

Program EVALPLOT  
(Version 2021-1)

by

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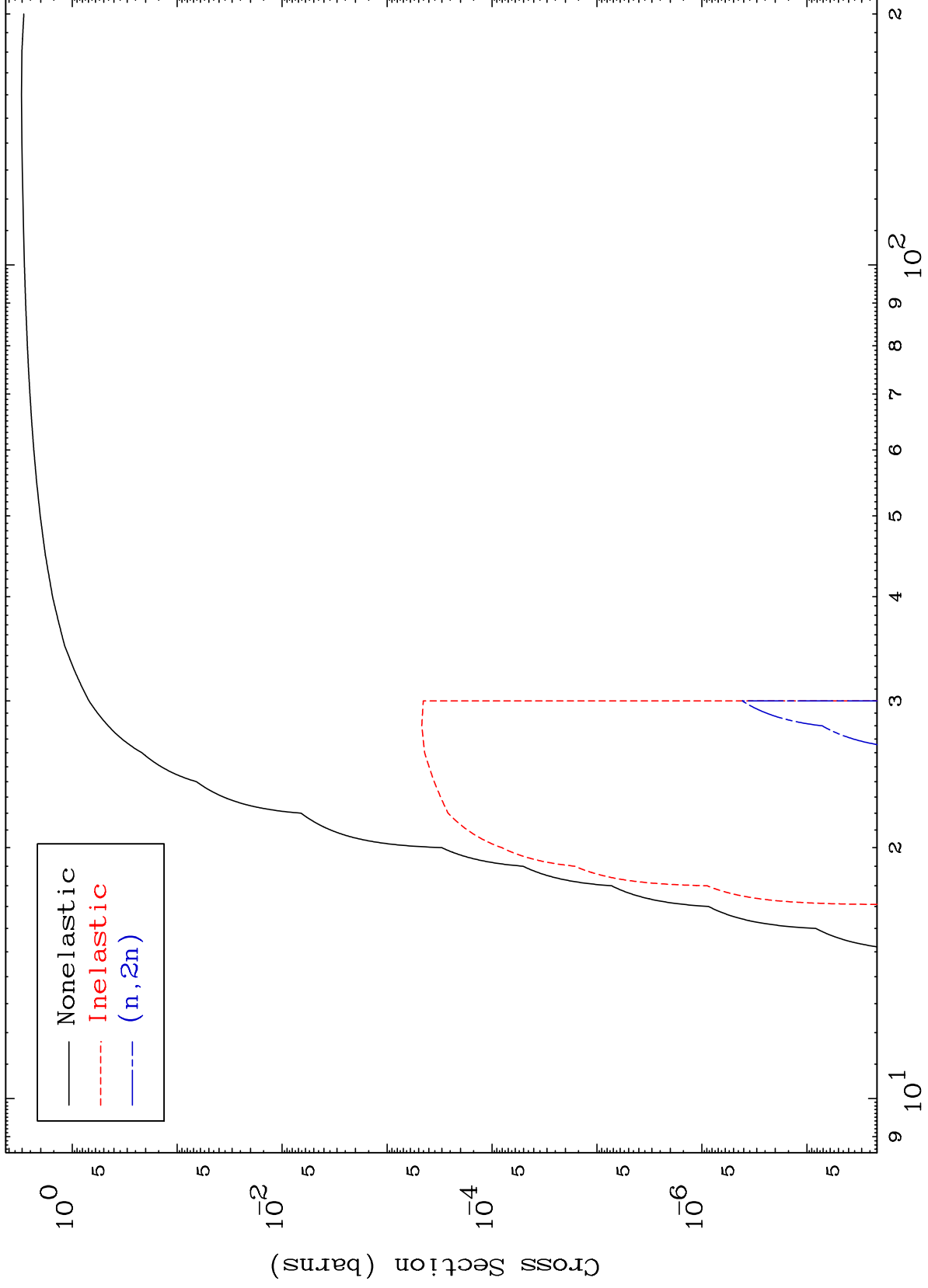
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9955

0 Kelvin  
 $\alpha$  Major  
Cross Sections

101-Md-249m



101-Md-249m

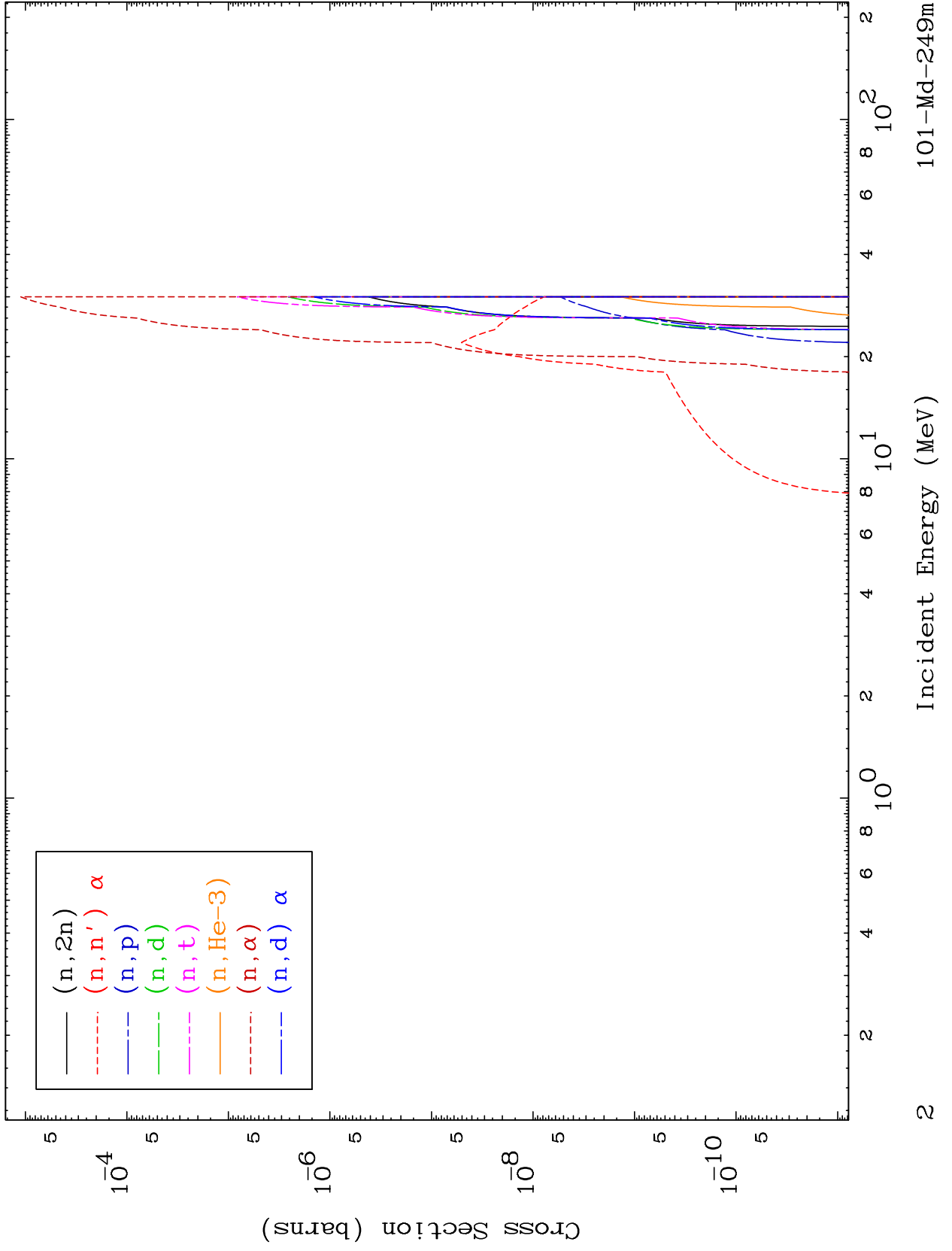
Incident Energy (MeV)

1

MAT 9955

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

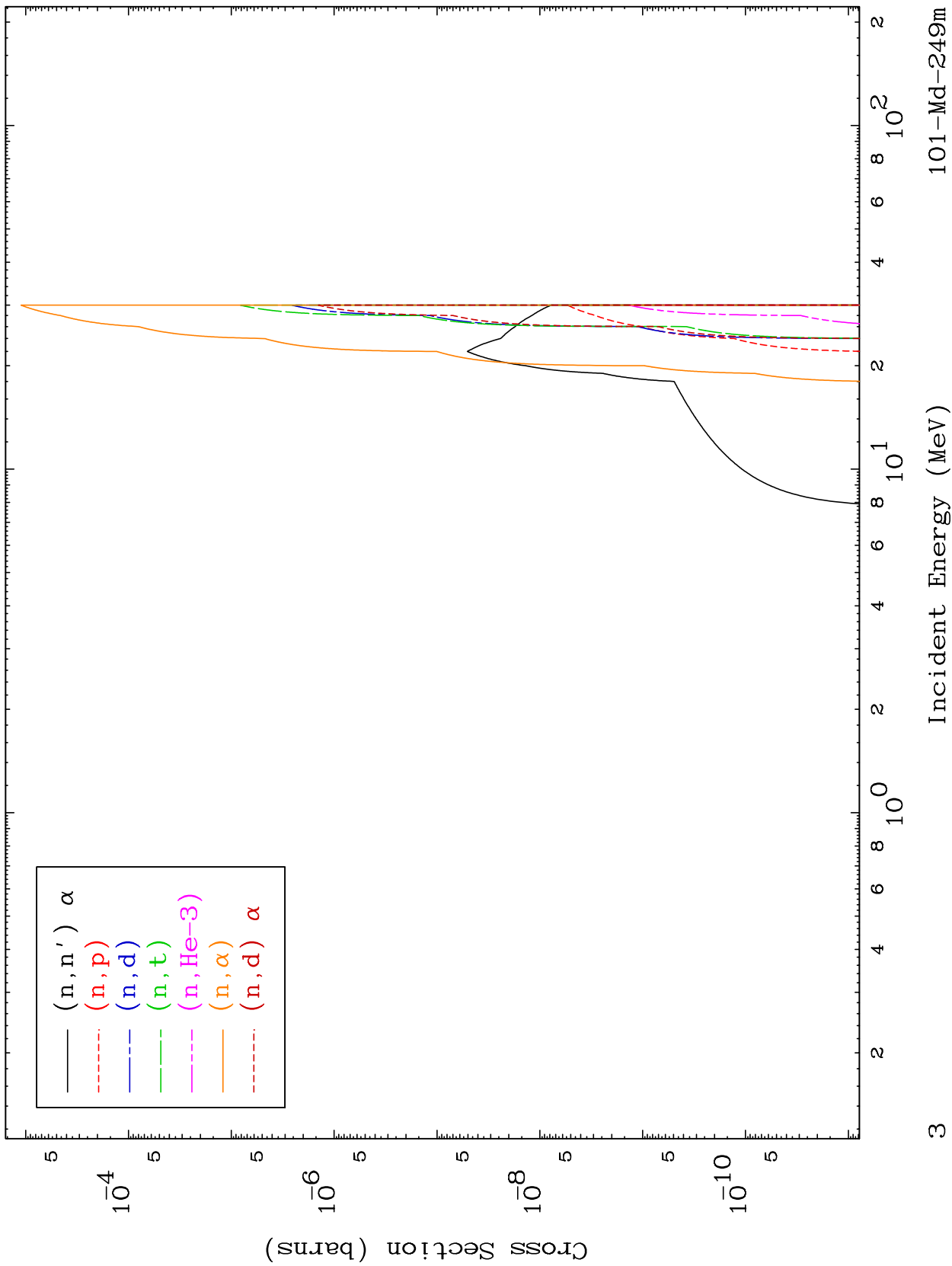
101-Md-249m



MAT 9955

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

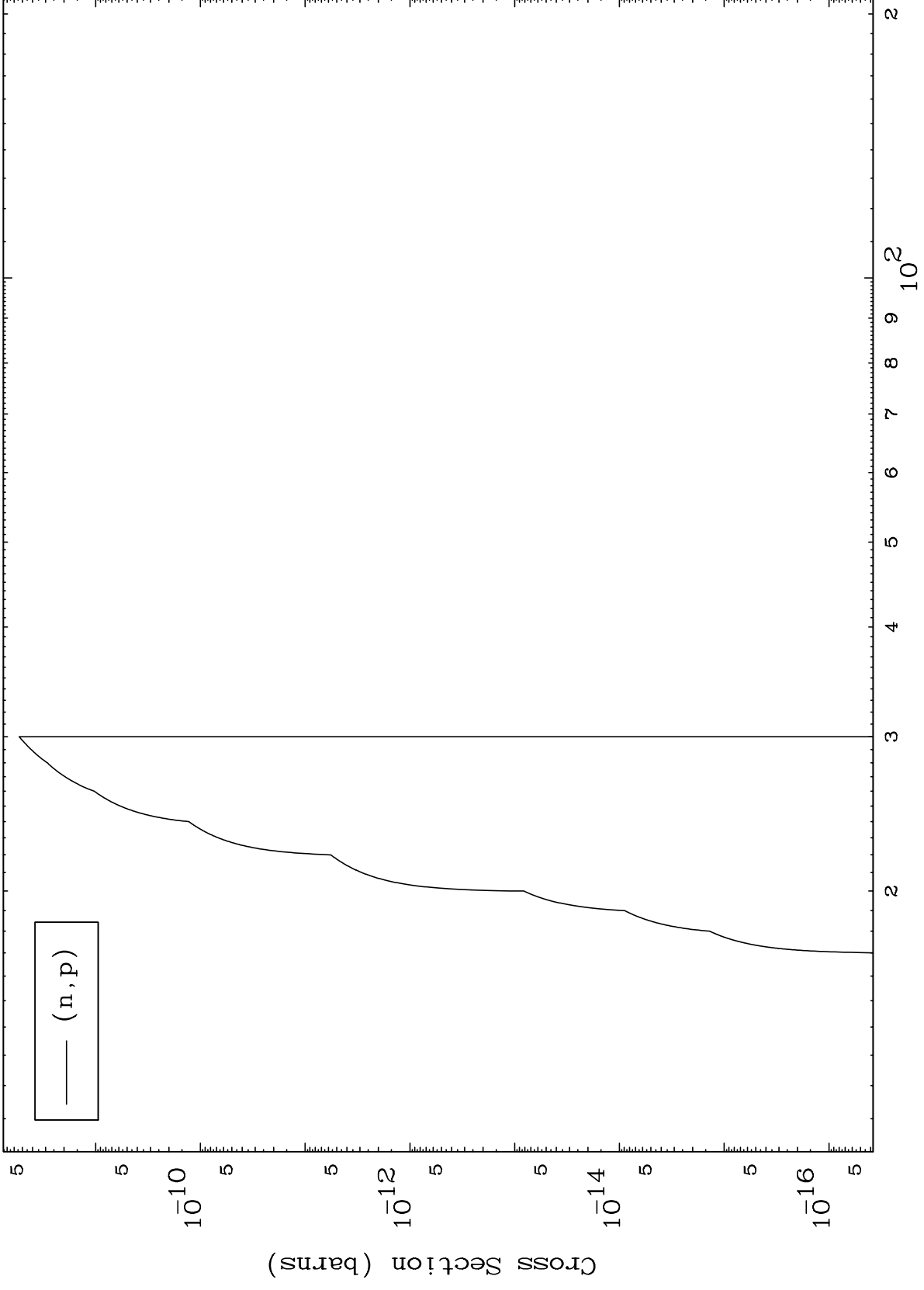
101-Md-249m



MAT 9955

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

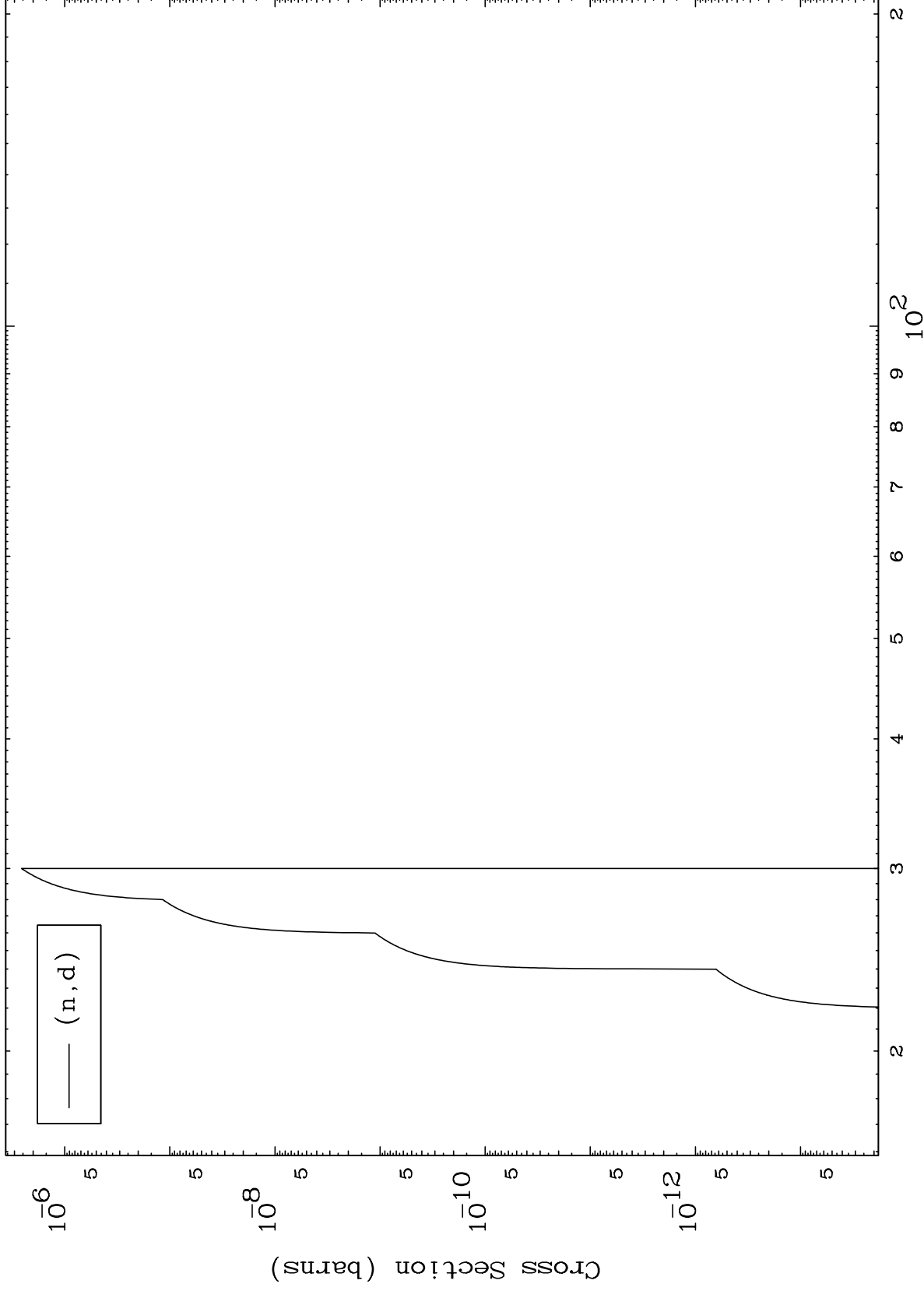
101-Md-249m



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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-249m



5

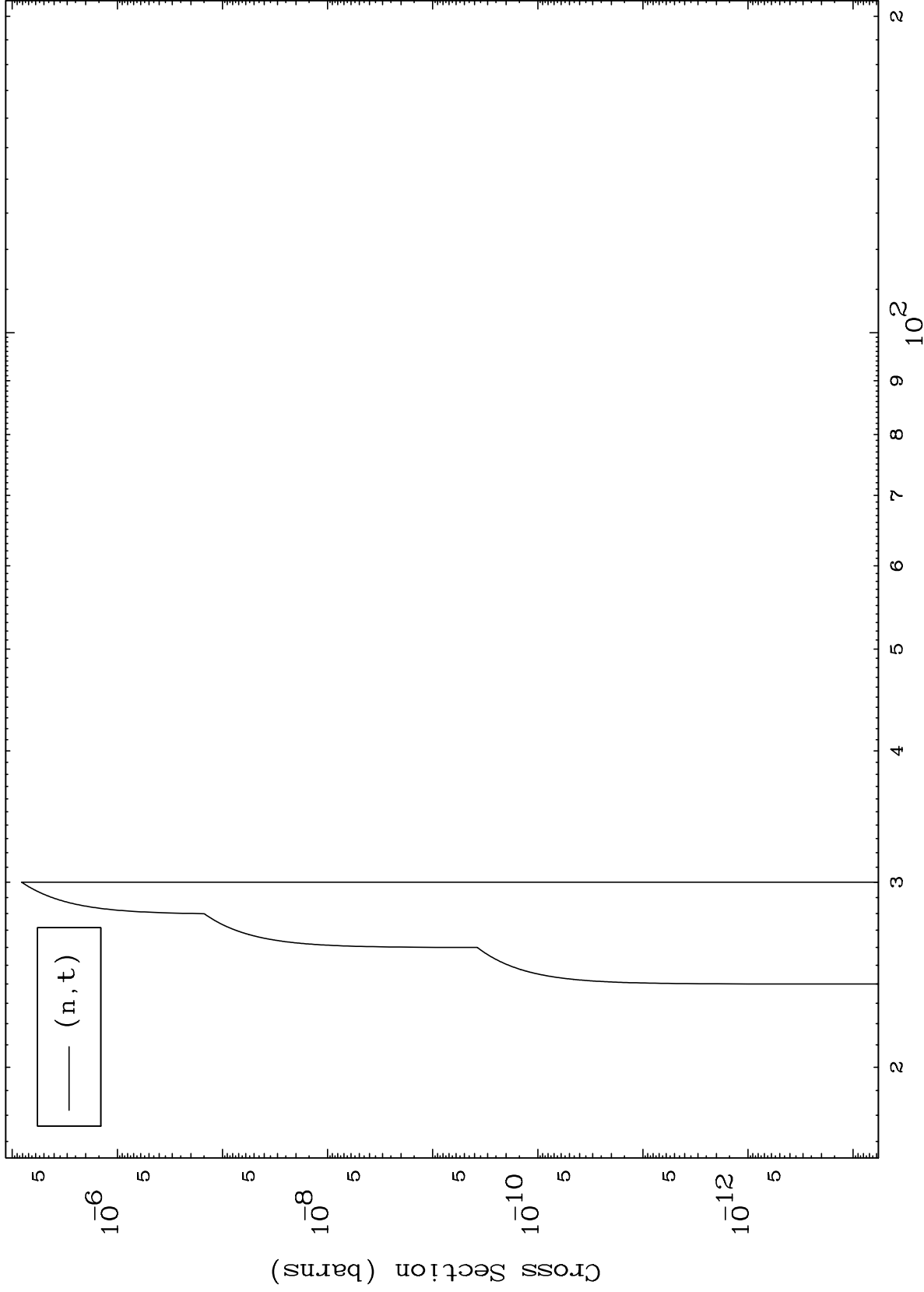
Incident Energy (MeV)

101-Md-249m

MAT 9955

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-249m



6

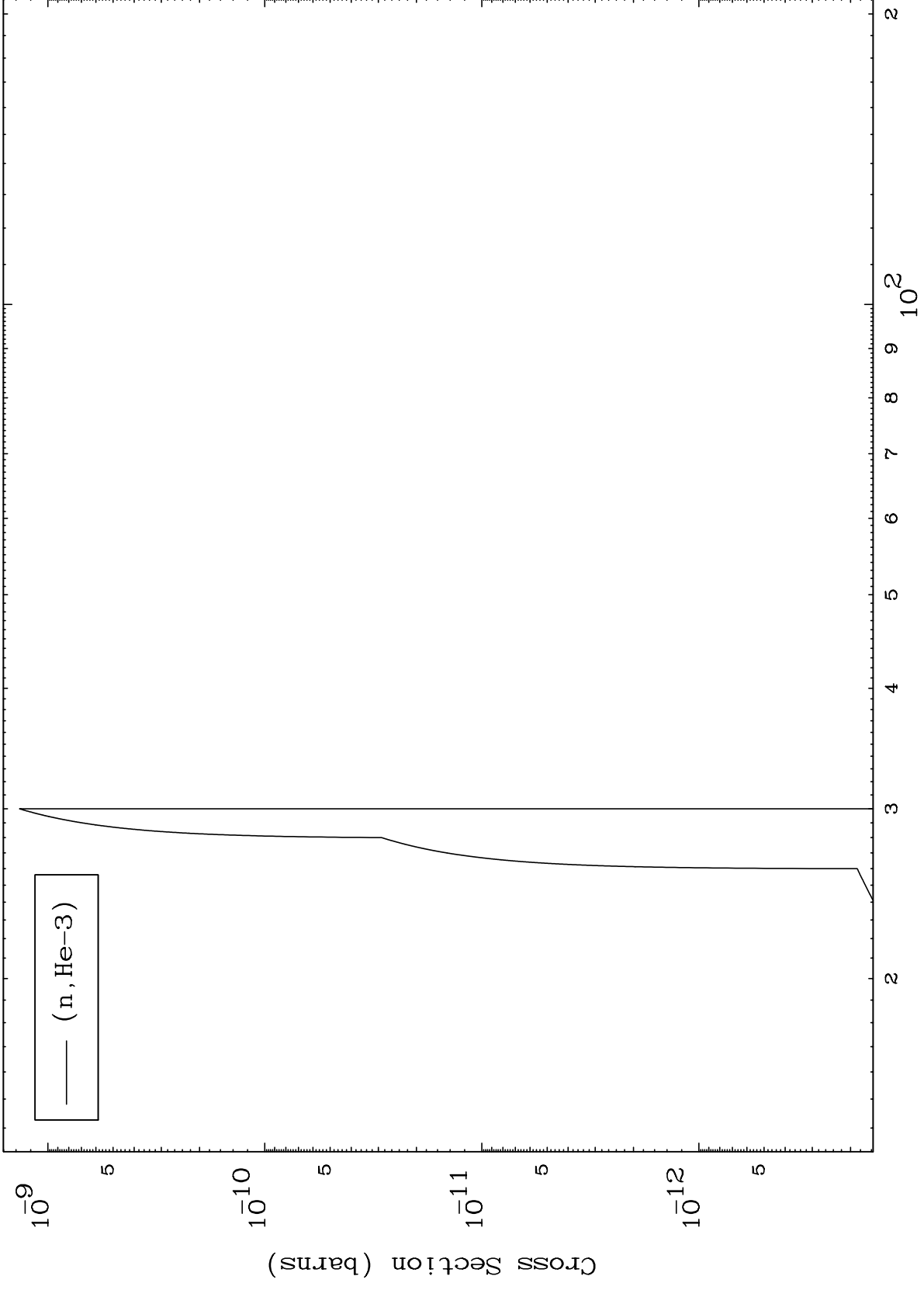
Incident Energy (MeV)

101-Md-249m

MAT 9955

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

101-Md-249m



7

Incident Energy (MeV)

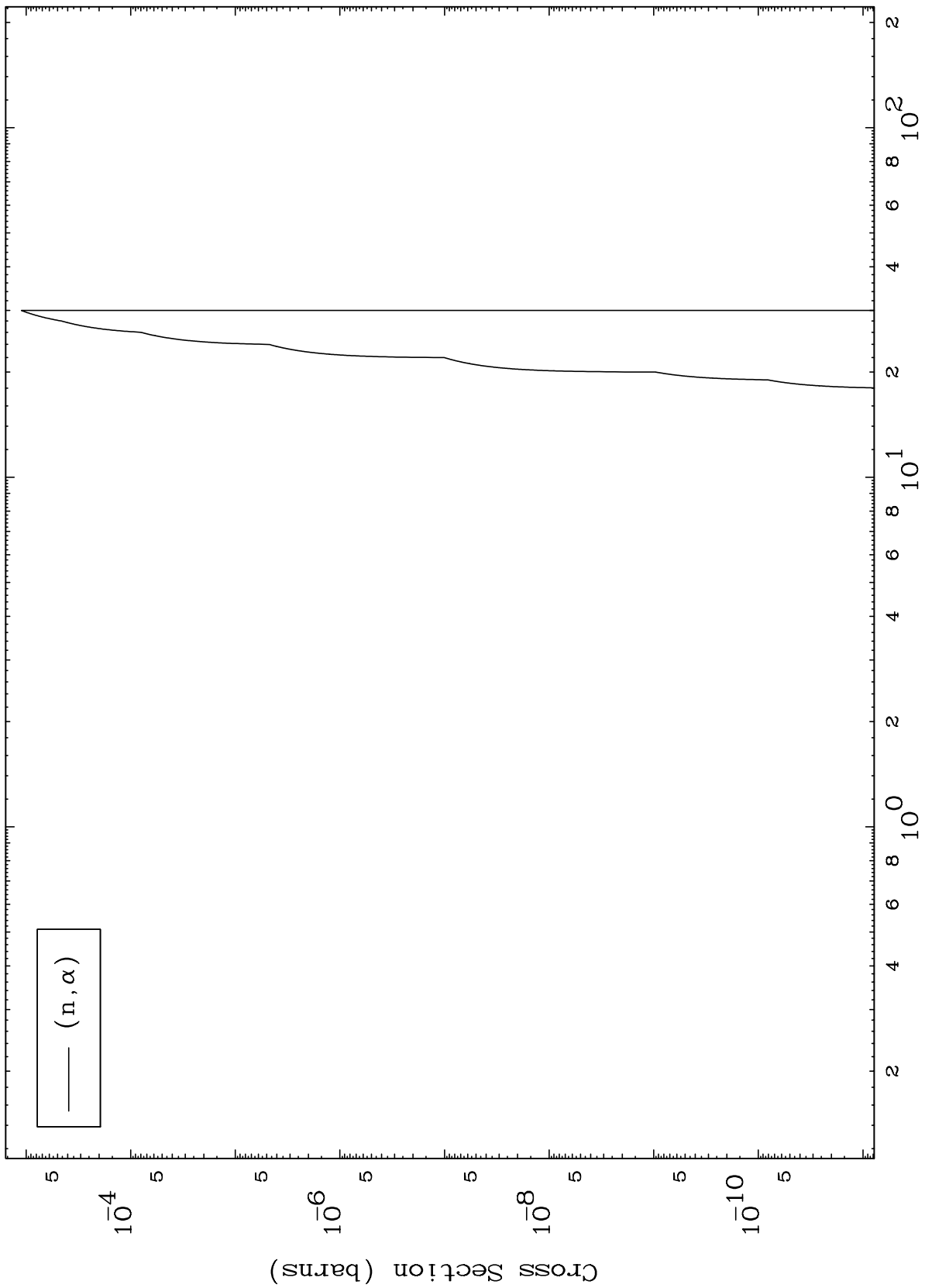
101-Md-249m

MAT 9955

( $\alpha, \alpha$ ) Levels

101-Md-249m

0 Kelvin Cross Sections

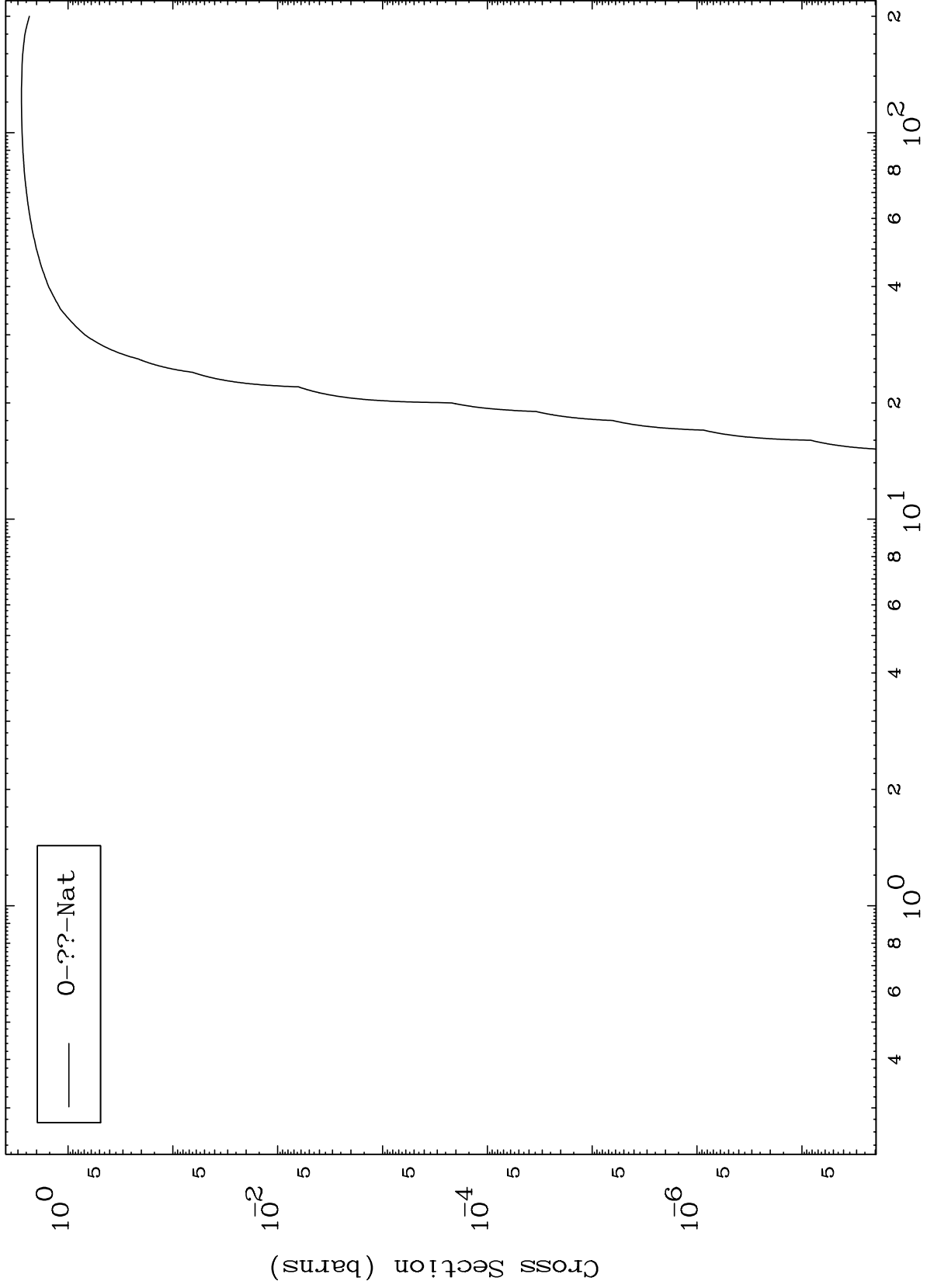


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Fission

101-Md-249m

Radionuclide Production Cross Section

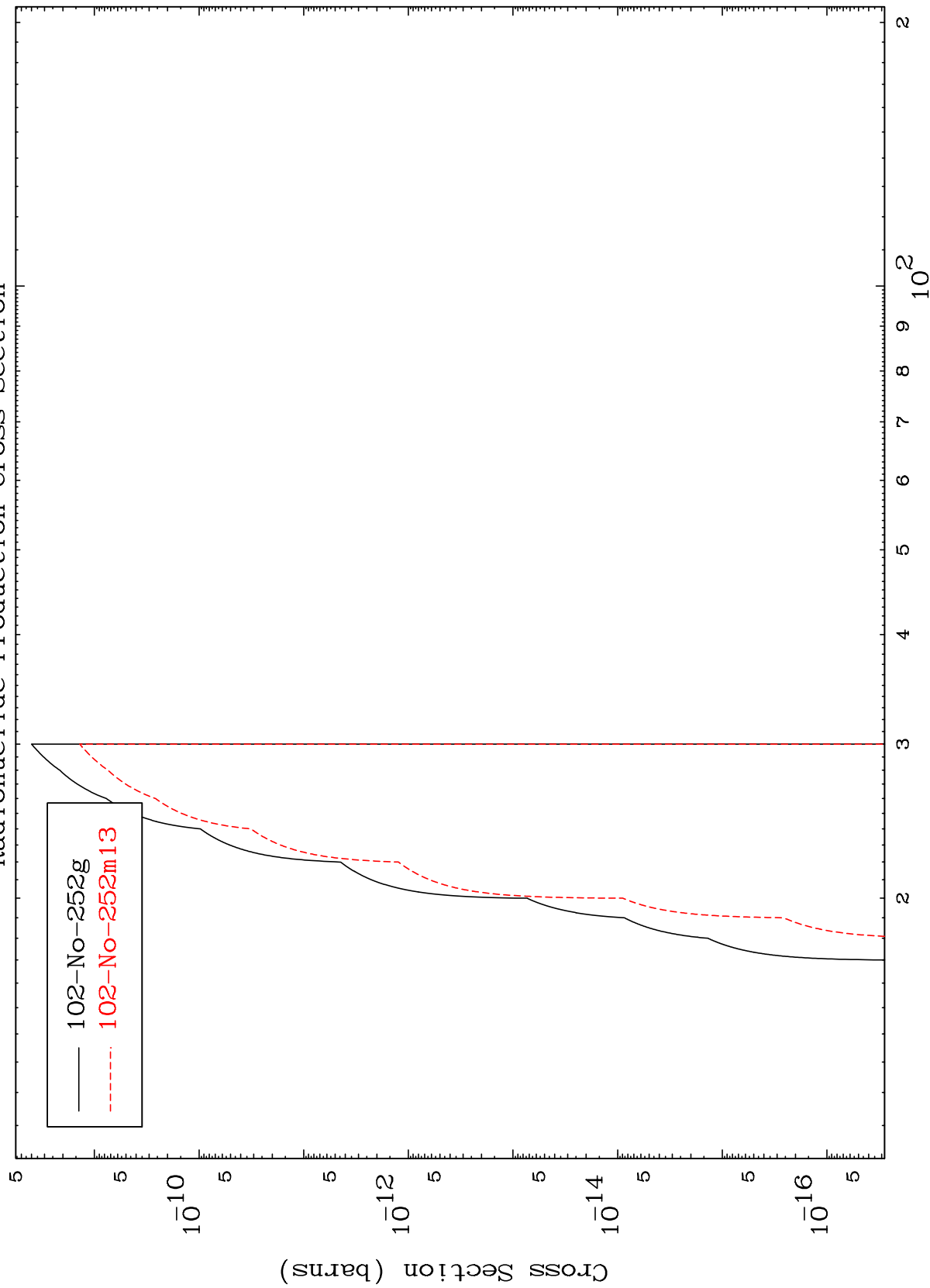


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(n,p)

101-Md-249m

Radionuclide Production Cross Section



10

Incident Energy (MeV)

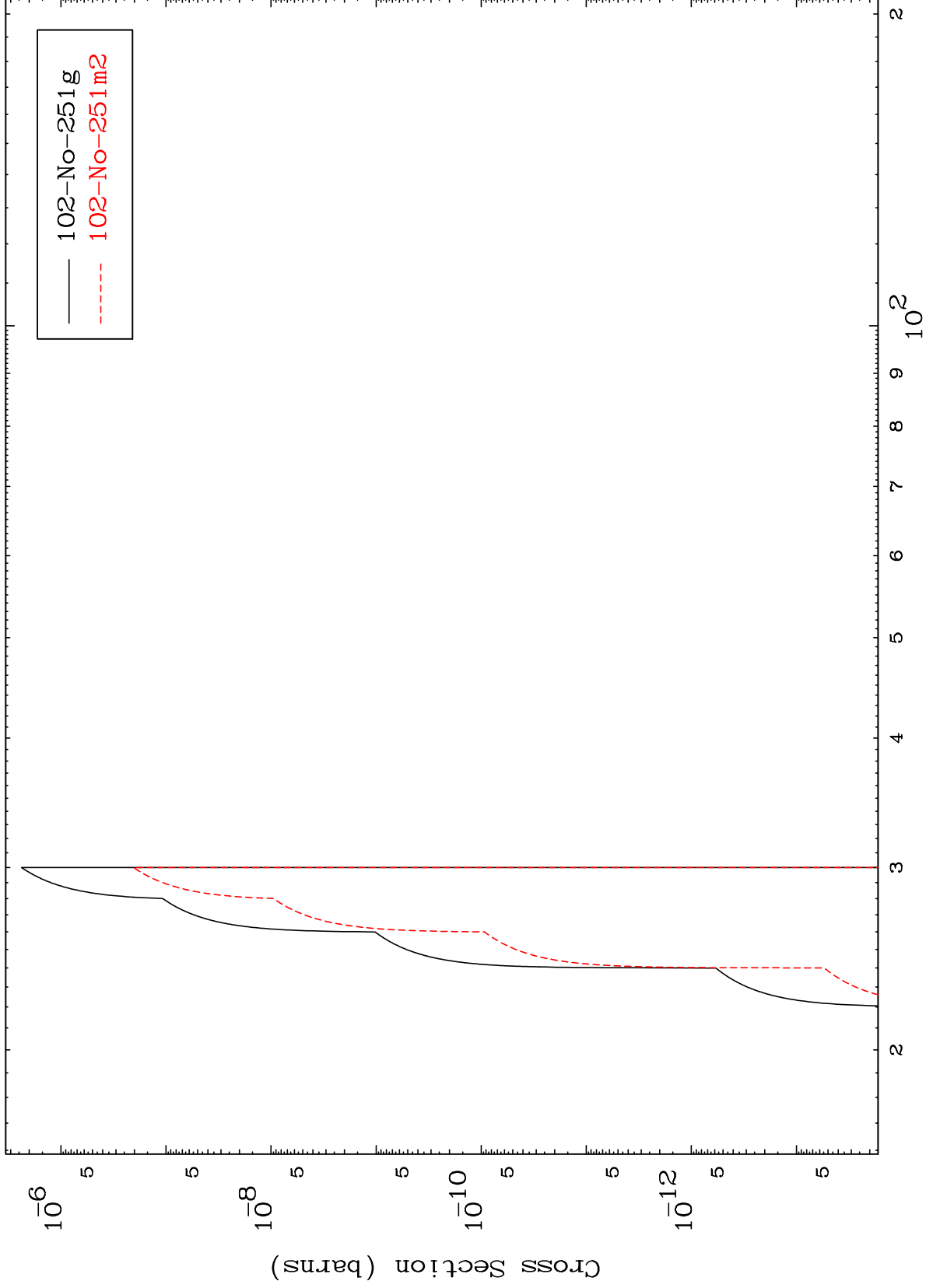
101-Md-249m

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(n,d)

101-Md-249m

Radionuclide Production Cross Section



11

Incident Energy (MeV)

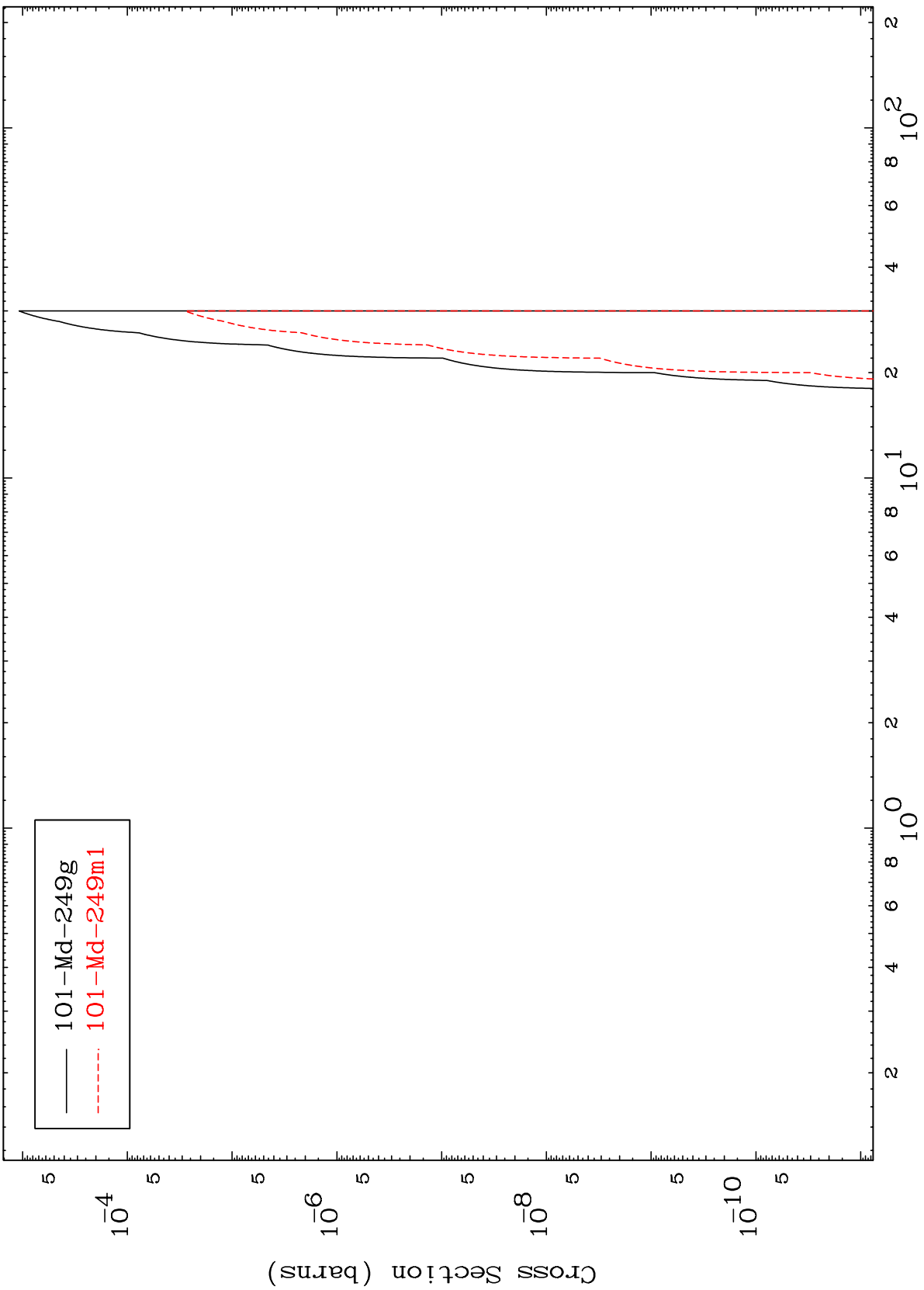
101-Md-249m

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(n,  $\alpha$ )

101-Md-249m

Radionuclide Production Cross Section



— 101-Md-249g  
- - - 101-Md-249m1

Incident Energy (MeV)

101-Md-249m

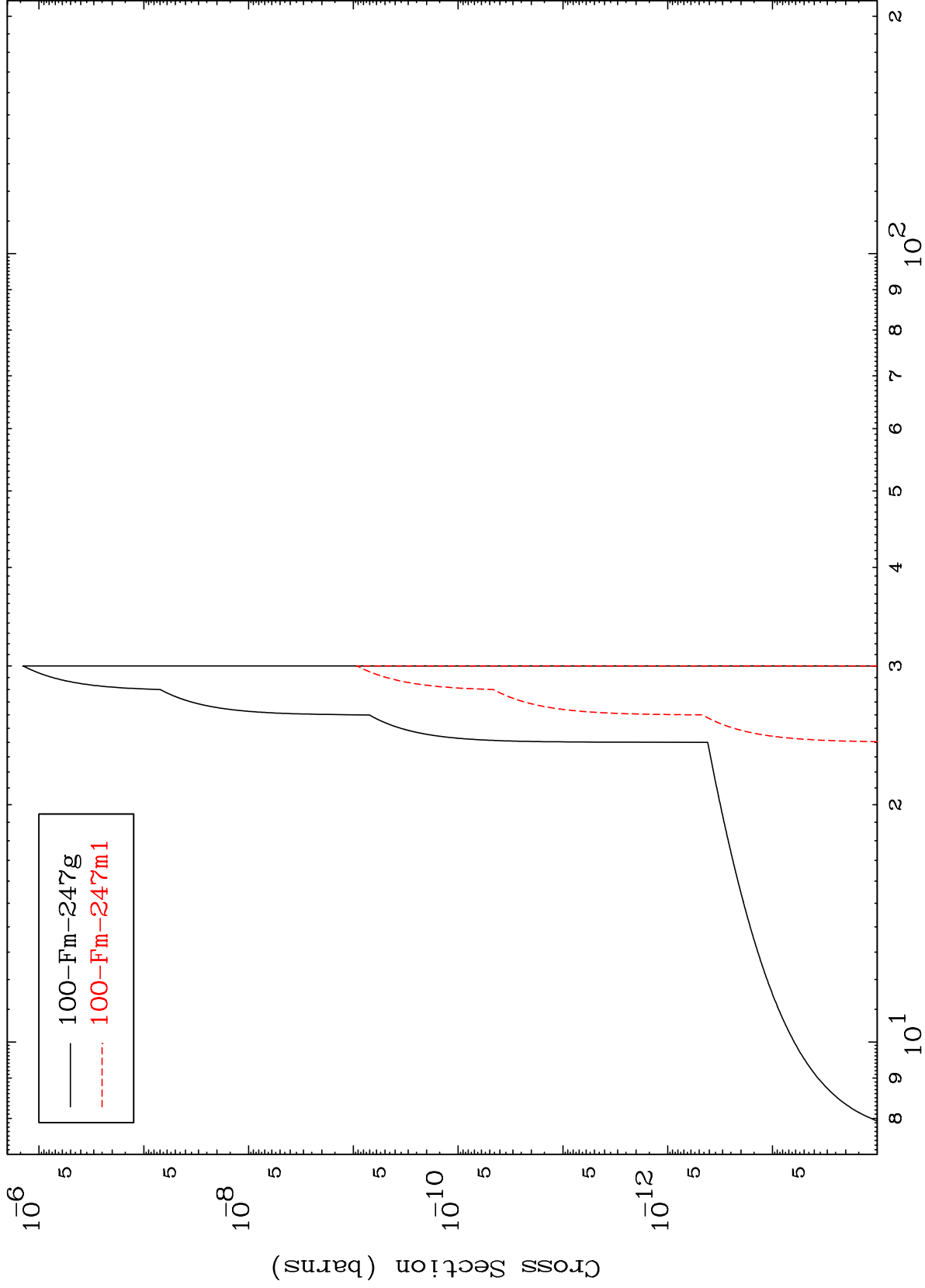
12

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(n,d)  $\alpha$

101-Md-249m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

101-Md-249m