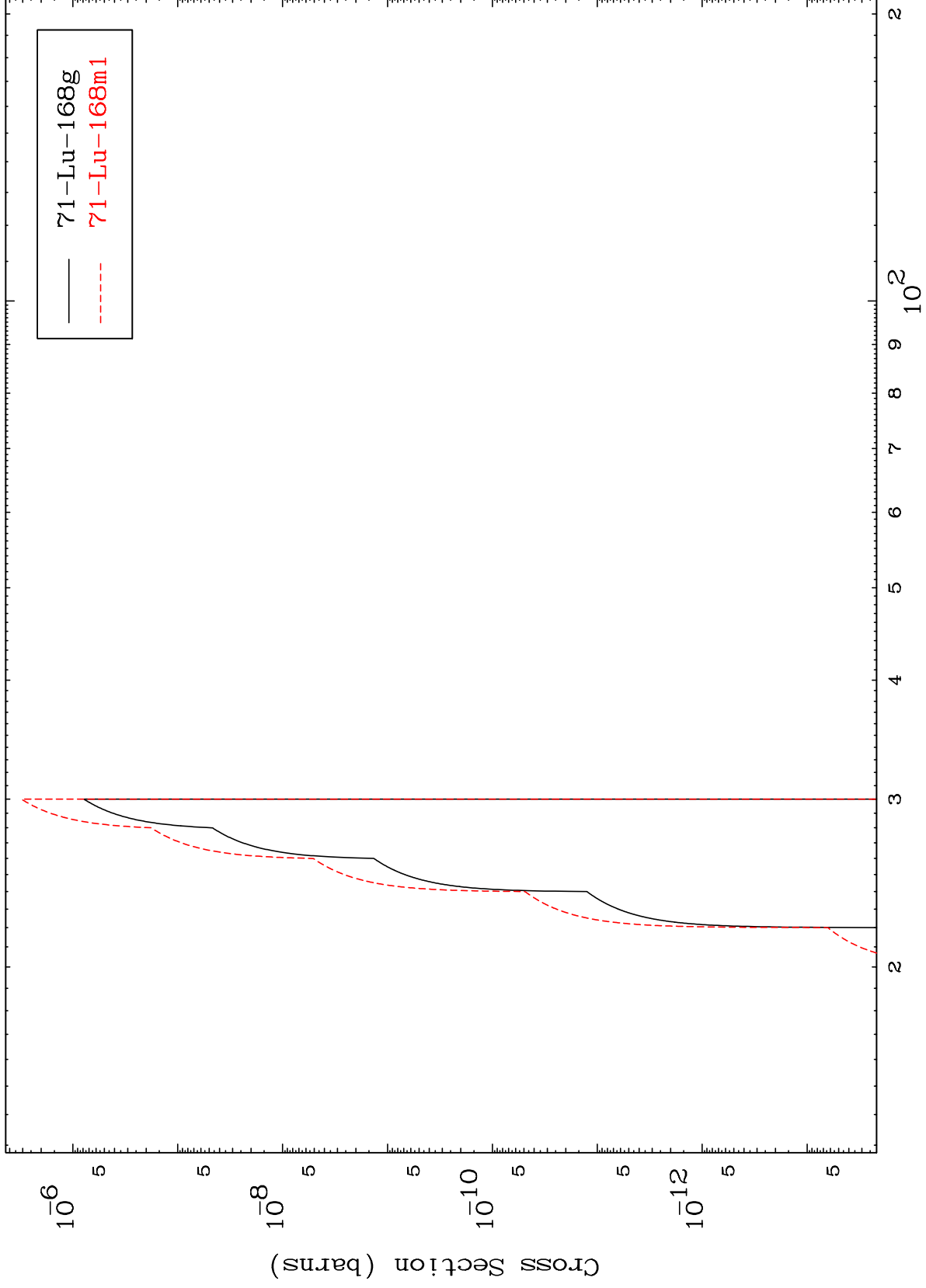
12

MAT 7101

(n,He-3)

71-Lu-167

Radionuclide Production Cross Section



13

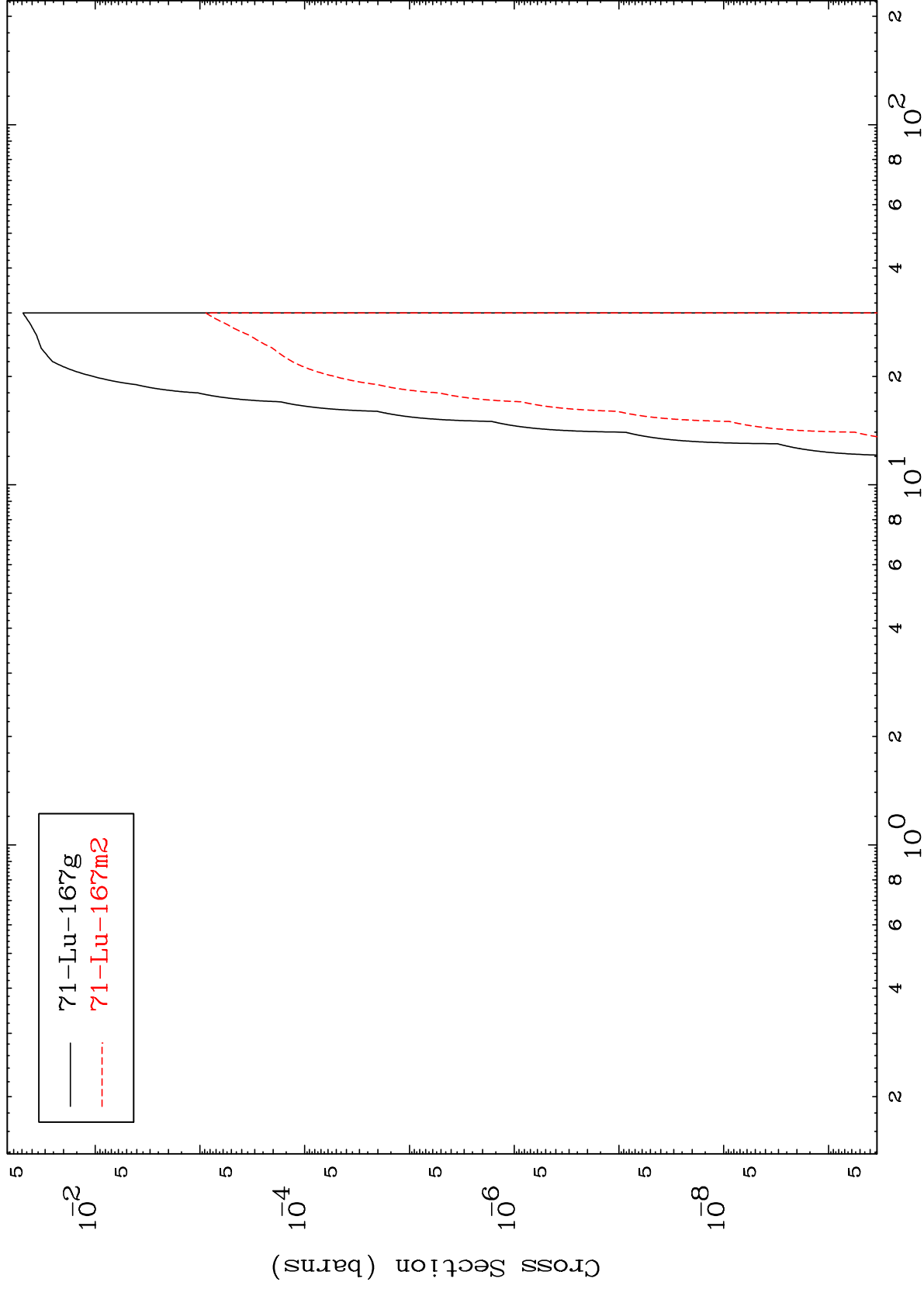
Incident Energy (MeV)

71-Lu-167

MAT 7101

71-Lu-167

Radionuclide Production Cross Section  
(n,  $\alpha$ )



14

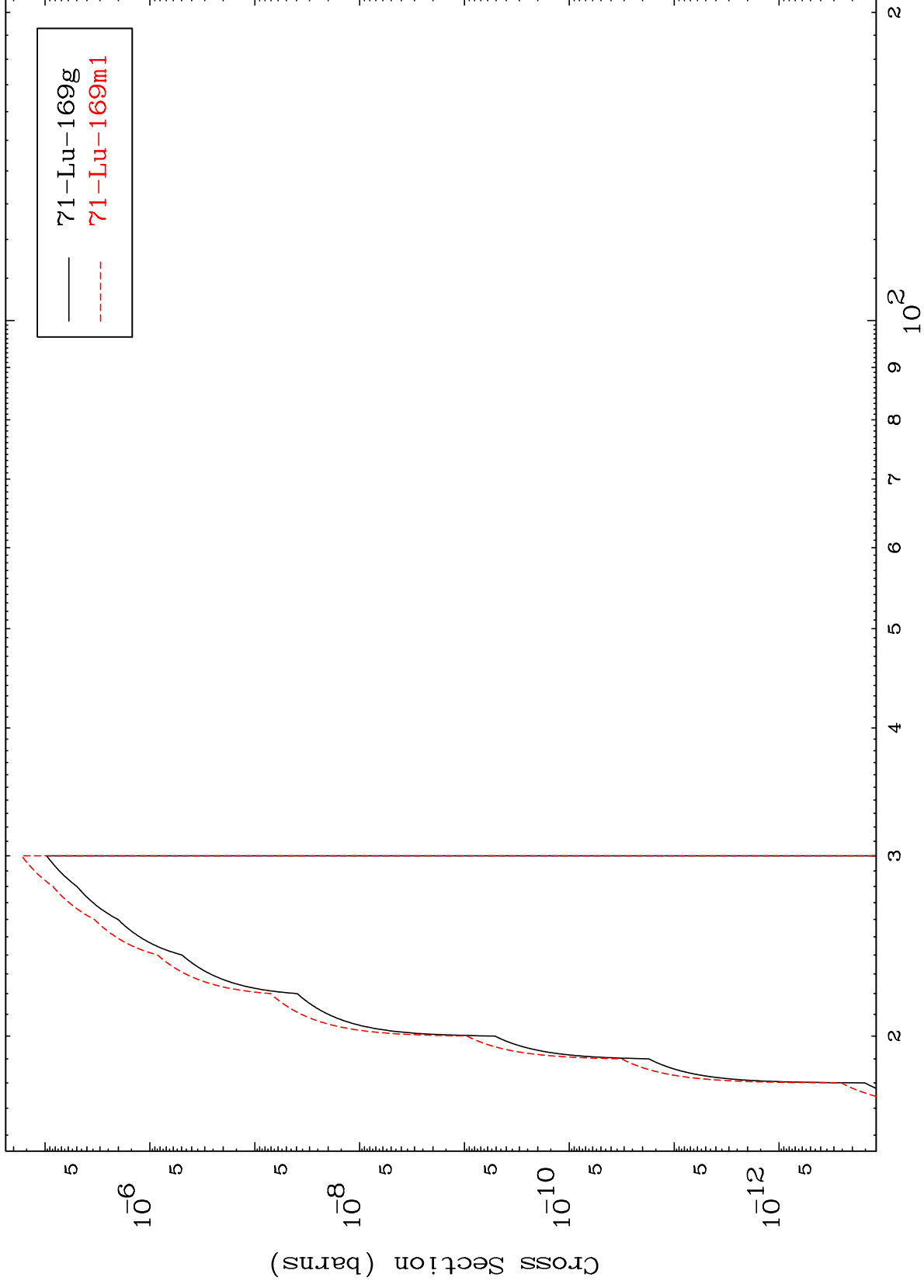
71-Lu-167

Incident Energy (MeV)

MAT 7101

71-Lu-167

(n,2p)  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

71-Lu-167