

Program EVALPLOT  
(Version 2021-1)

by

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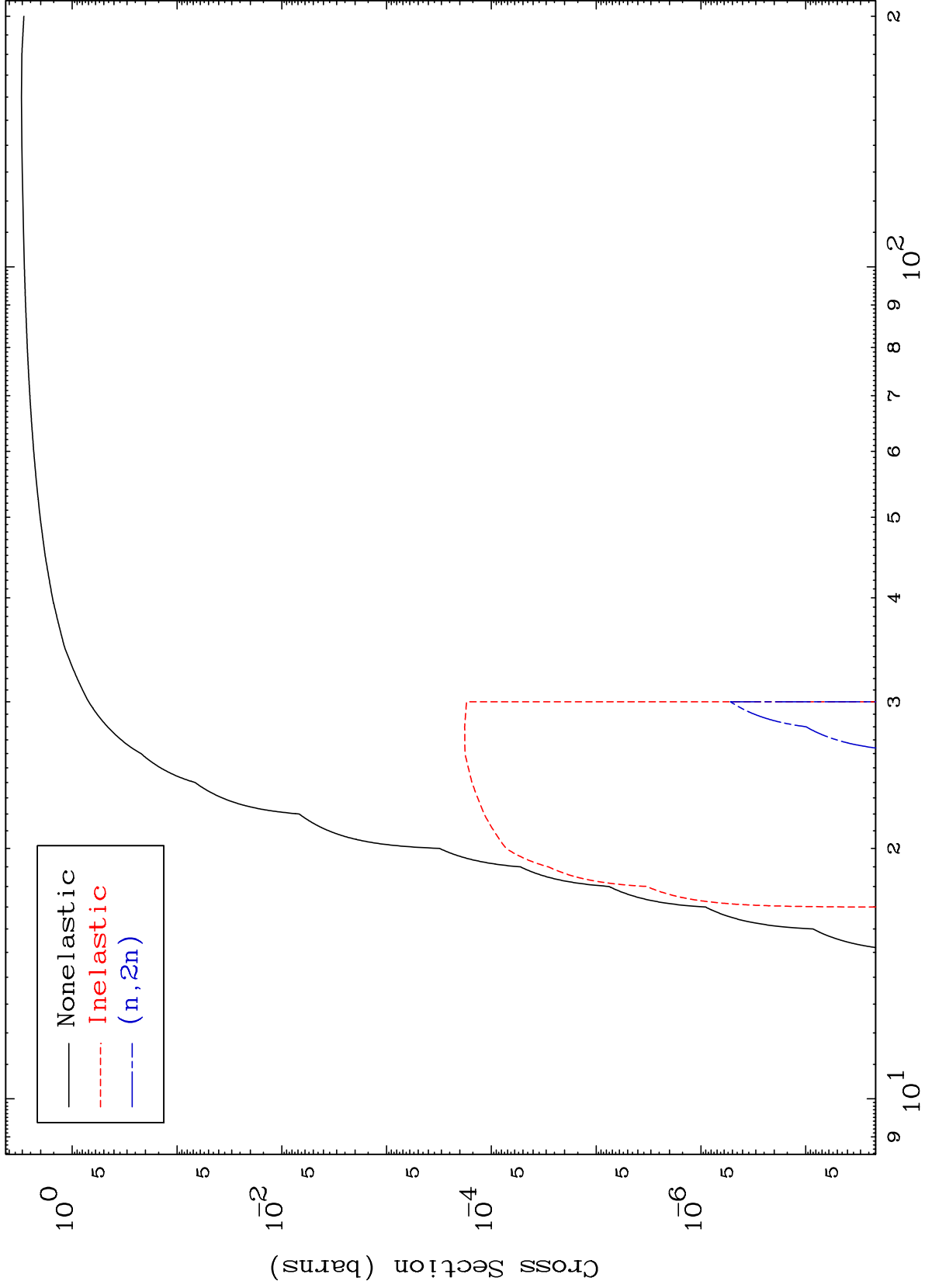
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9951

0 Kelvin  $\alpha$  Major Cross Sections

101-Md-250



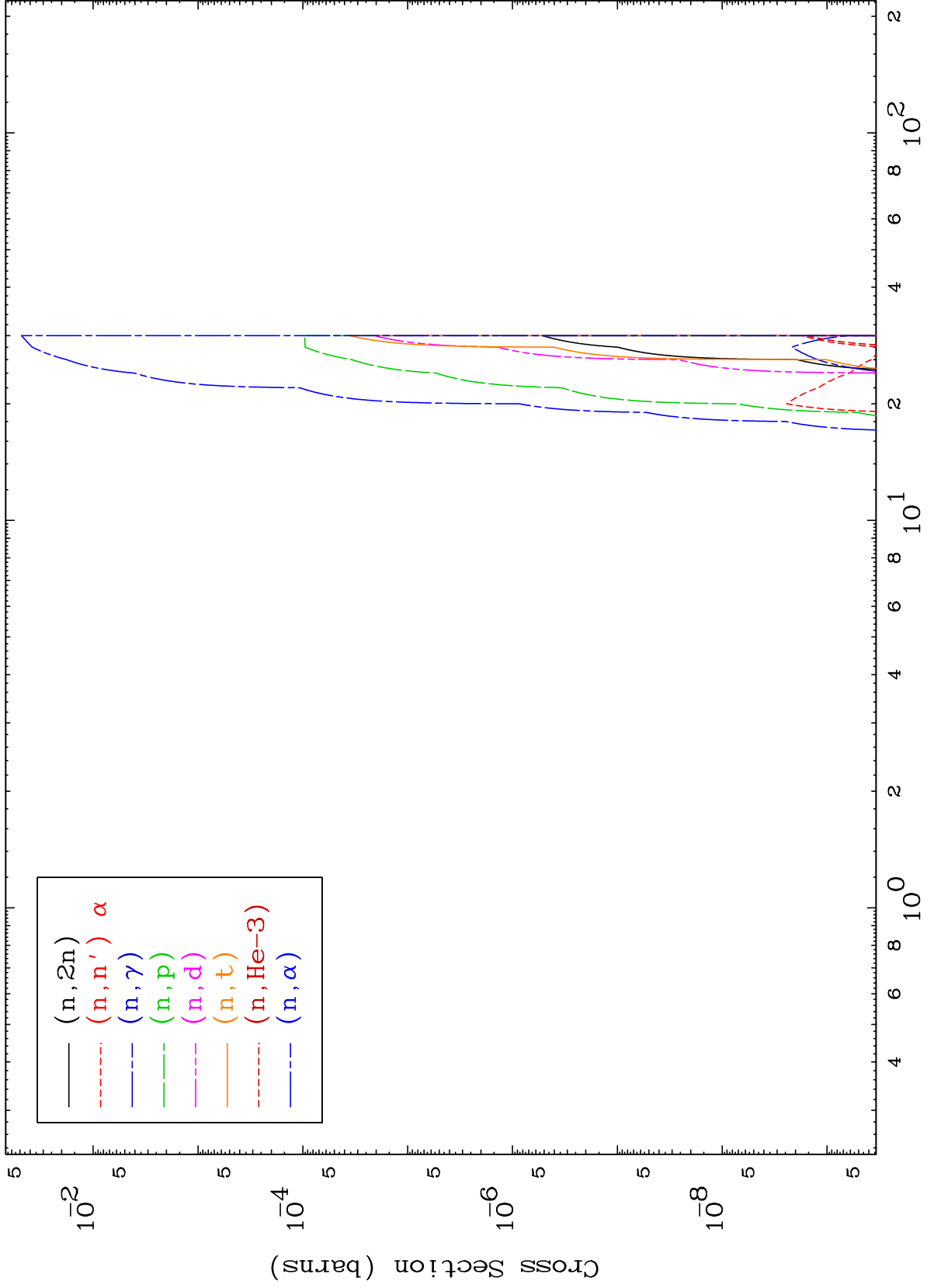
Incident Energy (MeV)

101-Md-250

MAT 9951

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

101-Md-250

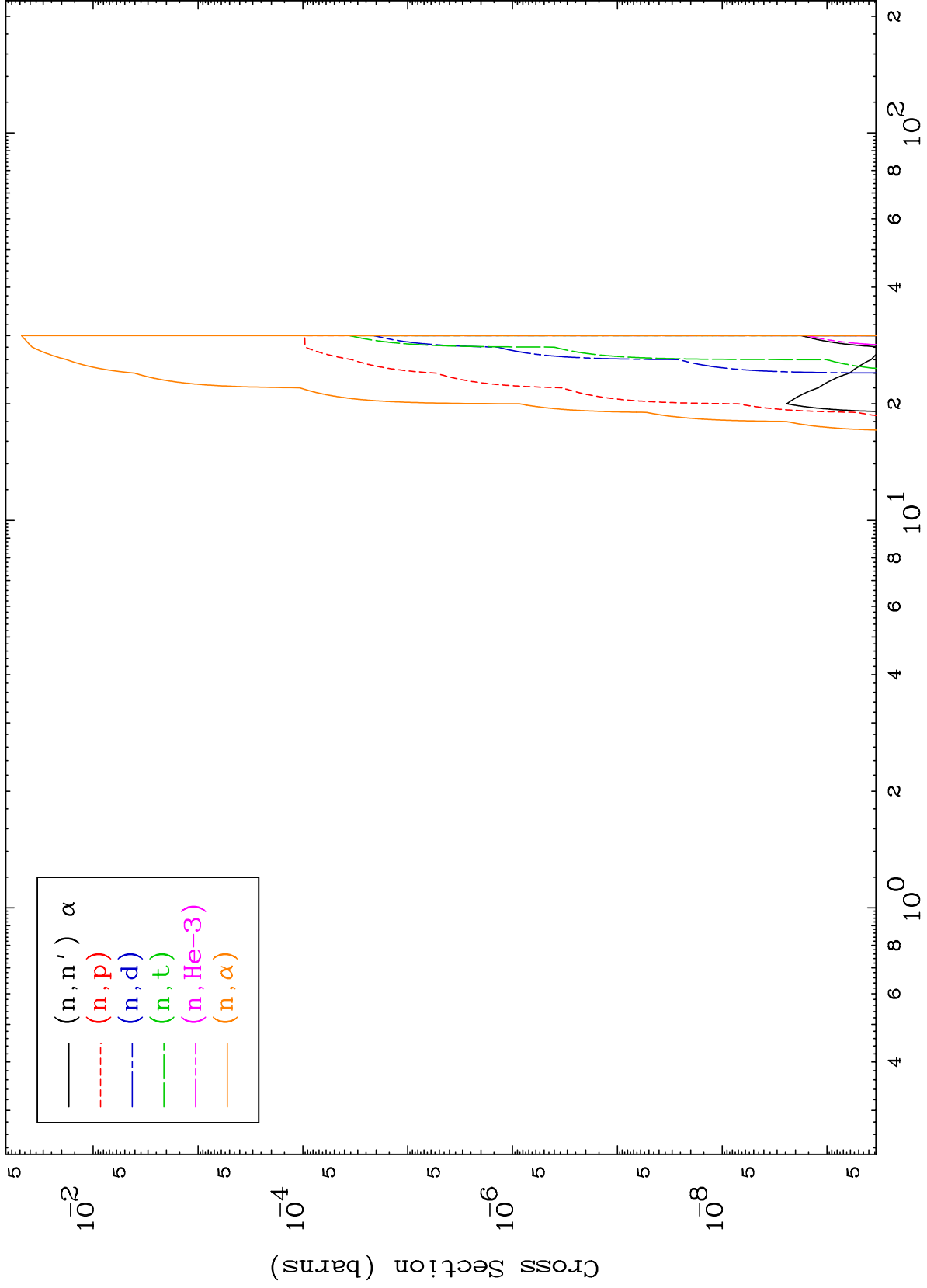


MAT 9951

$\alpha$  Charged Particle

101-Md-250

0 Kelvin Cross Sections



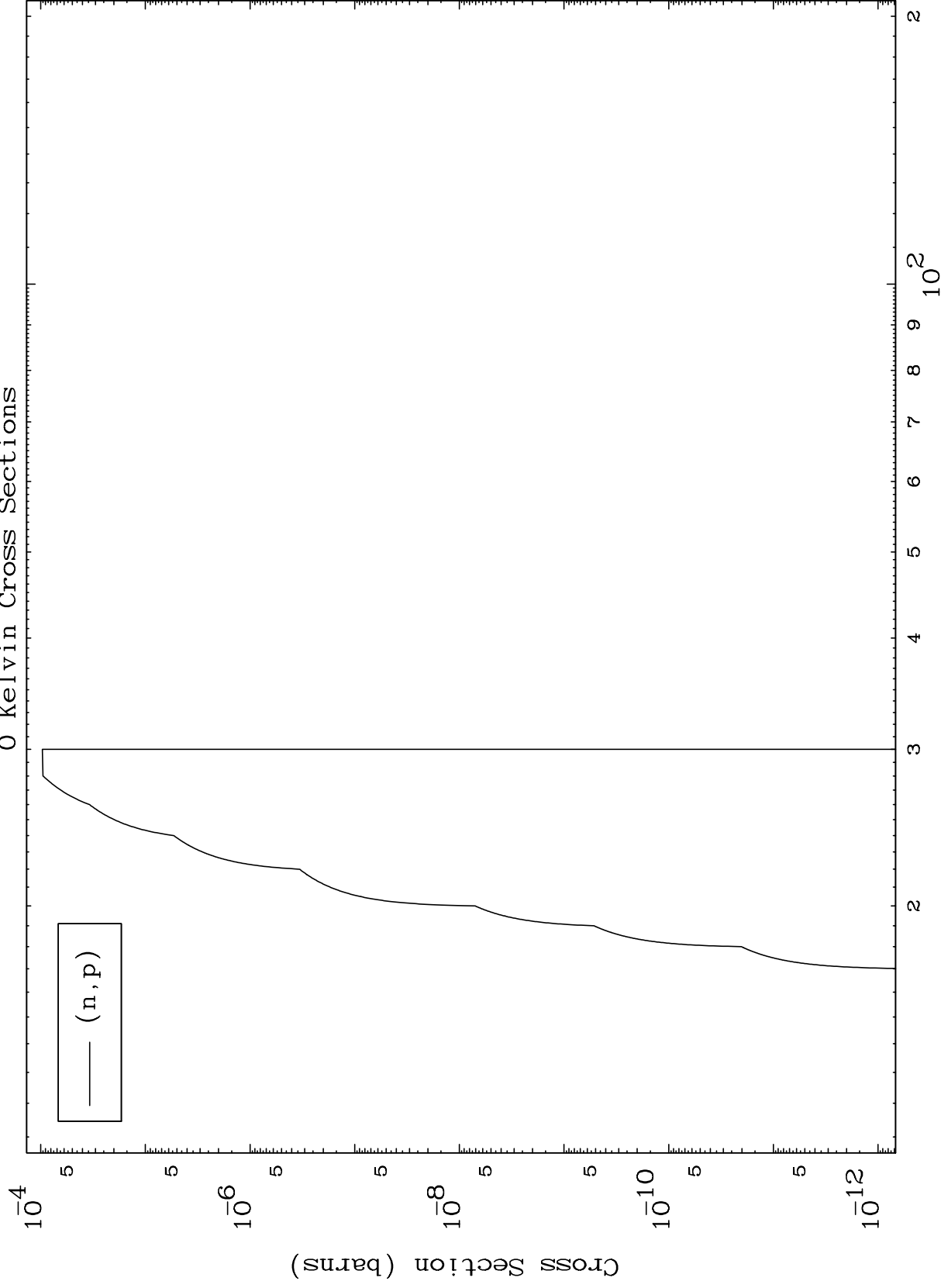
101-Md-250

Incident Energy (MeV)

MAT 9951

101-Md-250

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



101-Md-250

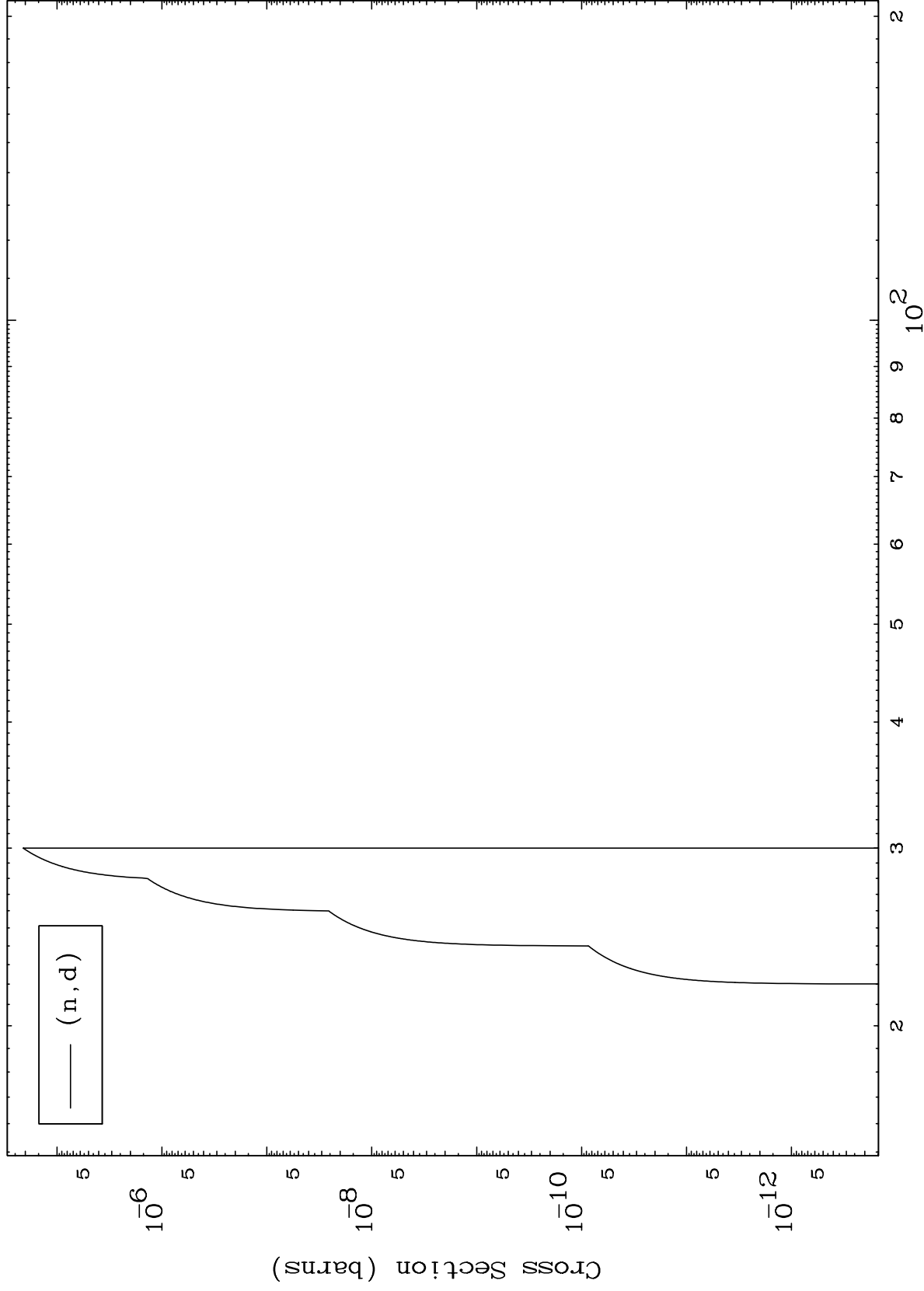
Incident Energy (MeV)

4

MAT 9951

101-Md-250

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



101-Md-250

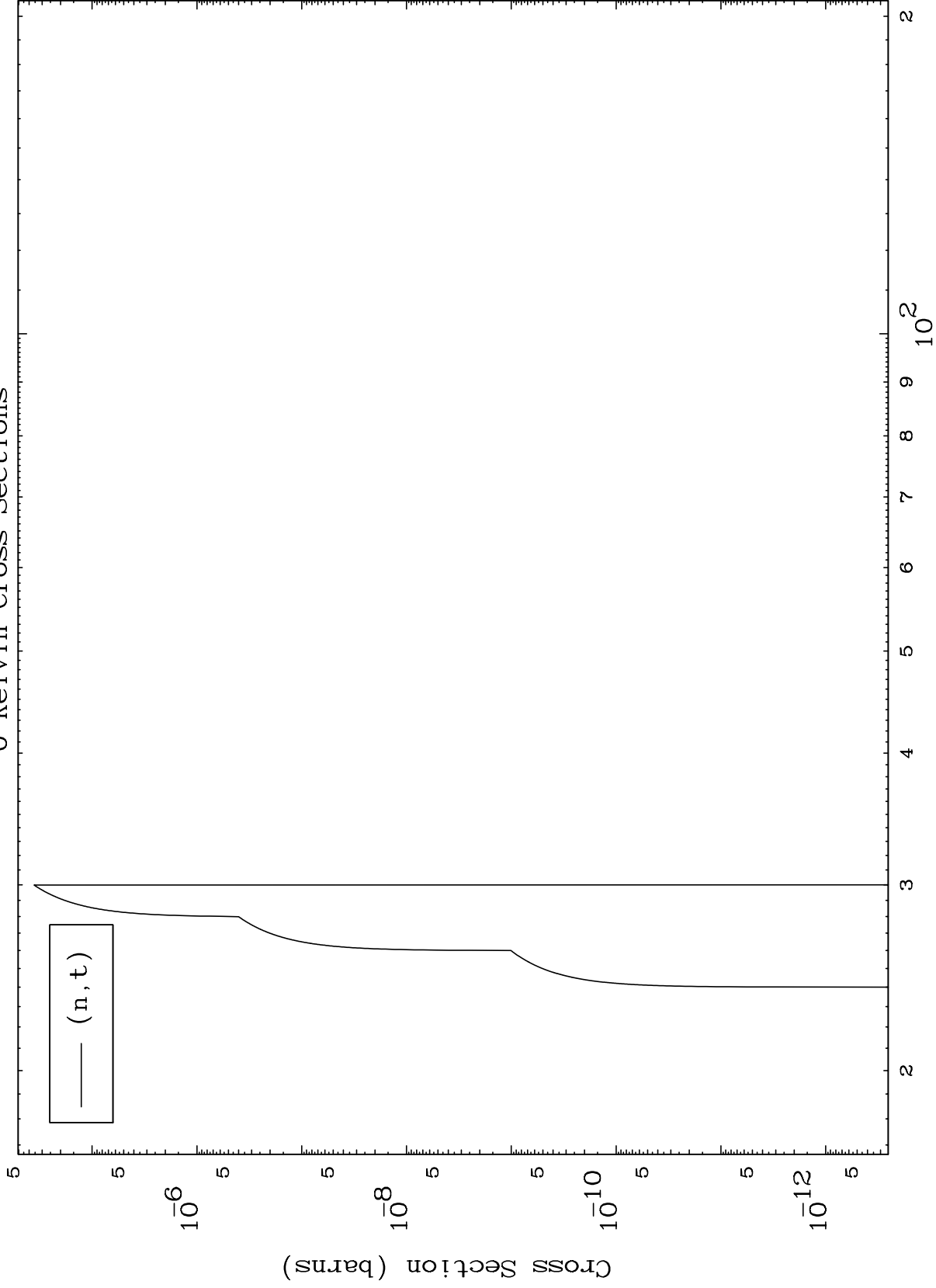
Incident Energy (MeV)

5

MAT 9951

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-250

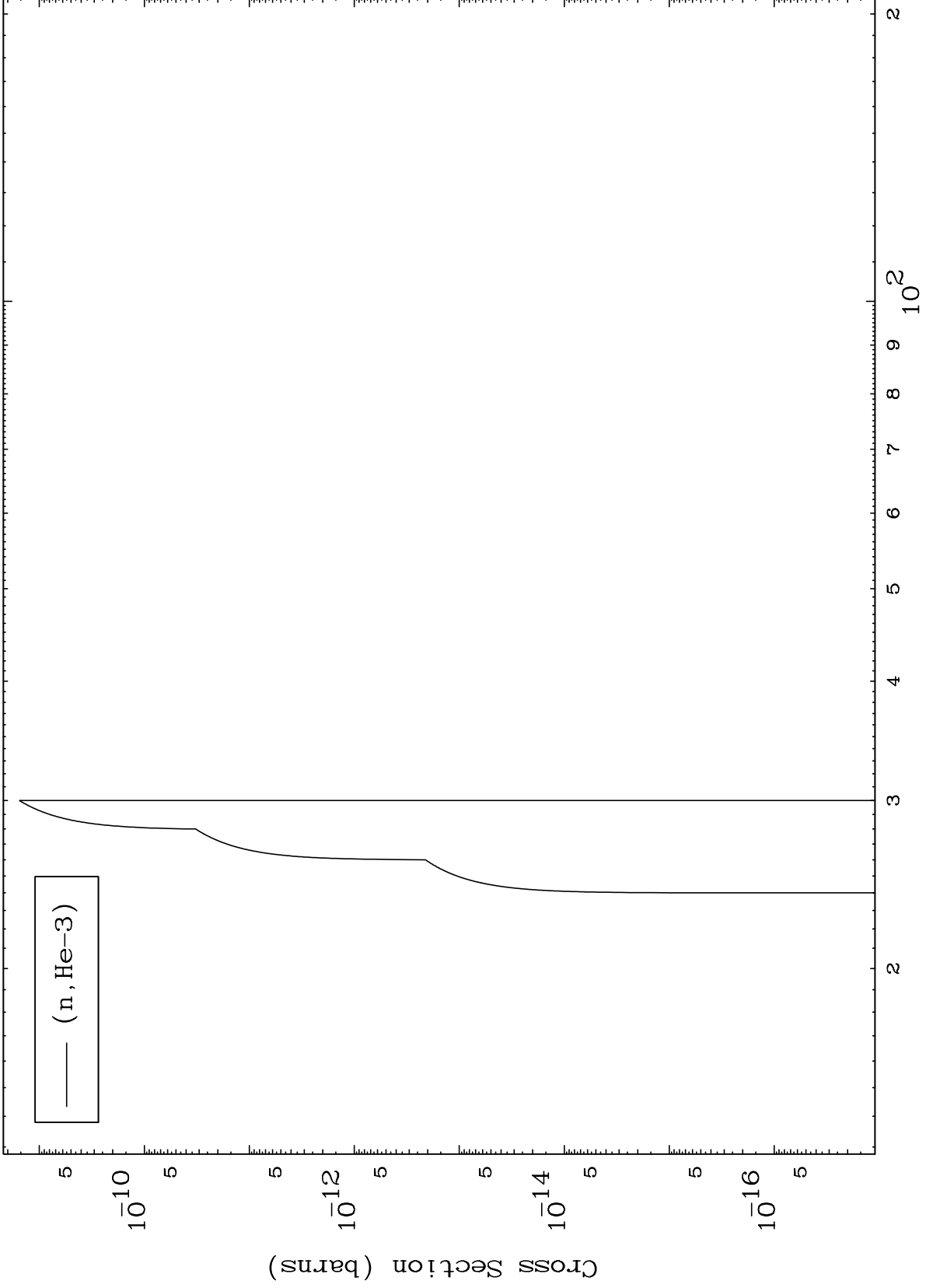


6

Incident Energy (MeV)

101-Md-250

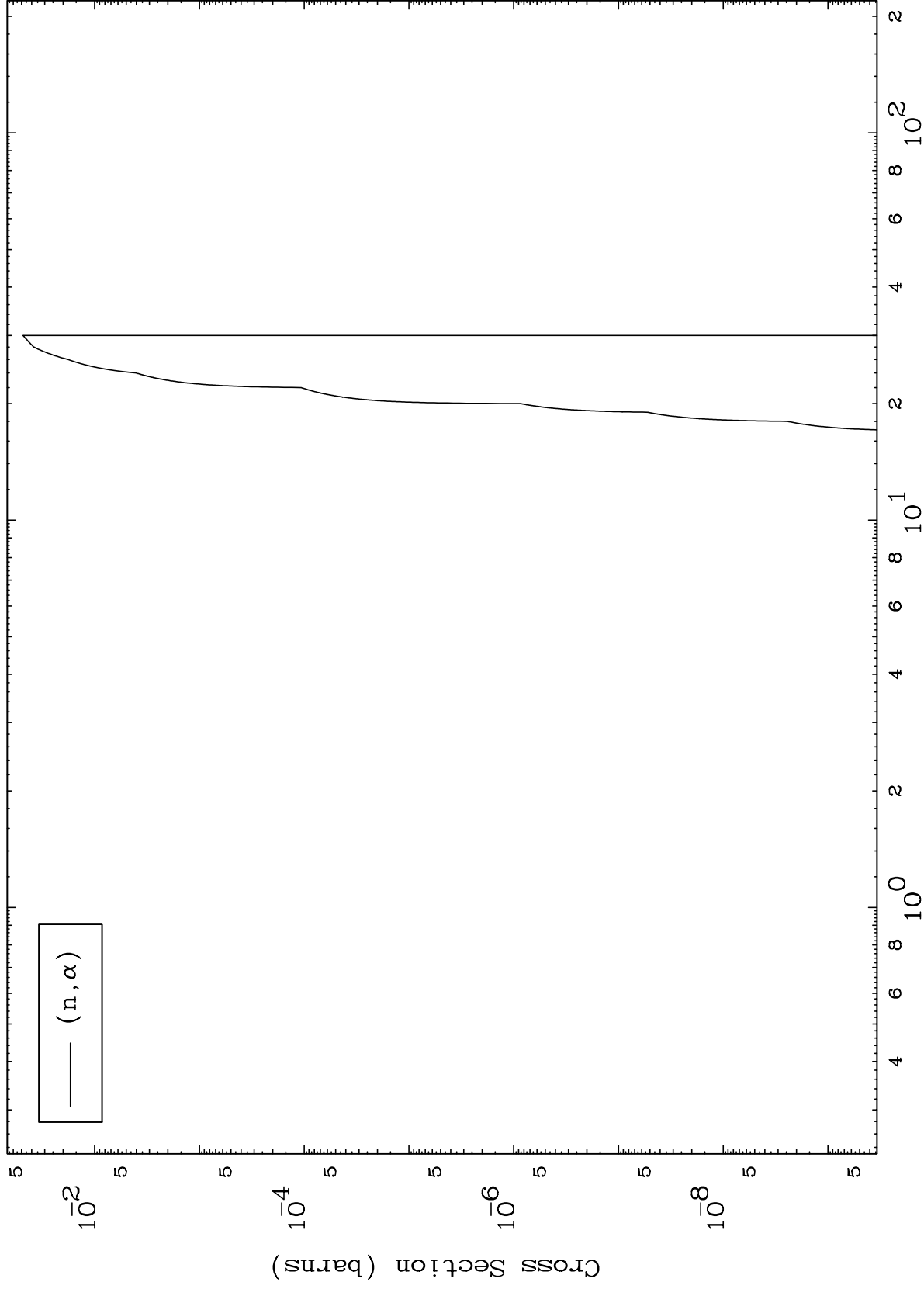
( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



MAT 9951

101-Md-250

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



8

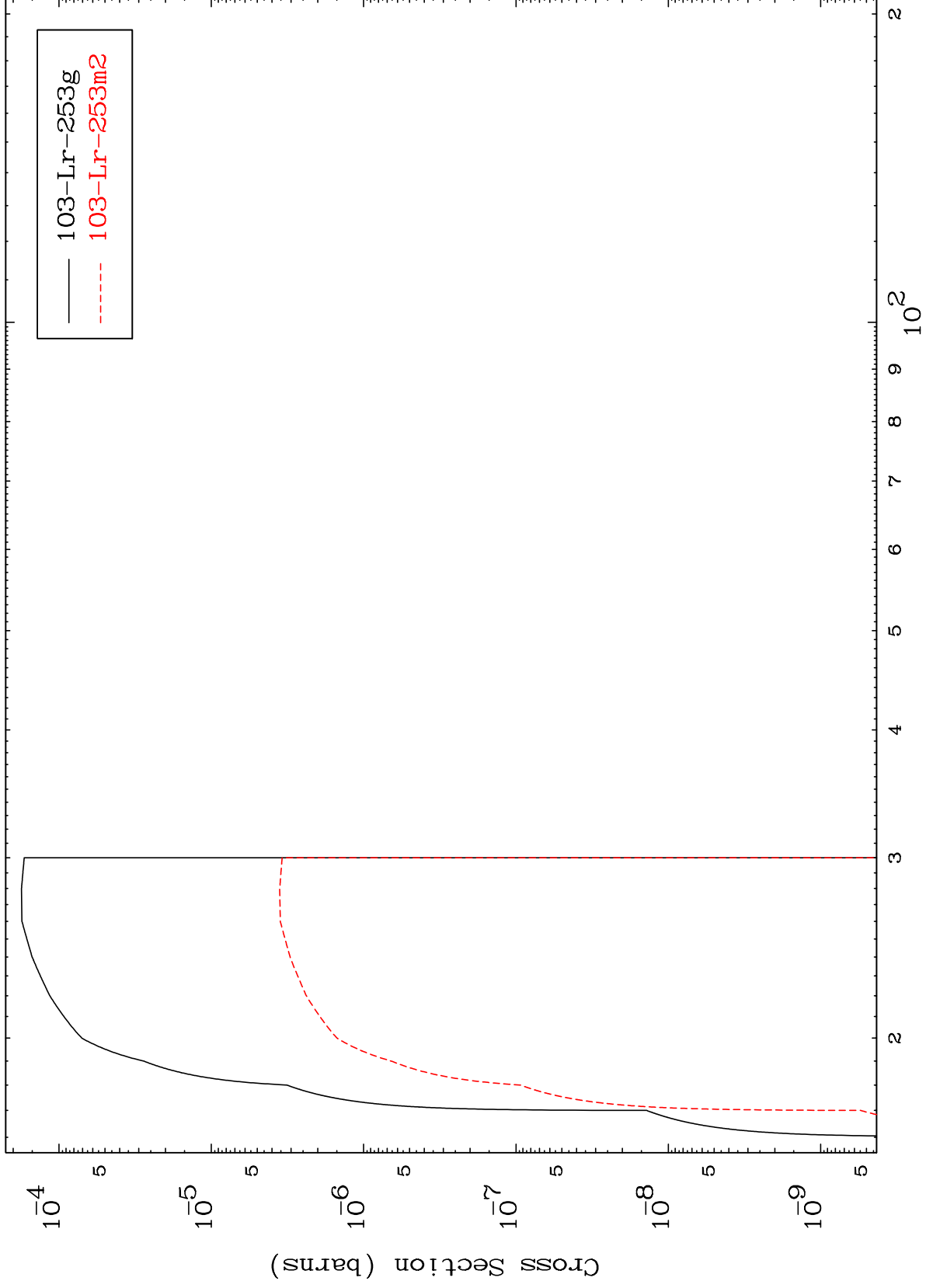
101-Md-250

Incident Energy (MeV)

MAT 9951

101-Md-250

Inelastic  
Radionuclide Production Cross Section



101-Md-250

Incident Energy (MeV)

9

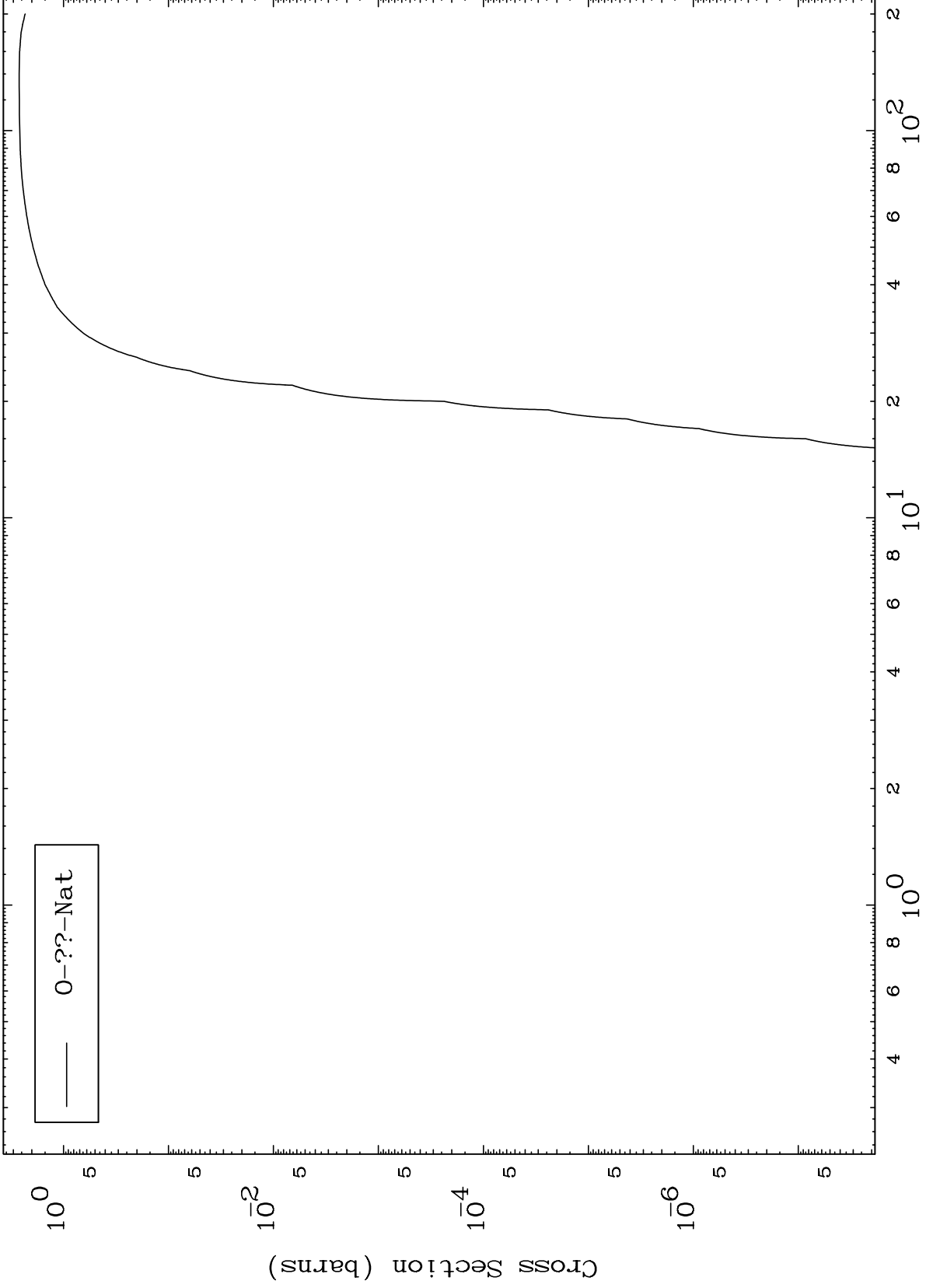
MAT 9951

Fission

101-Md-250

Radionuclide Production Cross Section

— 0-??-Nat



10

Incident Energy (MeV)

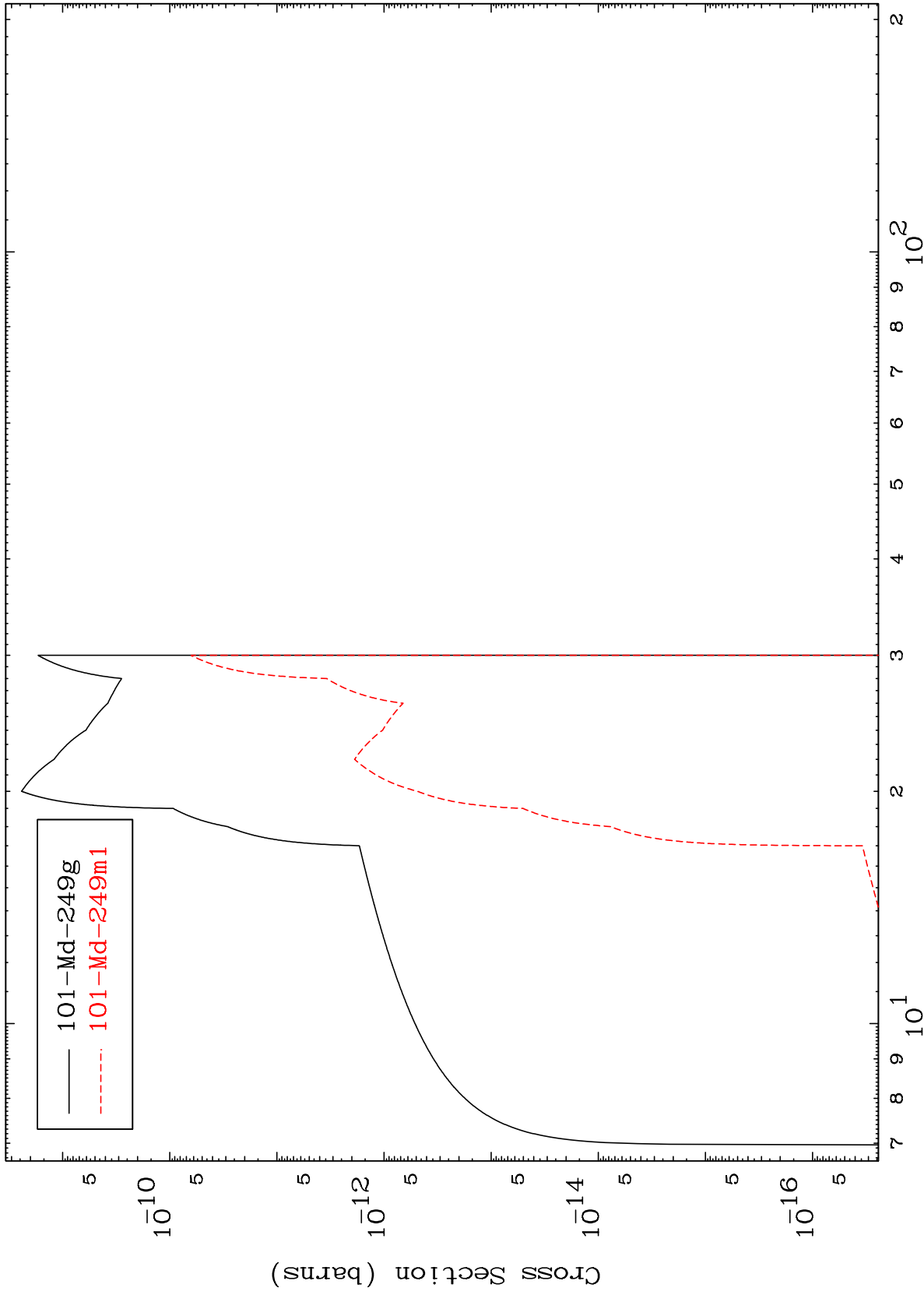
101-Md-250

MAT 9951

101-Md-250

(n,n')  $\alpha$

Radionuclide Production Cross Section



101-Md-250

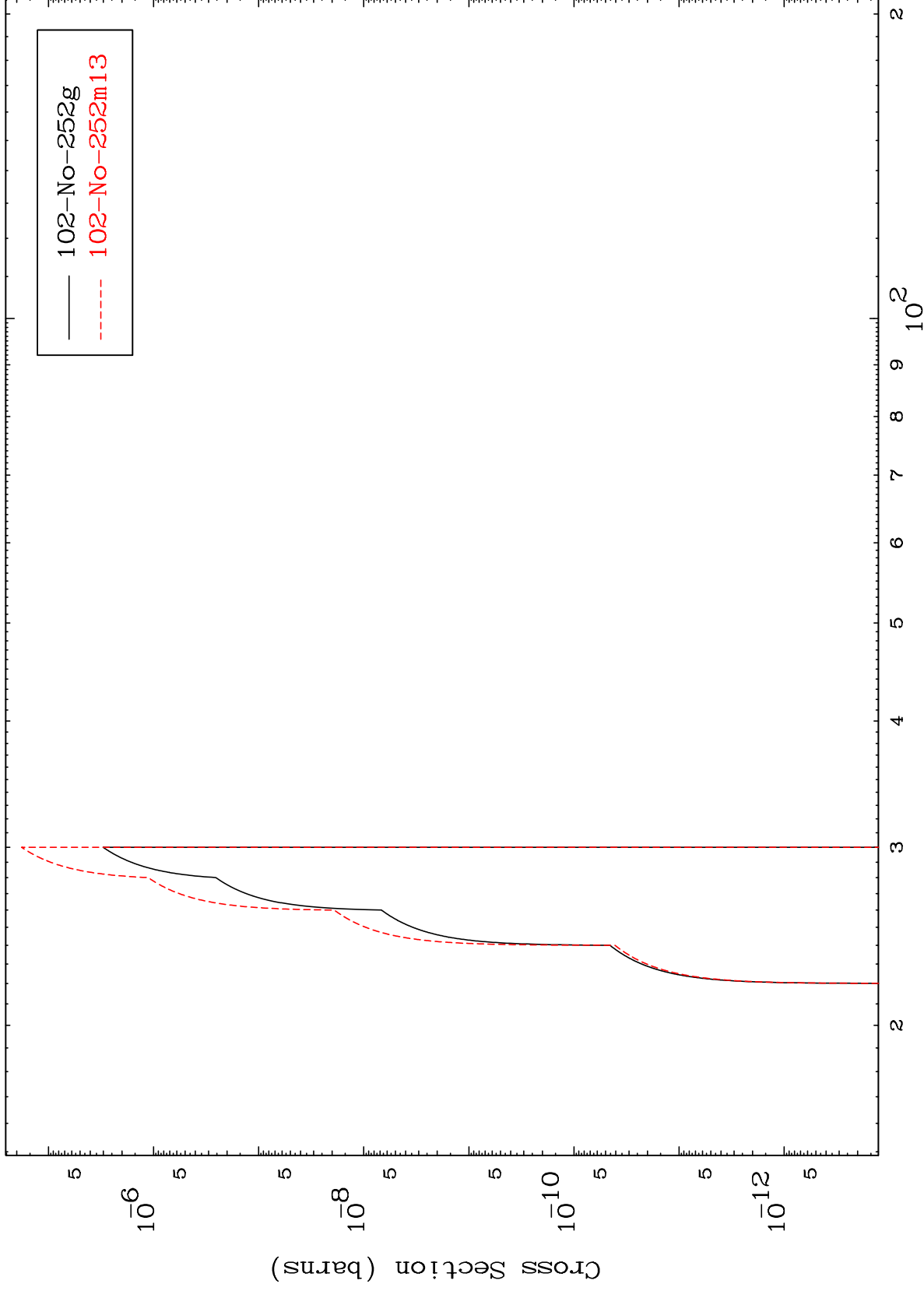
Incident Energy (MeV)

11

MAT 9951

101-Md-250

(n,d)  
Radionuclide Production Cross Section



12

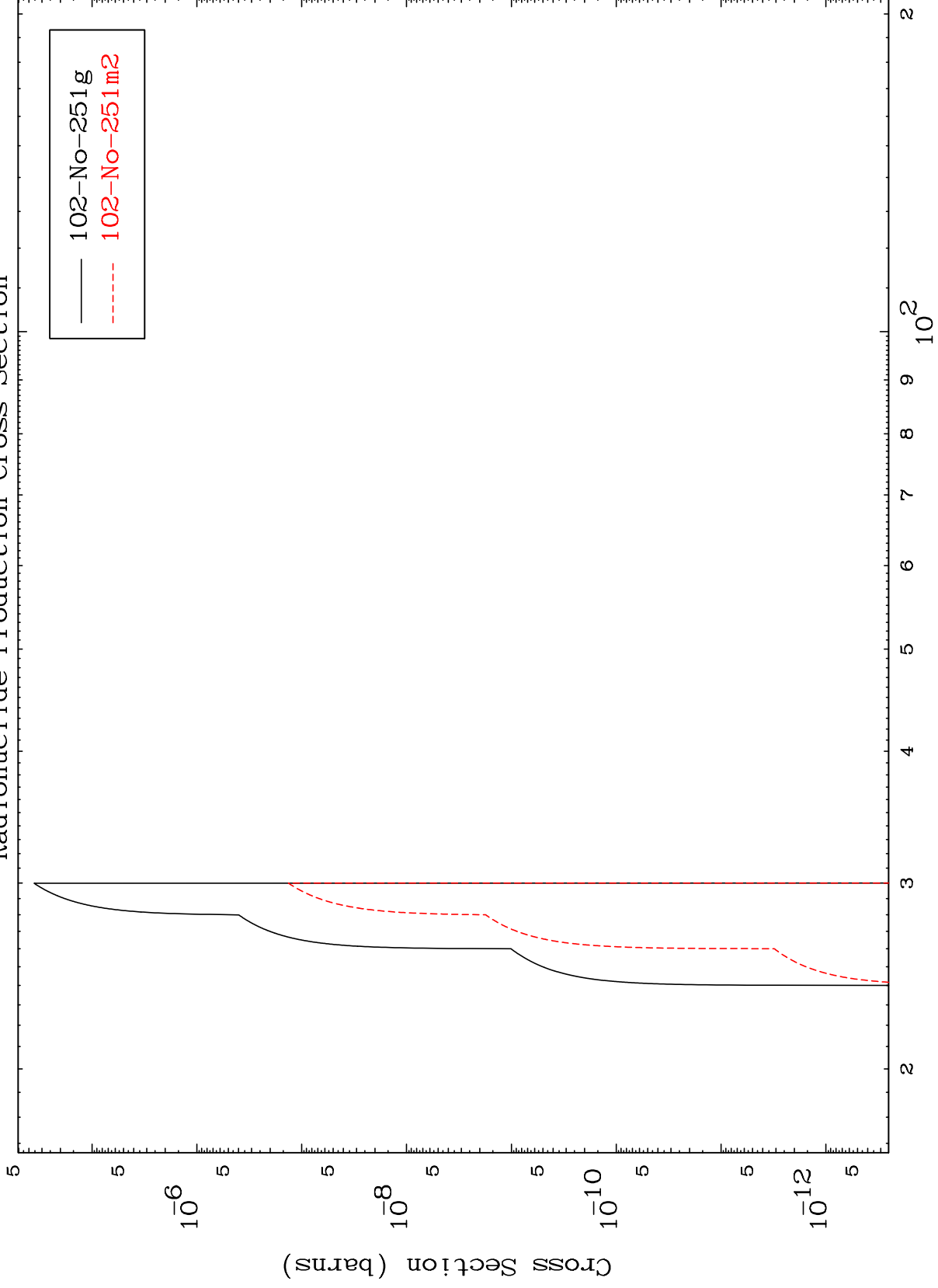
101-Md-250

Incident Energy (MeV)

MAT 9951

101-Md-250

(n,t)  
Radionuclide Production Cross Section



13

101-Md-250

Incident Energy (MeV)