

Program EVALPLOT  
(Version 2021-1)

by

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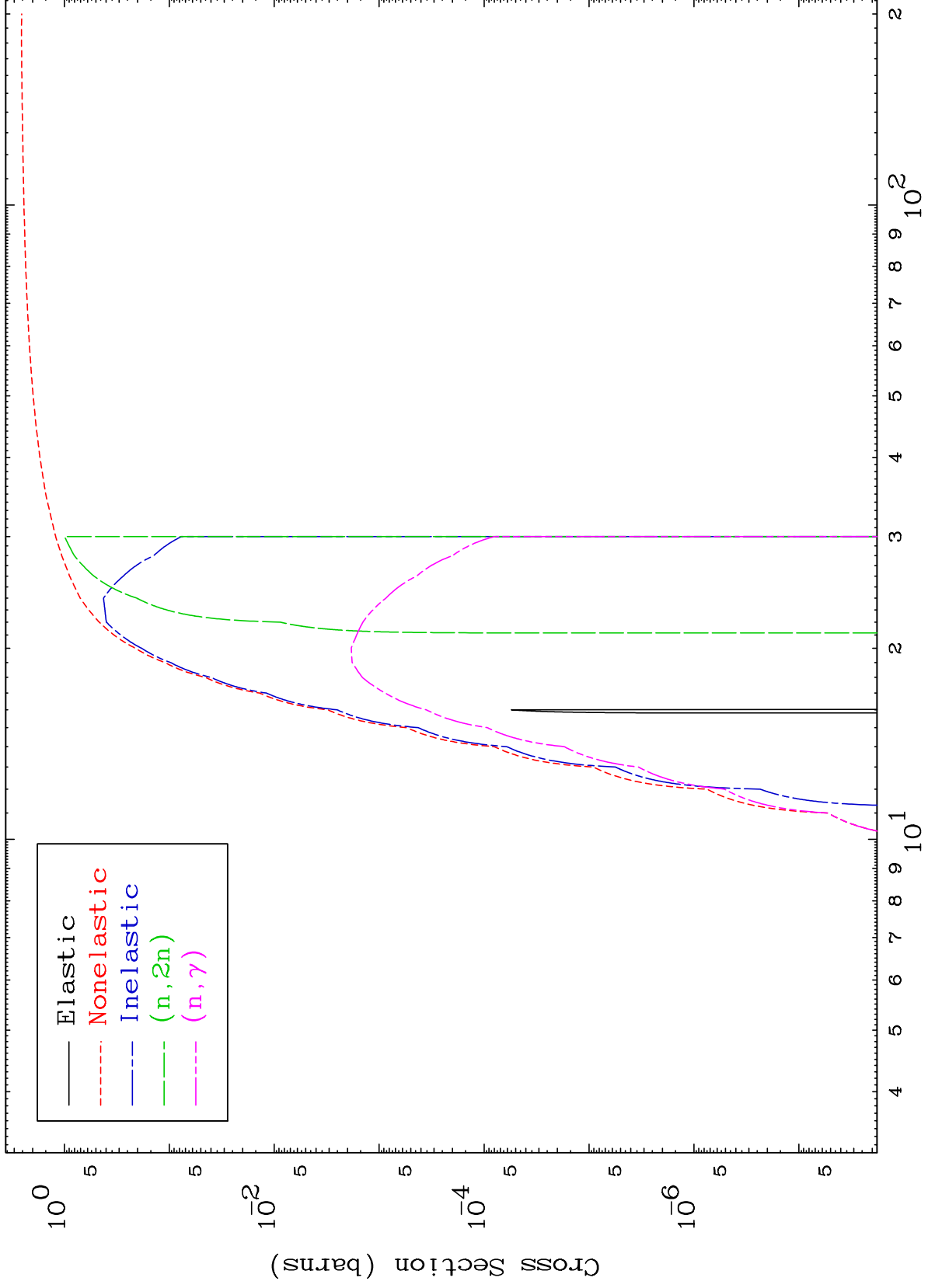
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 7104

0 Kelvin Cross Sections

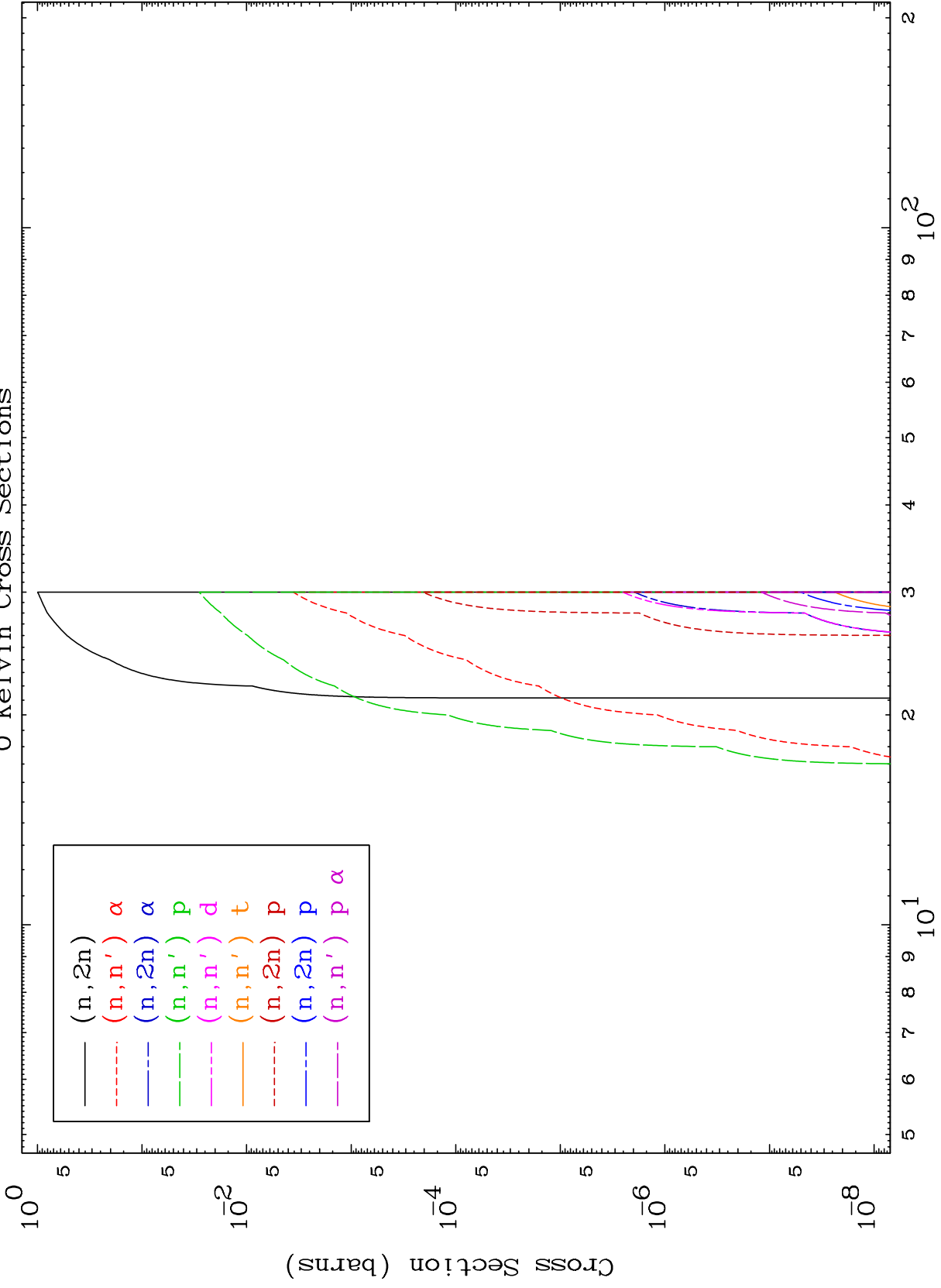
71-Lu-168



MAT 7104

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

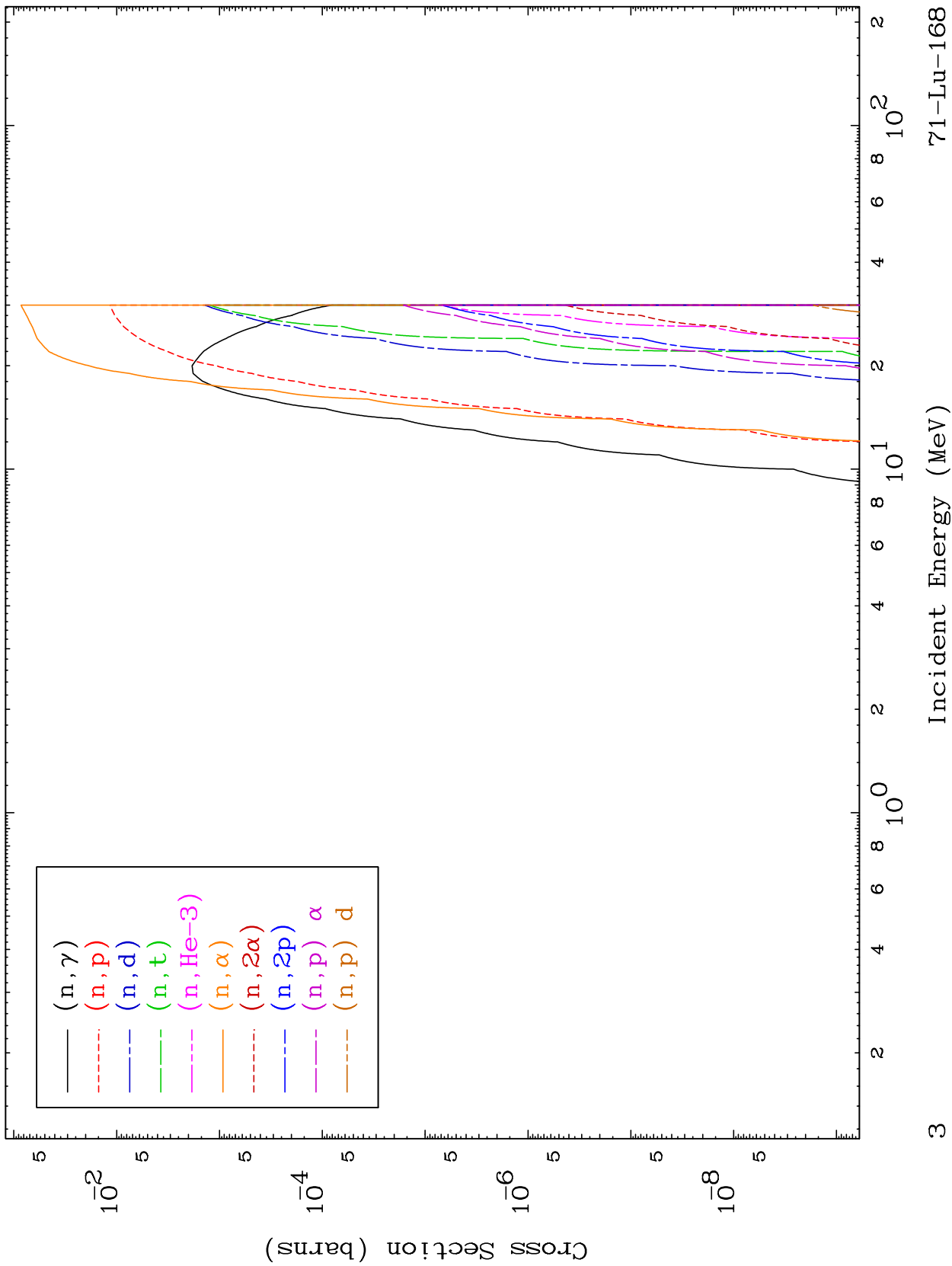
71-Lu-168

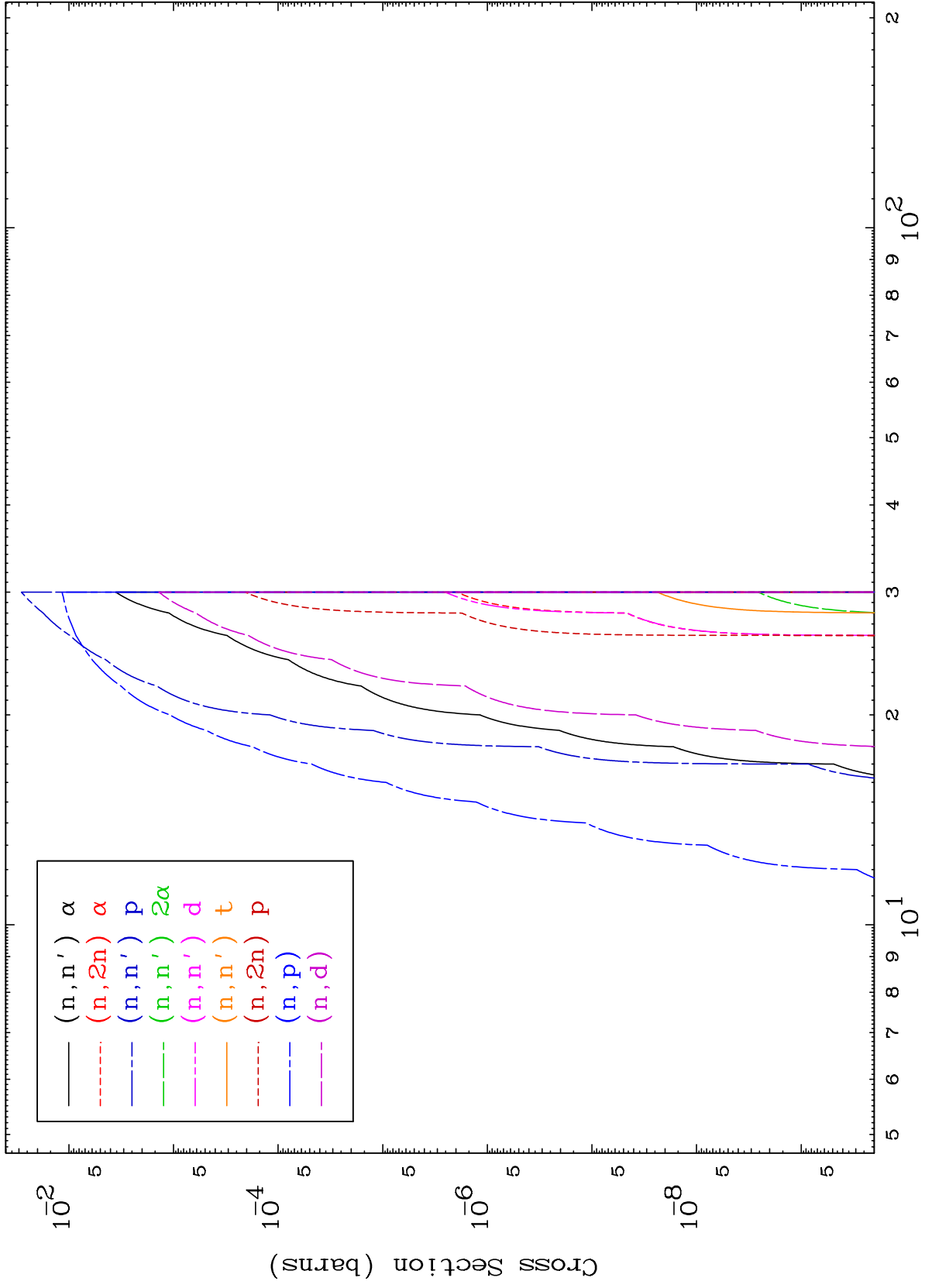


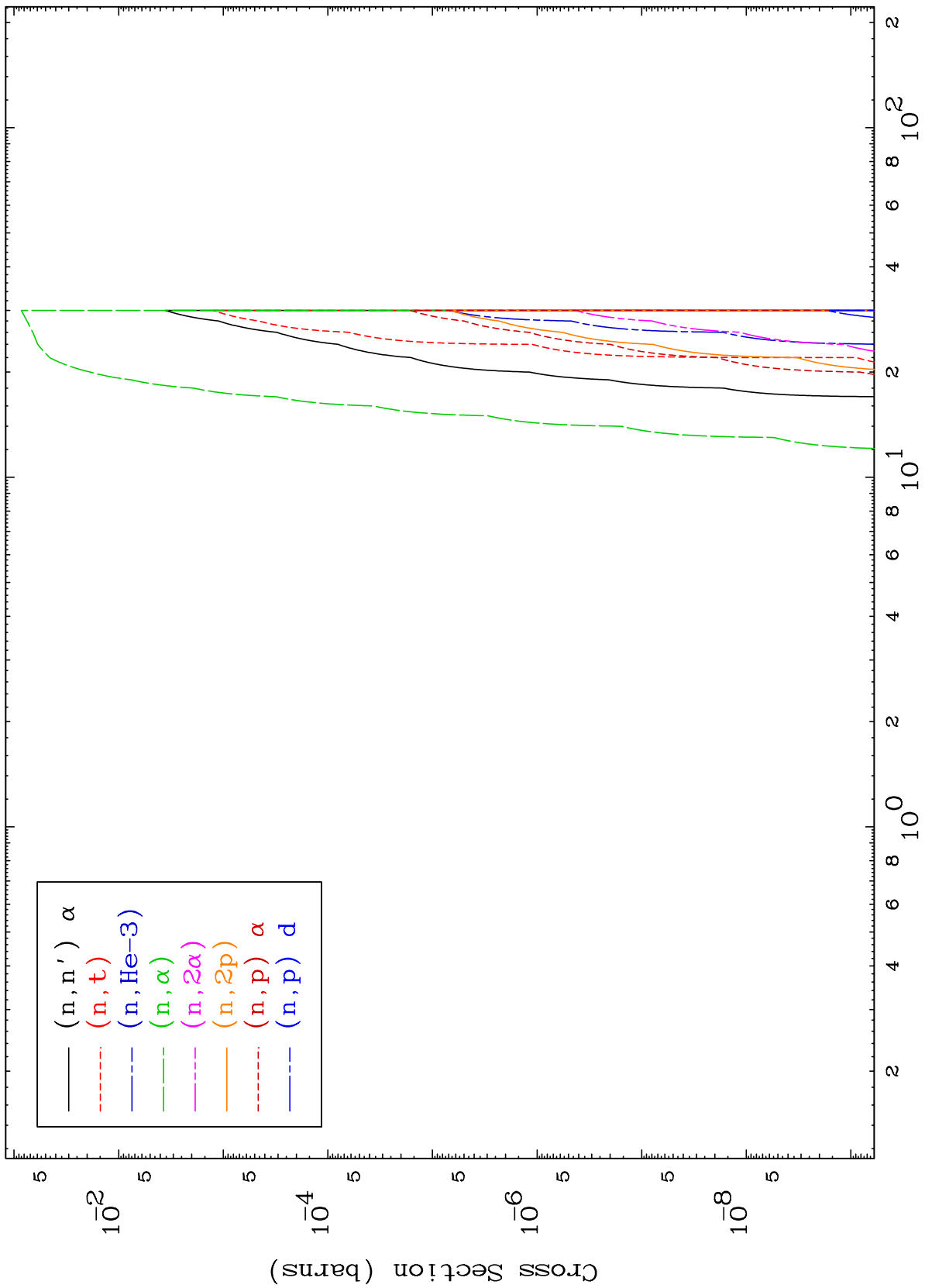
2

Incident Energy (MeV)

71-Lu-168



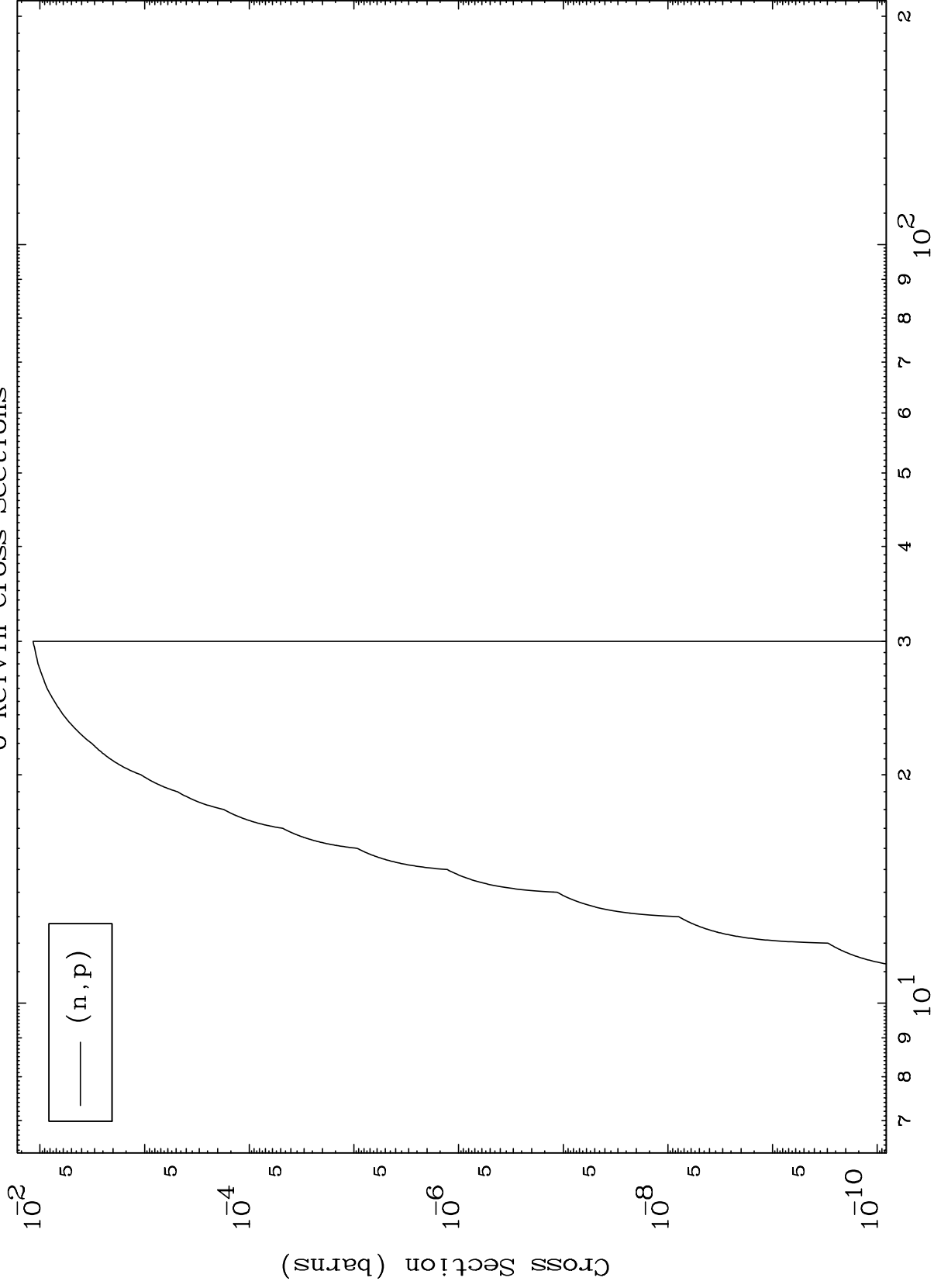




MAT 7104

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

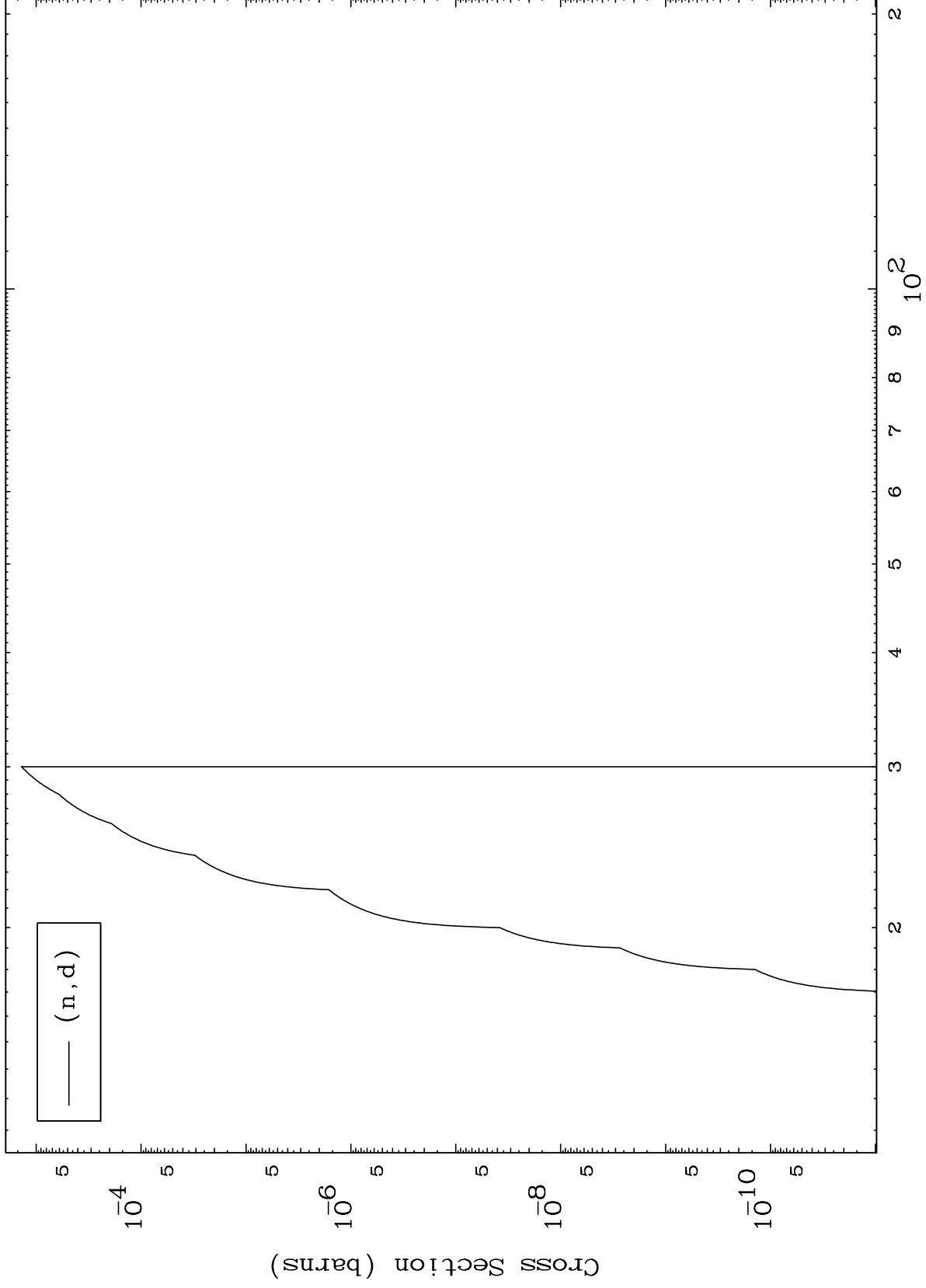
71-Lu-168

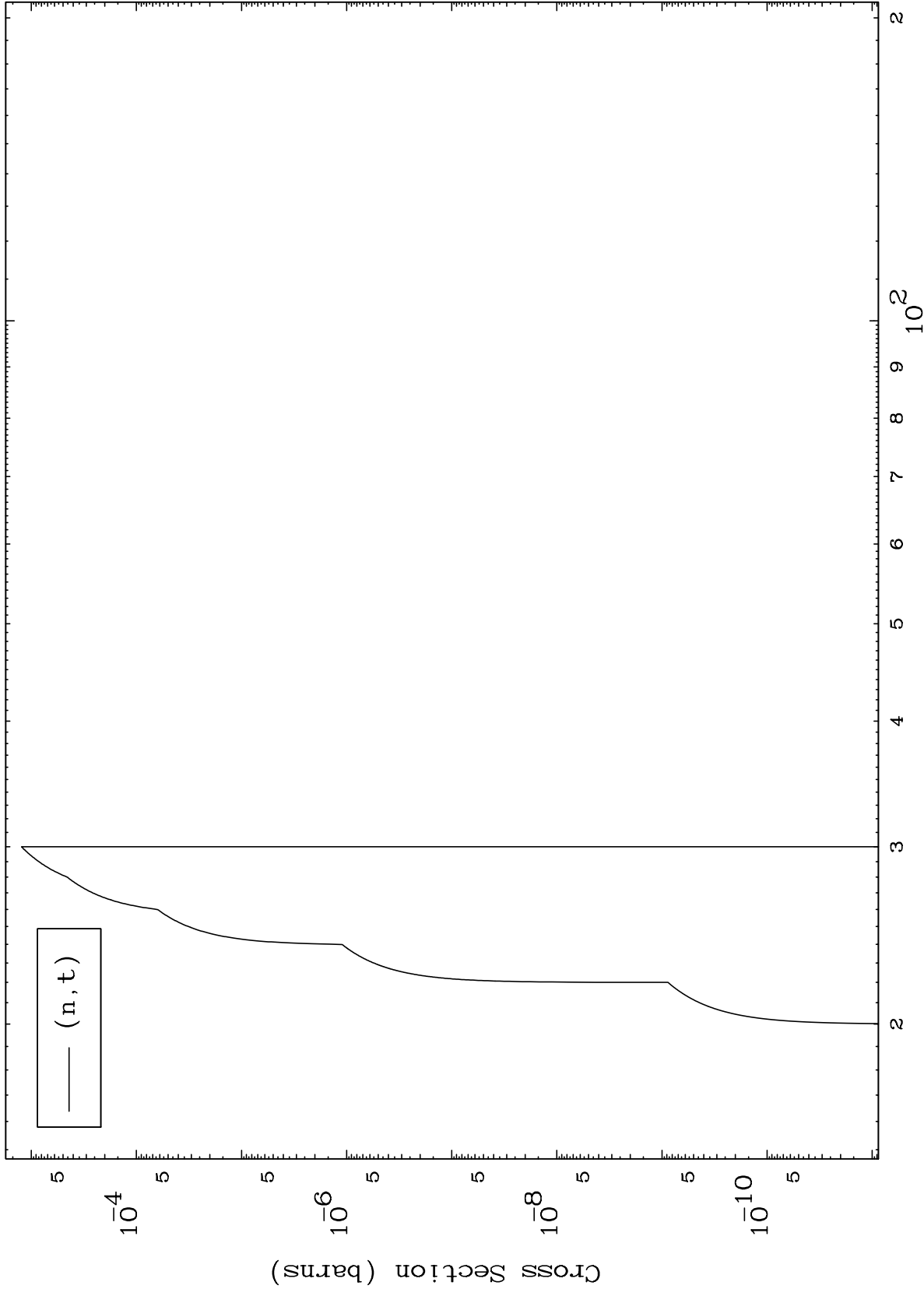


6

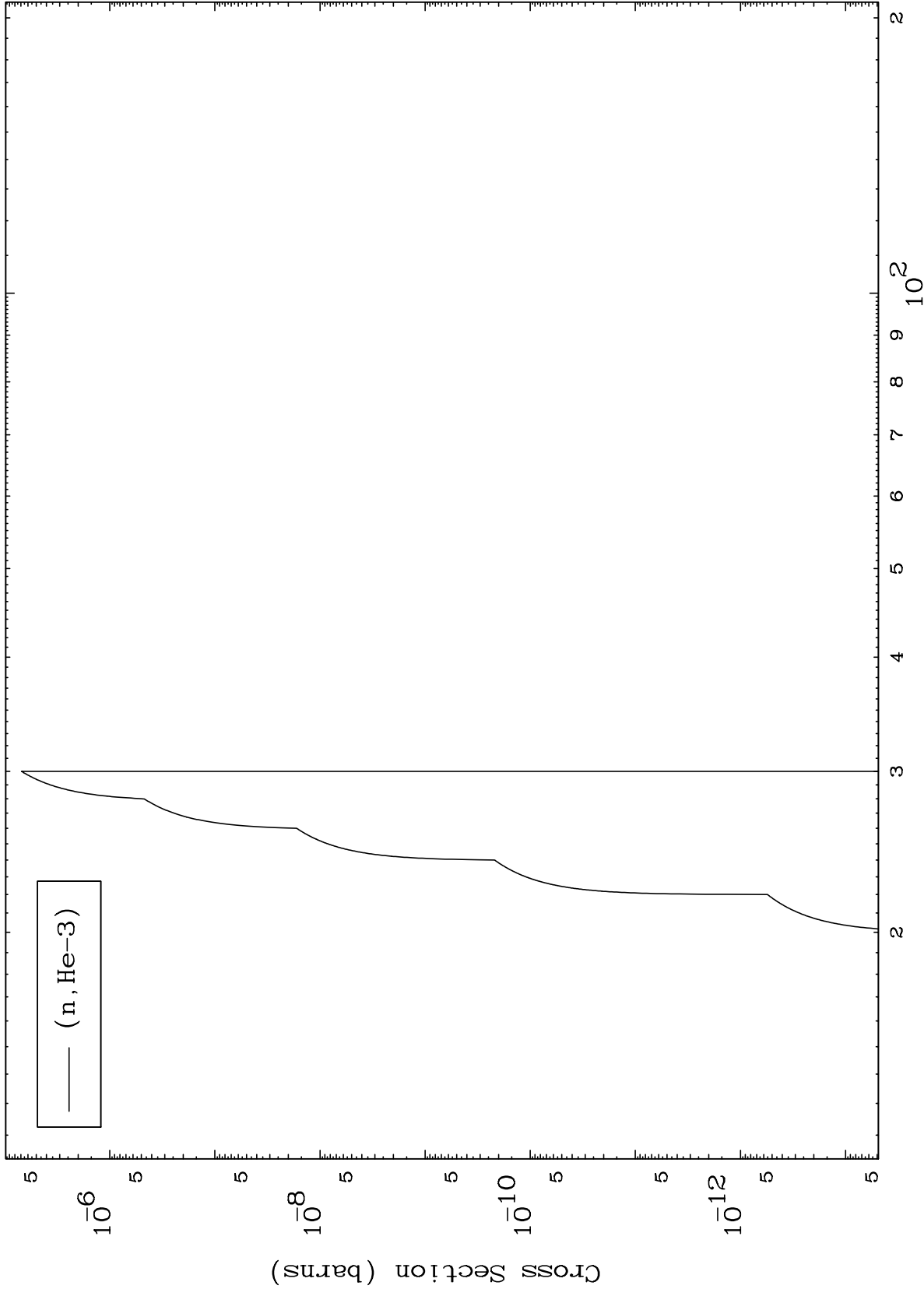
Incident Energy (MeV)

71-Lu-168





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

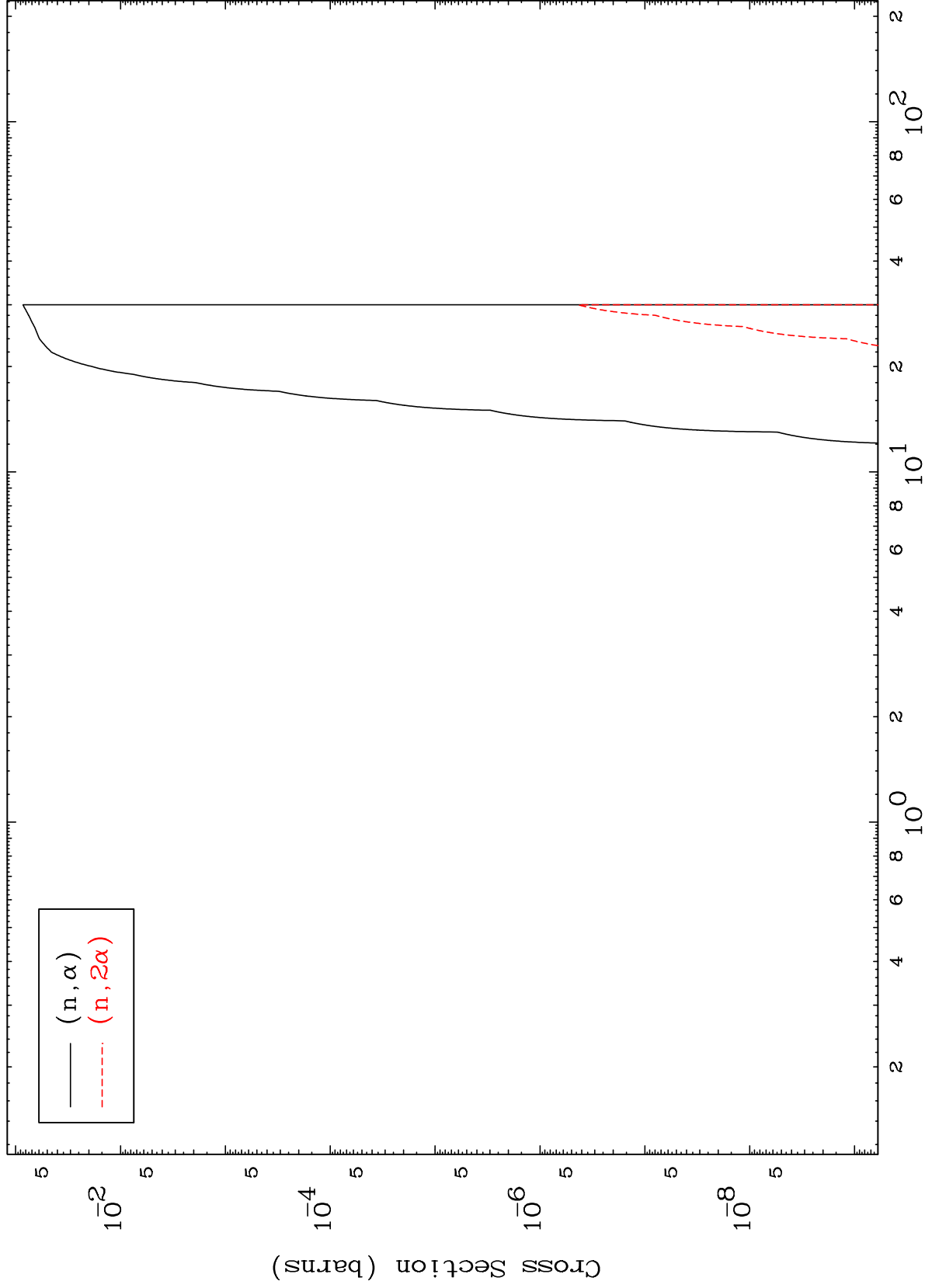


MAT 7104

( $\alpha, \alpha$ ) Levels

71-Lu-168

0 Kelvin Cross Sections

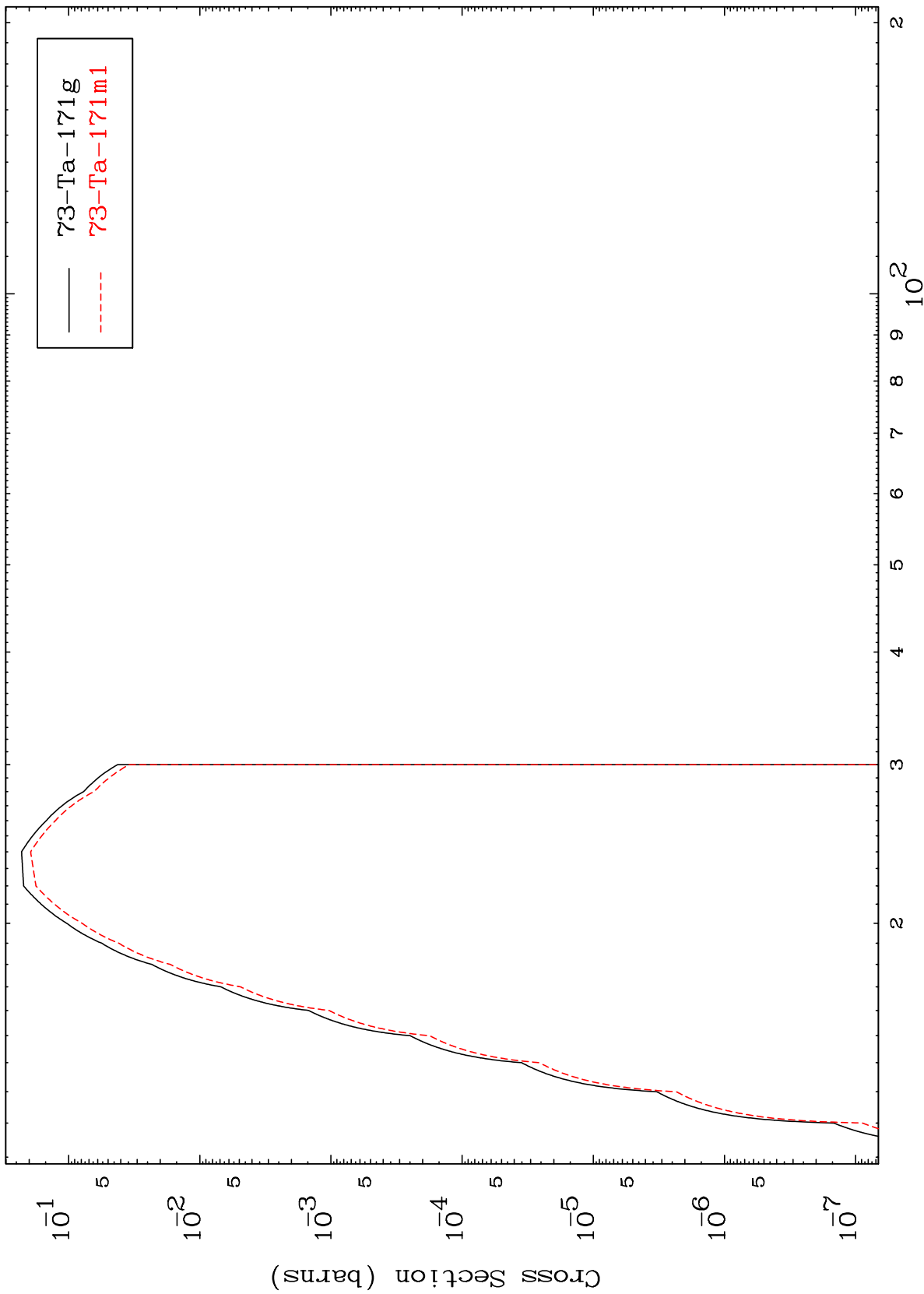


10

Incident Energy (MeV)

71-Lu-168

Inelastic  
Radionuclide Production Cross Section

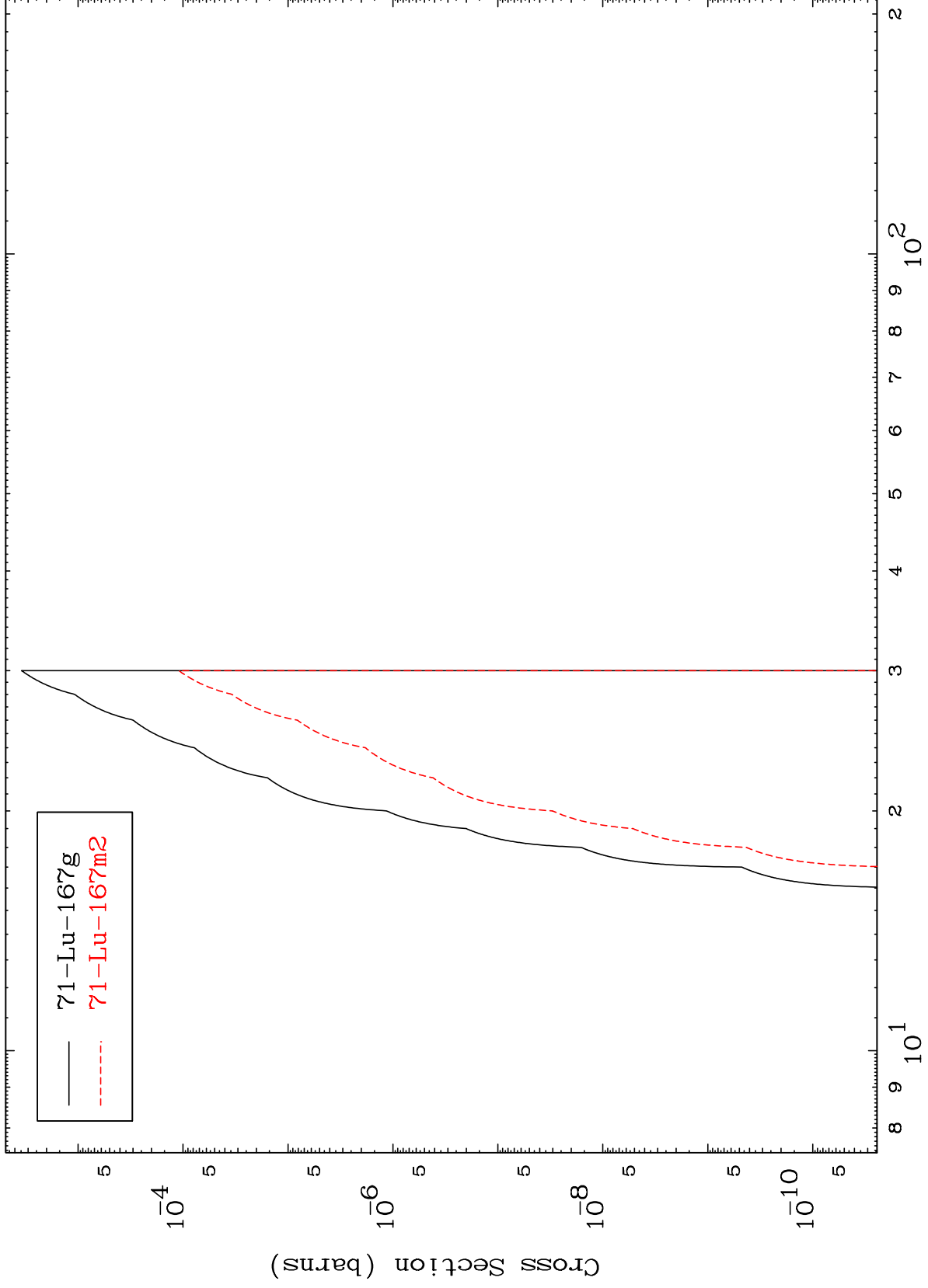


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$(n, n') \alpha$

$^{71}\text{Lu-168}$

Radionuclide Production Cross Section

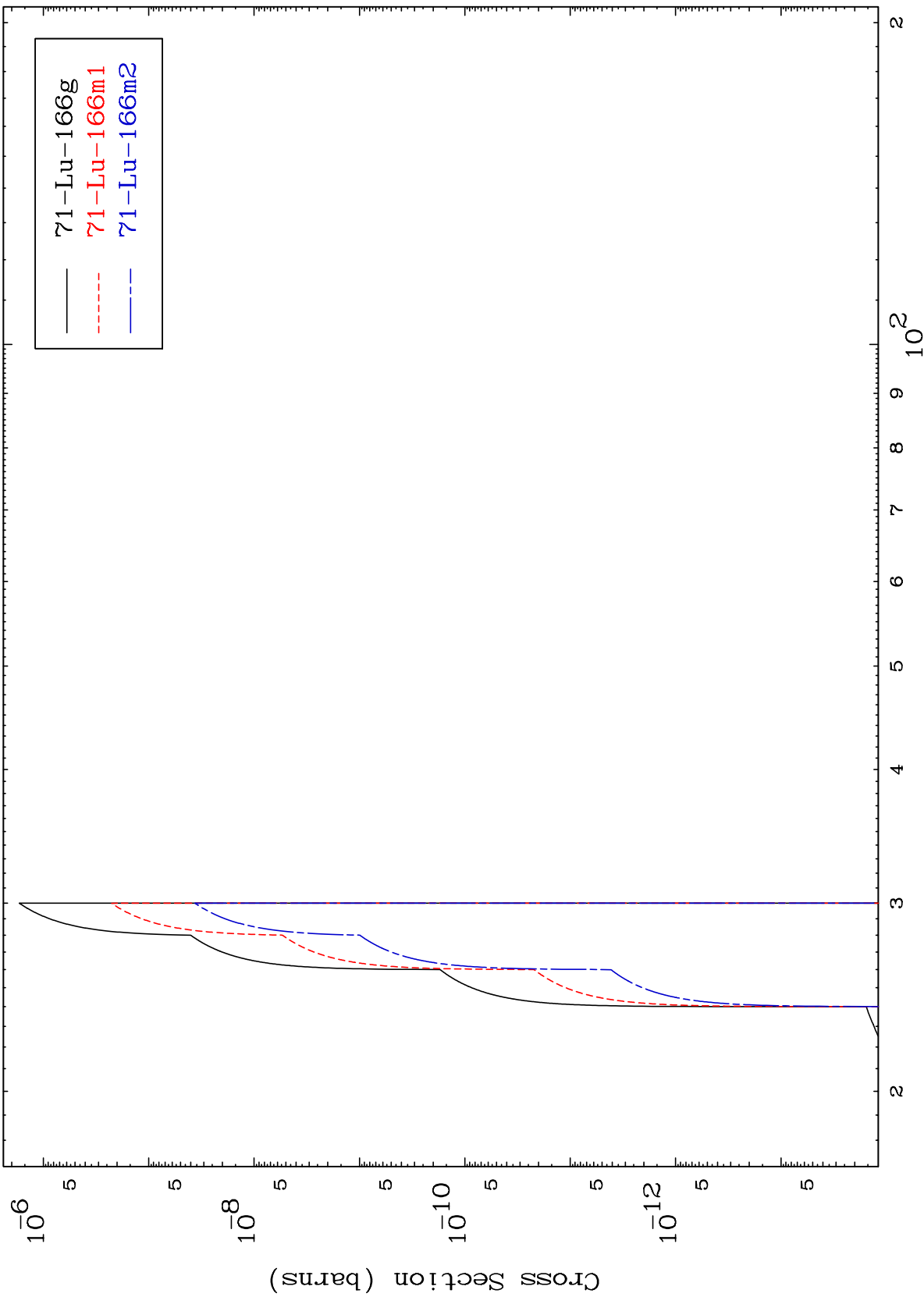


12

Incident Energy (MeV)

$^{71}\text{Lu-168}$

Radionuclide Production Cross Section

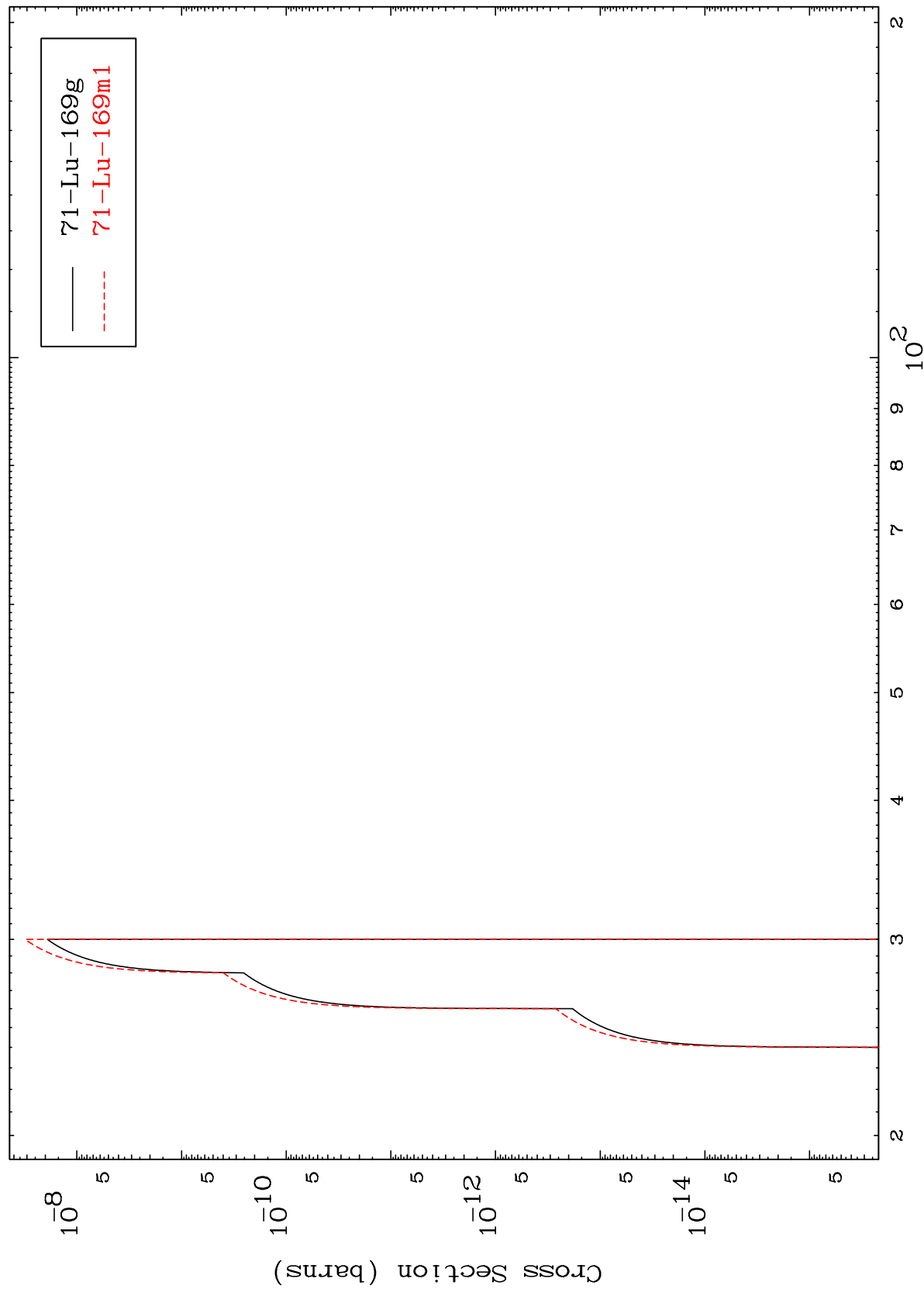


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(n,2n) p

71-Lu-168

Radionuclide Production Cross Section



14

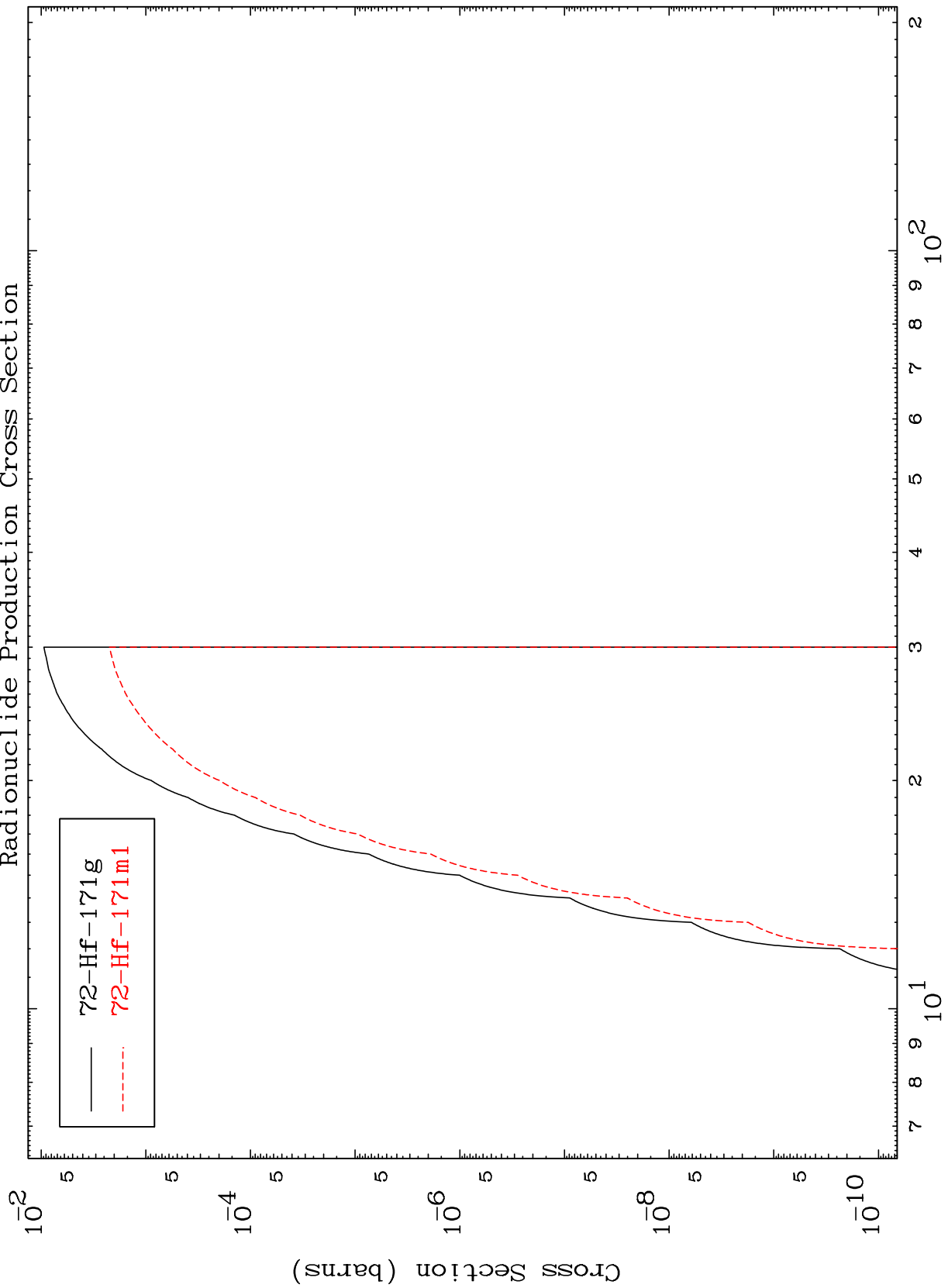
Incident Energy (MeV)

71-Lu-168

MAT 7104

71-Lu-168

Radionuclide Production Cross Section (n,p)



72-Hf-171g  
72-Hf-171m1

15

Incident Energy (MeV)

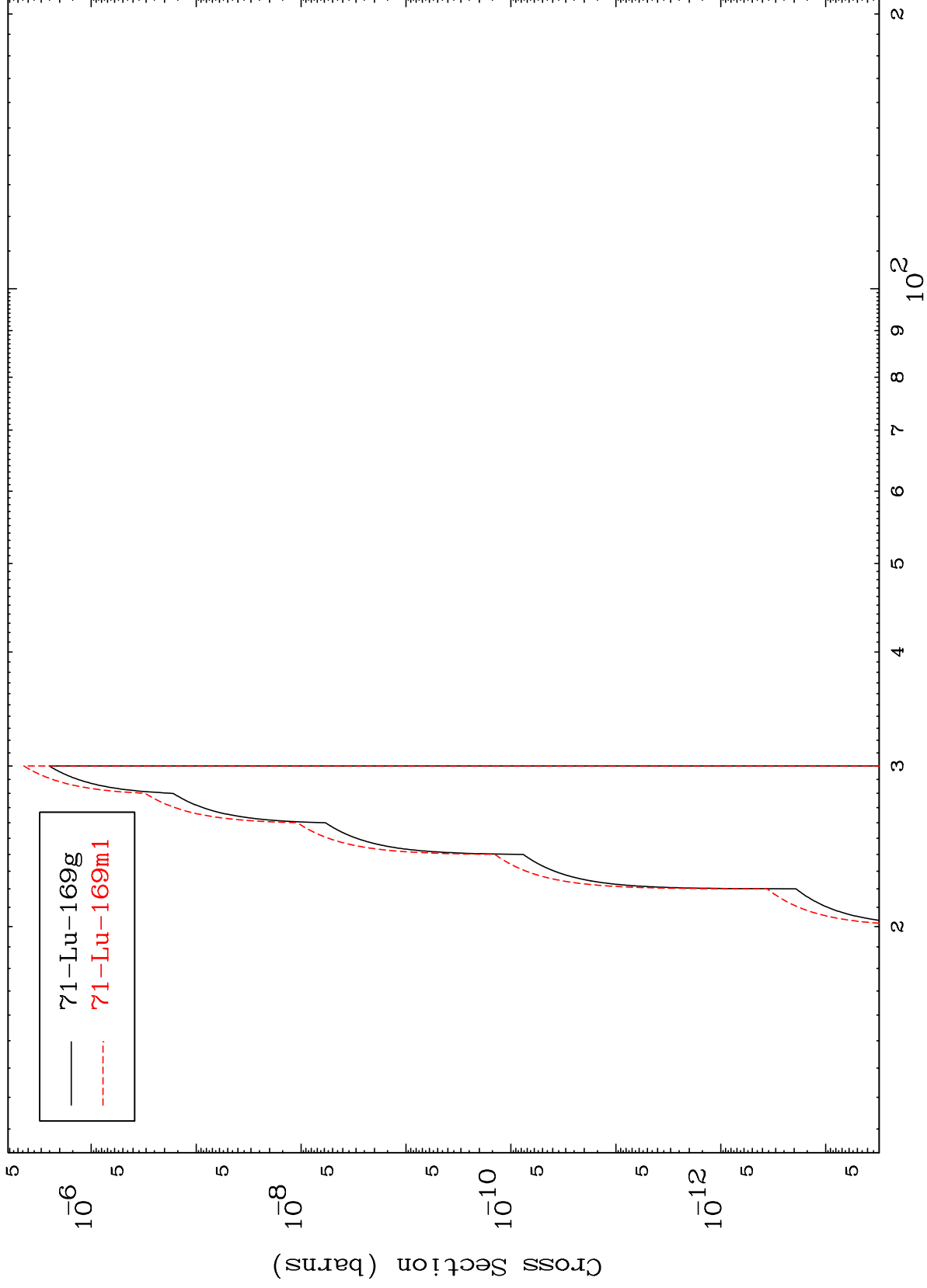
71-Lu-168

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(n,He-3)

71-Lu-168

Radionuclide Production Cross Section



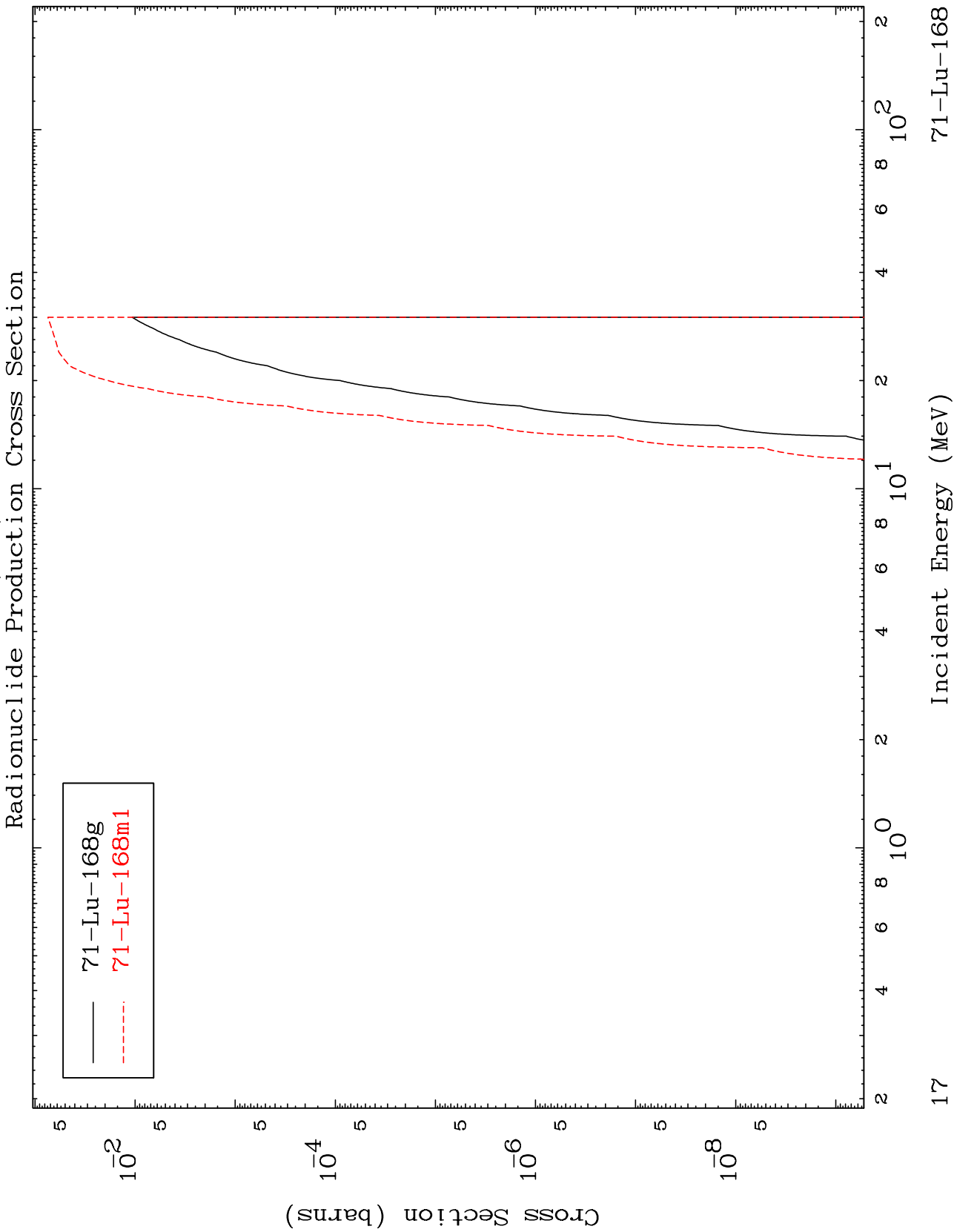
16

Incident Energy (MeV)

71-Lu-168

MAT 7104

<sup>71</sup>Lu-168



<sup>71</sup>Lu-168

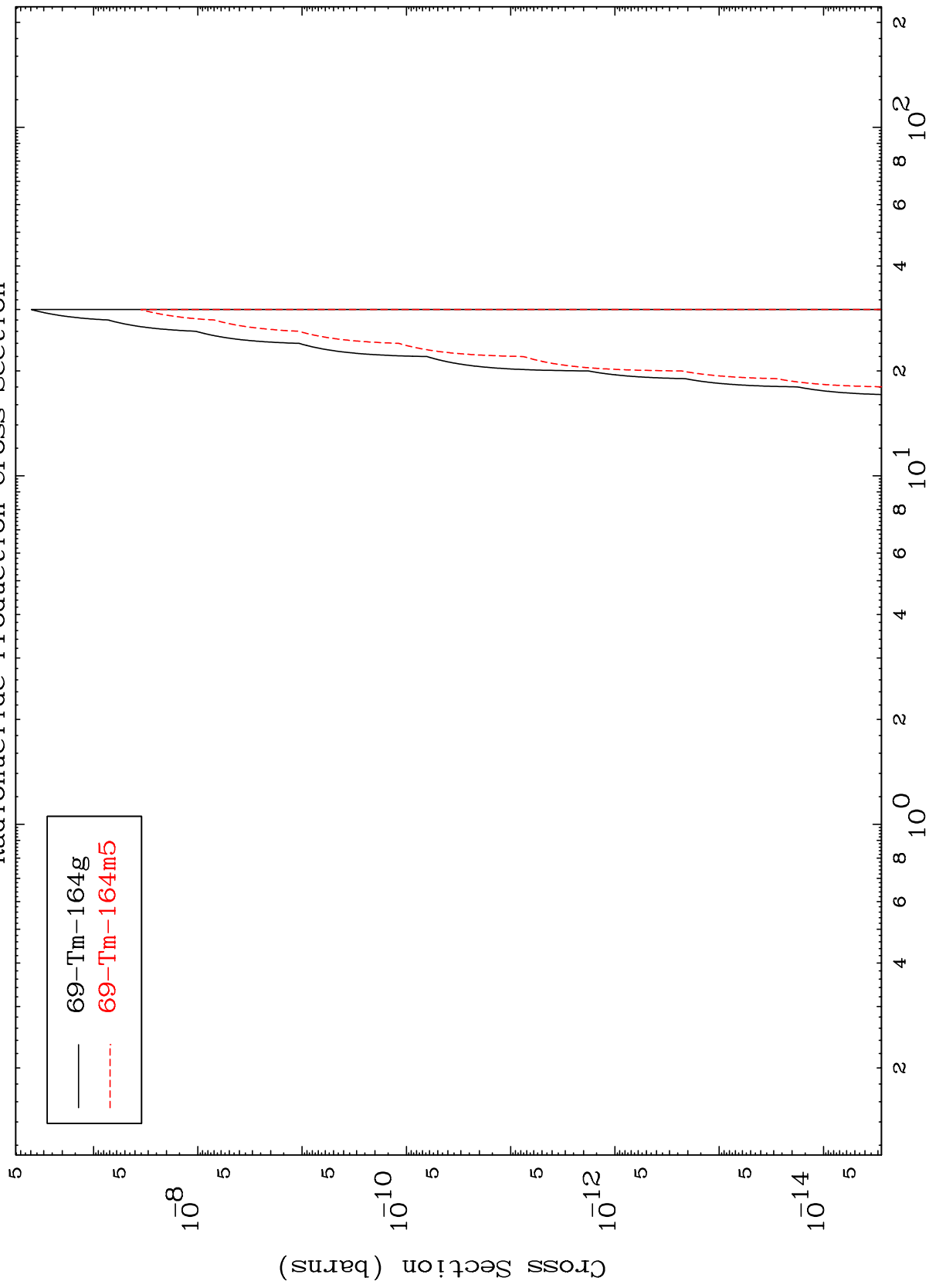
Incident Energy (MeV)

17

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71-Lu-168

Radionuclide Production Cross Section  
(n,2α)



71-Lu-168

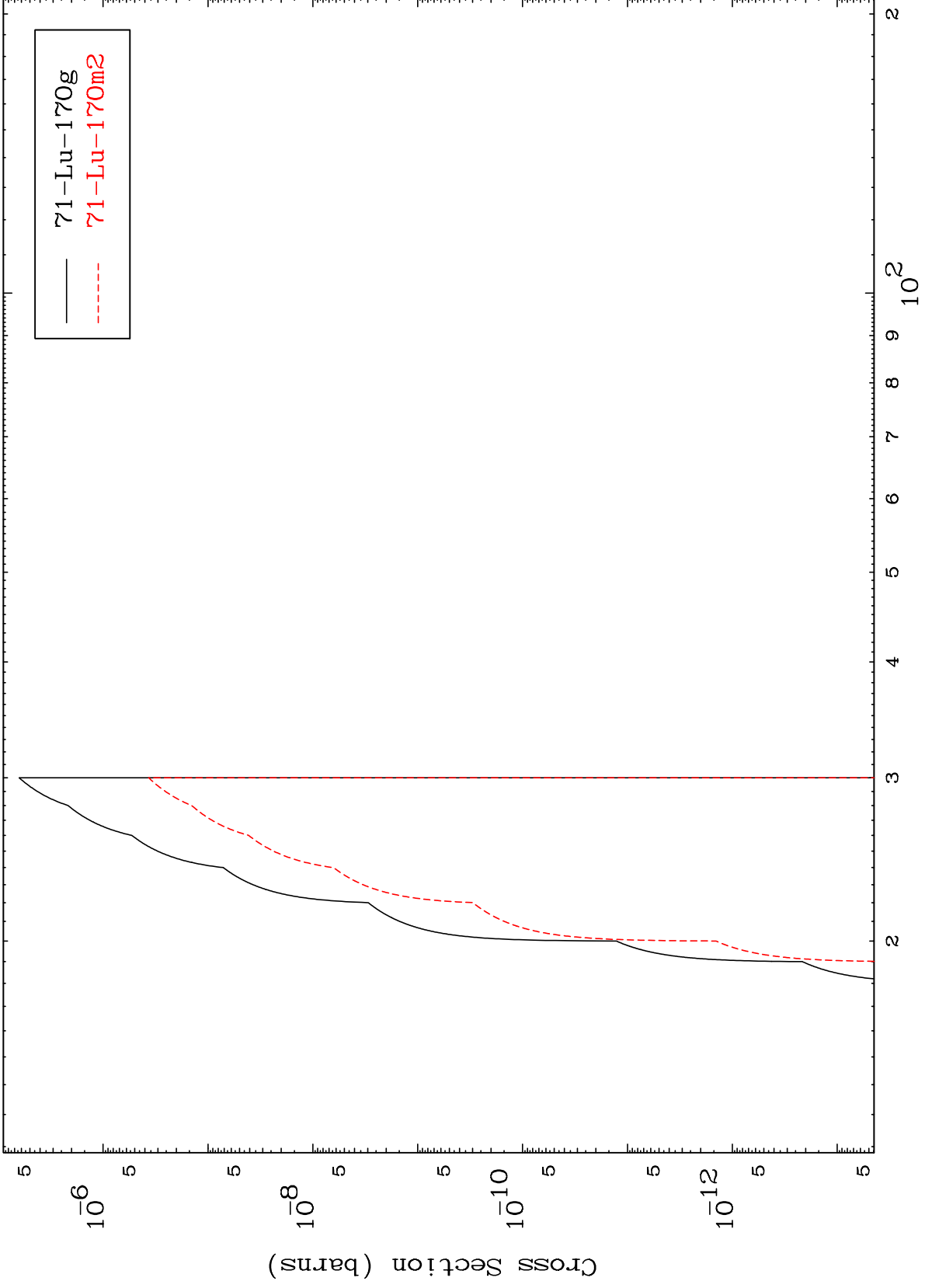
Incident Energy (MeV)

18

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71-Lu-168

(n,2p)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

71-Lu-168

Radionuclide Production Cross Section

