

Program EVALPLOT  
(Version 2021-1)

by

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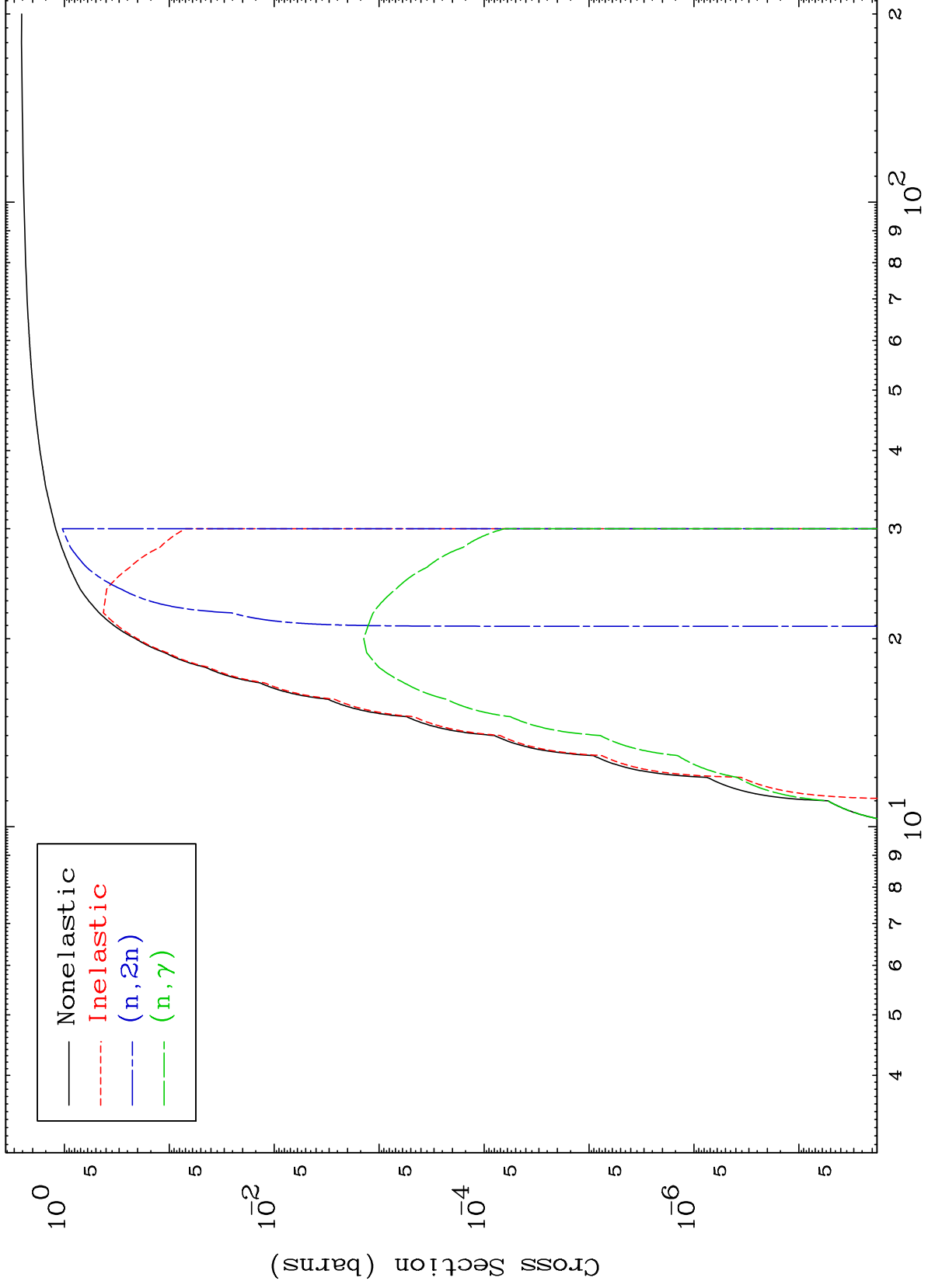
Press Mouse Button to Start

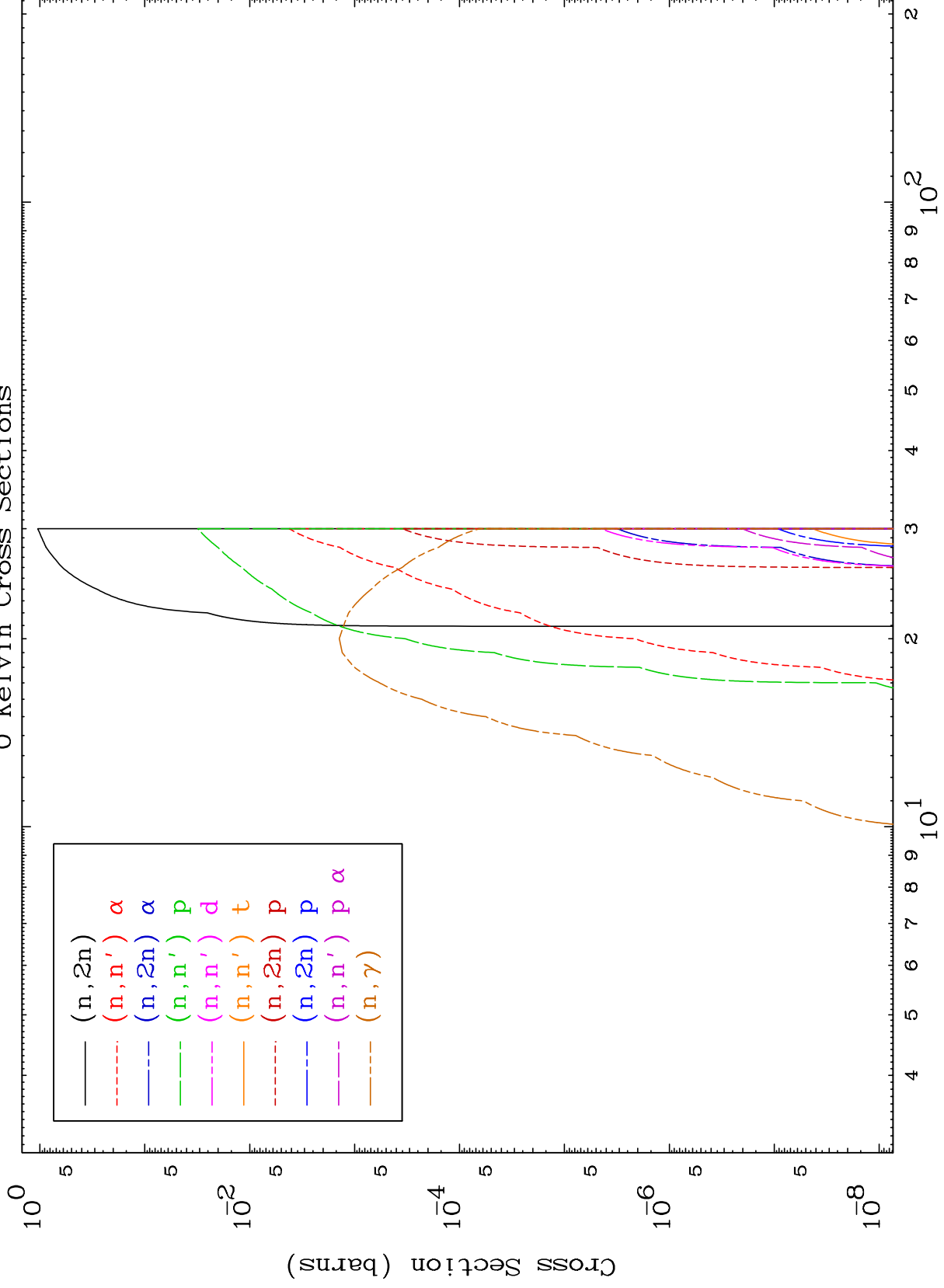
MAT 7105

0 Kelvin Cross Sections

$\alpha$  Major

<sup>71</sup>Lu-168m

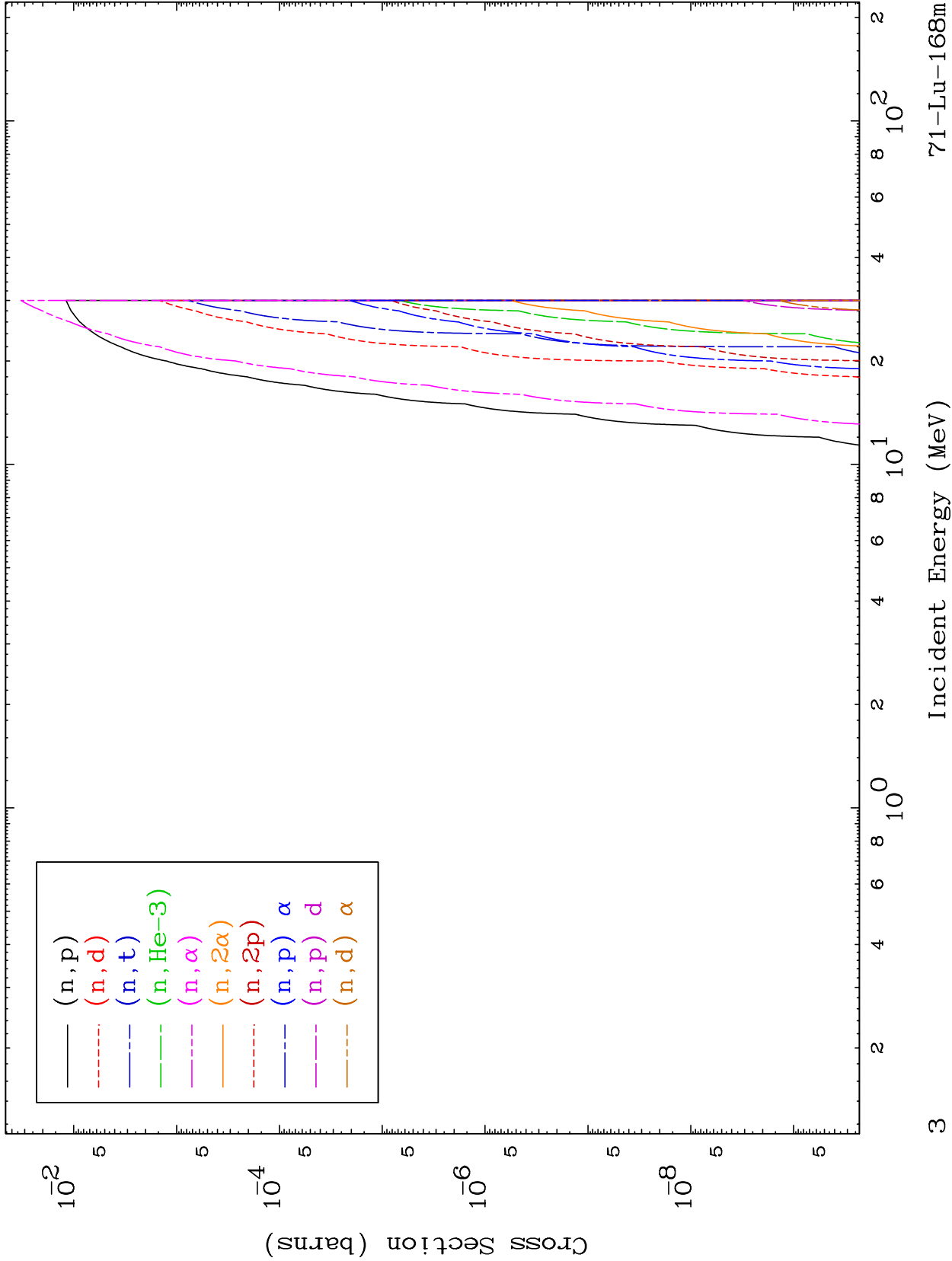


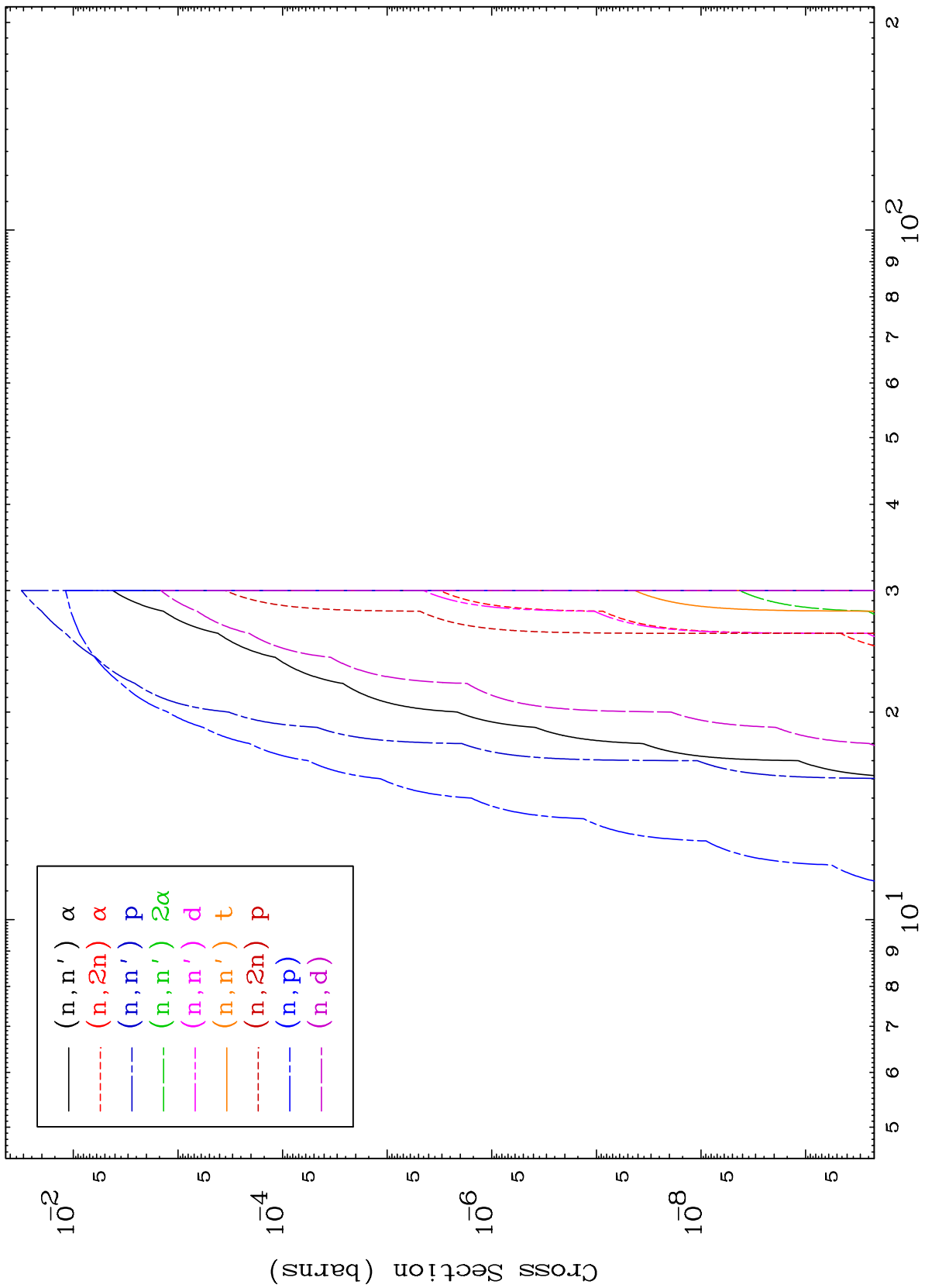


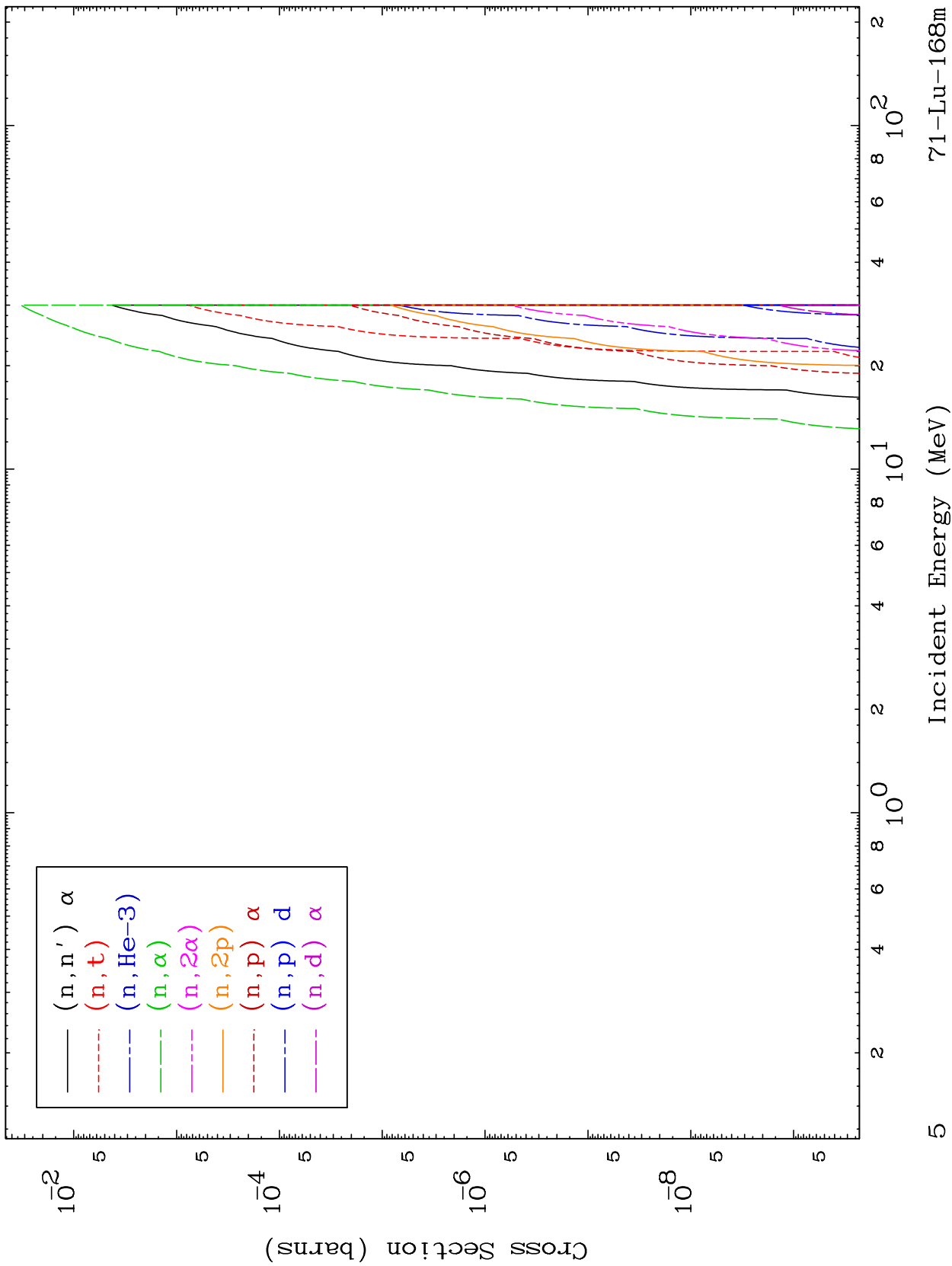
MAT 7105

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

<sup>71</sup>Lu-168m





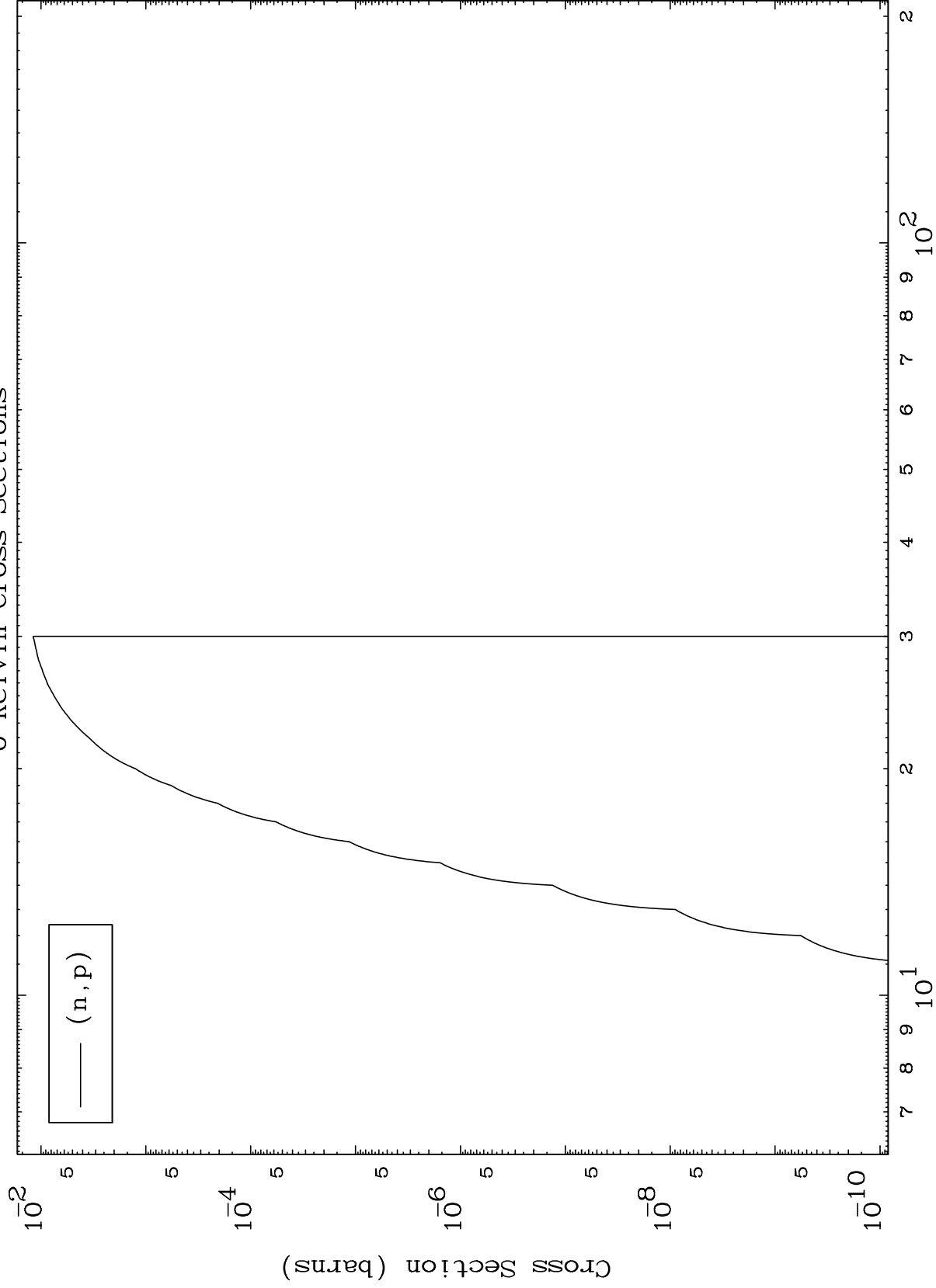


MAT 7105

( $\alpha, p$ ) Levels

<sup>71</sup>Lu-168m

0 Kelvin Cross Sections



6

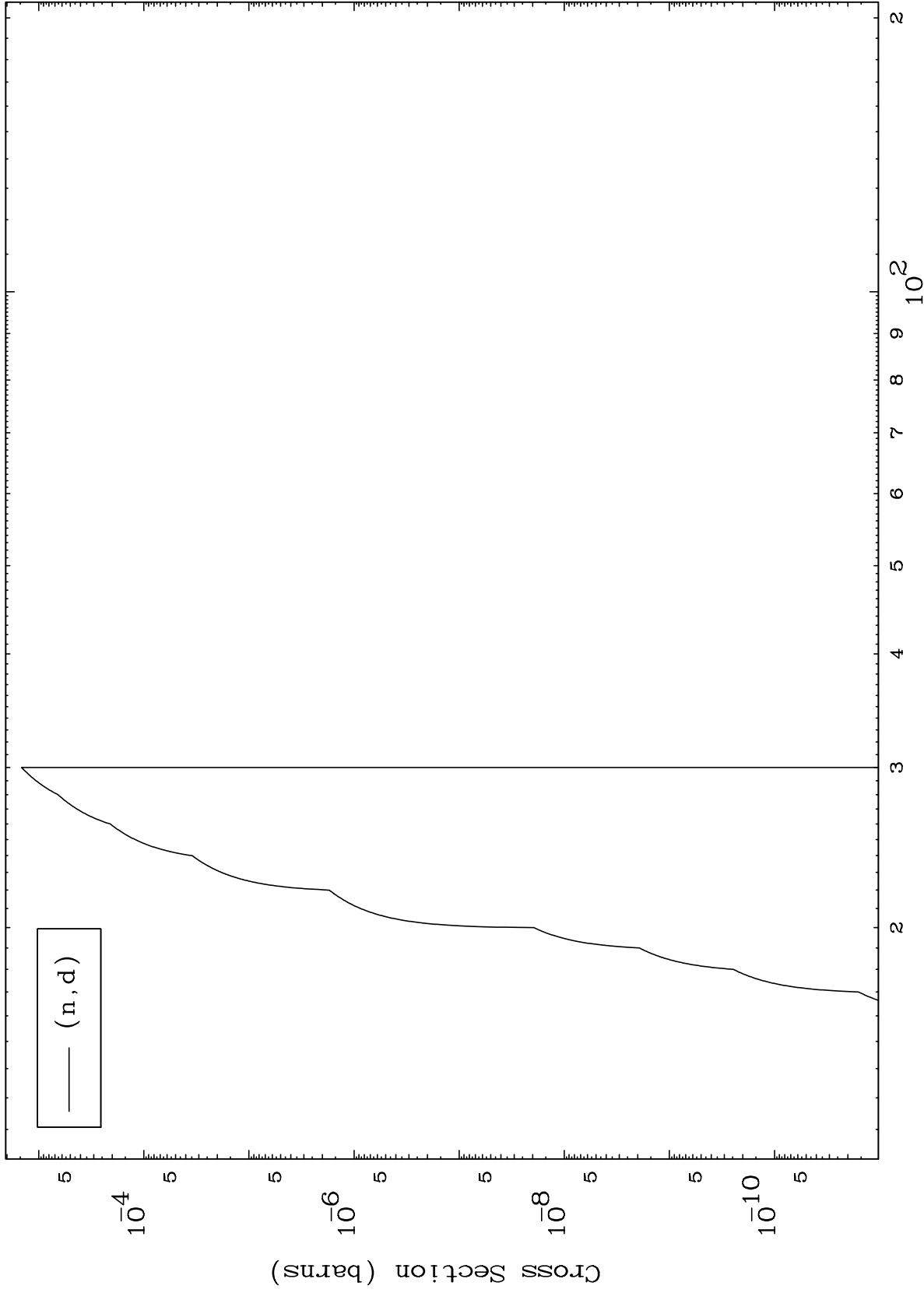
Incident Energy (MeV)

<sup>71</sup>Lu-168m

MAT 7105

71-Lu-168m

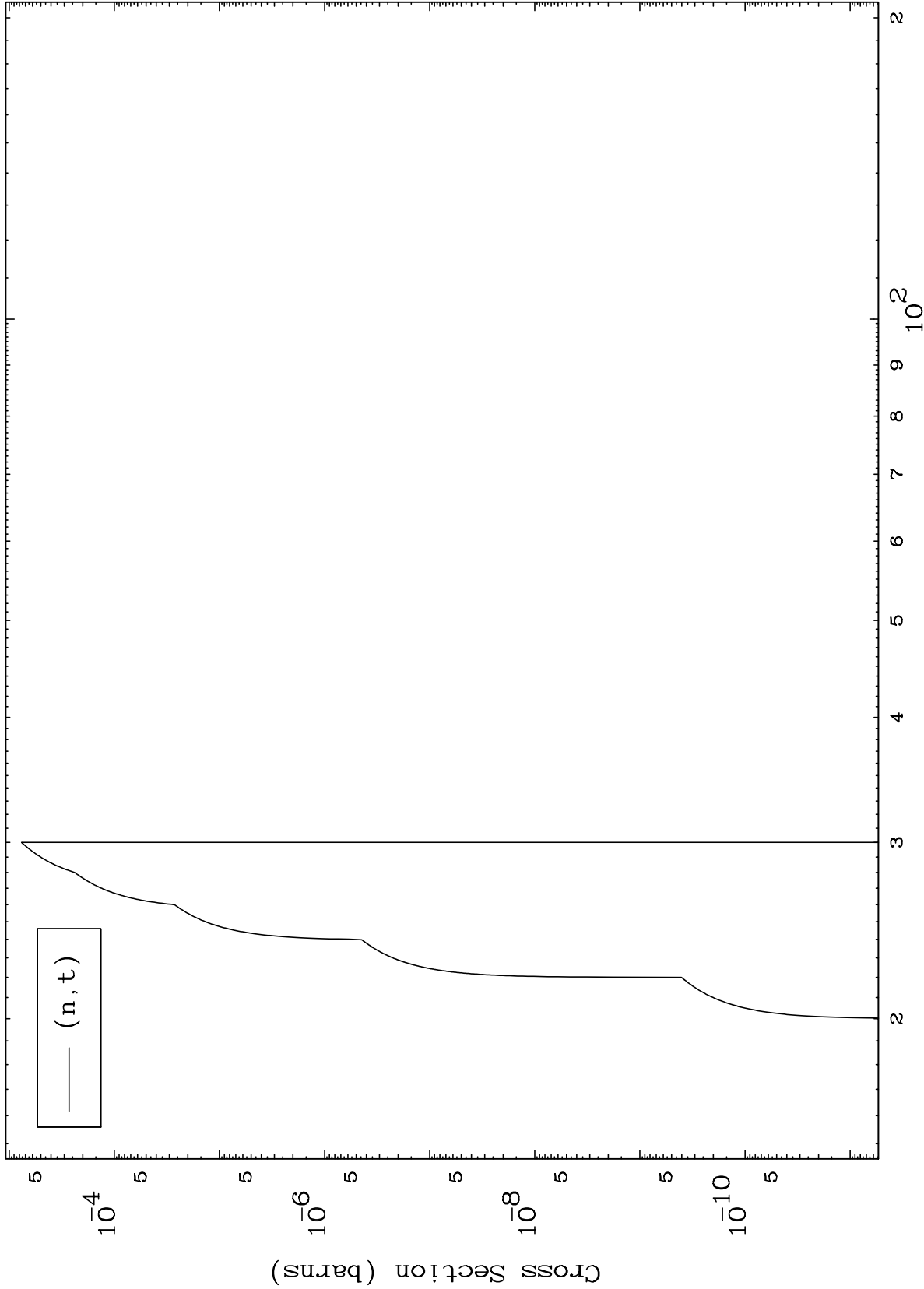
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

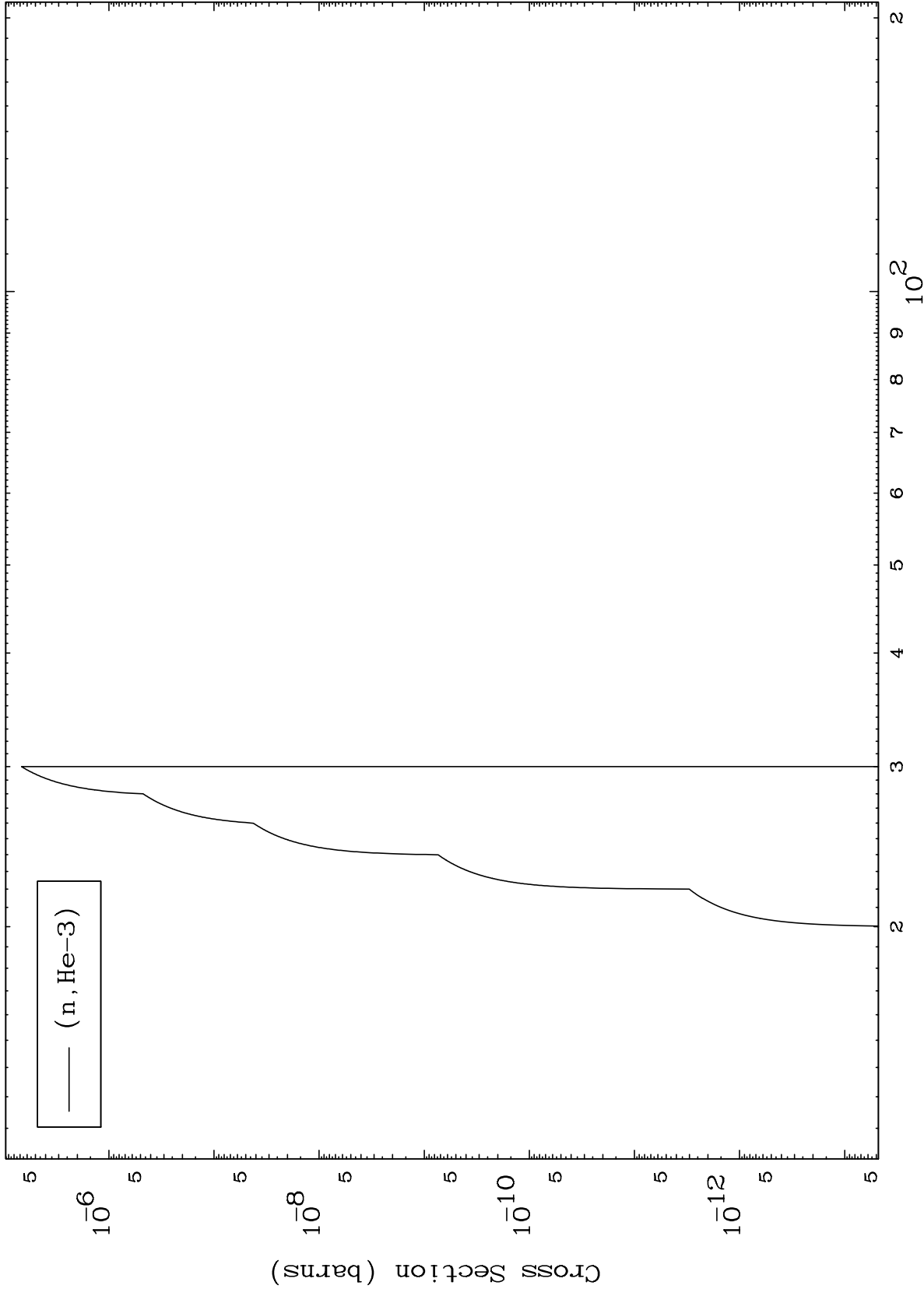


71-Lu-168m

Incident Energy (MeV)

7



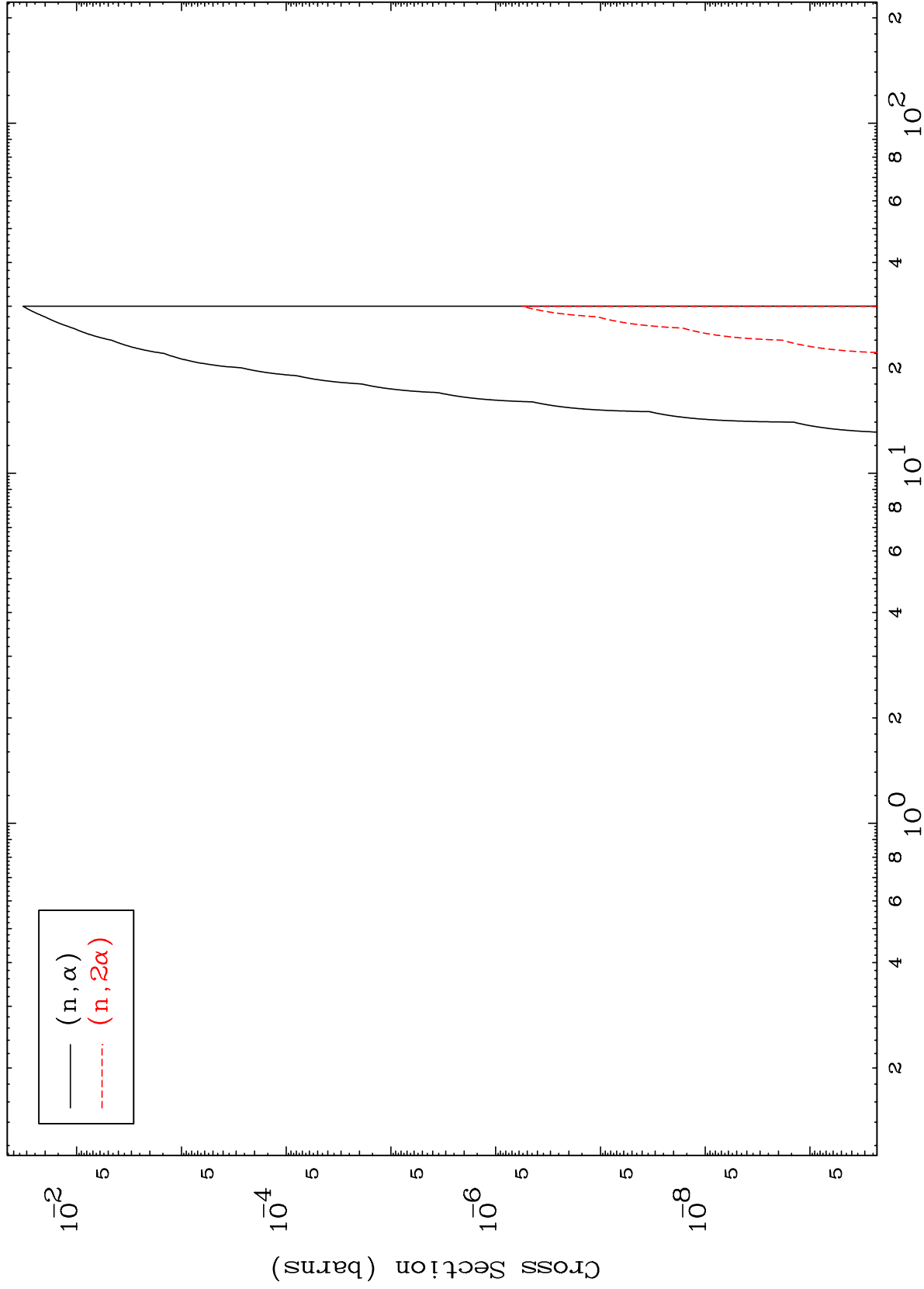


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( $\alpha, \alpha$ ) Levels

71-Lu-168m

0 Kelvin Cross Sections

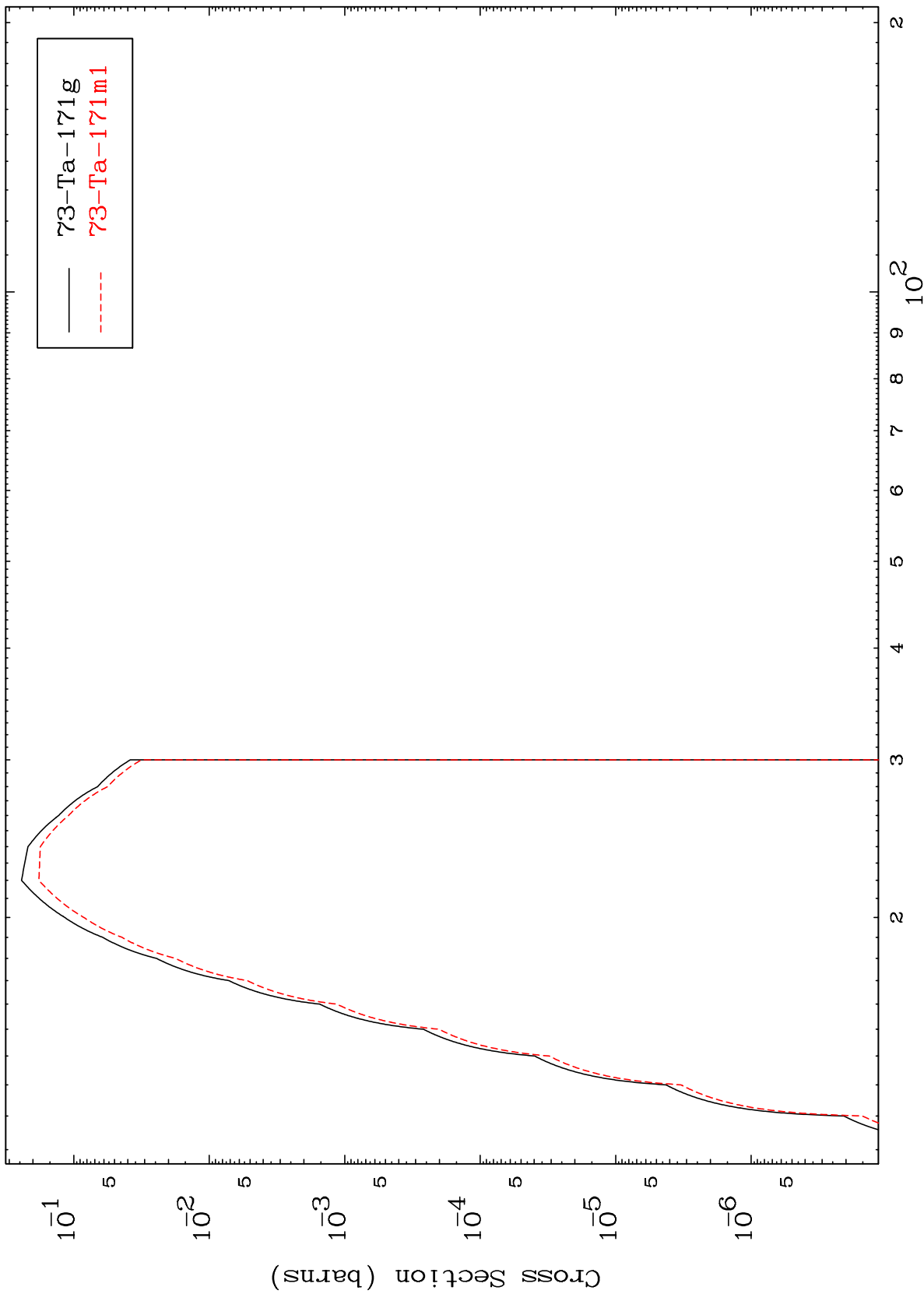


10

Incident Energy (MeV)

71-Lu-168m

Inelastic  
Radionuclide Production Cross Section

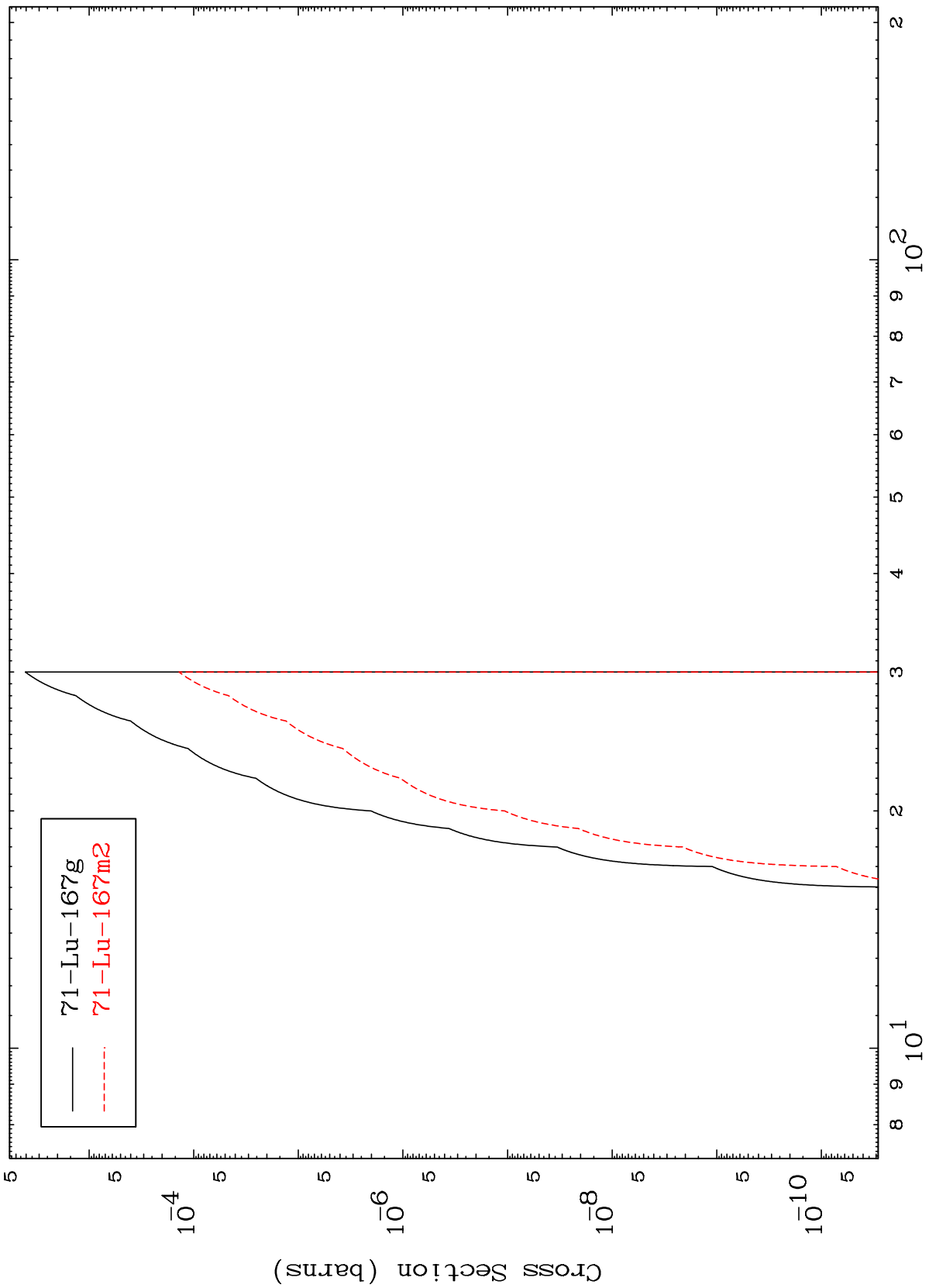


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$(n, n') \alpha$

$^{71}\text{Lu-168m}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

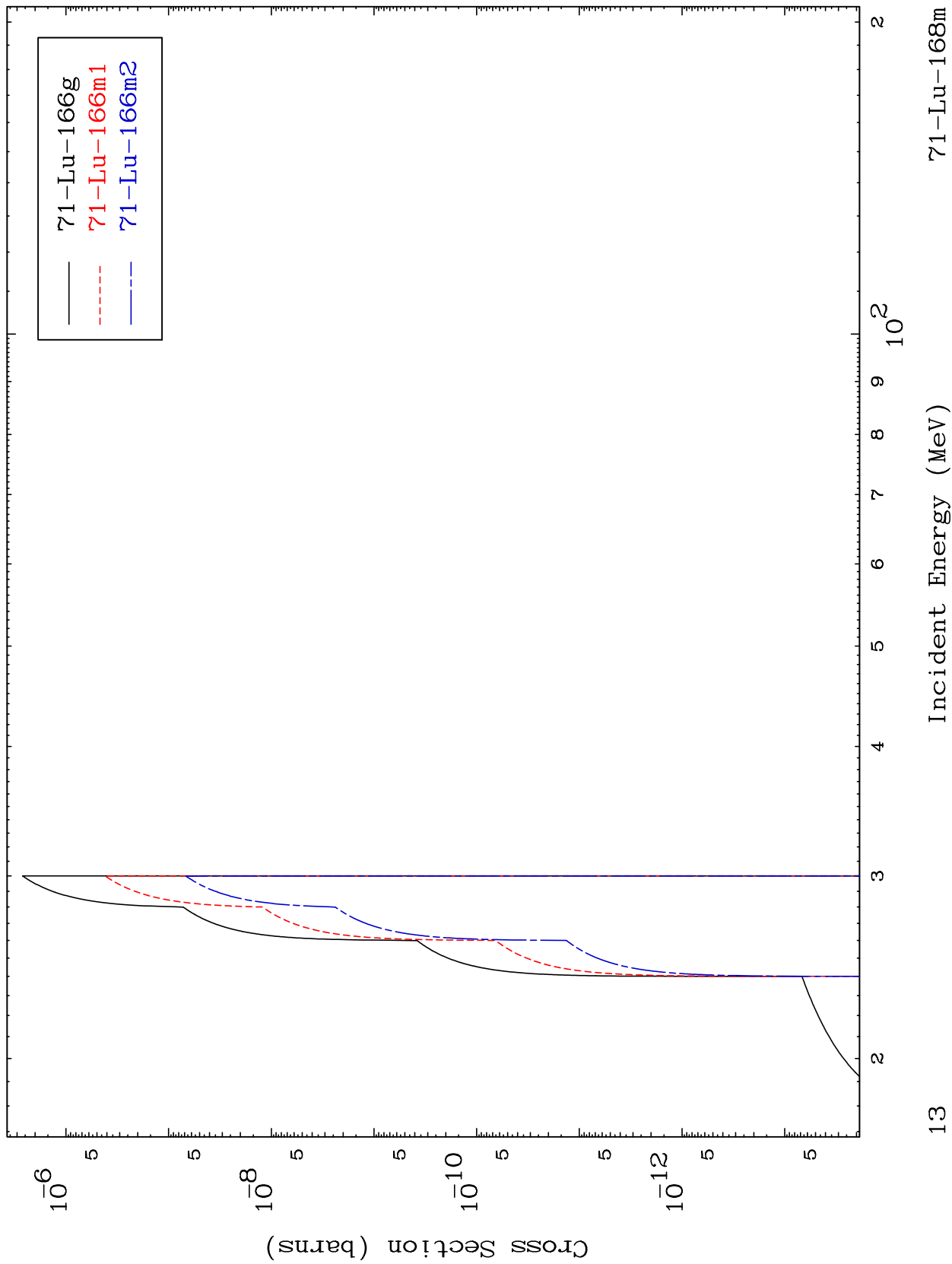
$^{71}\text{Lu-168m}$

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$(n,2n) \alpha$

$^{71}\text{Lu-168m}$

Radionuclide Production Cross Section



13

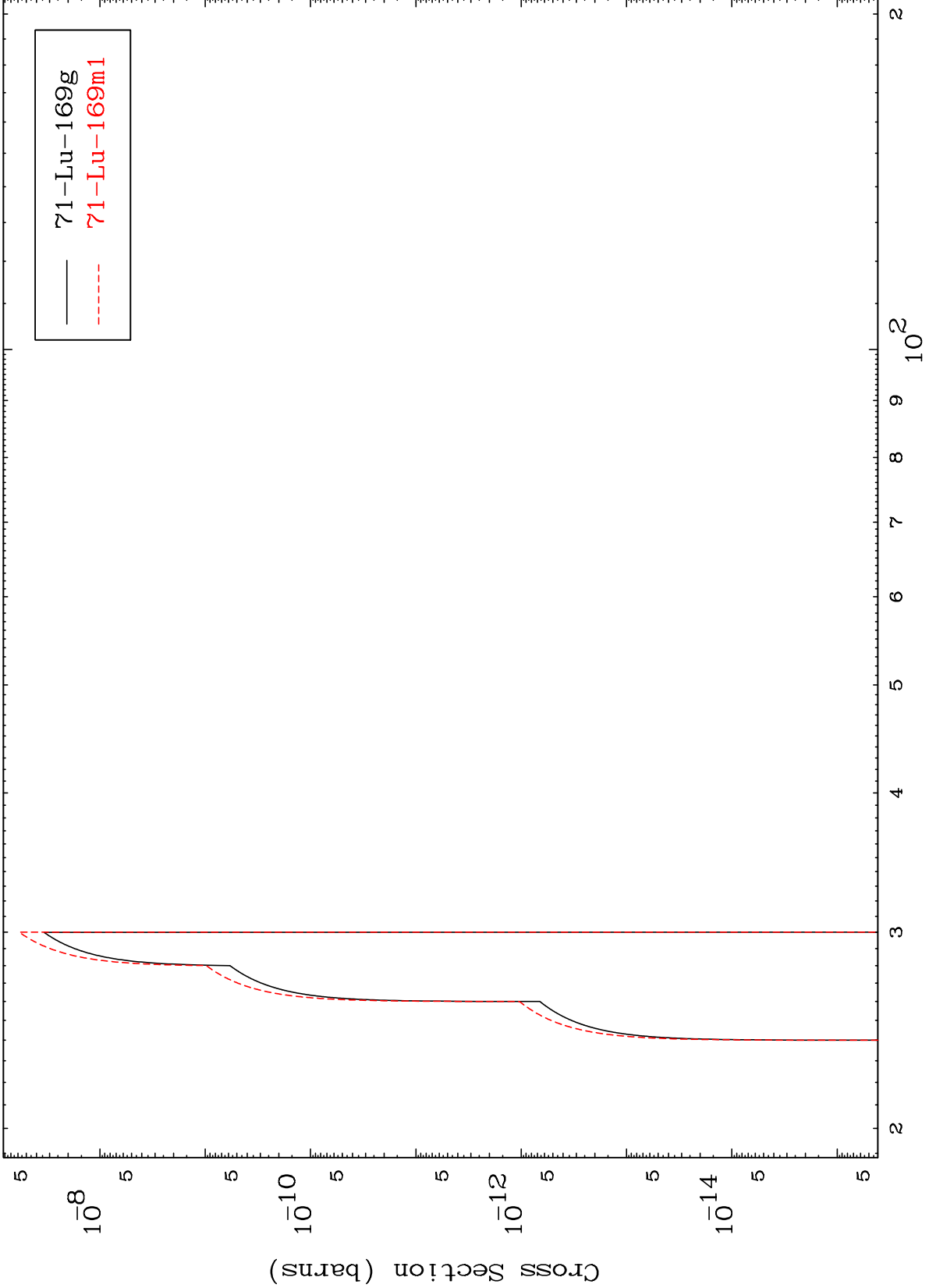
$^{71}\text{Lu-168m}$

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(n,2n) p

<sup>71</sup>Lu-168m

Radionuclide Production Cross Section



14

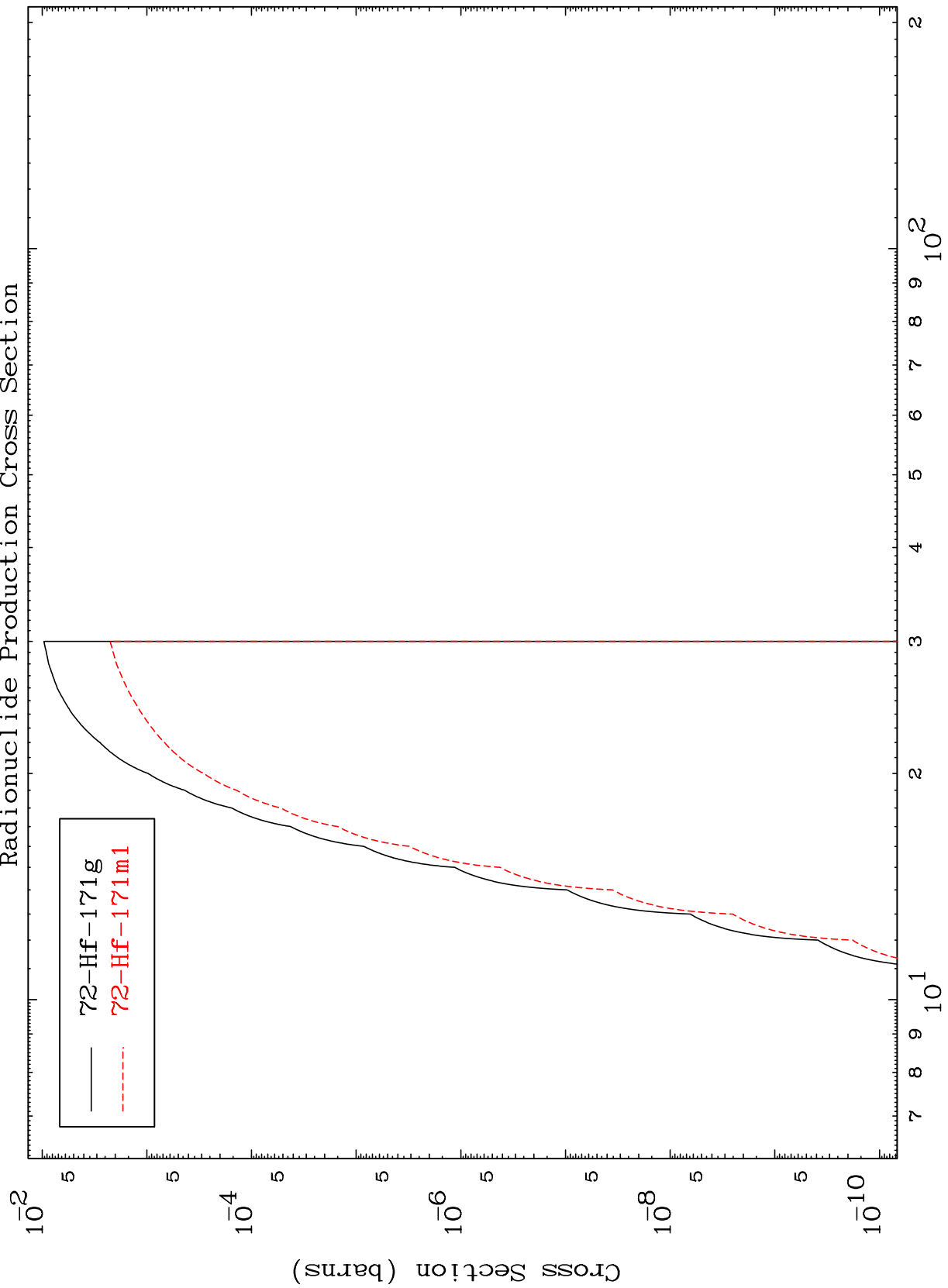
Incident Energy (MeV)

<sup>71</sup>Lu-168m

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71-Lu-168m

Radionuclide Production Cross Section (n,p)



Incident Energy (MeV)

71-Lu-168m

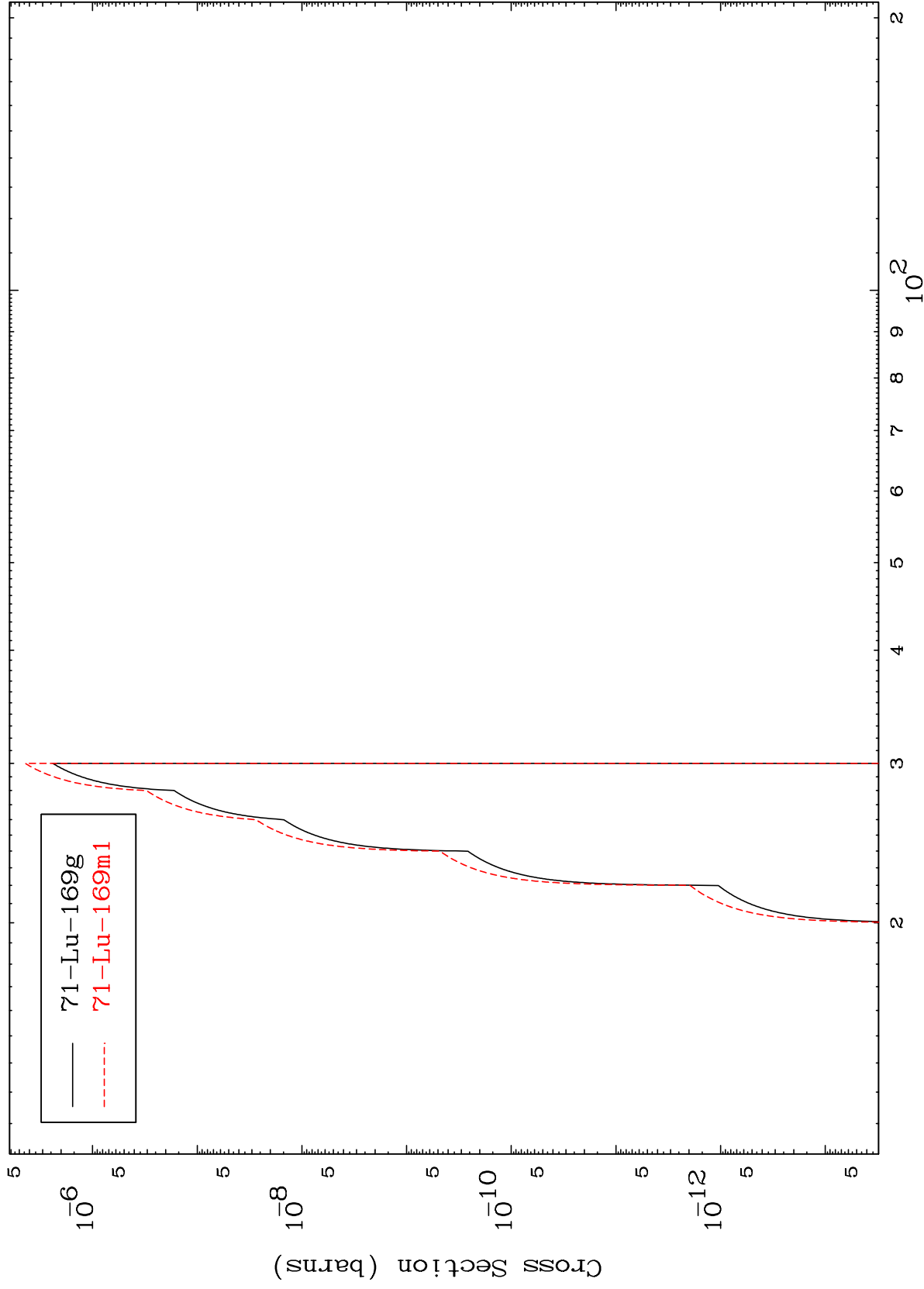
15

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(n,He-3)

71-Lu-168m

Radionuclide Production Cross Section



16

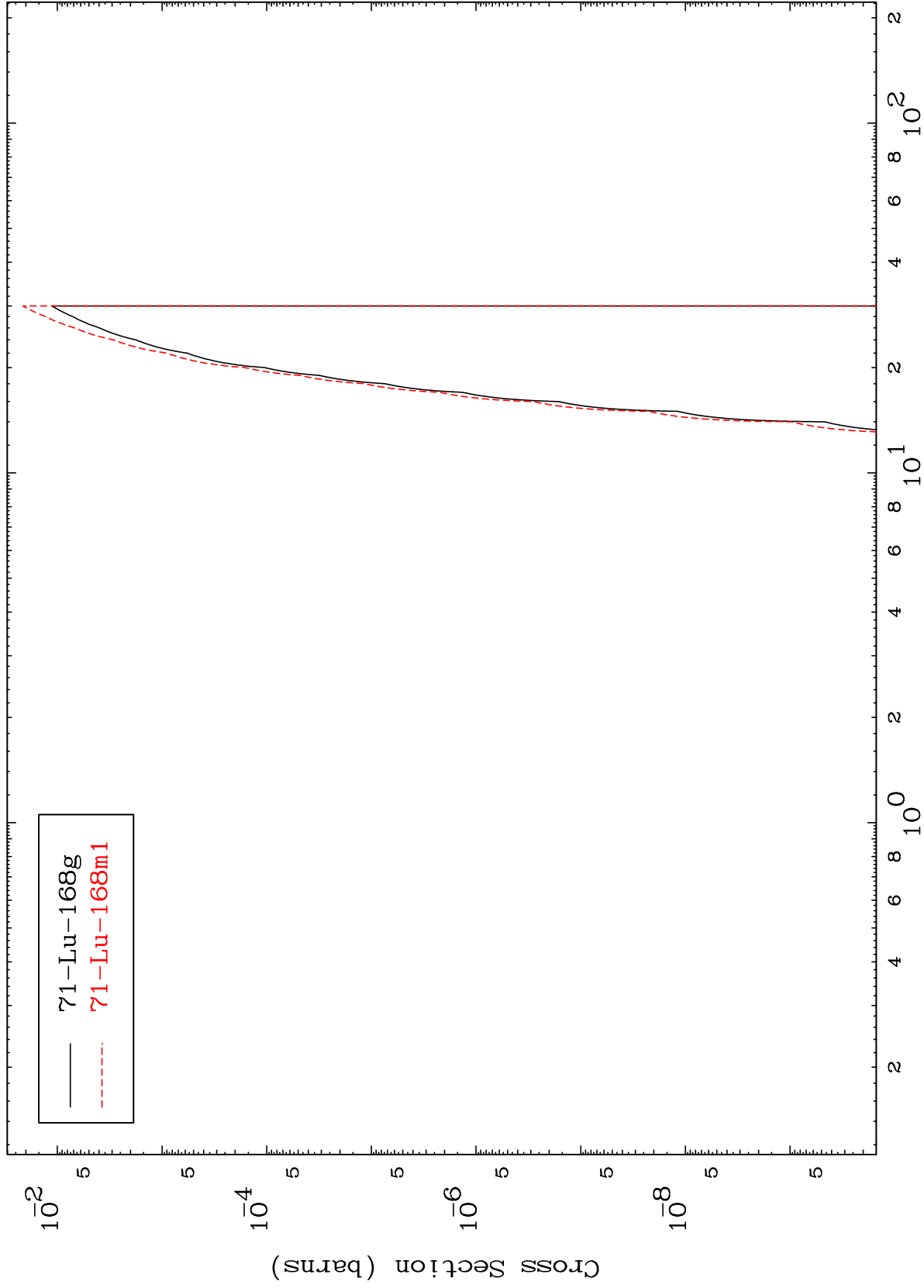
Incident Energy (MeV)

71-Lu-168m

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<sup>71</sup>Lu-168m

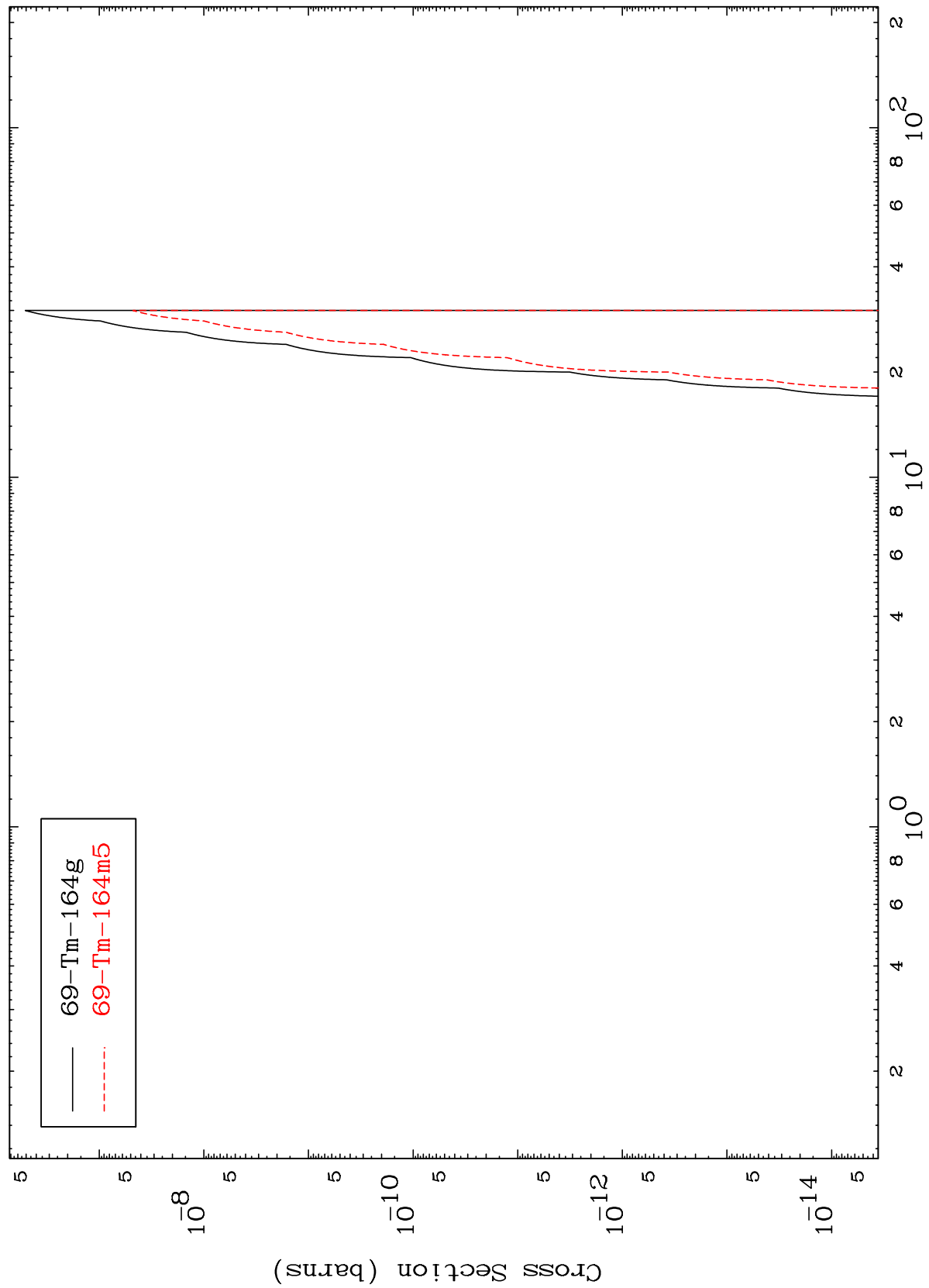
Radionuclide Production Cross Section



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71-Lu-168m

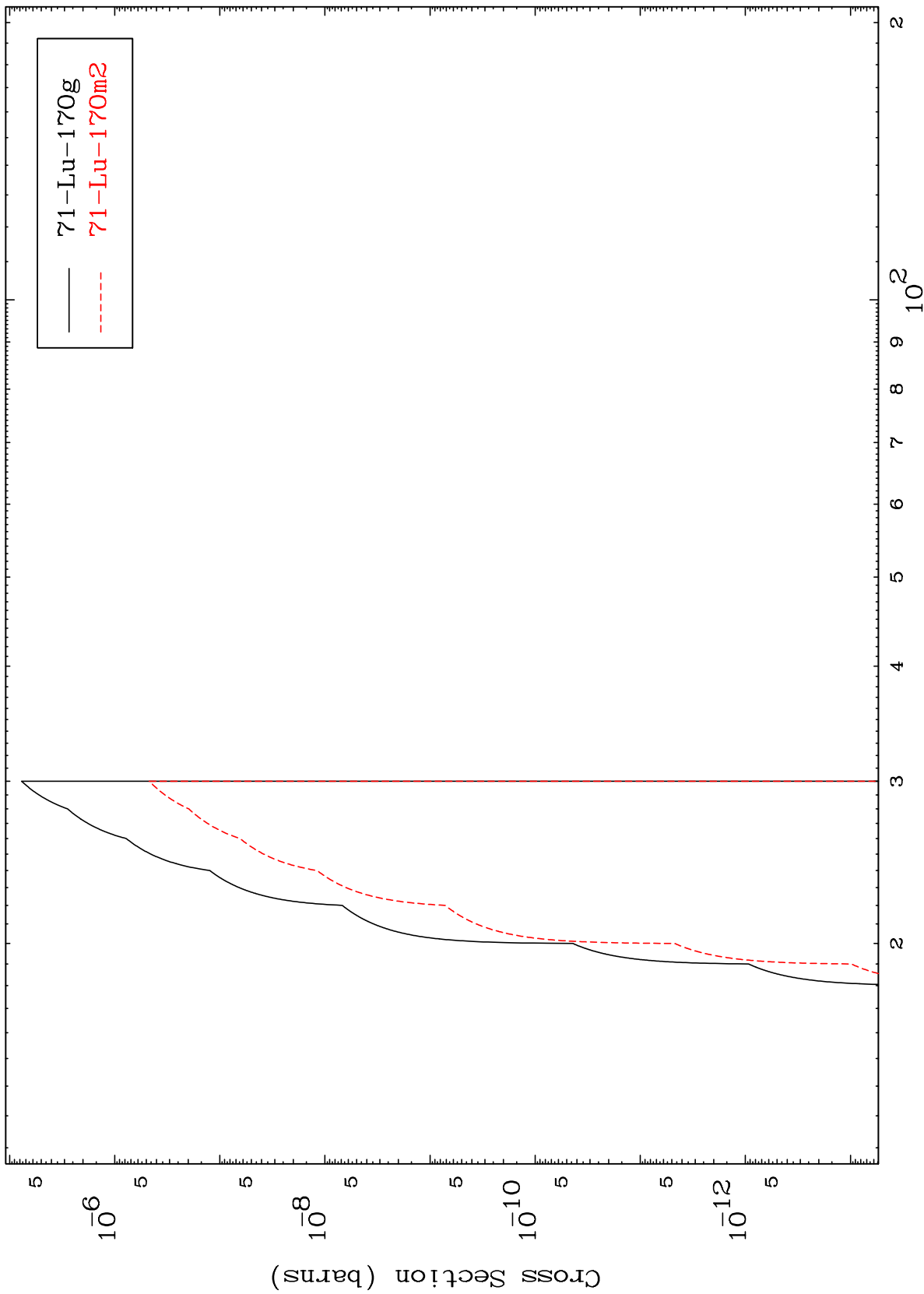
Radionuclide Production Cross Section  
(n,2 $\alpha$ )



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<sup>71</sup>Lu-168m

(n,2p)  
Radionuclide Production Cross Section

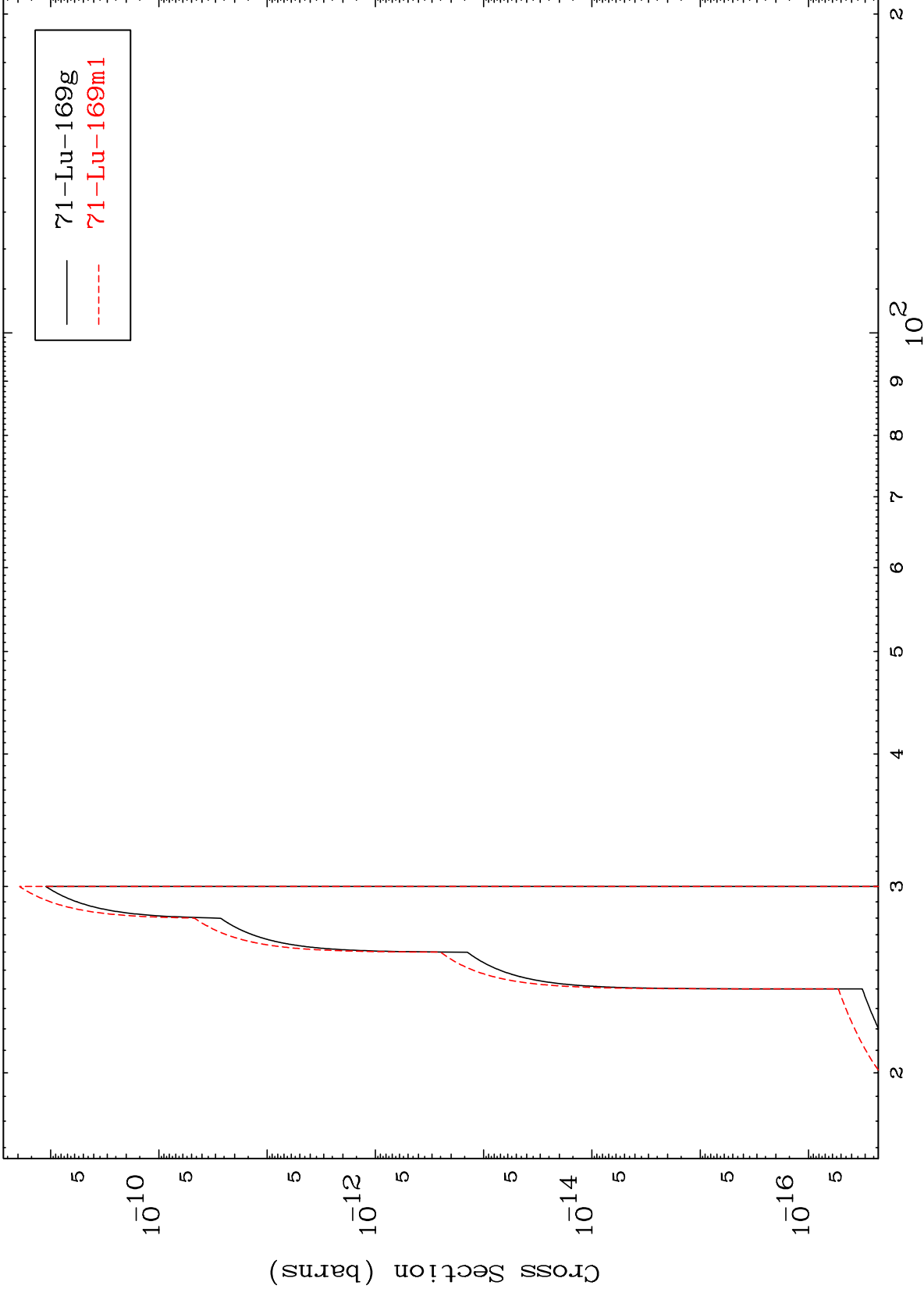


MAT 7105

(n,p) d

71-Lu-168m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

71-Lu-168m