

Program EVALPLOT
(Version 2021-1)

by

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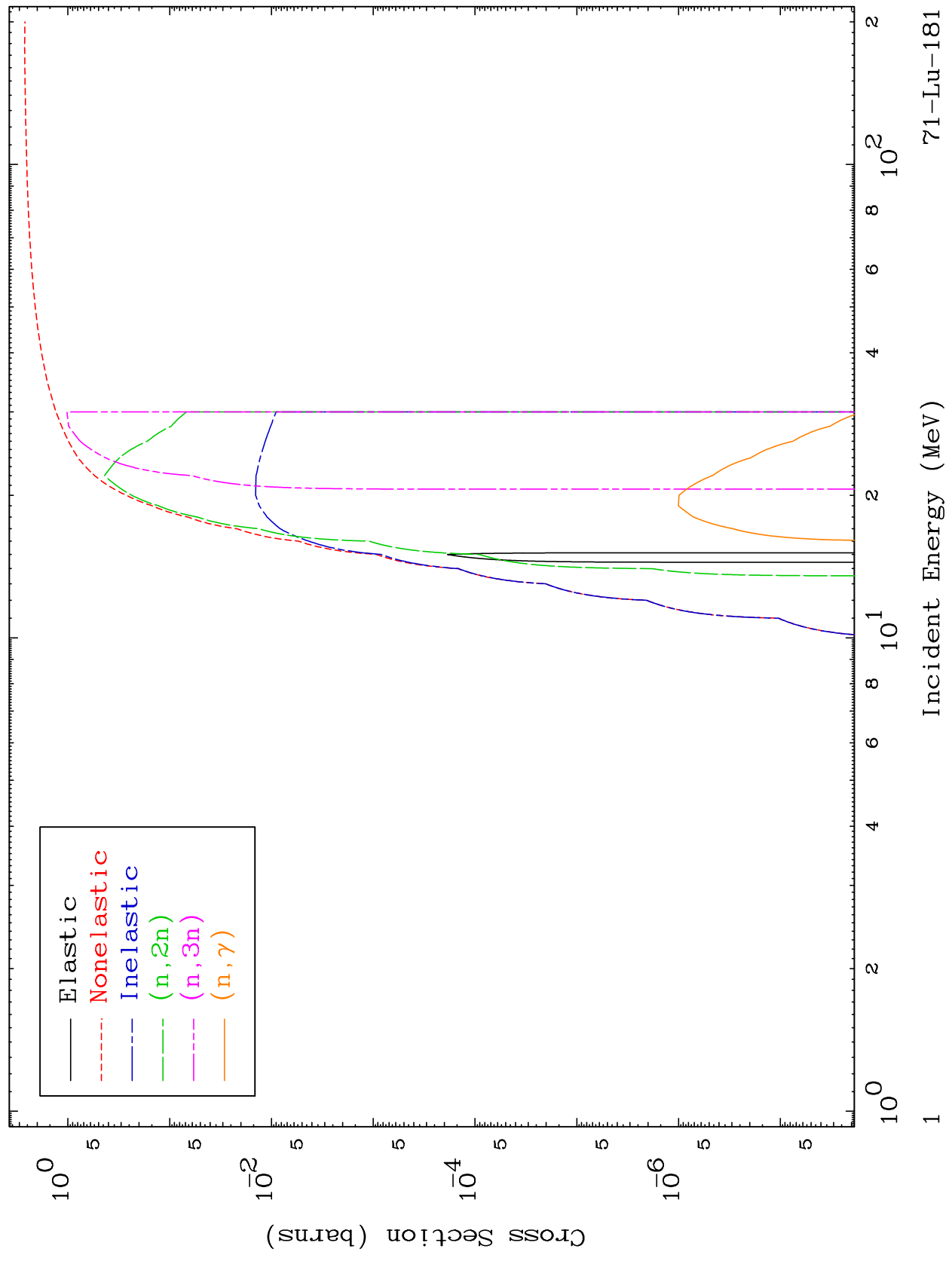
Press Mouse Button to Start

MAT 7143

0 Kelvin

Major Cross Sections

⁷¹Lu-181



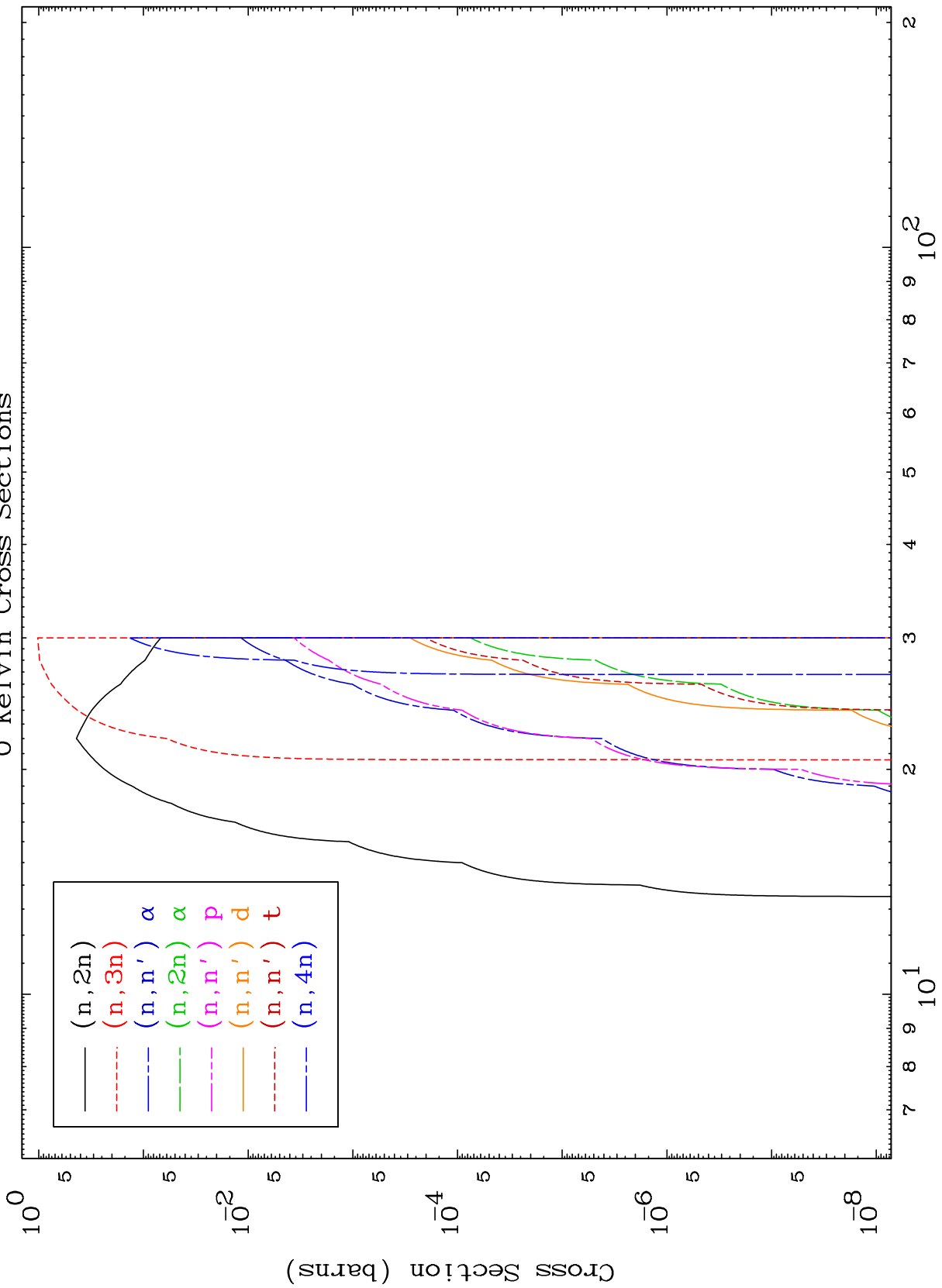
⁷¹Lu-181

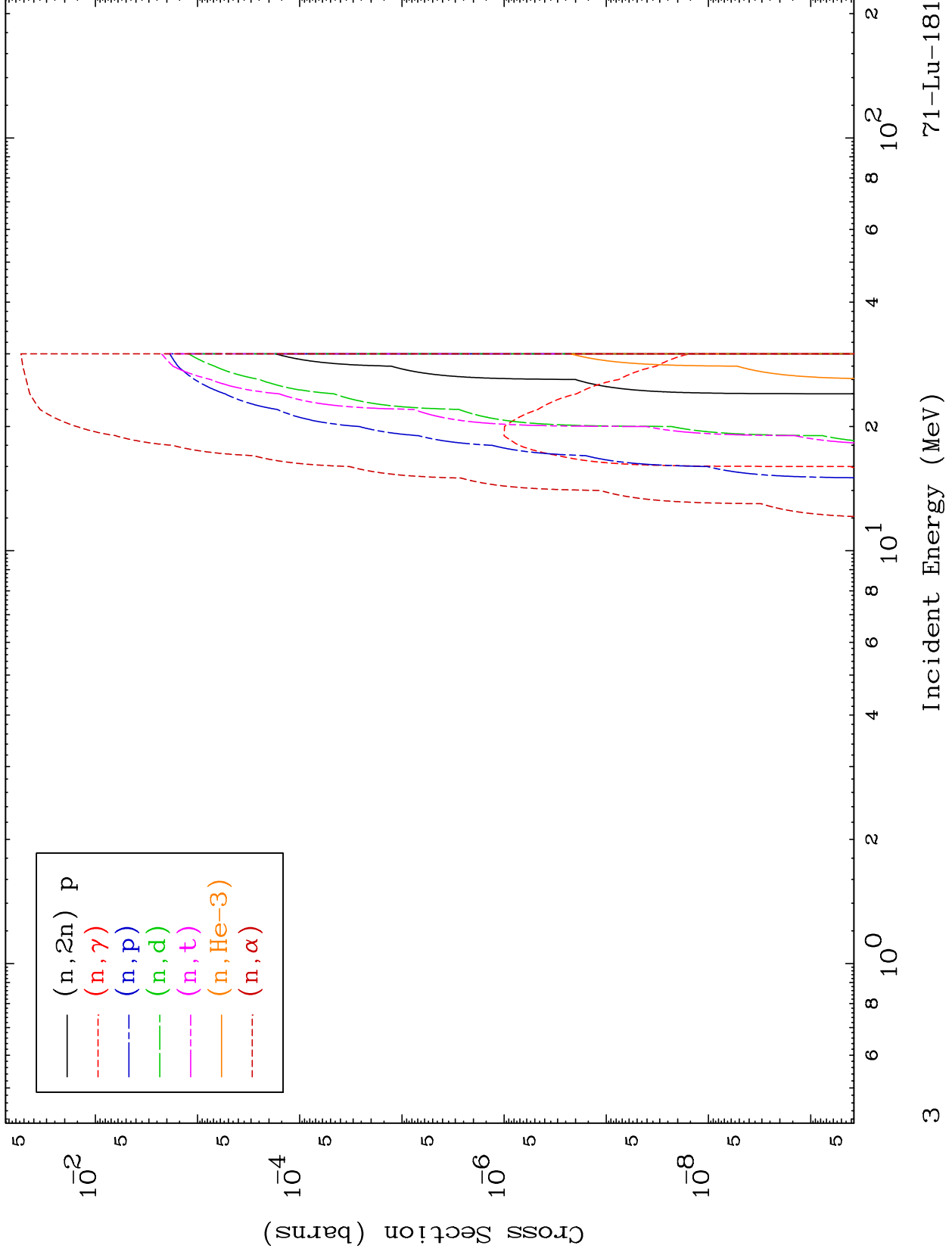
Incident Energy (MeV)

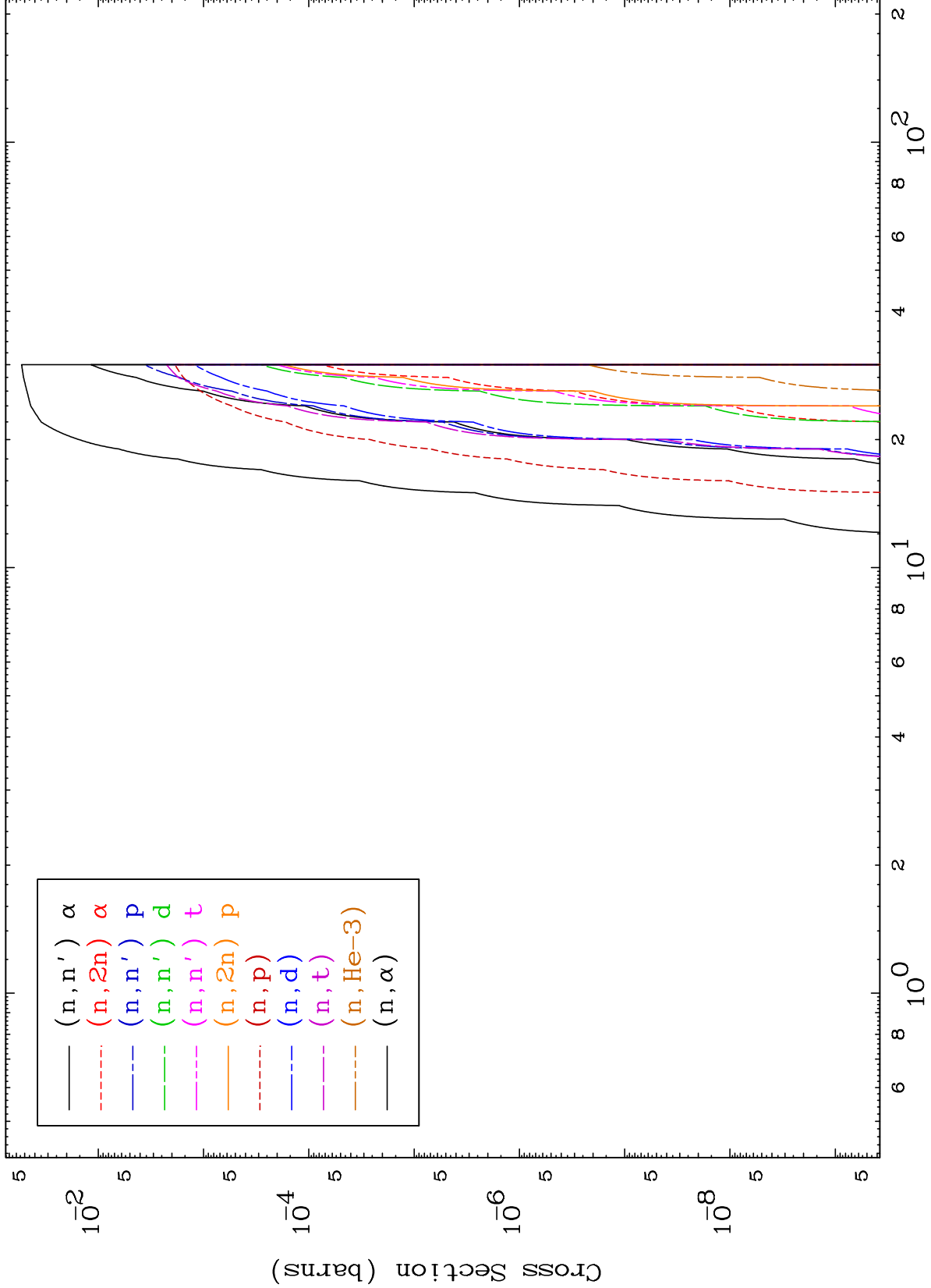
MAT 7143

α Neutron Absorption
0 Kelvin Cross Sections

71-Lu-181





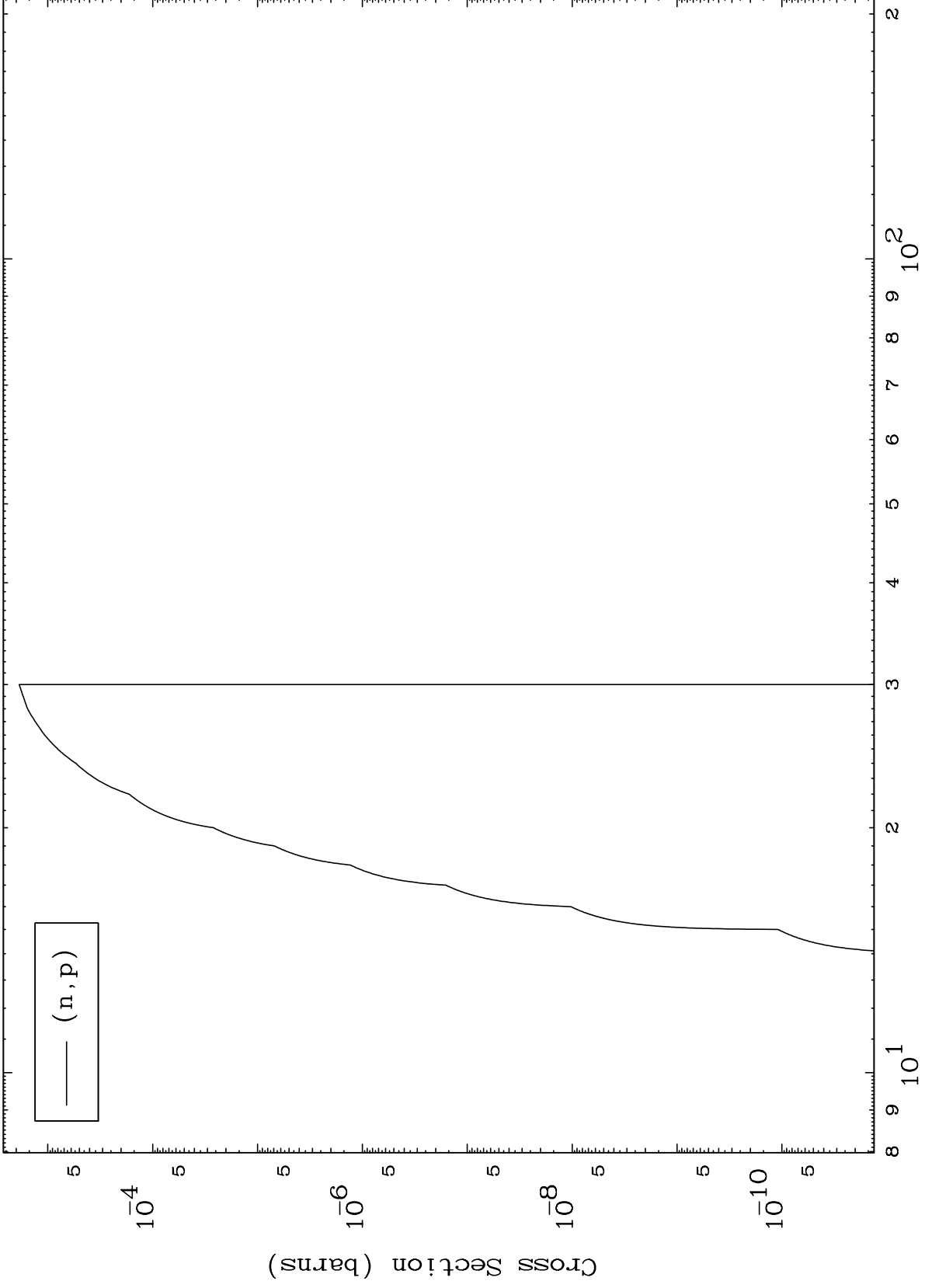


MAT 7143

(α, p) Levels

71-Lu-181

0 Kelvin Cross Sections

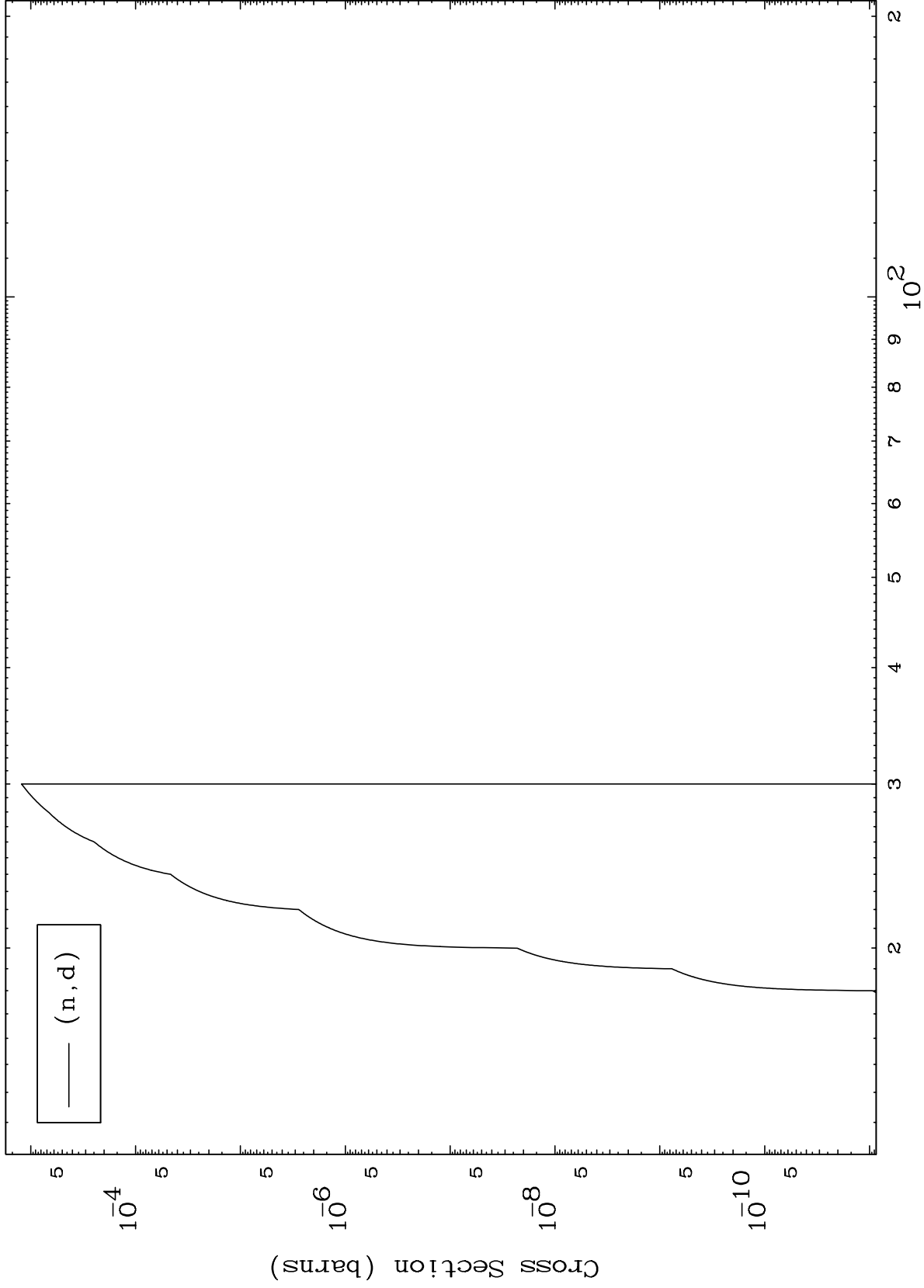


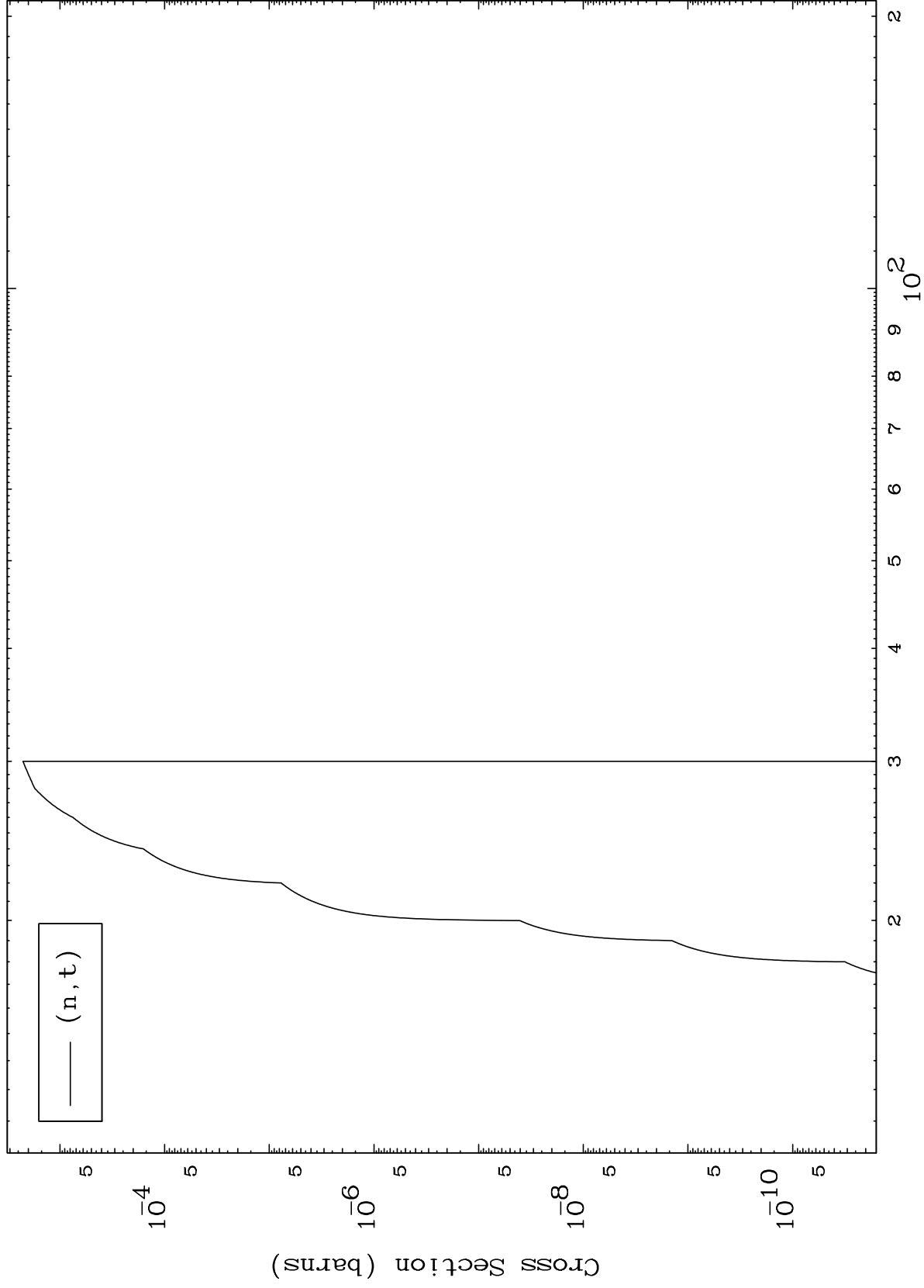
(n, p)

5

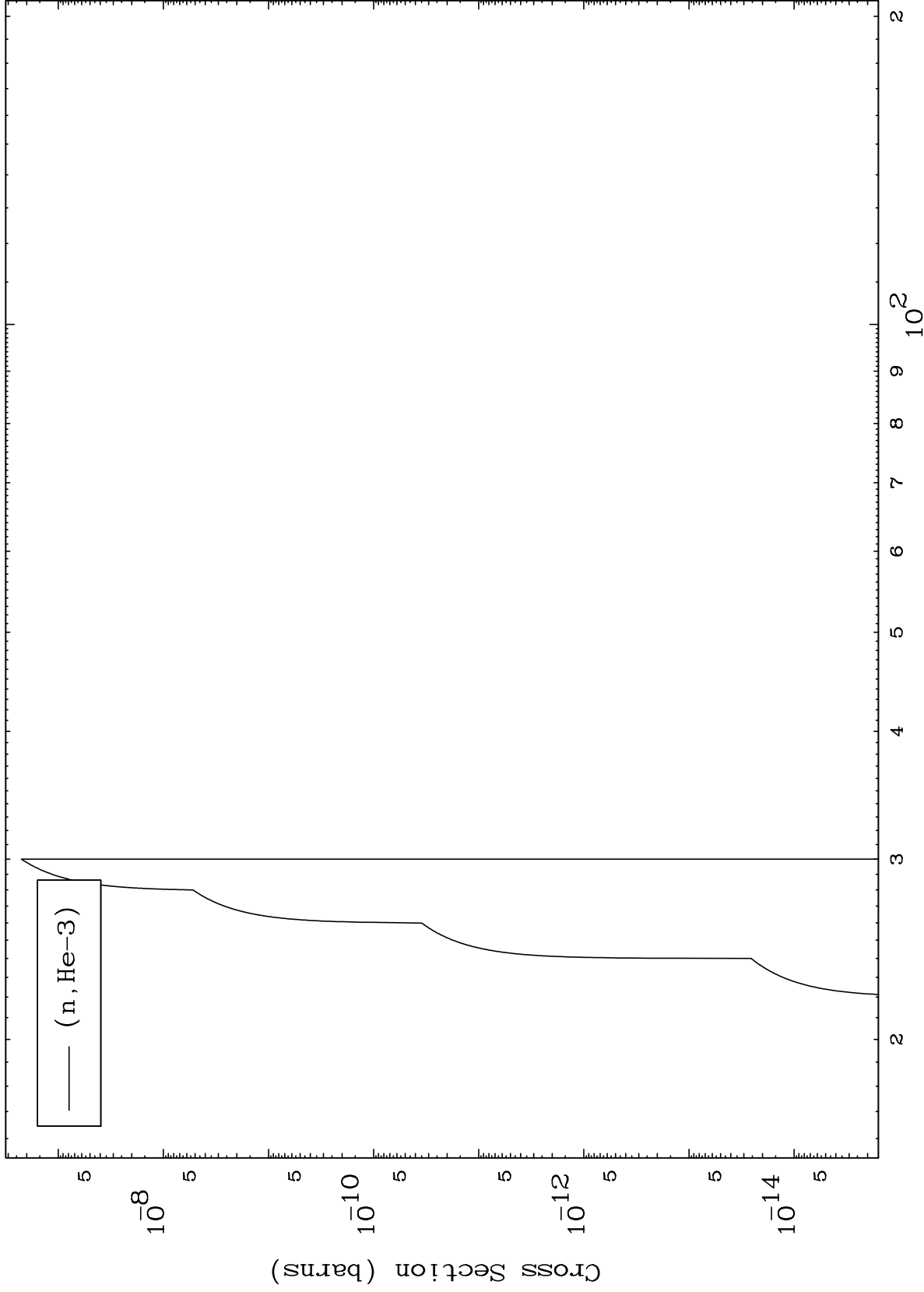
Incident Energy (MeV)

71-Lu-181





($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

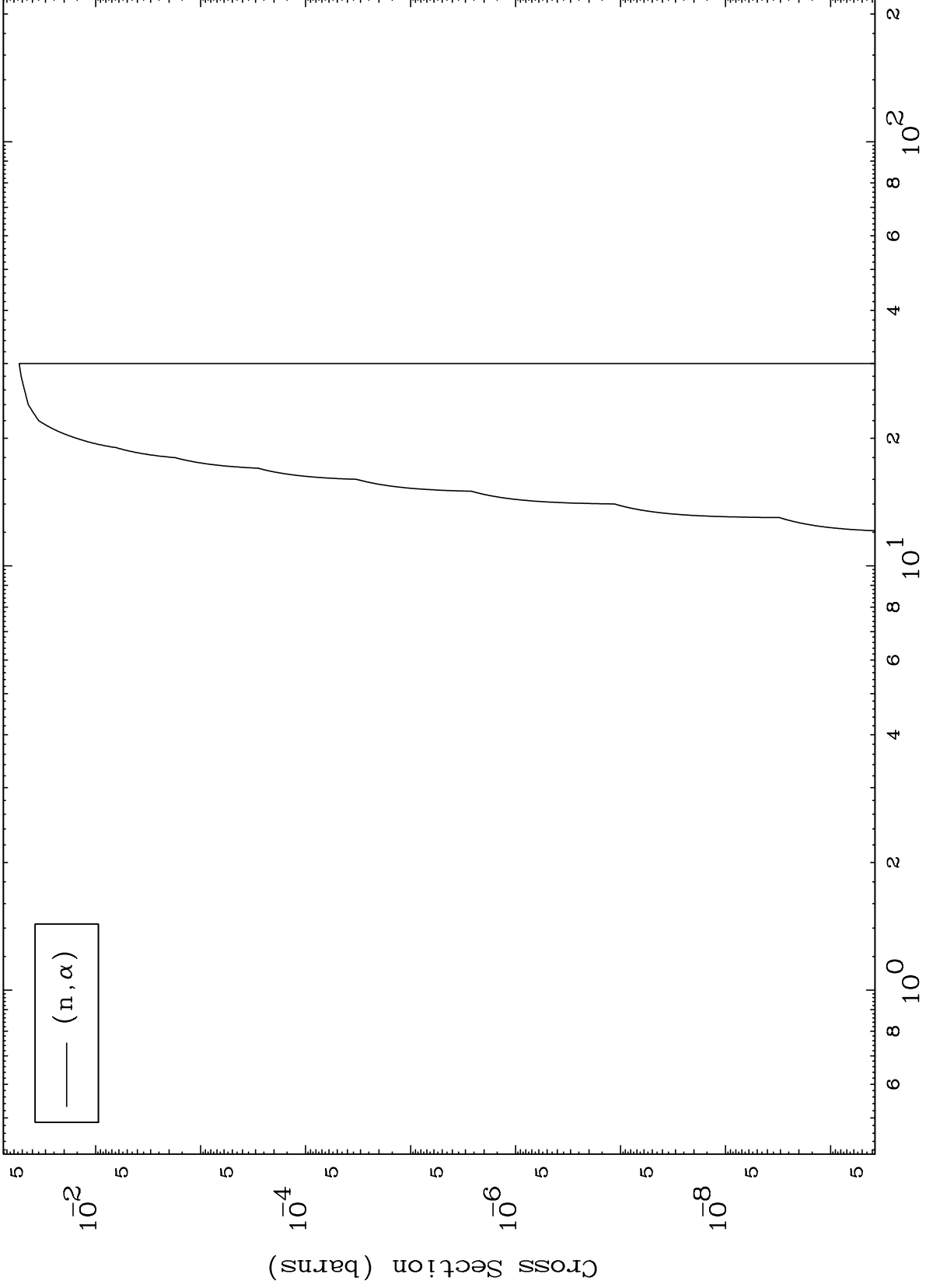


MAT 7143

(α, α) Levels

71-Lu-181

0 Kelvin Cross Sections



9

Incident Energy (MeV)

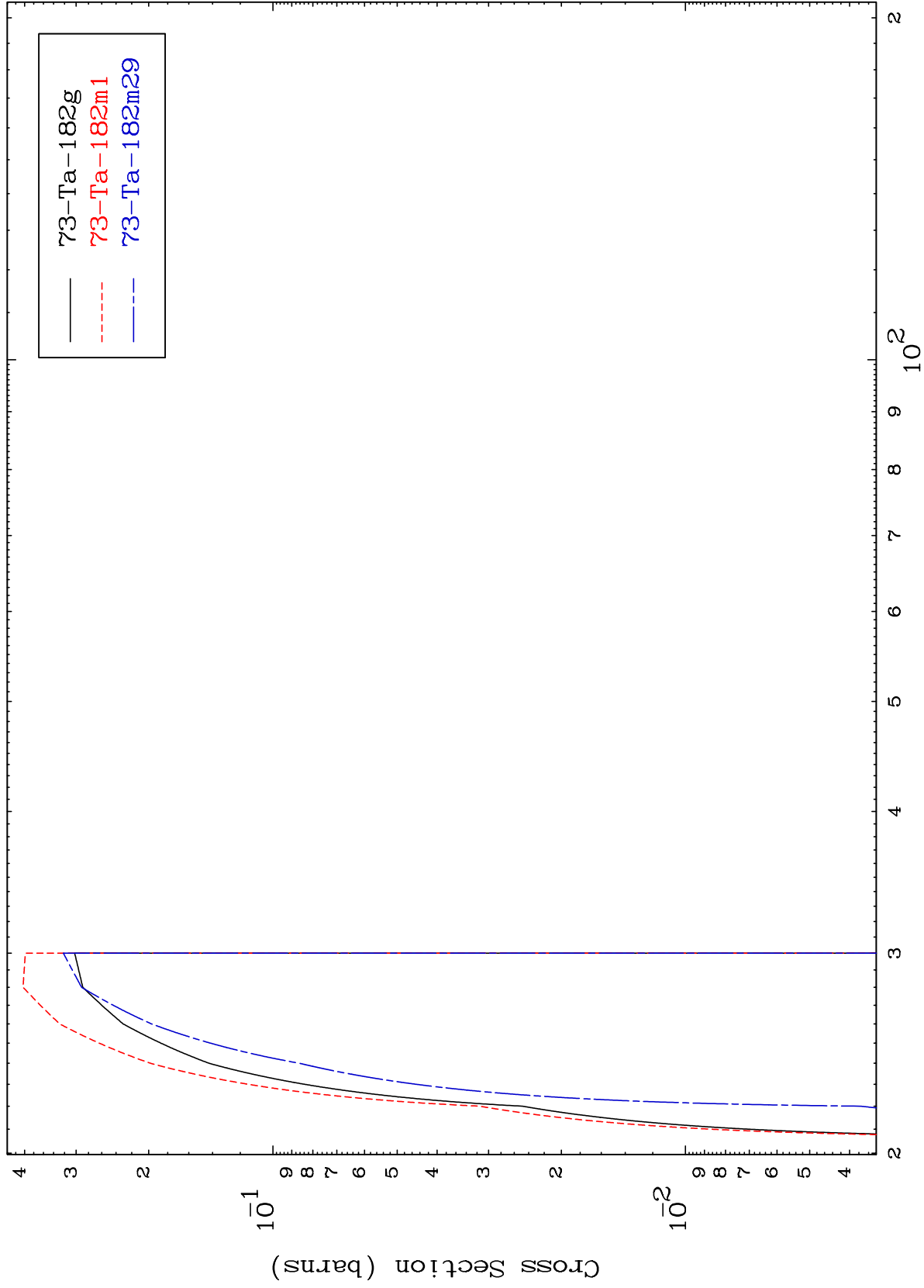
71-Lu-181

MAT 7143

(n,3n)

71-Lu-181

Radionuclide Production Cross Section



Incident Energy (MeV)

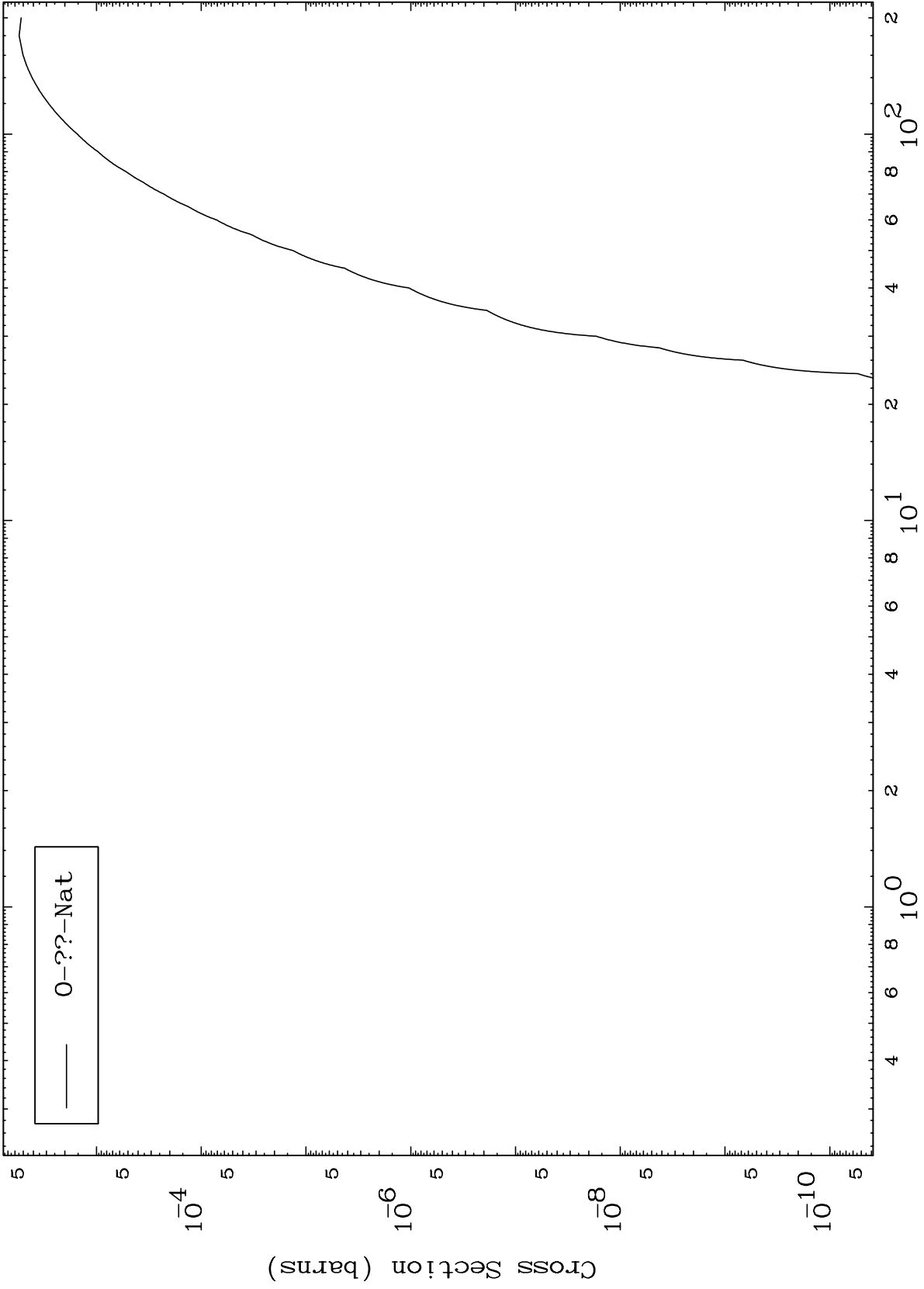
71-Lu-181

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Fission

71-Lu-181

Radionuclide Production Cross Section

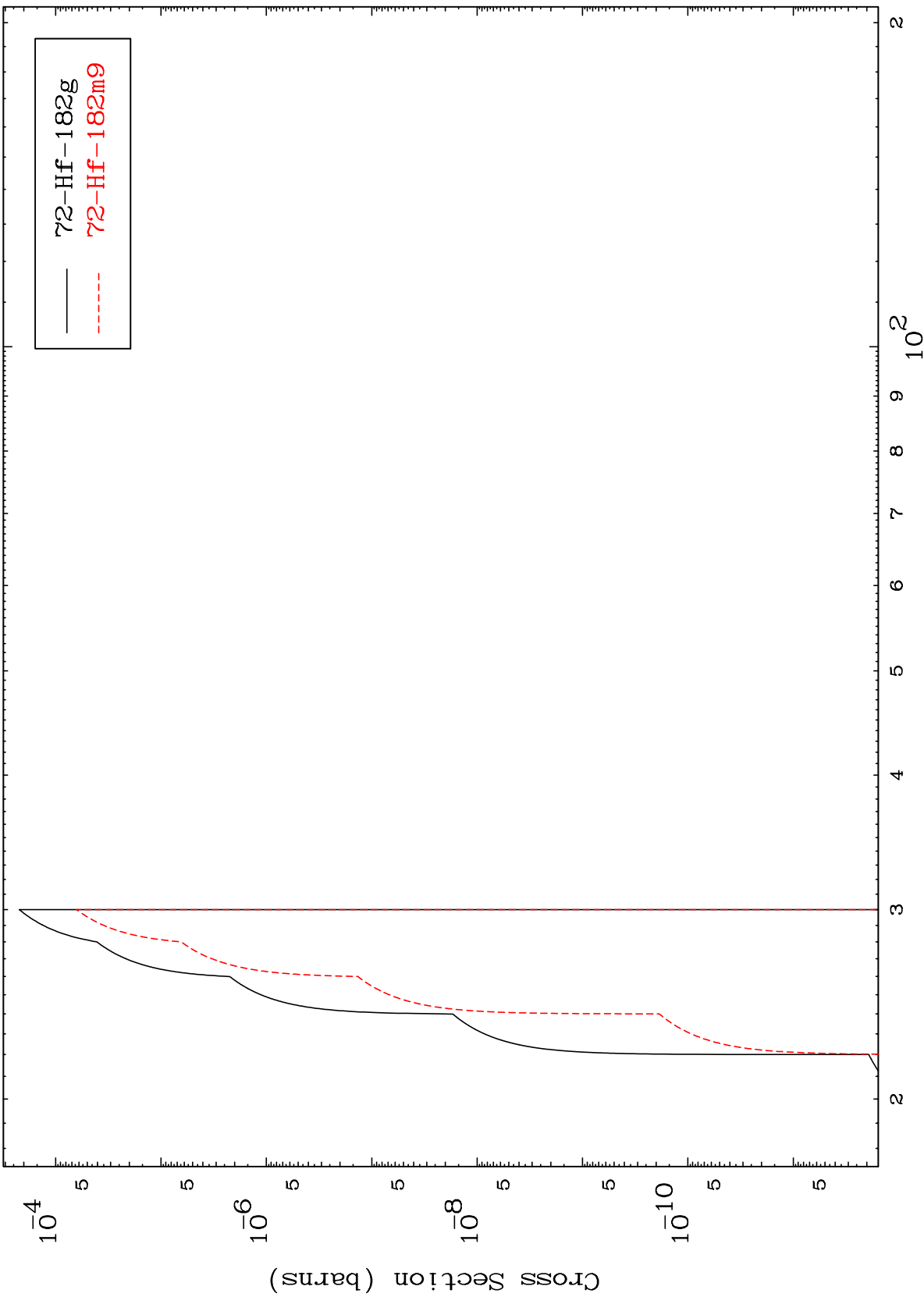


MAT 7143

(n,n') d

71-Lu-181

Radionuclide Production Cross Section



12

Incident Energy (MeV)

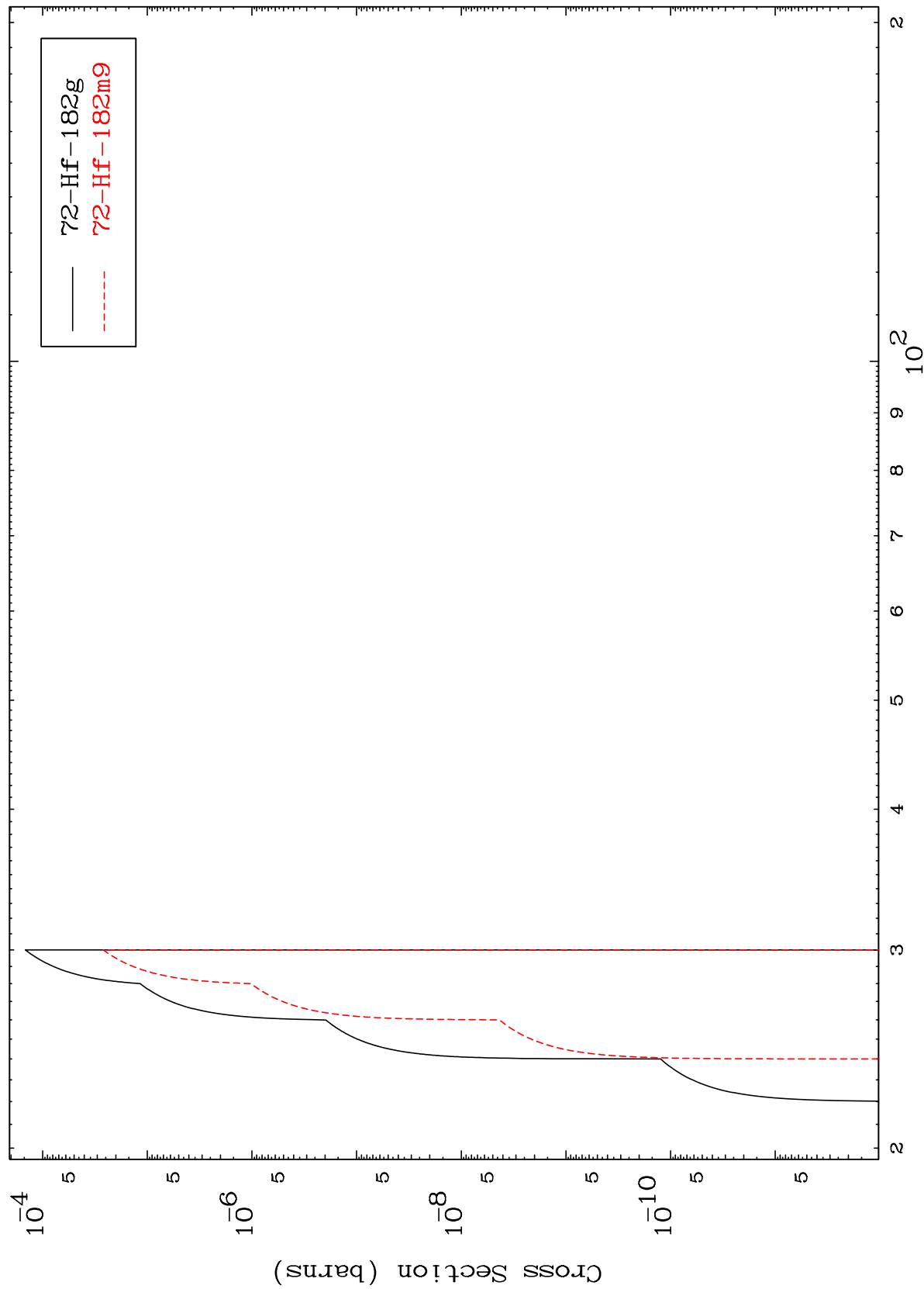
71-Lu-181

MAT 7143

(n,2n) p

71-Lu-181

Radionuclide Production Cross Section



13

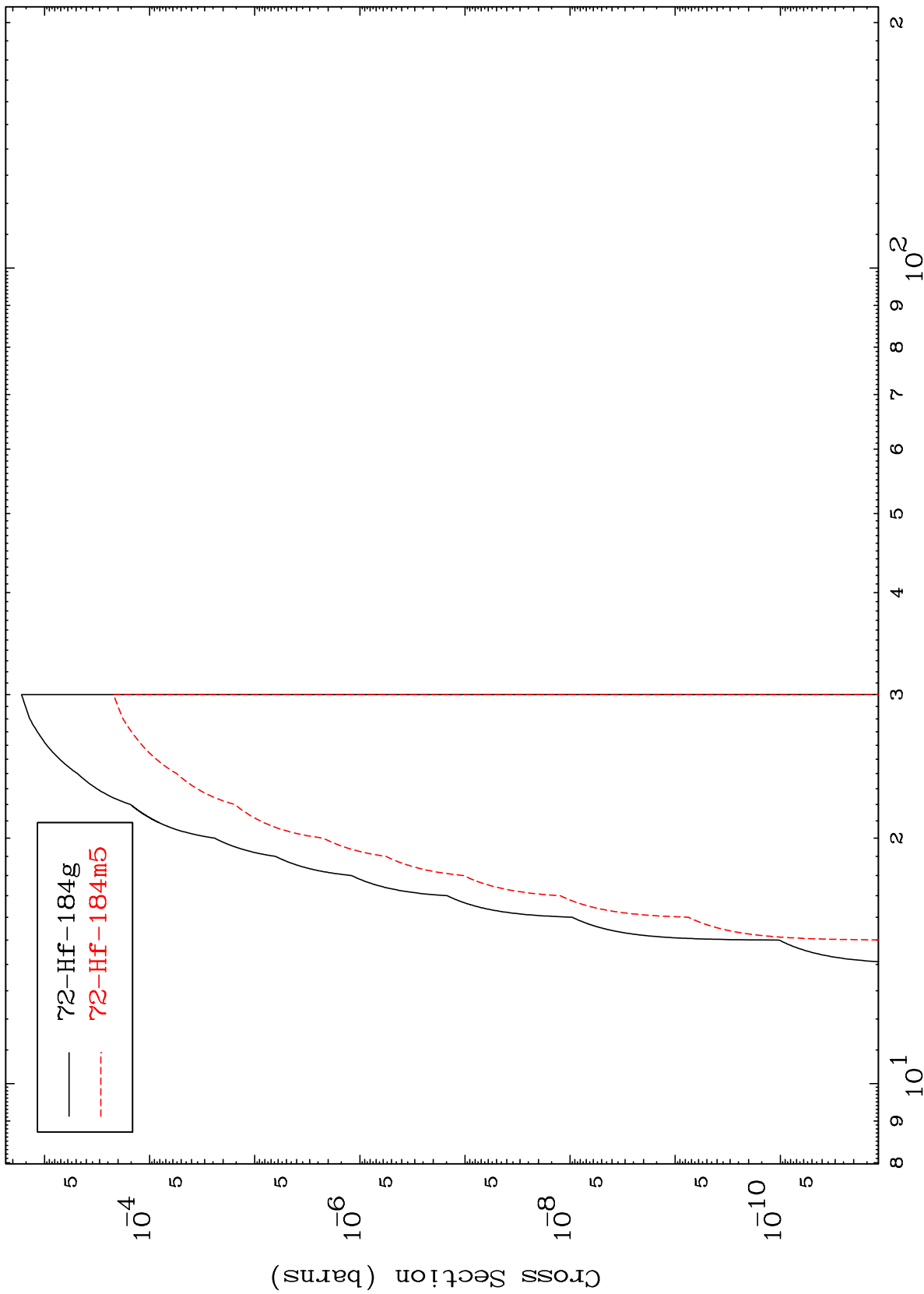
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(n,p)
Radionuclide Production Cross Section



14

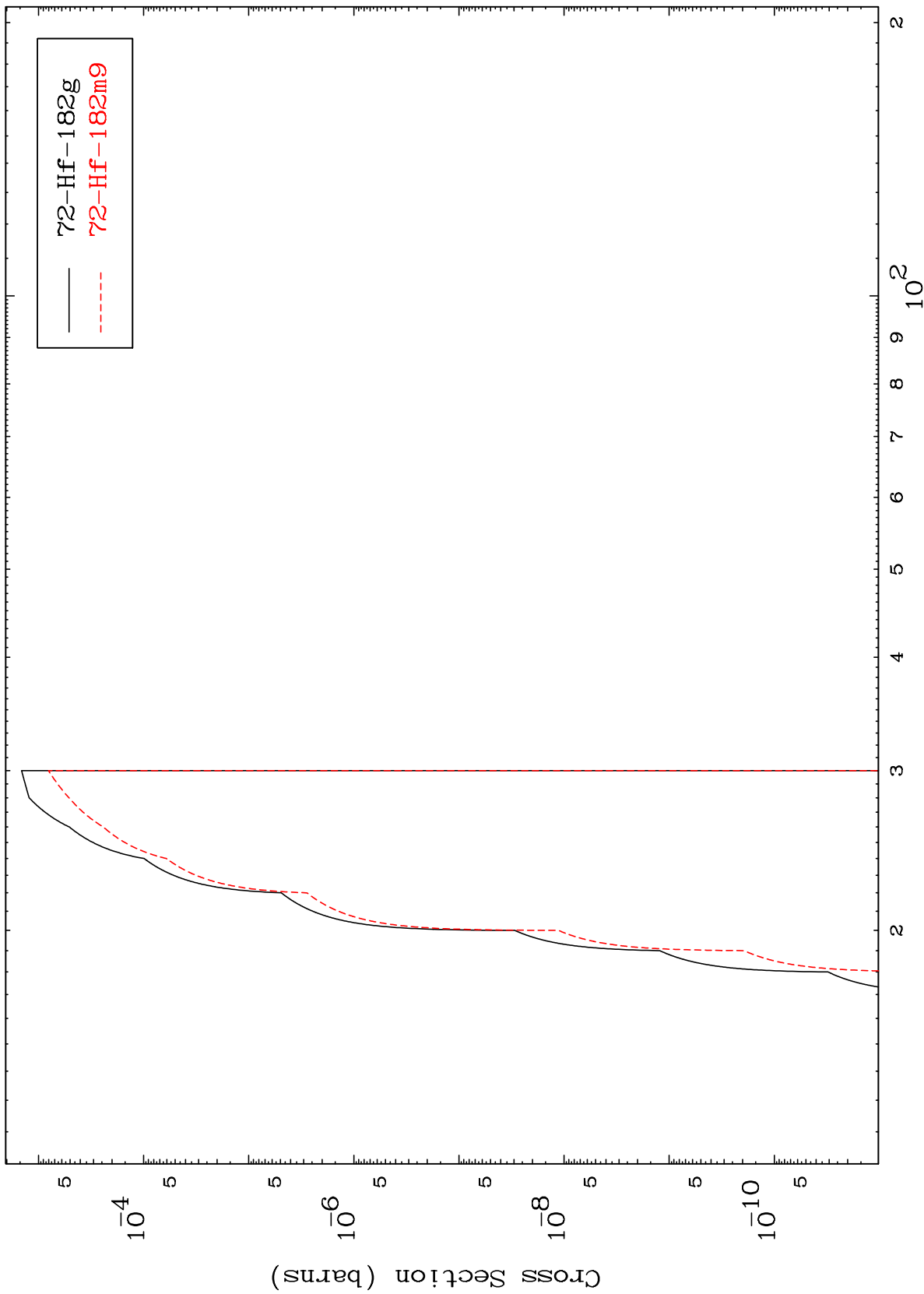
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(n, t)
Radionuclide Production Cross Section



15

Incident Energy (MeV)

71-Lu-181