

Program EVALPLOT  
(Version 2021-1)

by

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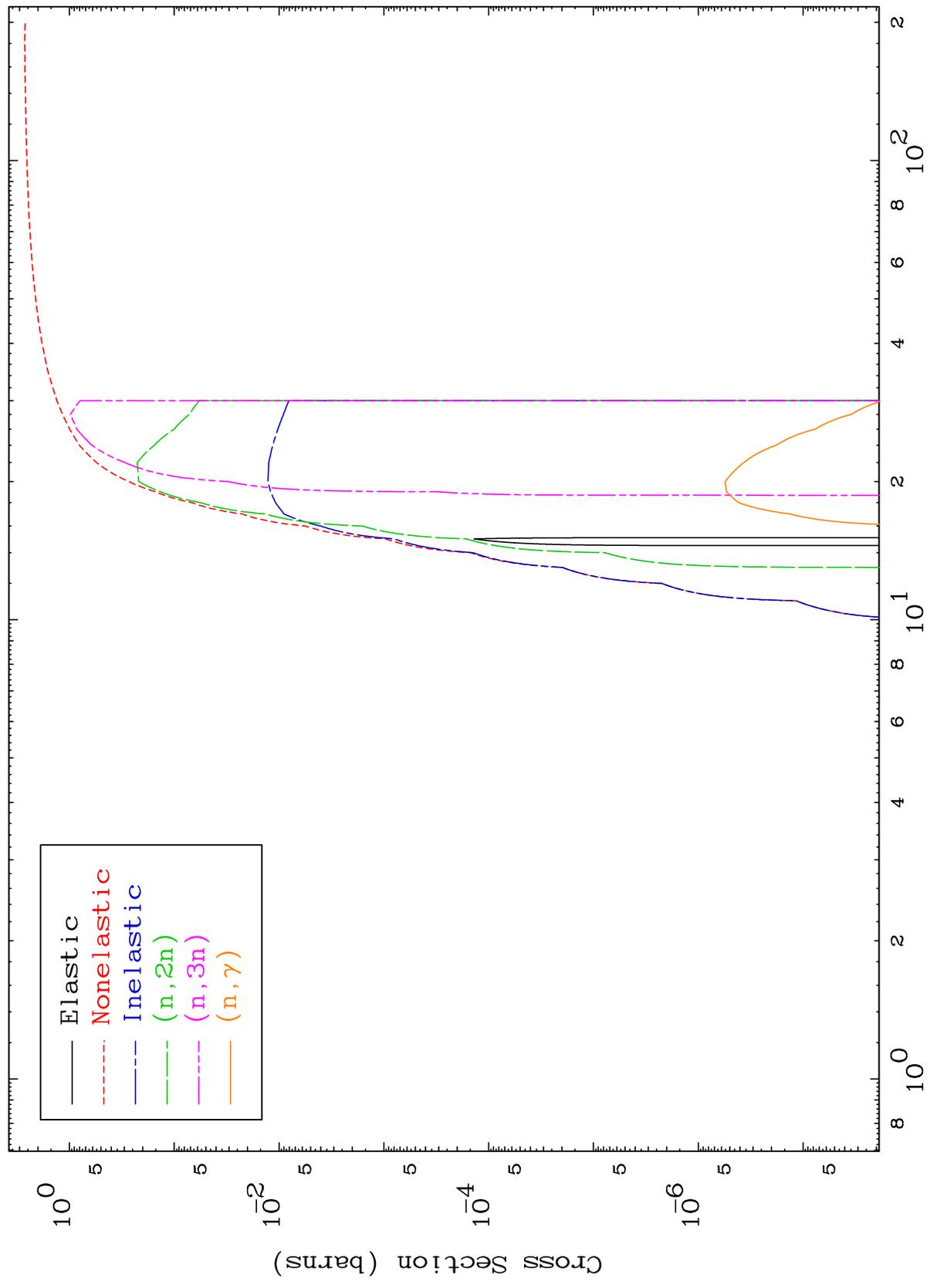
Press Mouse Button to Start

MAT 7146

0 Kelvin

Major Cross Sections

<sup>71</sup>Lu-182



1

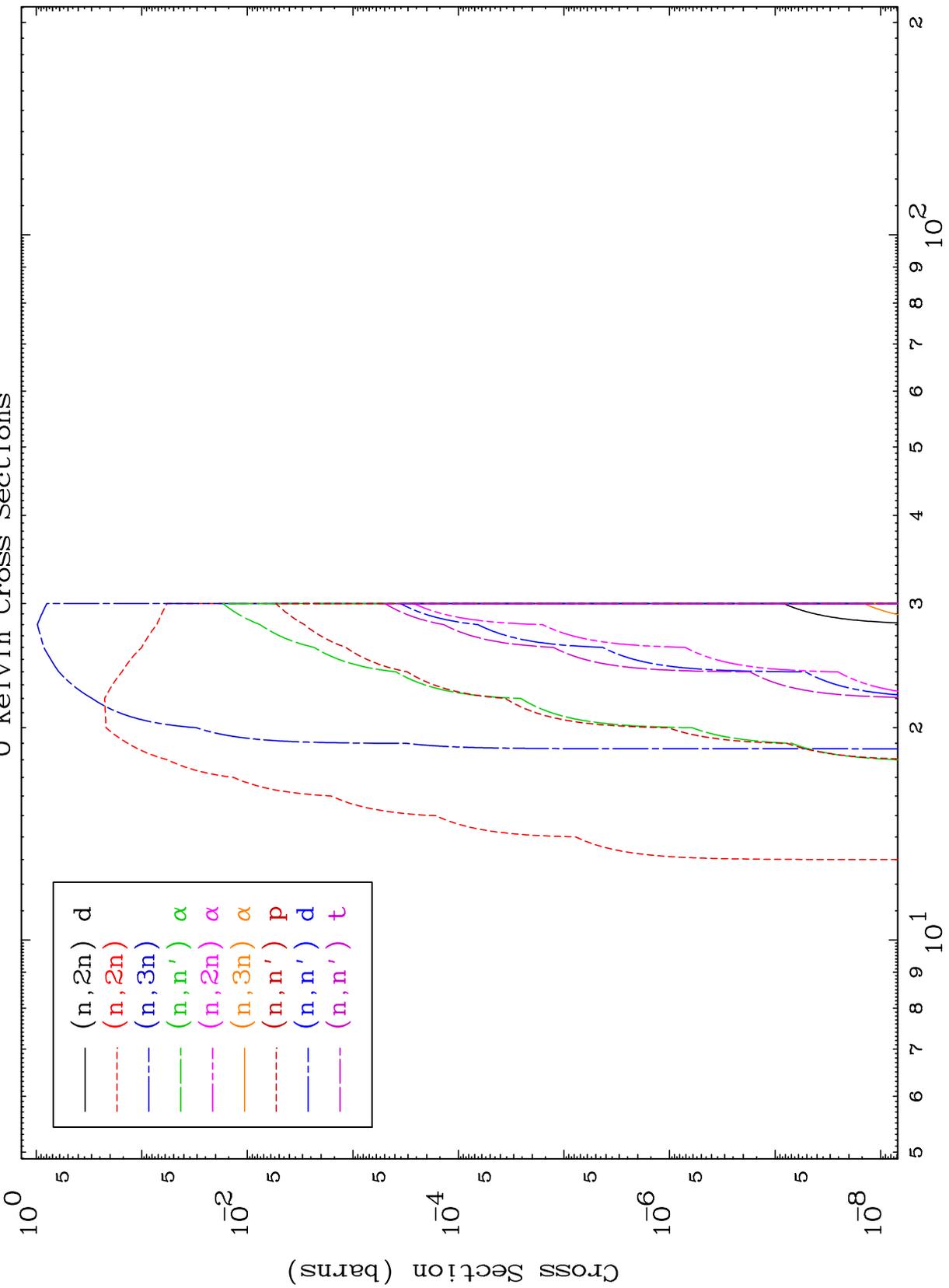
Incident Energy (MeV)

<sup>71</sup>Lu-182

MAT 7146

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-182



2

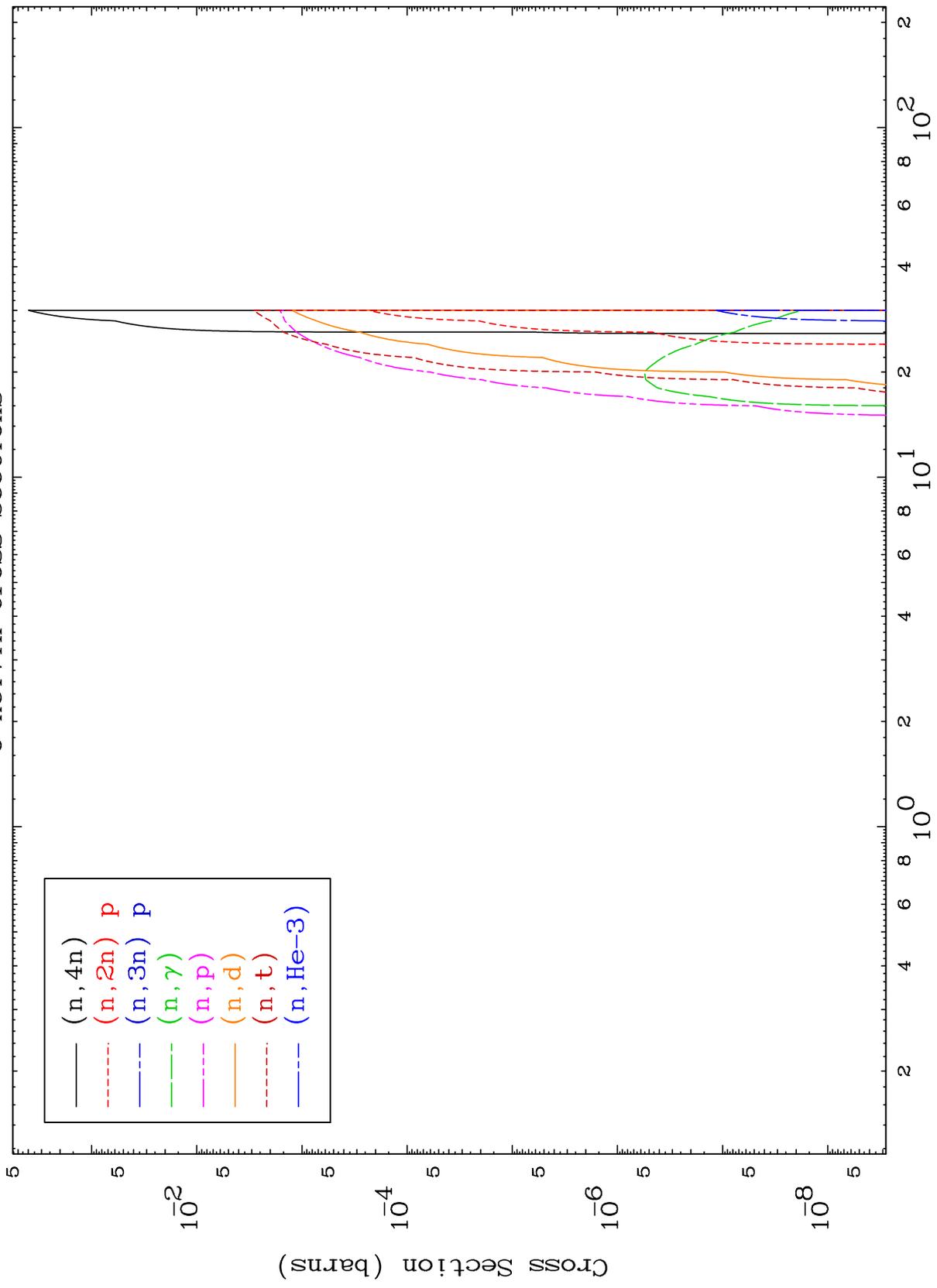
Incident Energy (MeV)

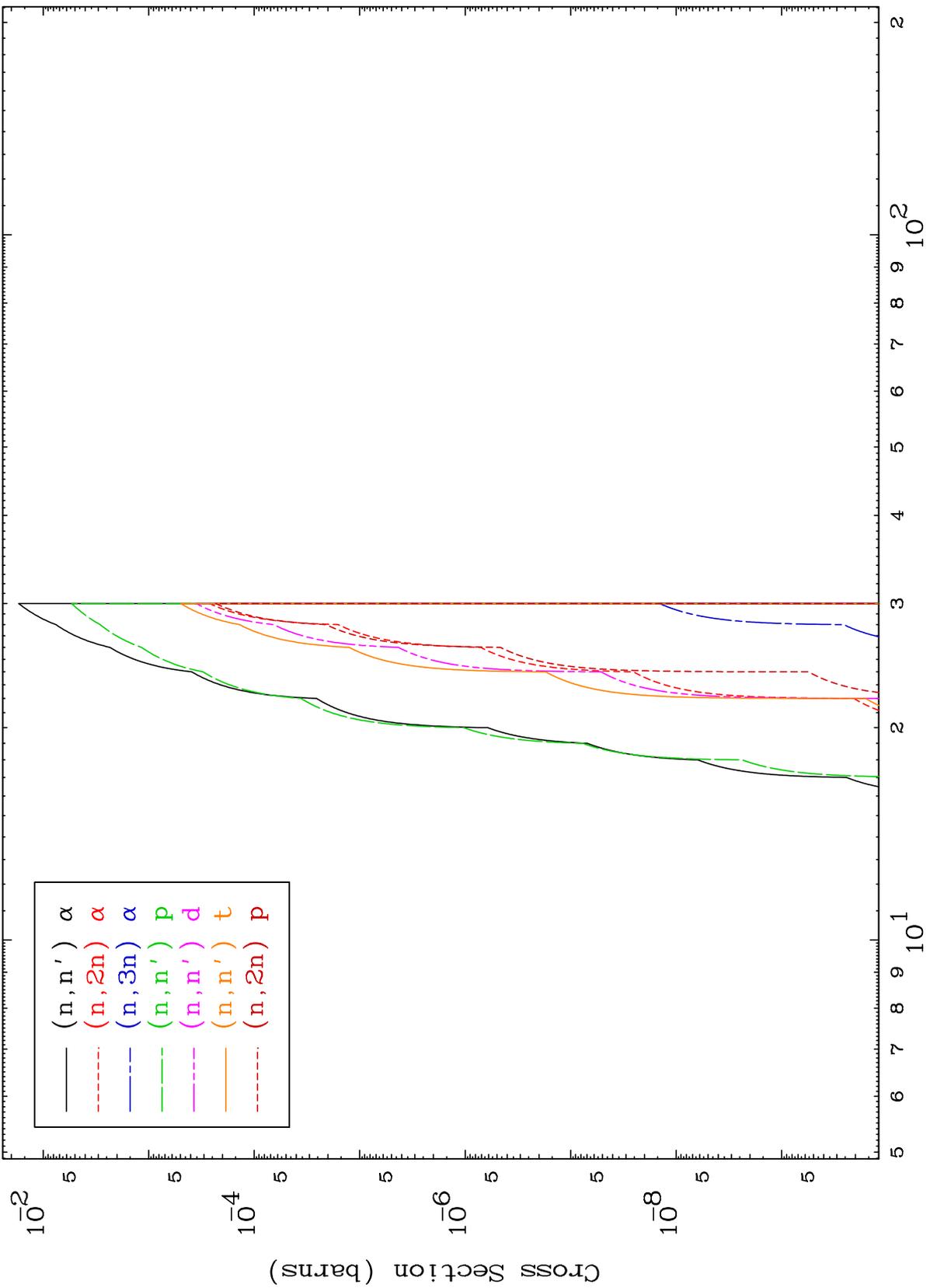
71-Lu-182

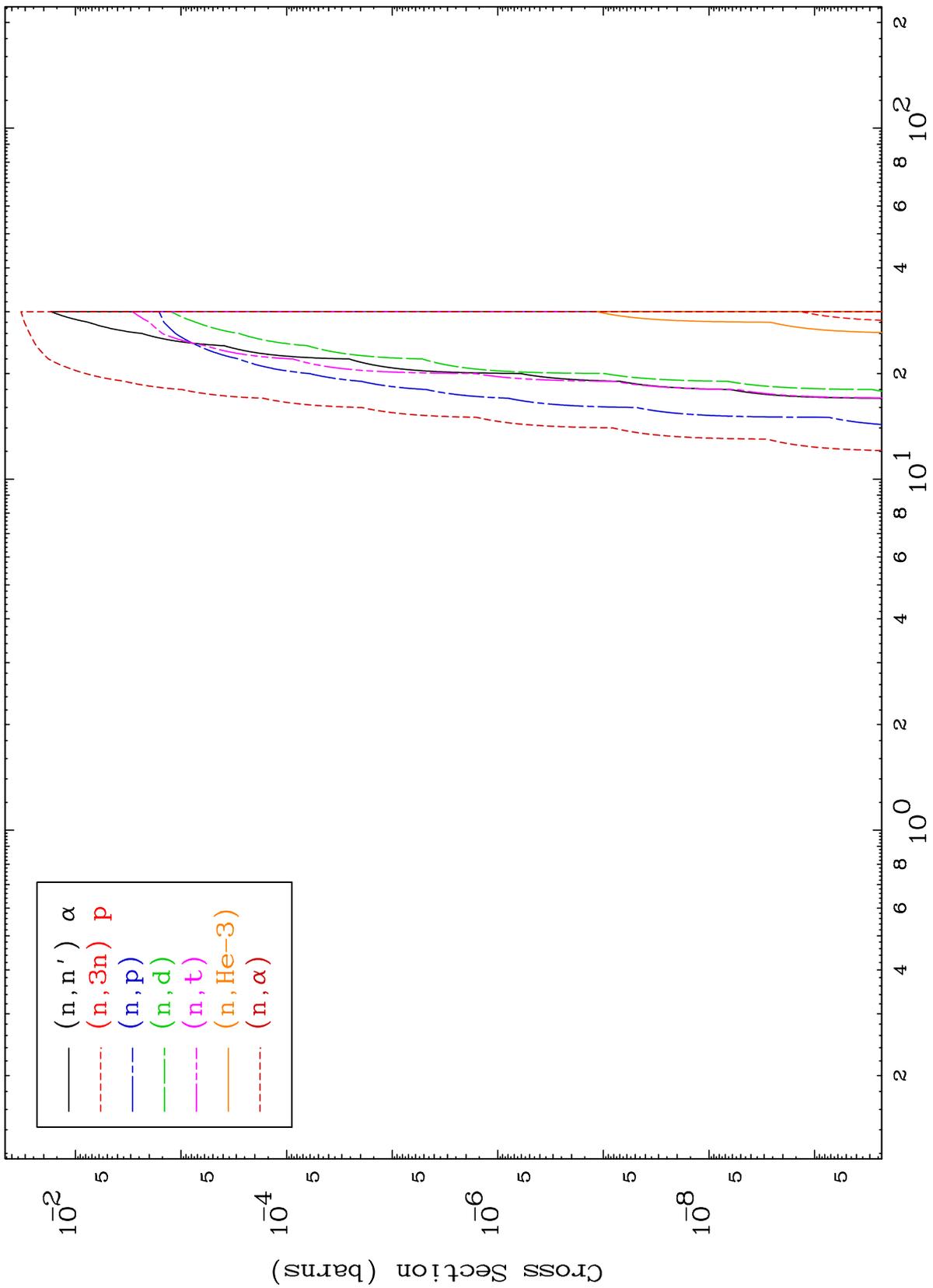
MAT 7146

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

<sup>71</sup>Lu-182



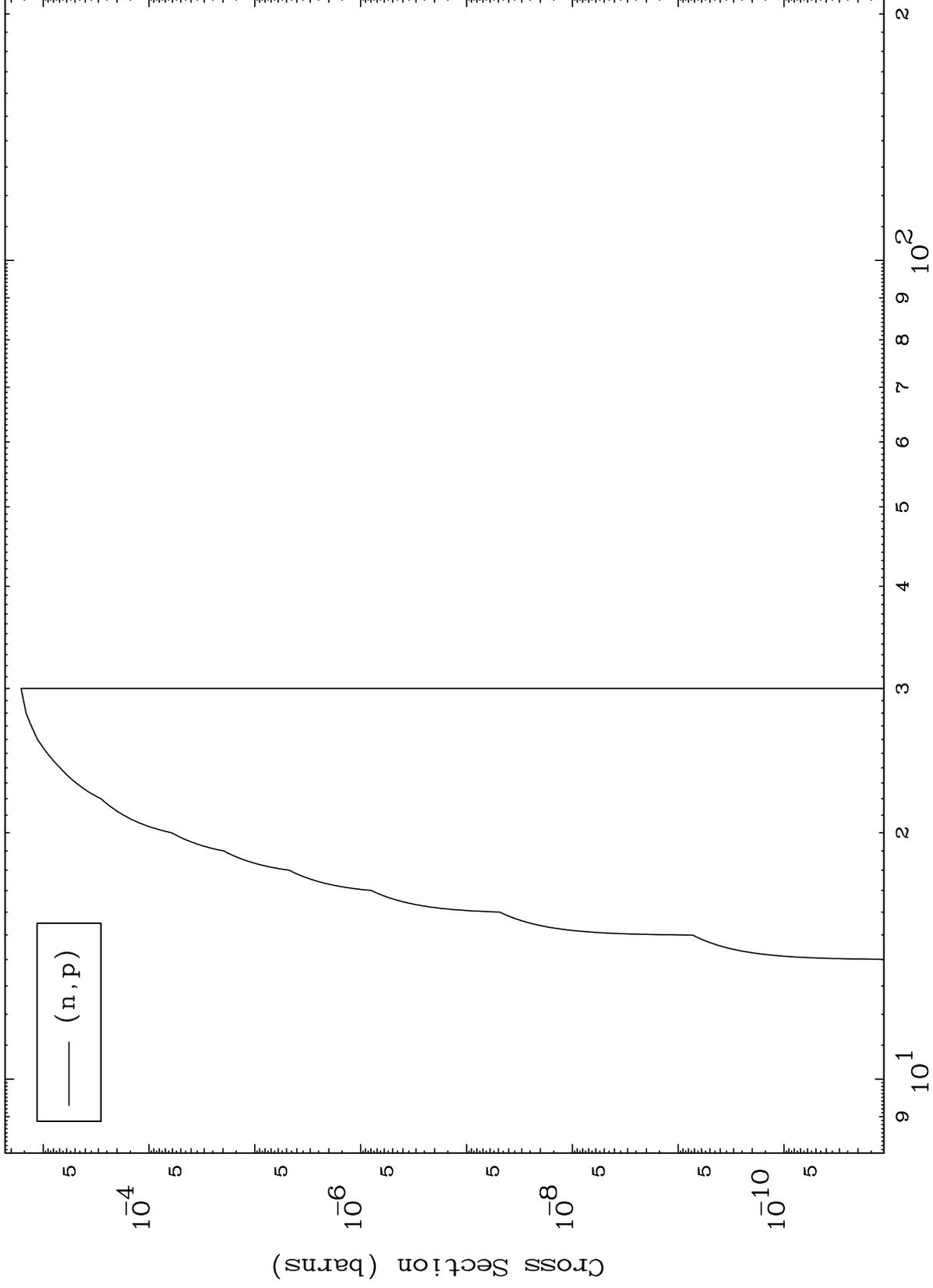




MAT 7146

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

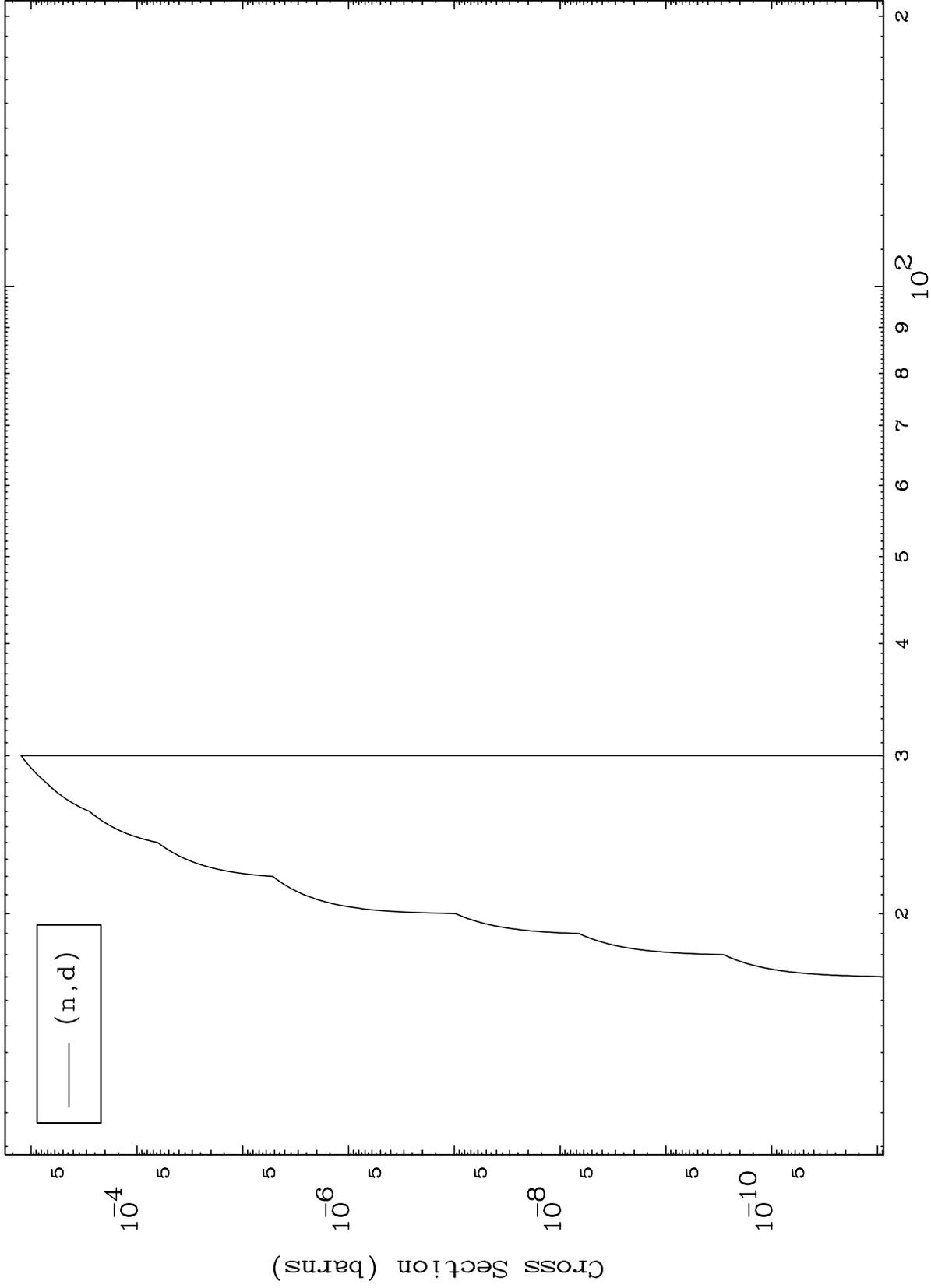
71-Lu-182

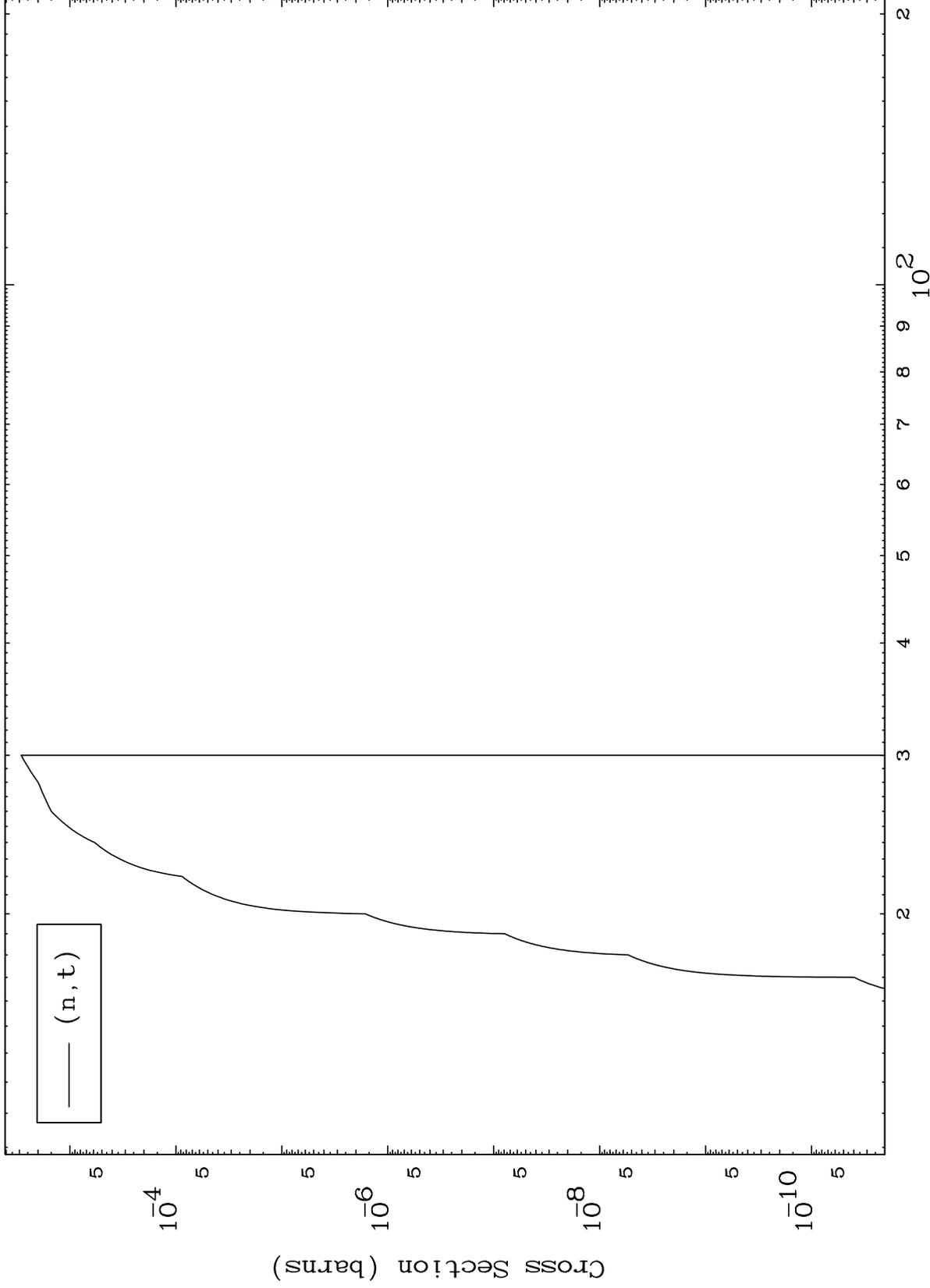


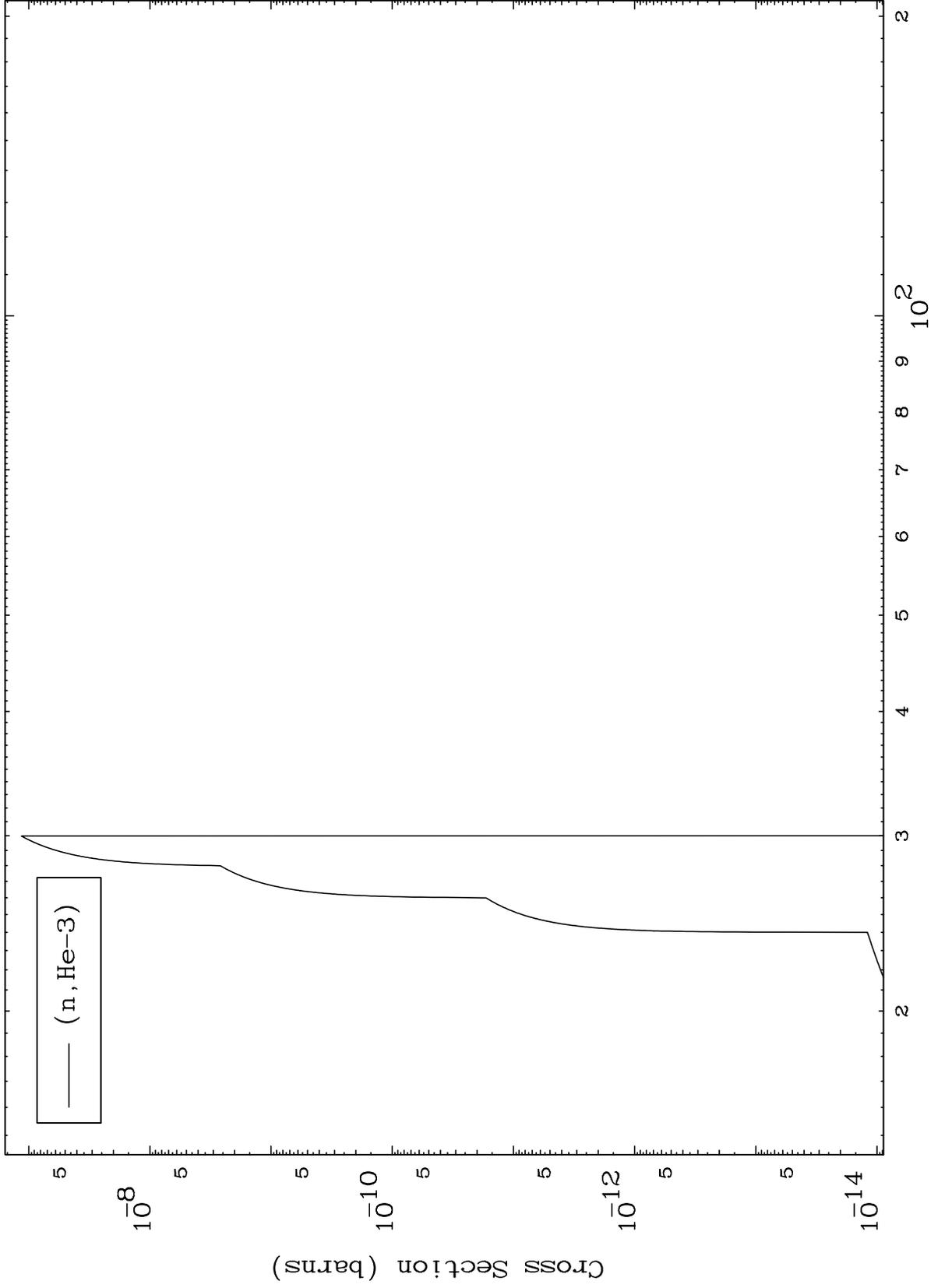
Incident Energy (MeV)

71-Lu-182

6





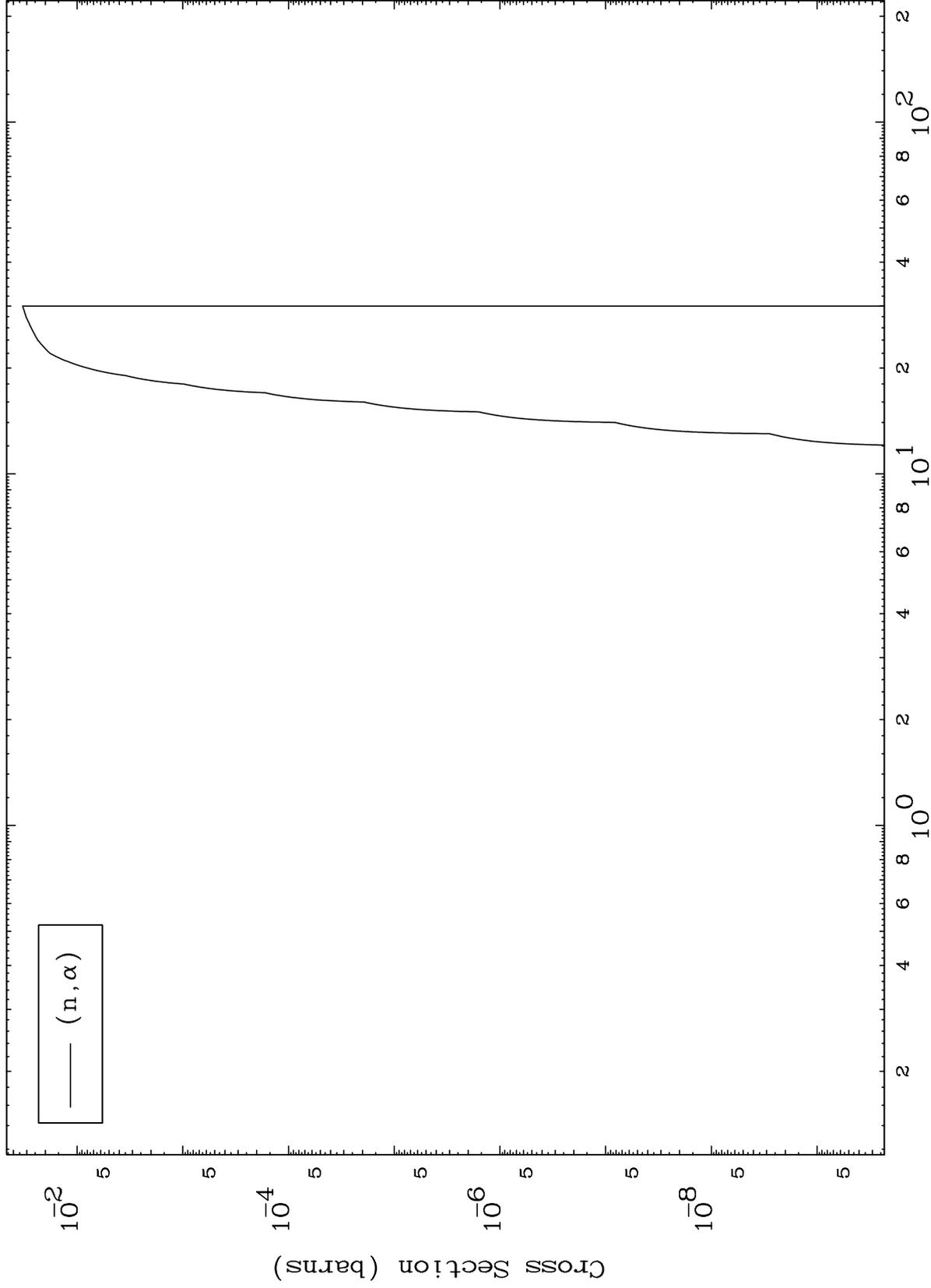


MAT 7146

( $\alpha, \alpha$ ) Levels

71-Lu-182

0 Kelvin Cross Sections

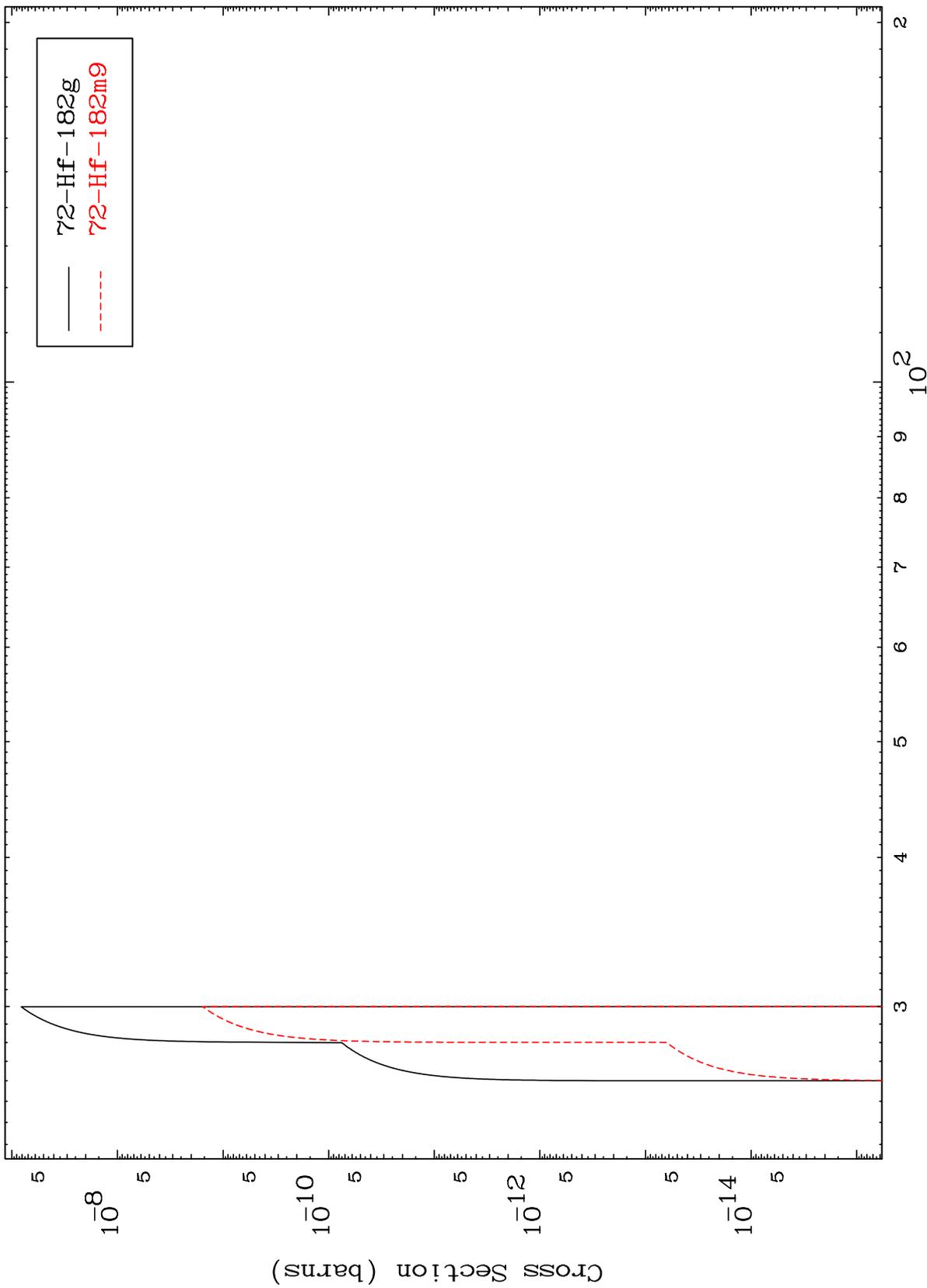


10

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

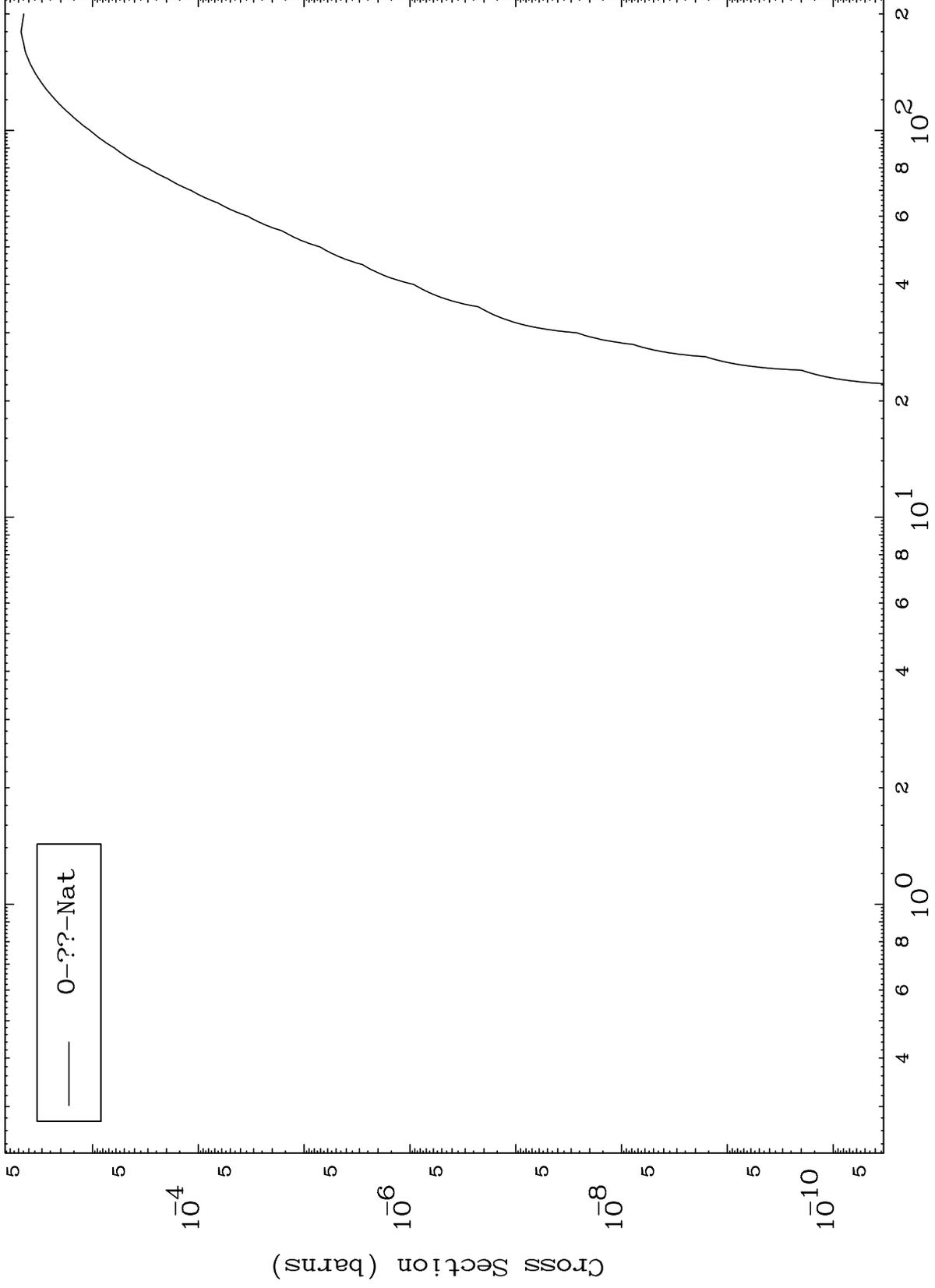


72-Hf-182g  
72-Hf-182m9

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Fission  
Radionuclide Production Cross Section

71-Lu-182



12

Incident Energy (MeV)

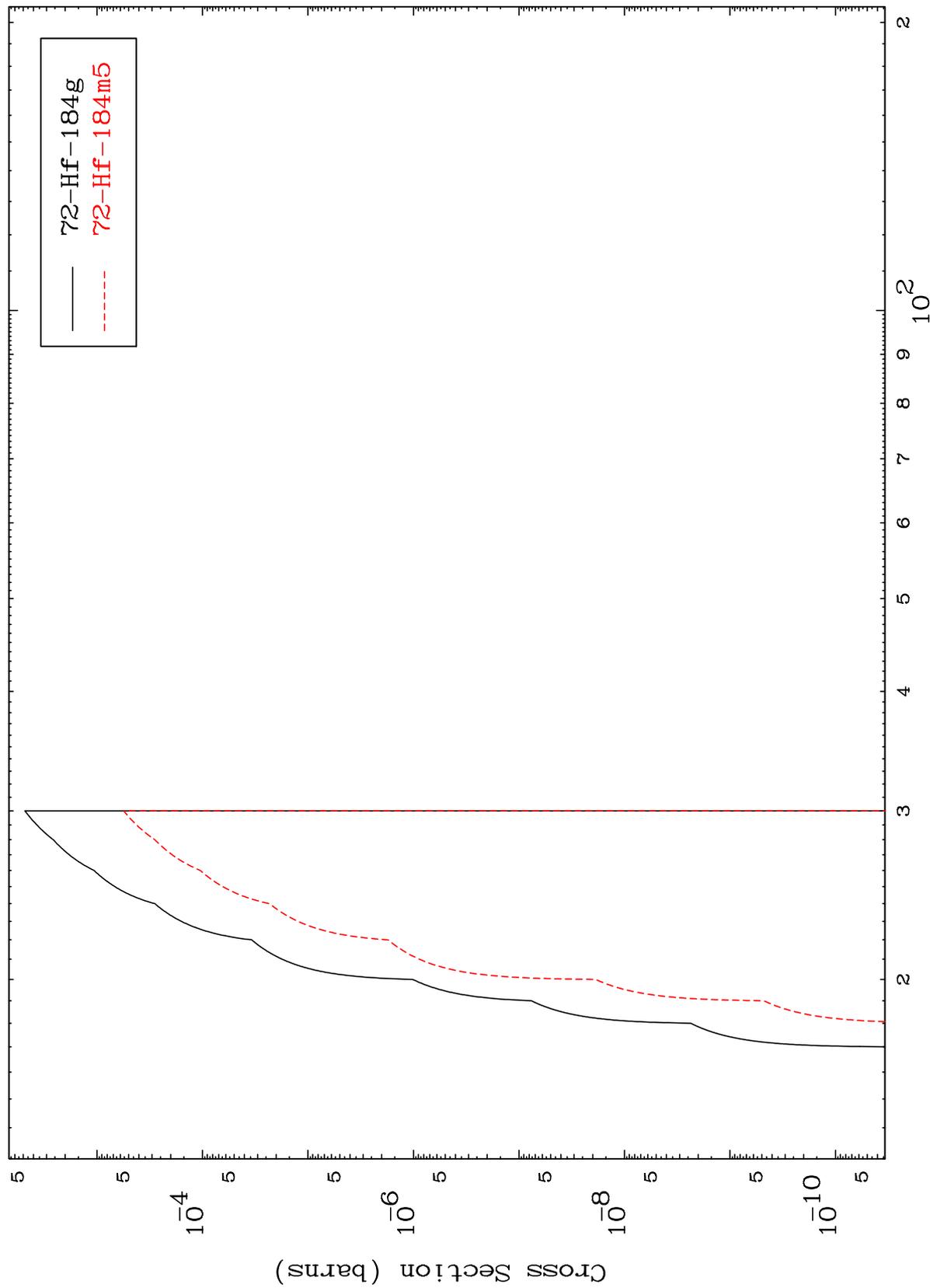
71-Lu-182

MAT 7146

$(n, n')$  p

71-Lu-182

Radionuclide Production Cross Section



13

Incident Energy (MeV)

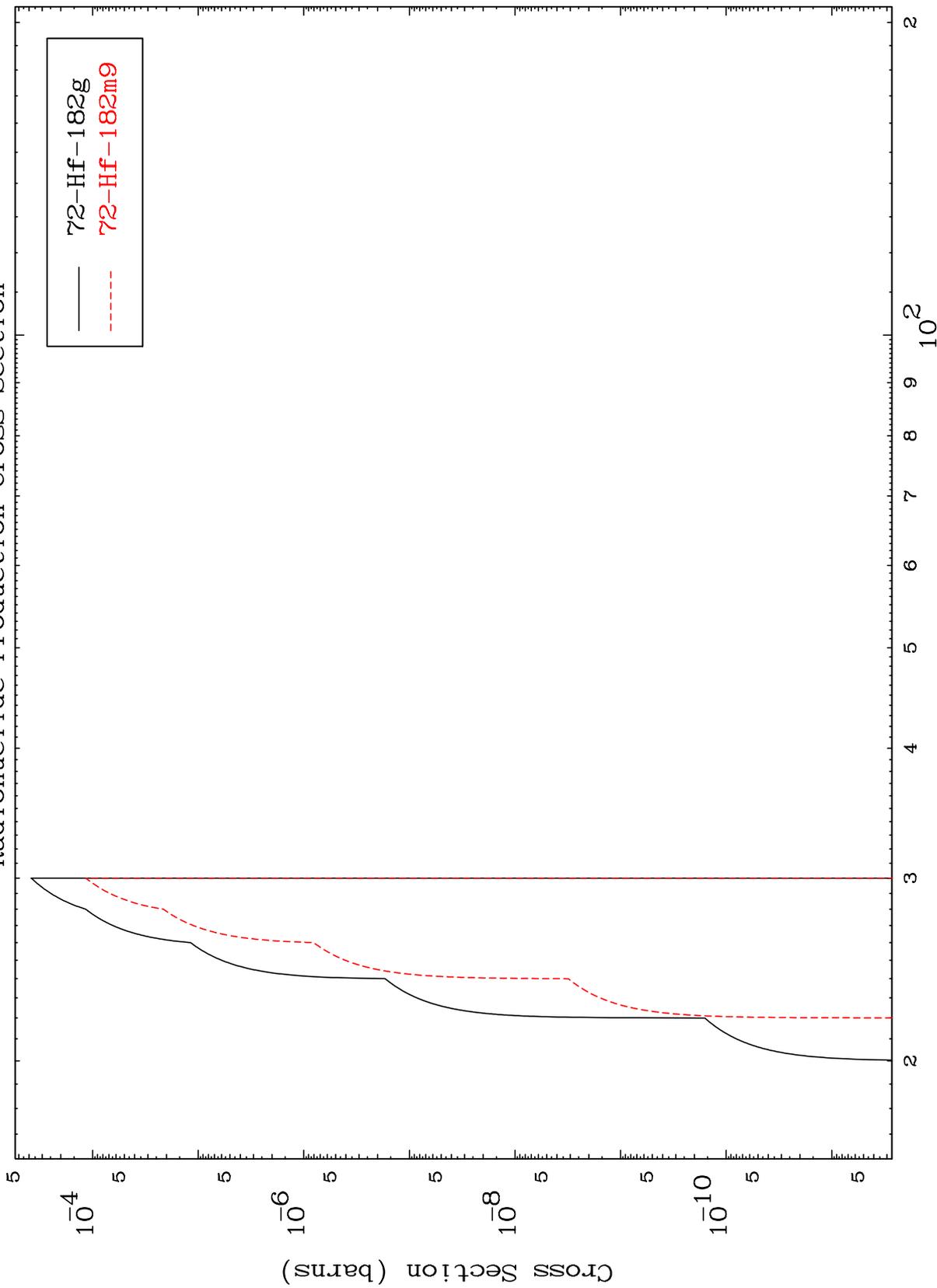
71-Lu-182

MAT 7146

(n,n') t

71-Lu-182

Radionuclide Production Cross Section



14

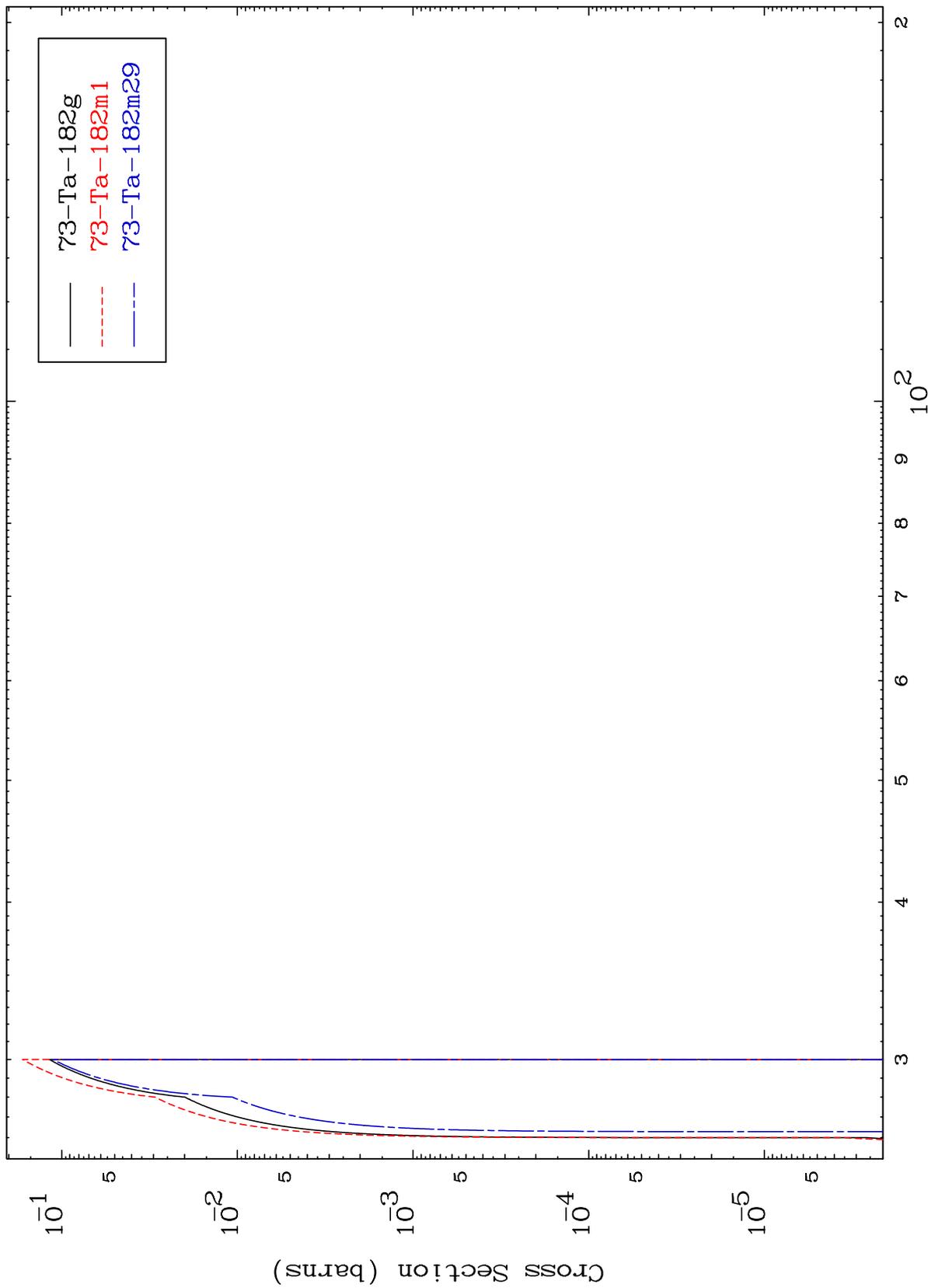
Incident Energy (MeV)

71-Lu-182

MAT 7146

71-Lu-182

(n,4n)  
Radionuclide Production Cross Section

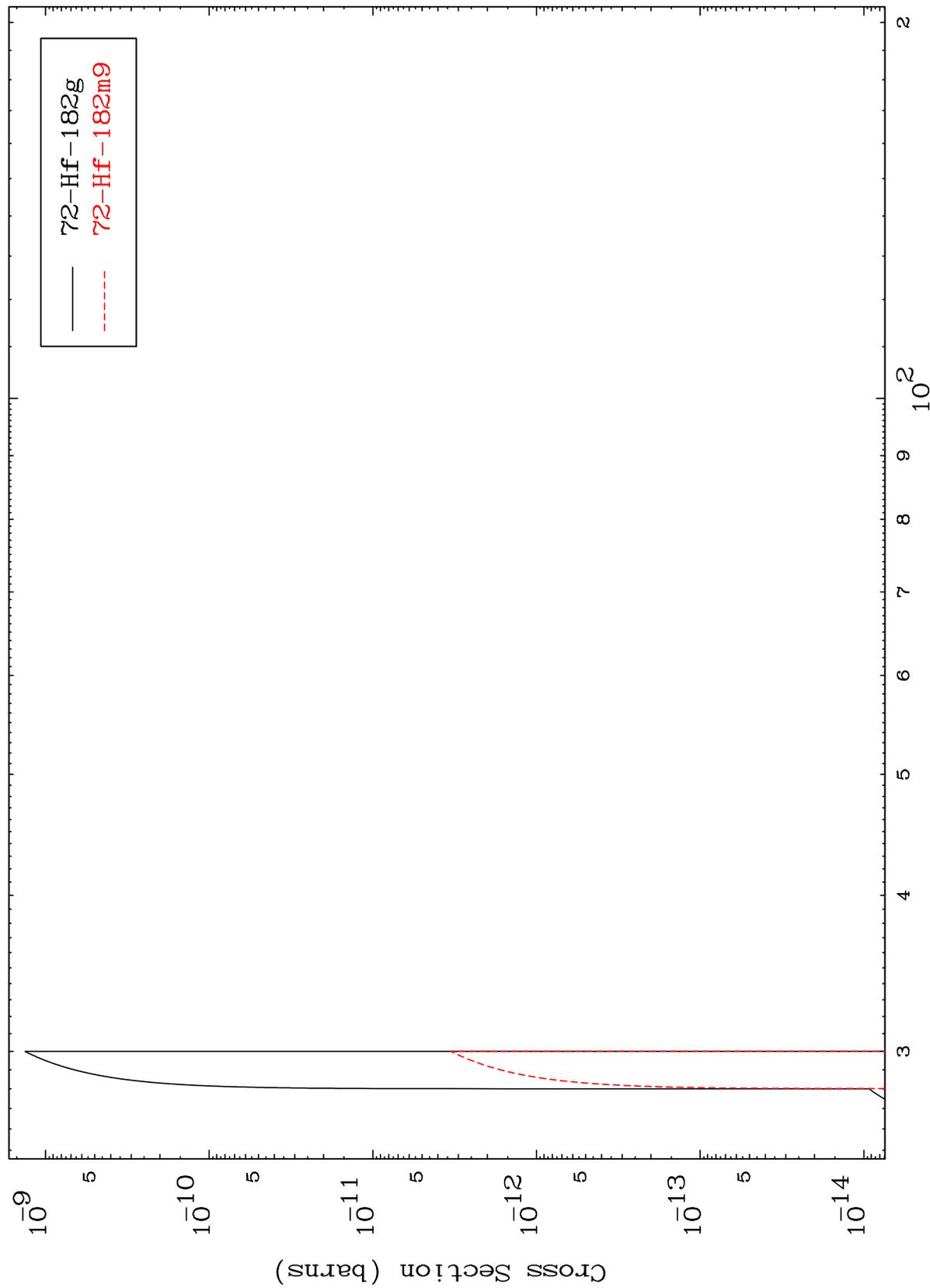


15

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section



(n,d)  
Radionuclide Production Cross Section

