

Program EVALPLOT  
(Version 2021-1)

by

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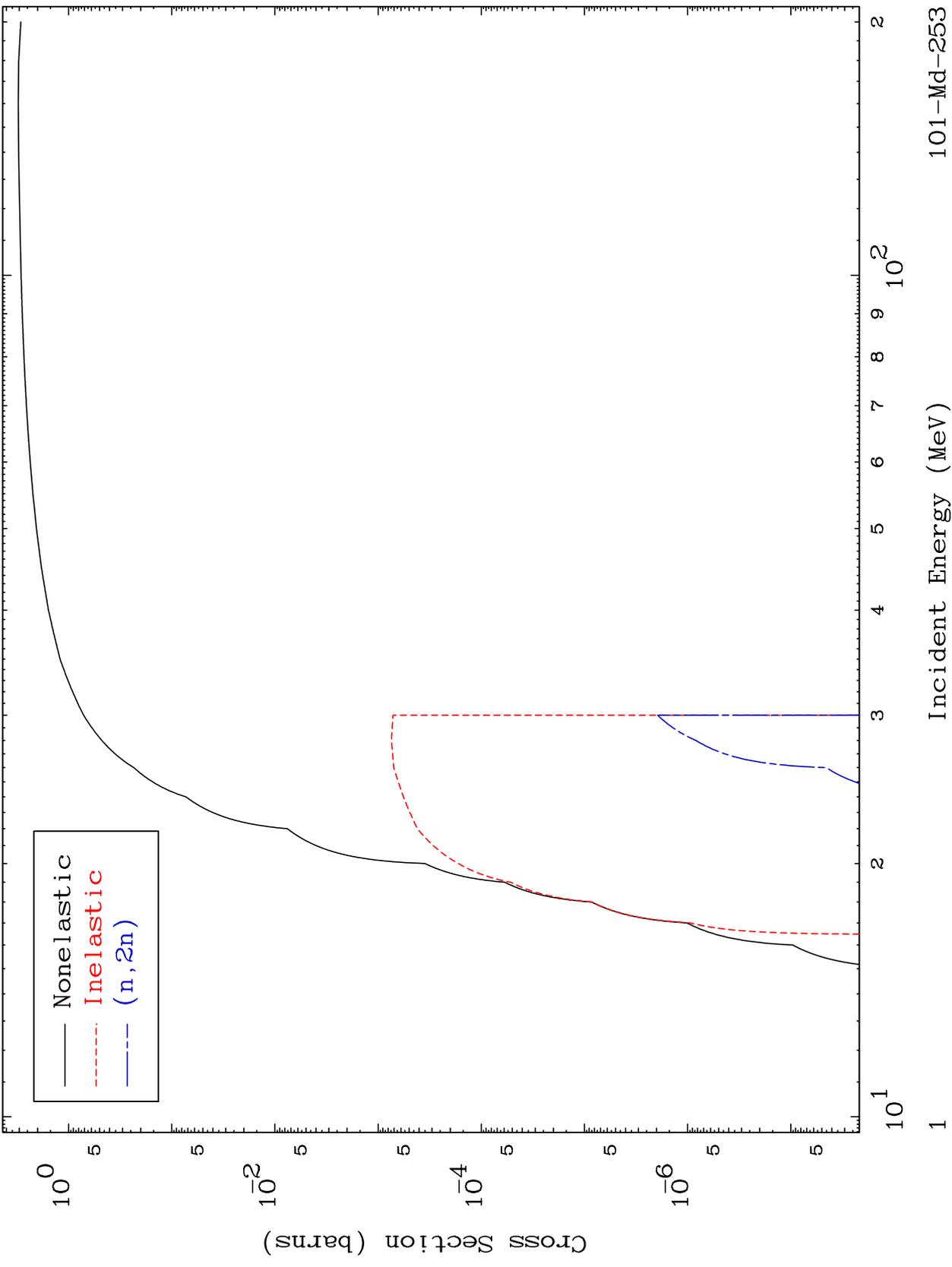
Press Mouse Button to Start

MAT 9954

$\alpha$  Major

101-Md-253

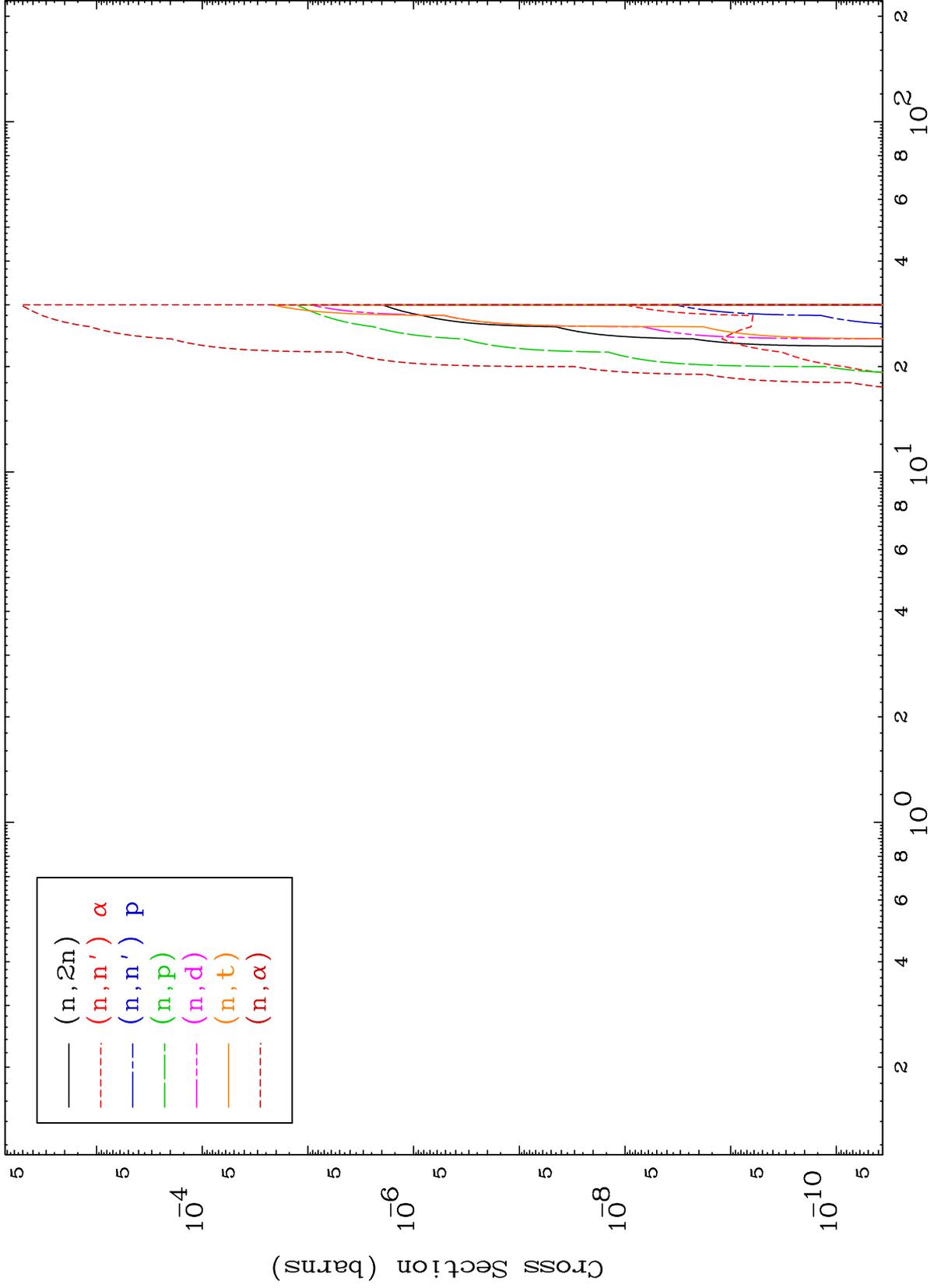
0 Kelvin Cross Sections



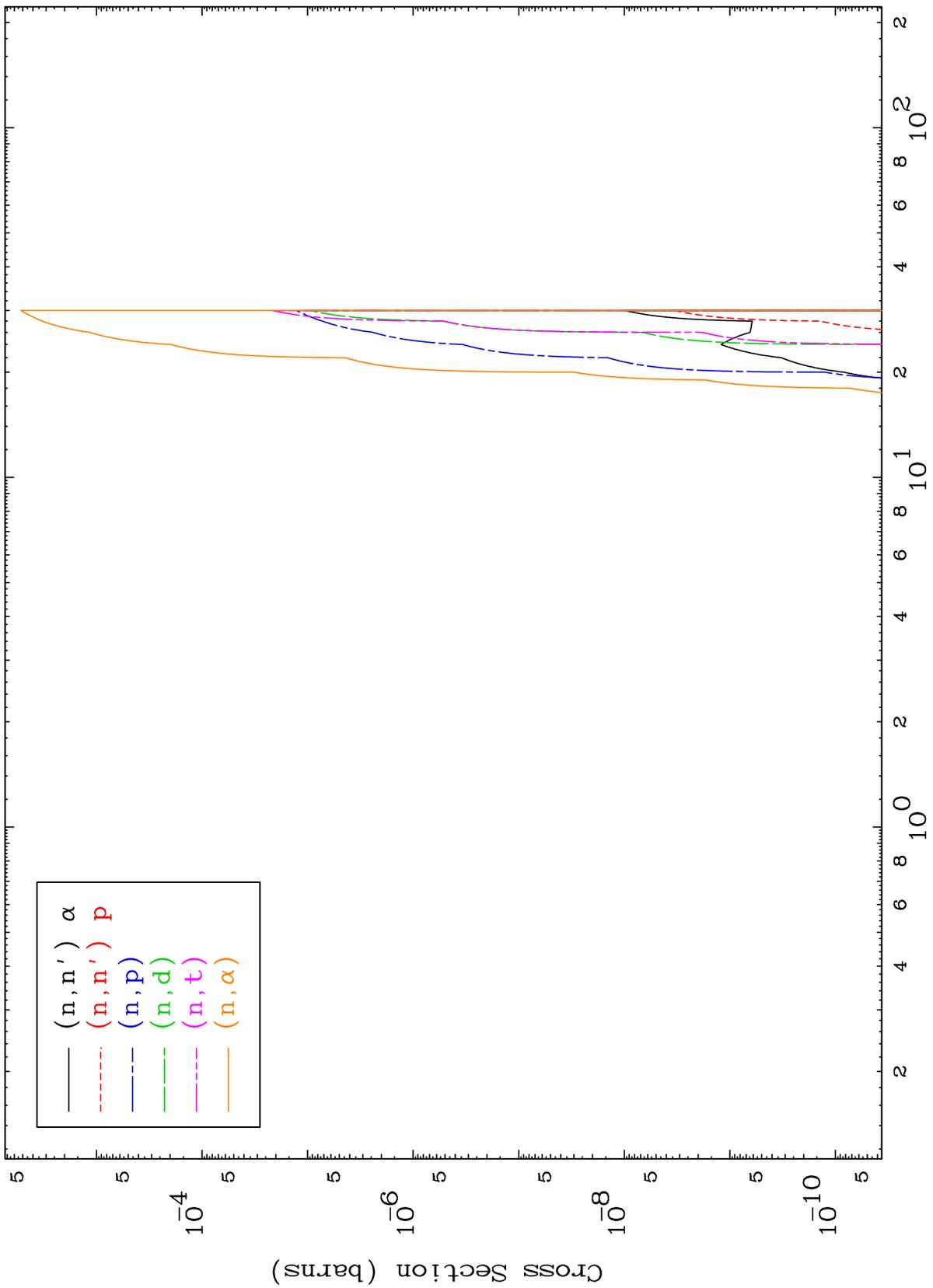
MAT 9954

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

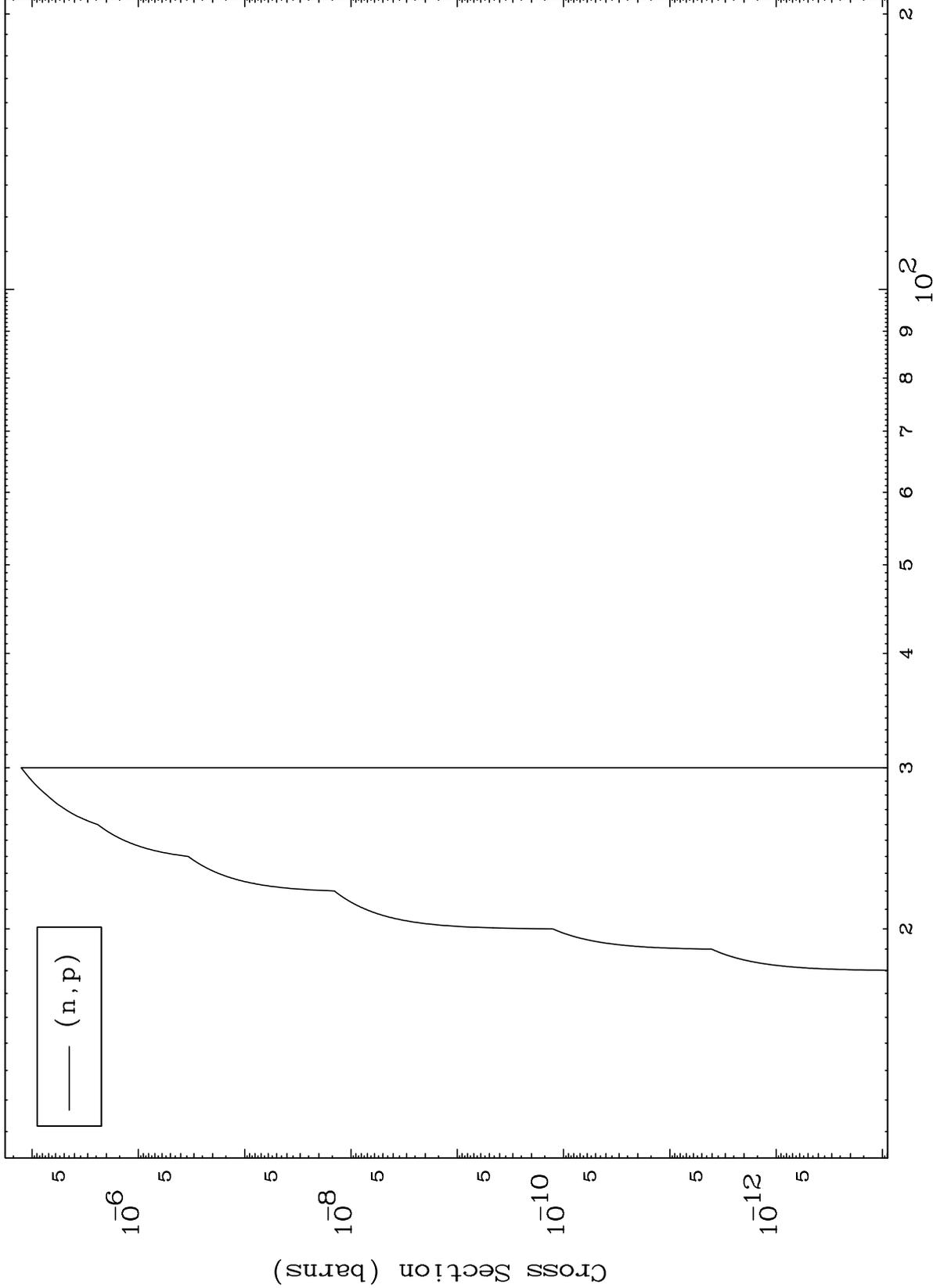
101-Md-253



0 Kelvin Cross Sections



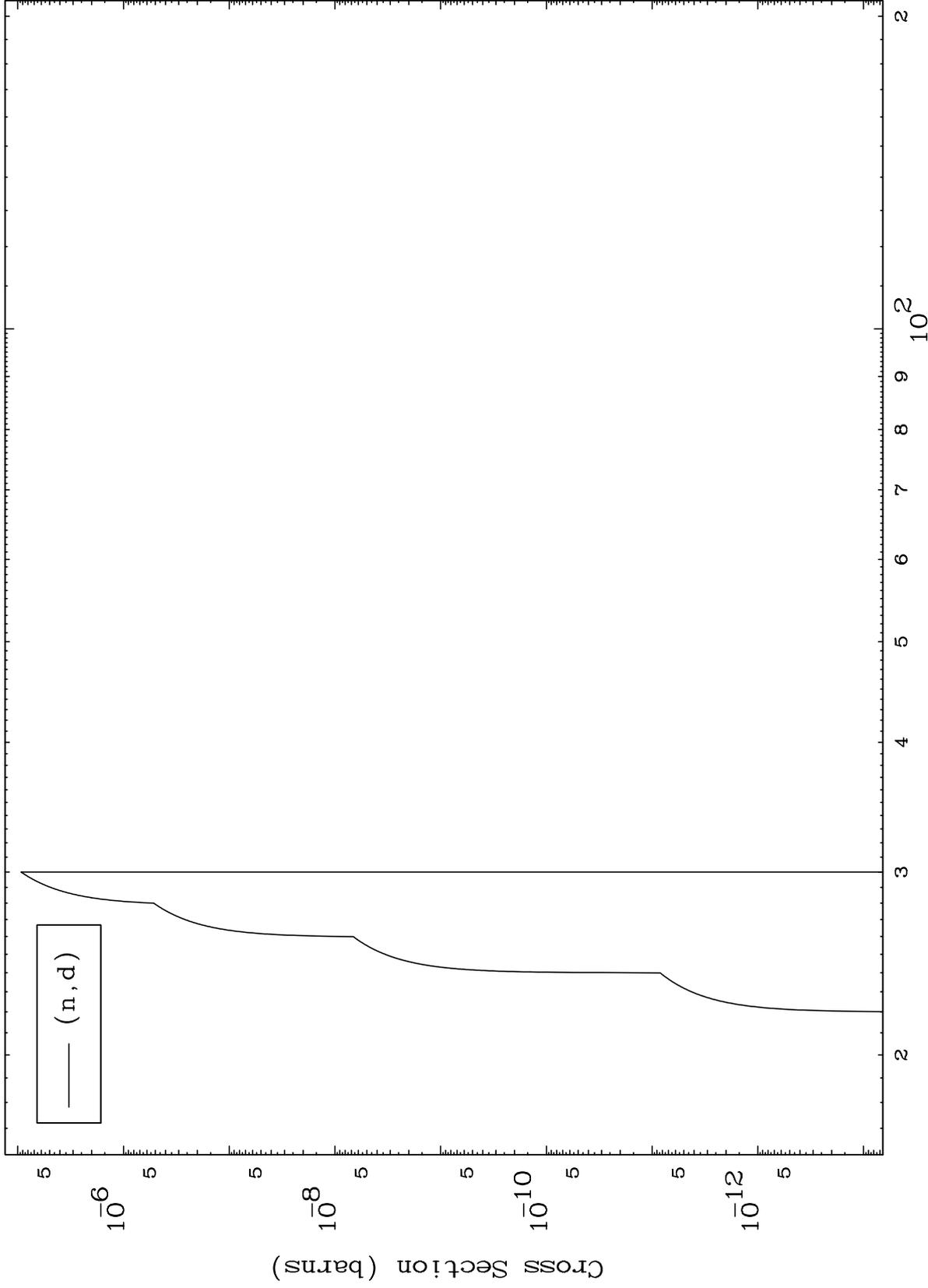
Incident Energy (MeV)



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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

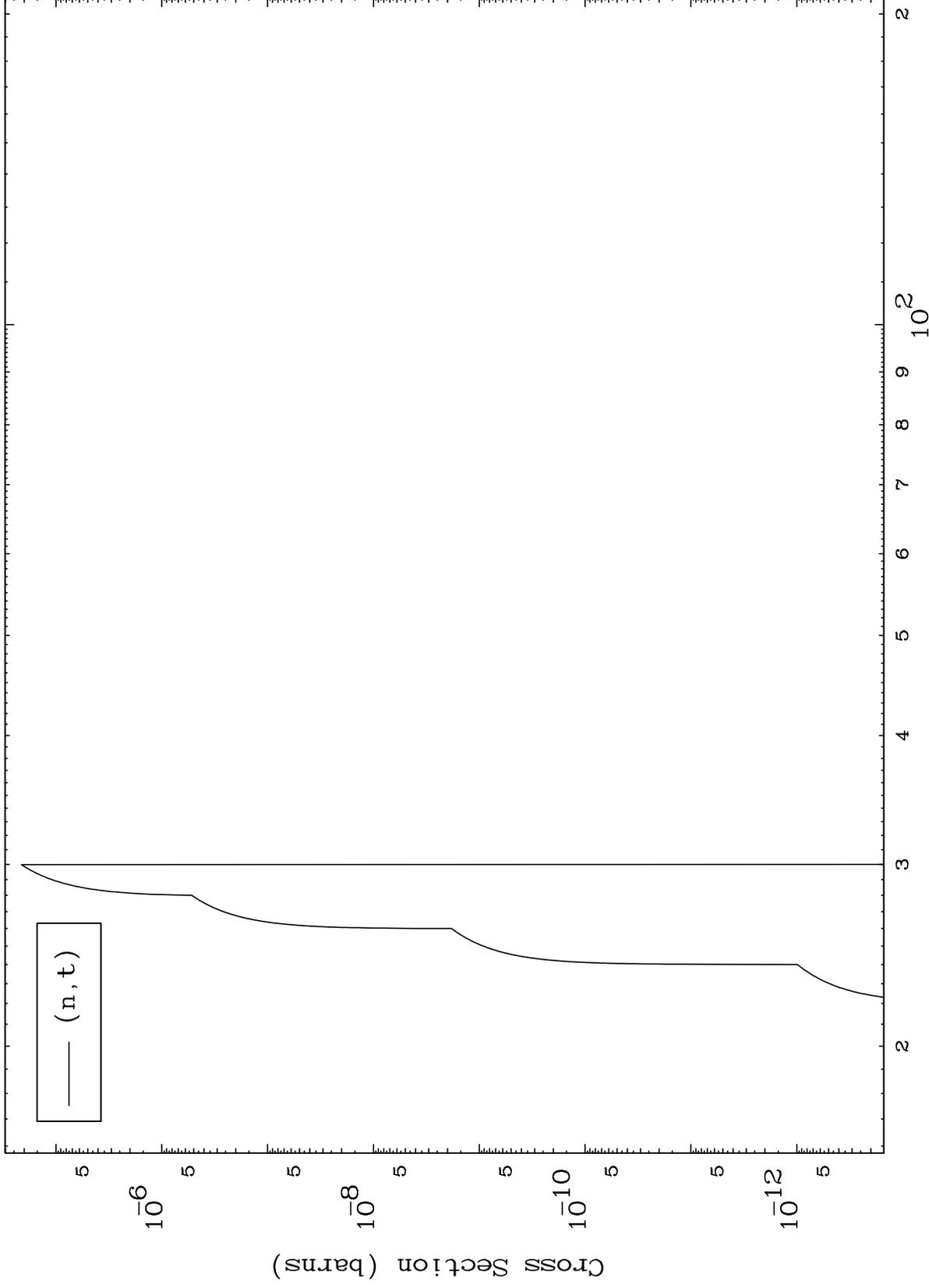
101-Md-253



5

Incident Energy (MeV)

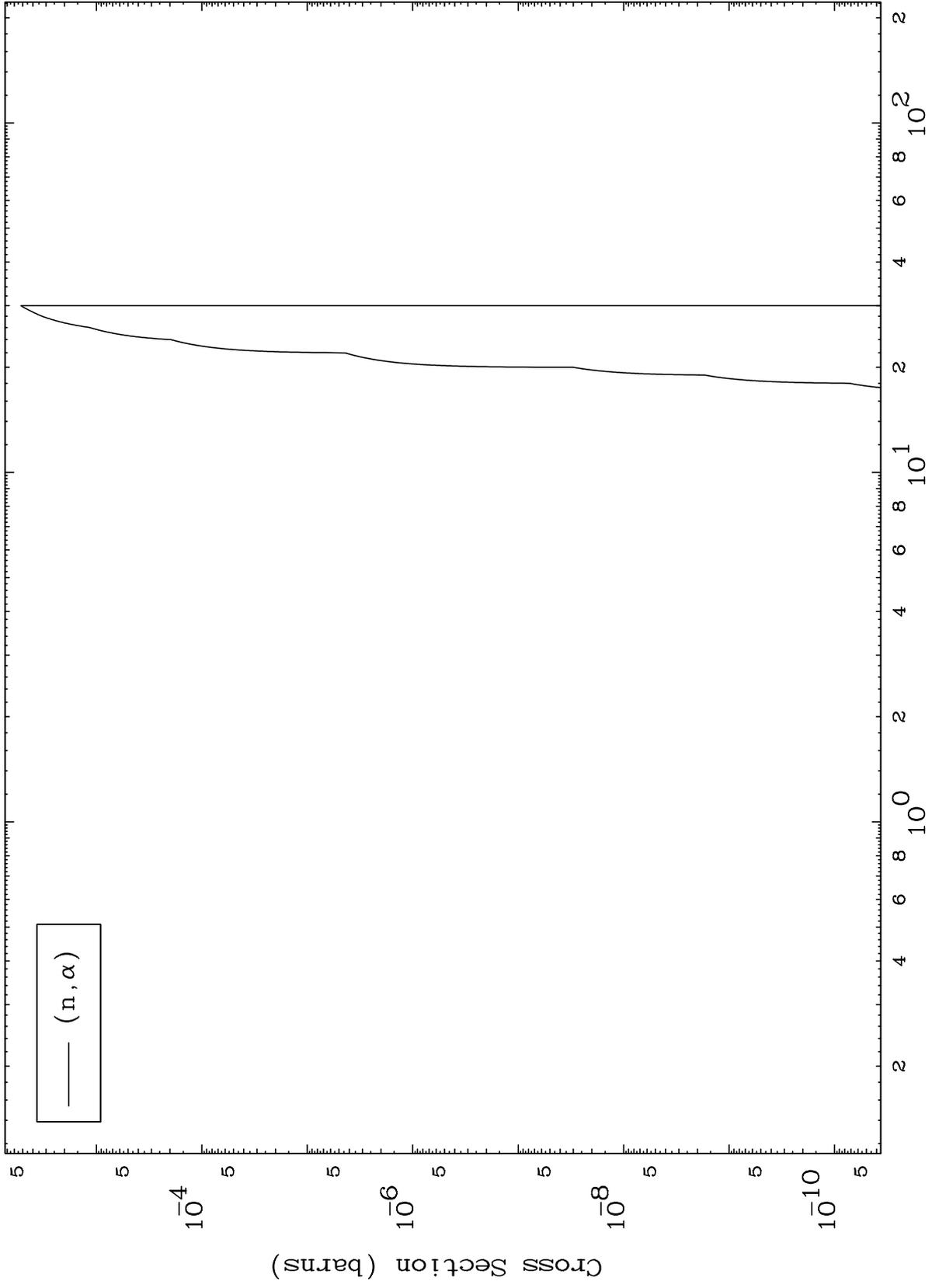
101-Md-253



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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-253

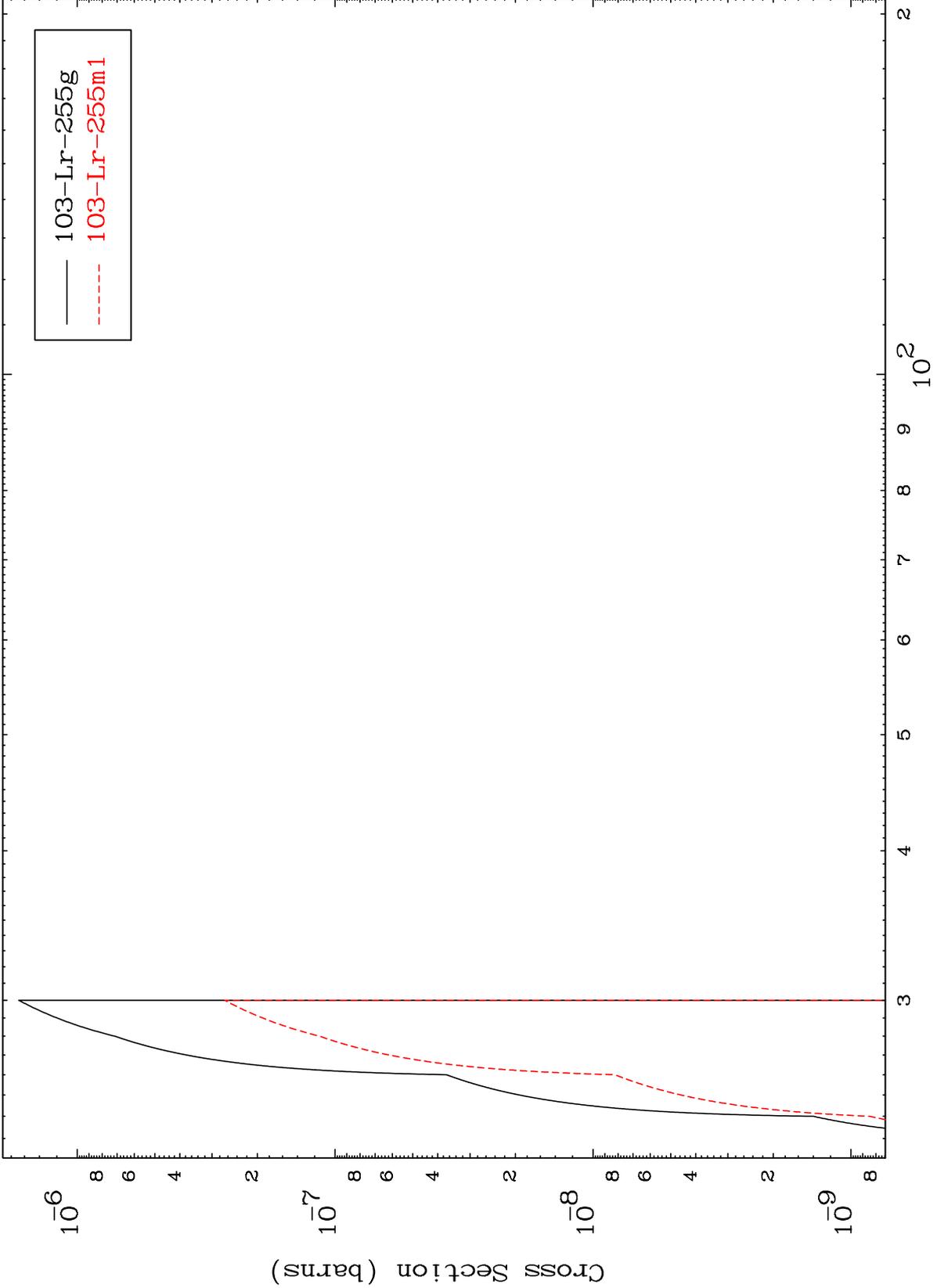


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(n,2n)

101-Md-253

Radionuclide Production Cross Section



103-Lr-255g  
103-Lr-255m1

8

Incident Energy (MeV)

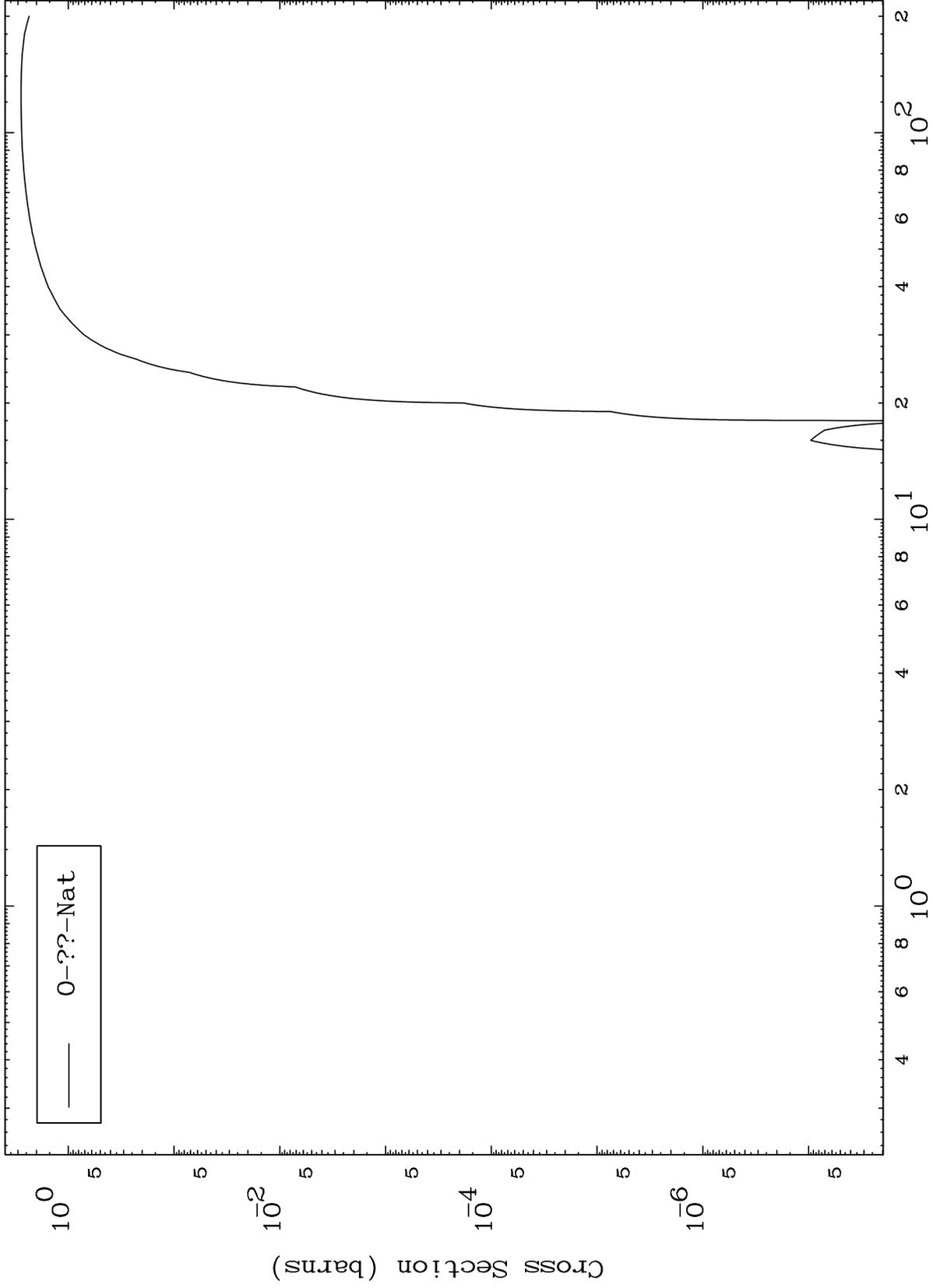
101-Md-253

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Fission

101-Md-253

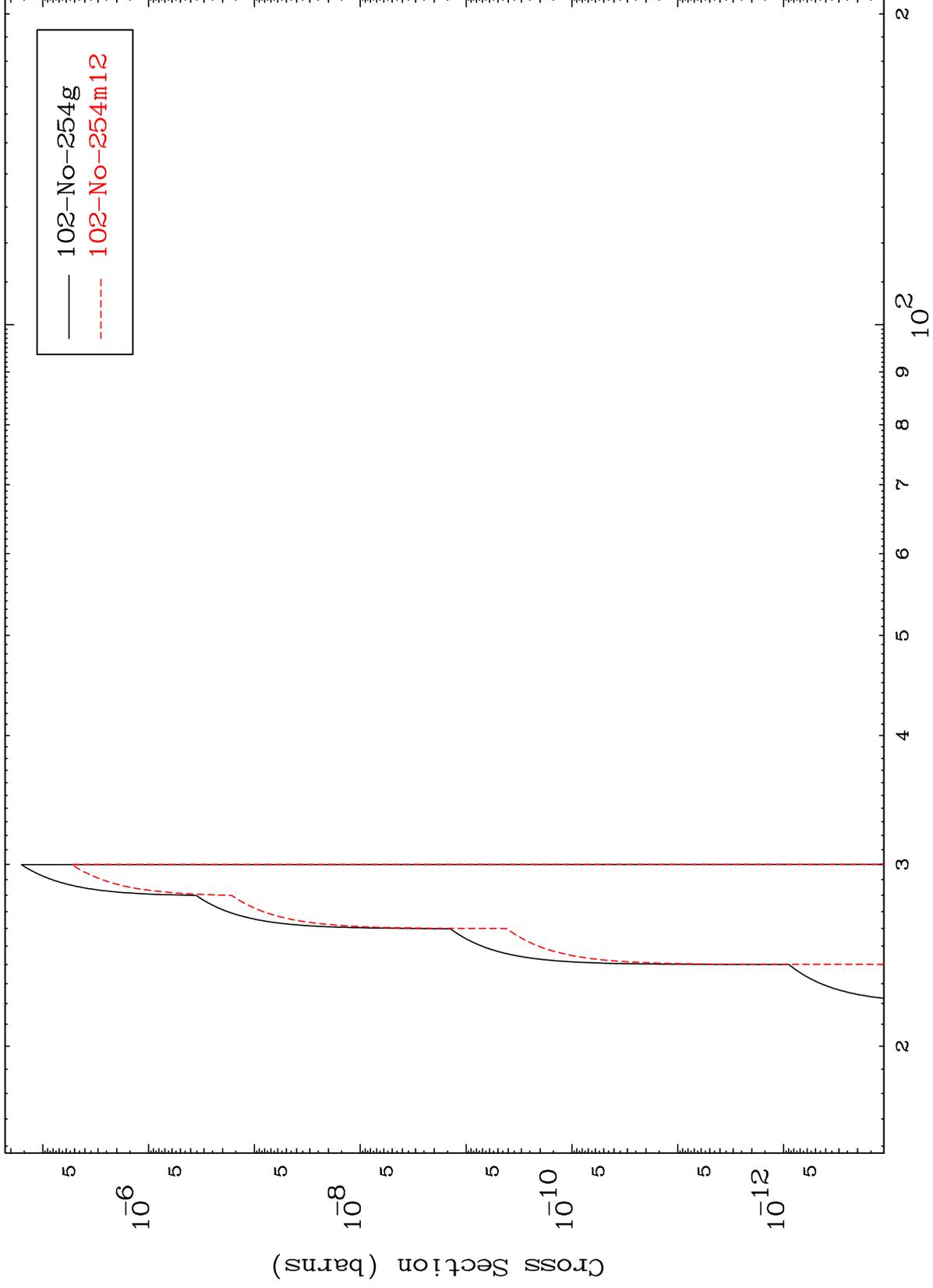
Radionuclide Production Cross Section



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101-Md-253

(n,t)  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

101-Md-253