

Program EVALPLOT  
(Version 2021-1)

by

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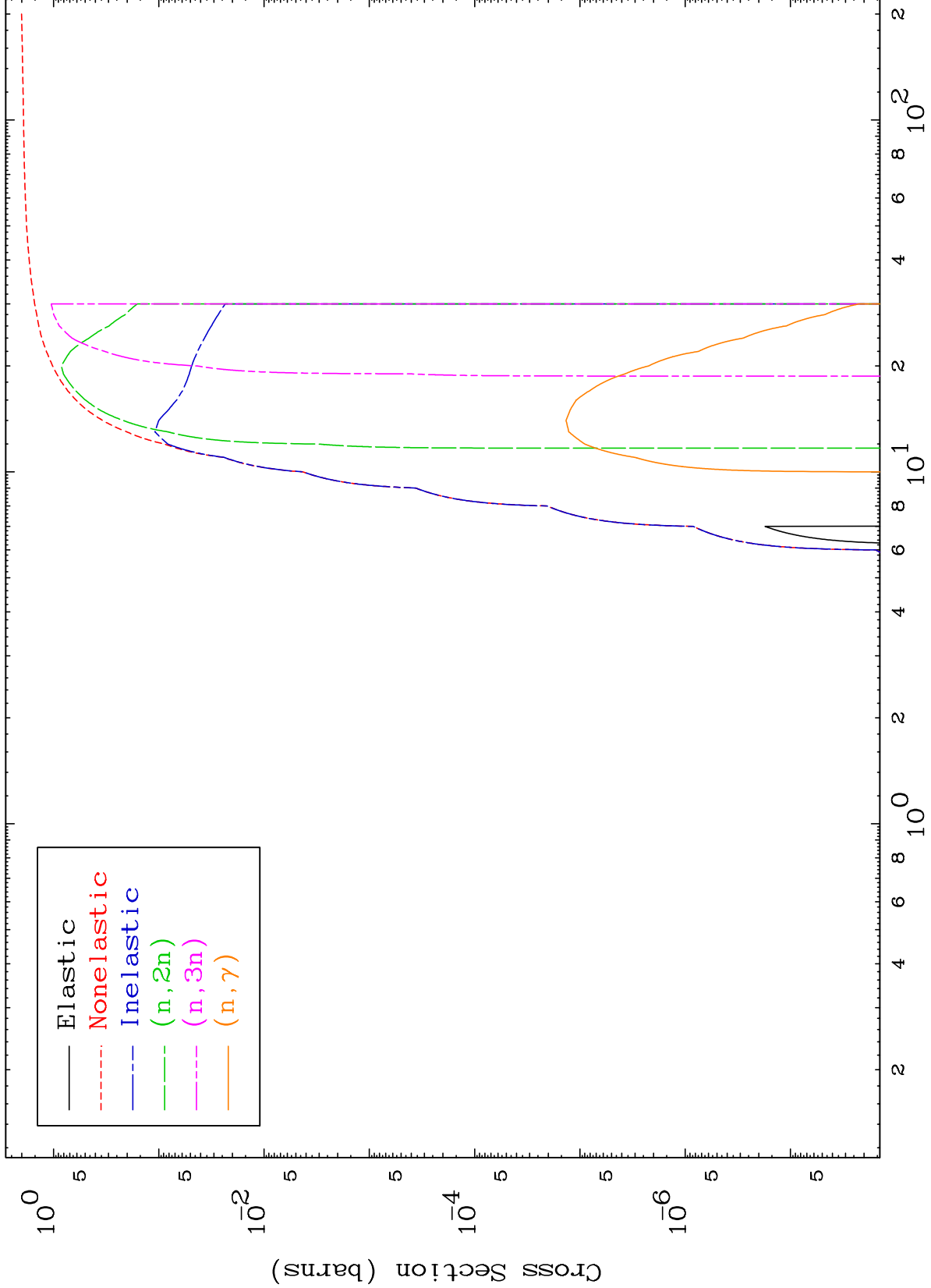
Press Mouse Button to Start

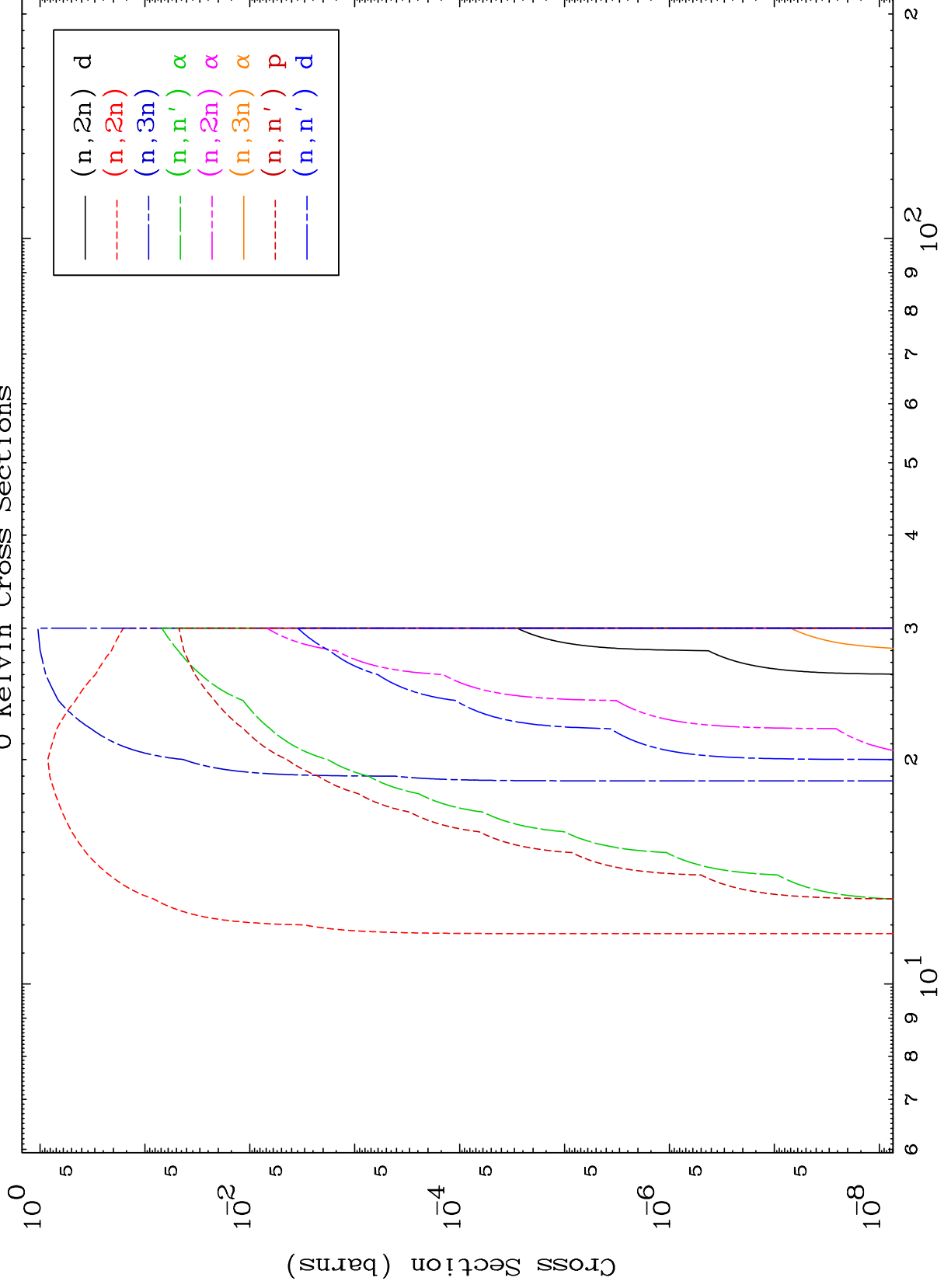
MAT 4140

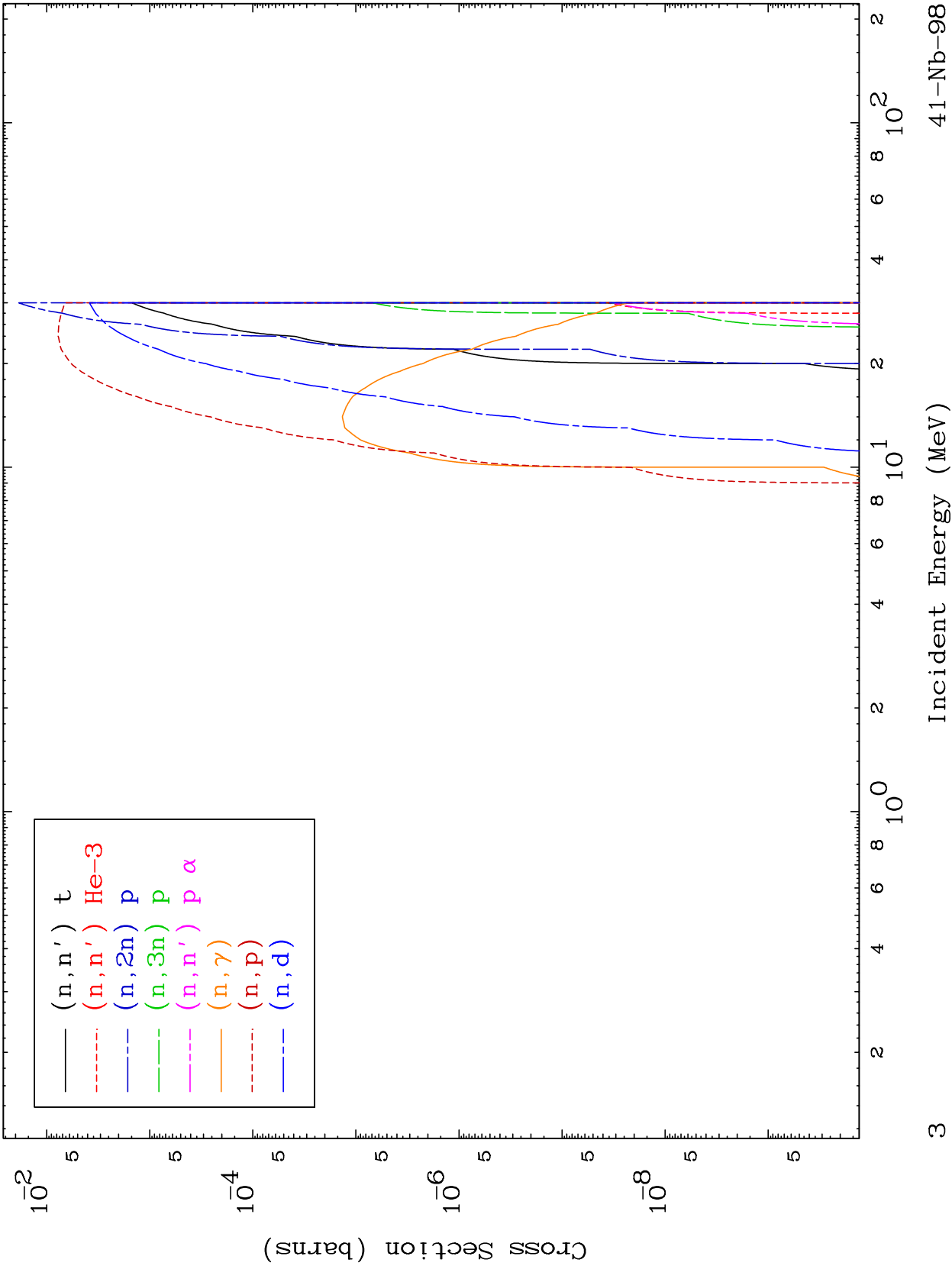
$\alpha$  Major

41-Nb-98

0 Kelvin Cross Sections



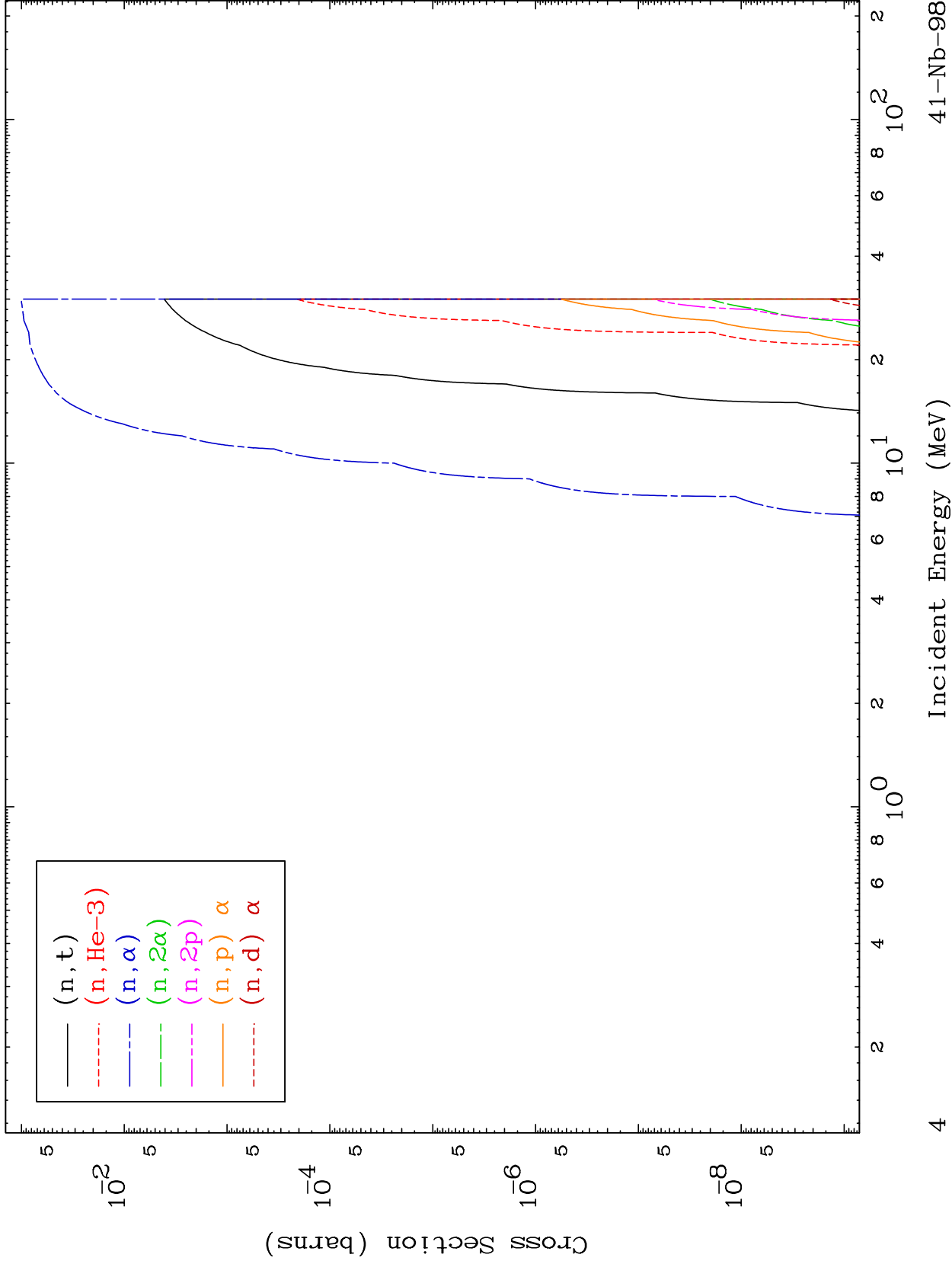


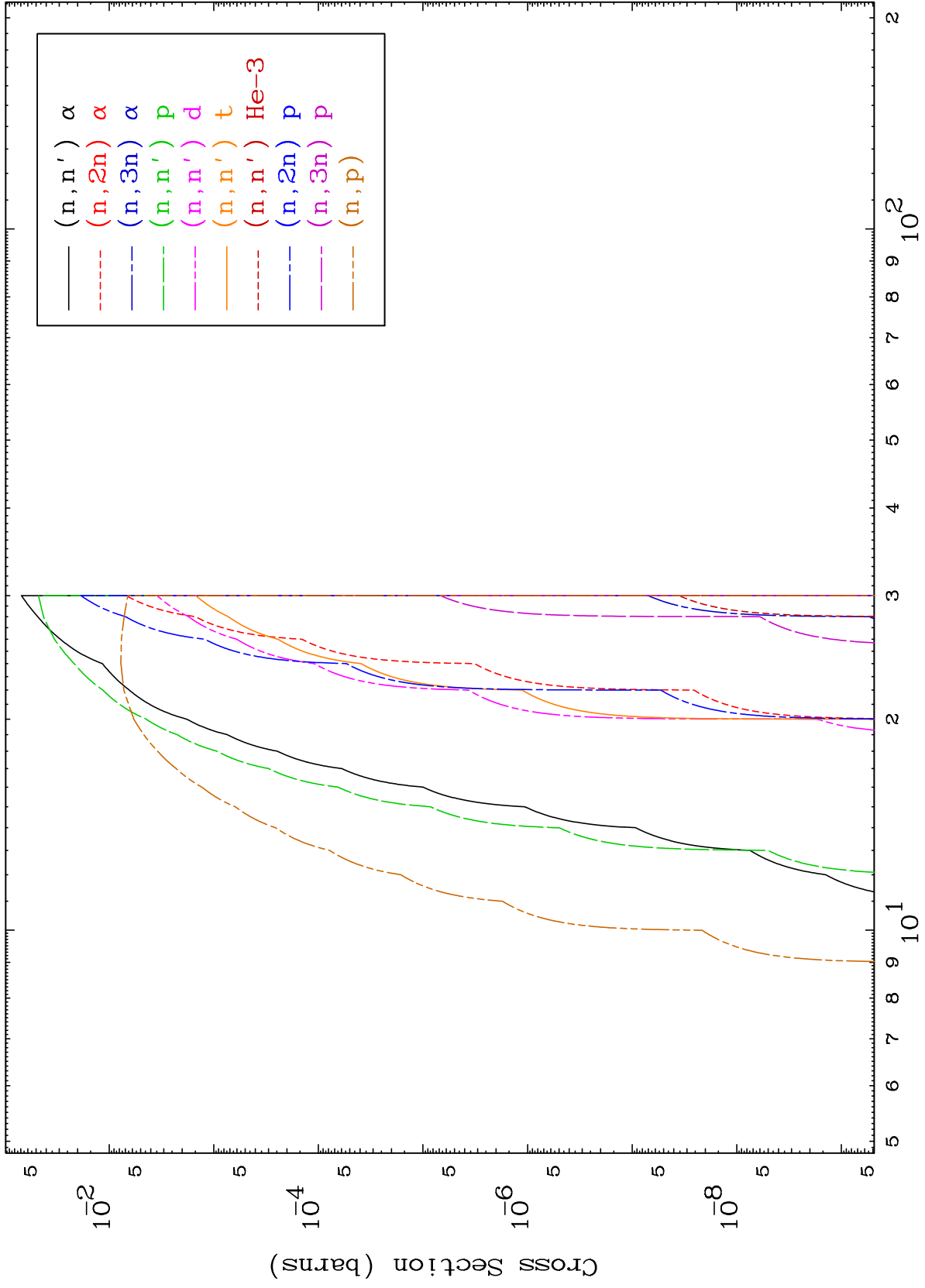


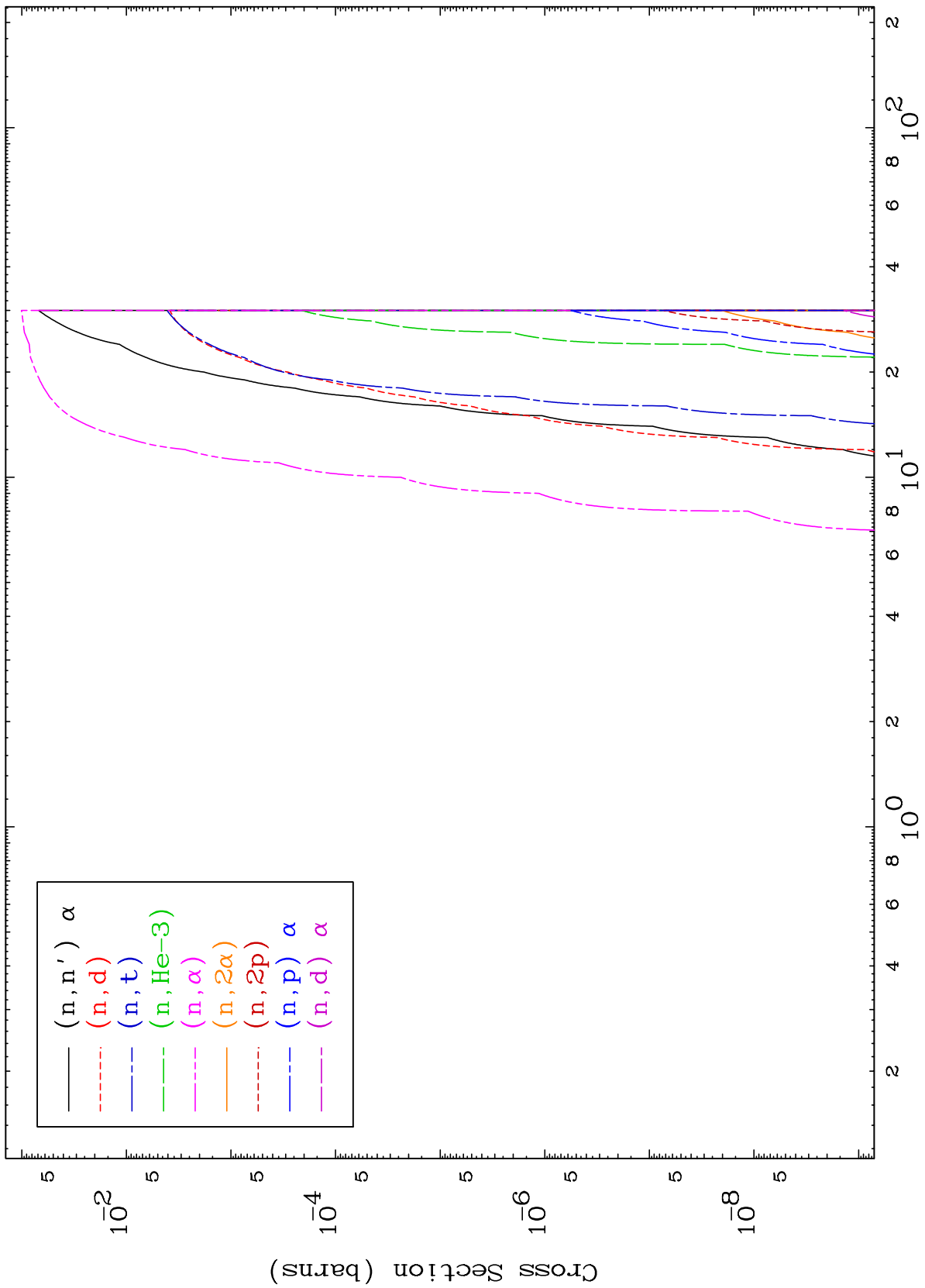
MAT 4140

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

41-Nb-98



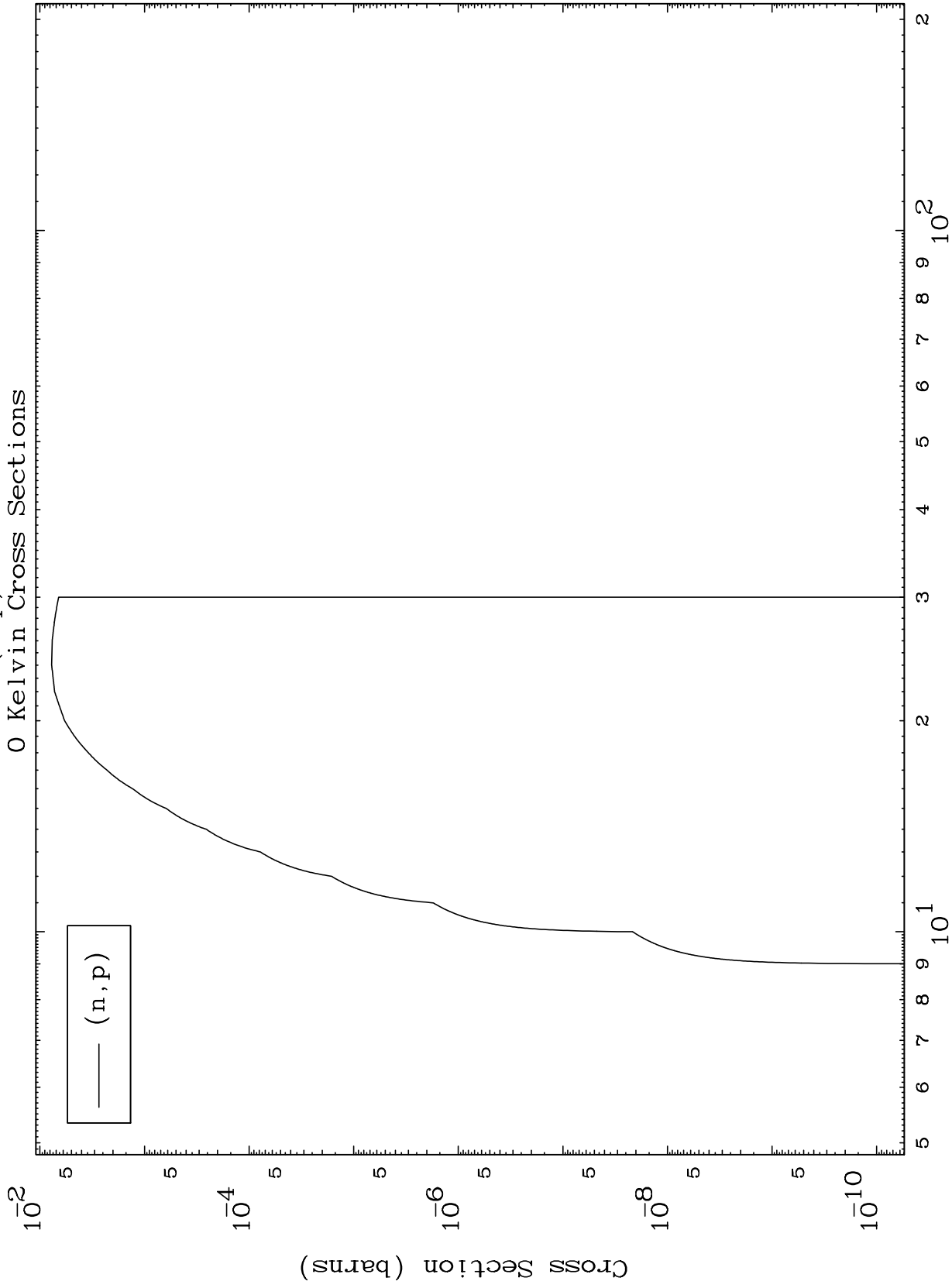




MAT 4140

( $\alpha, p$ ) Levels

41-Nb-98



7

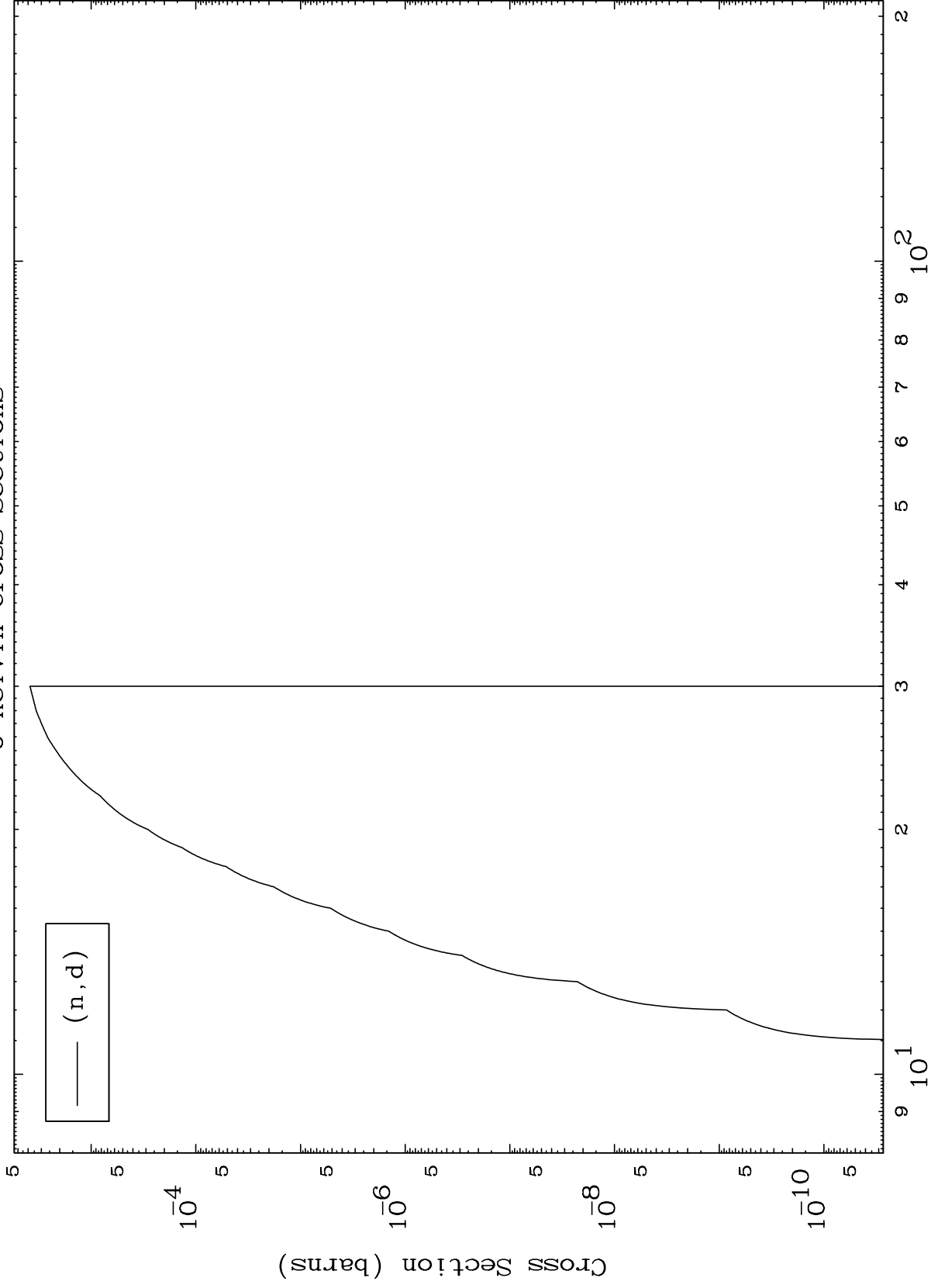
Incident Energy (MeV)

41-Nb-98

MAT 4140

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

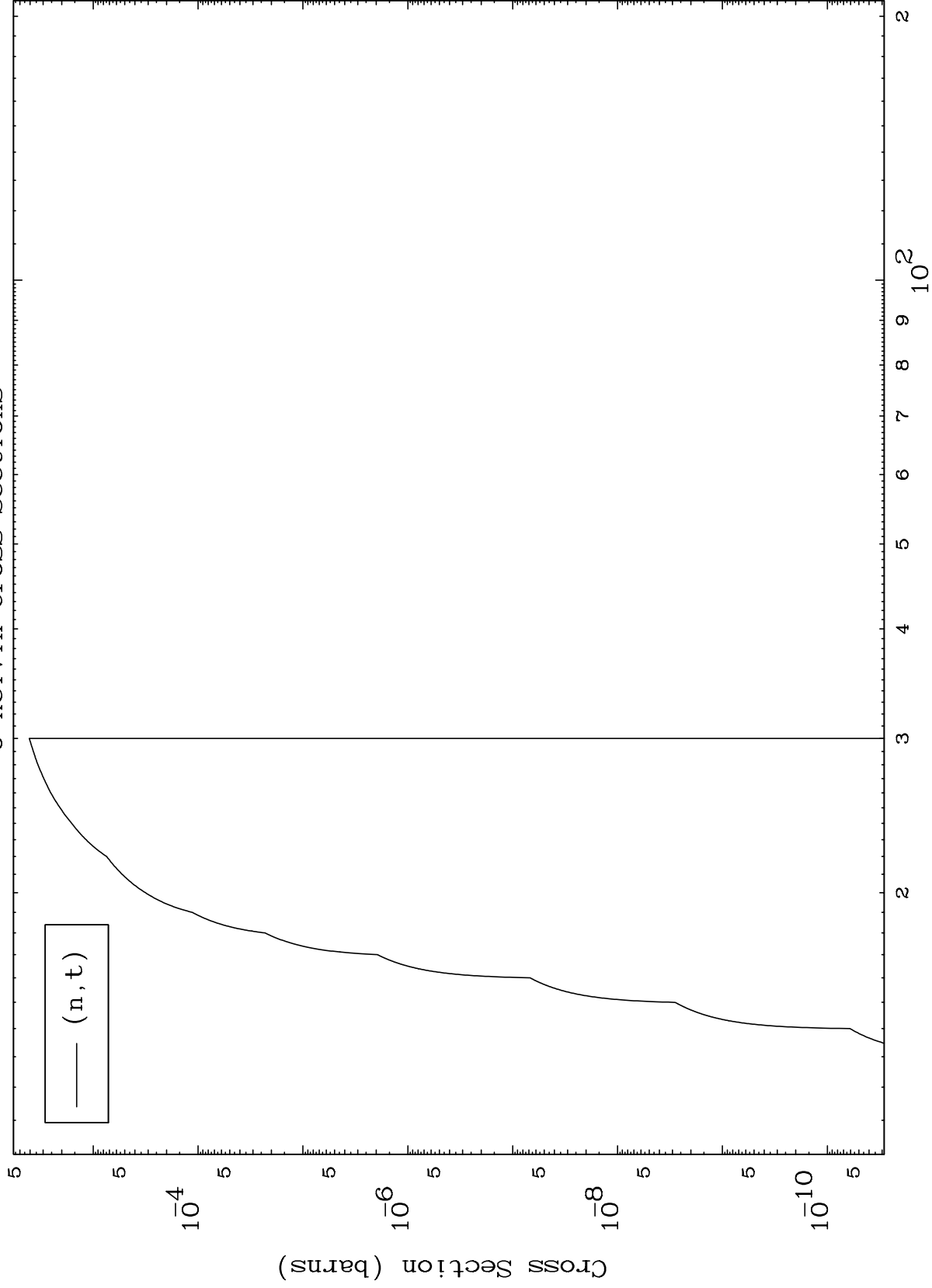
41-Nb-98



8

Incident Energy (MeV)

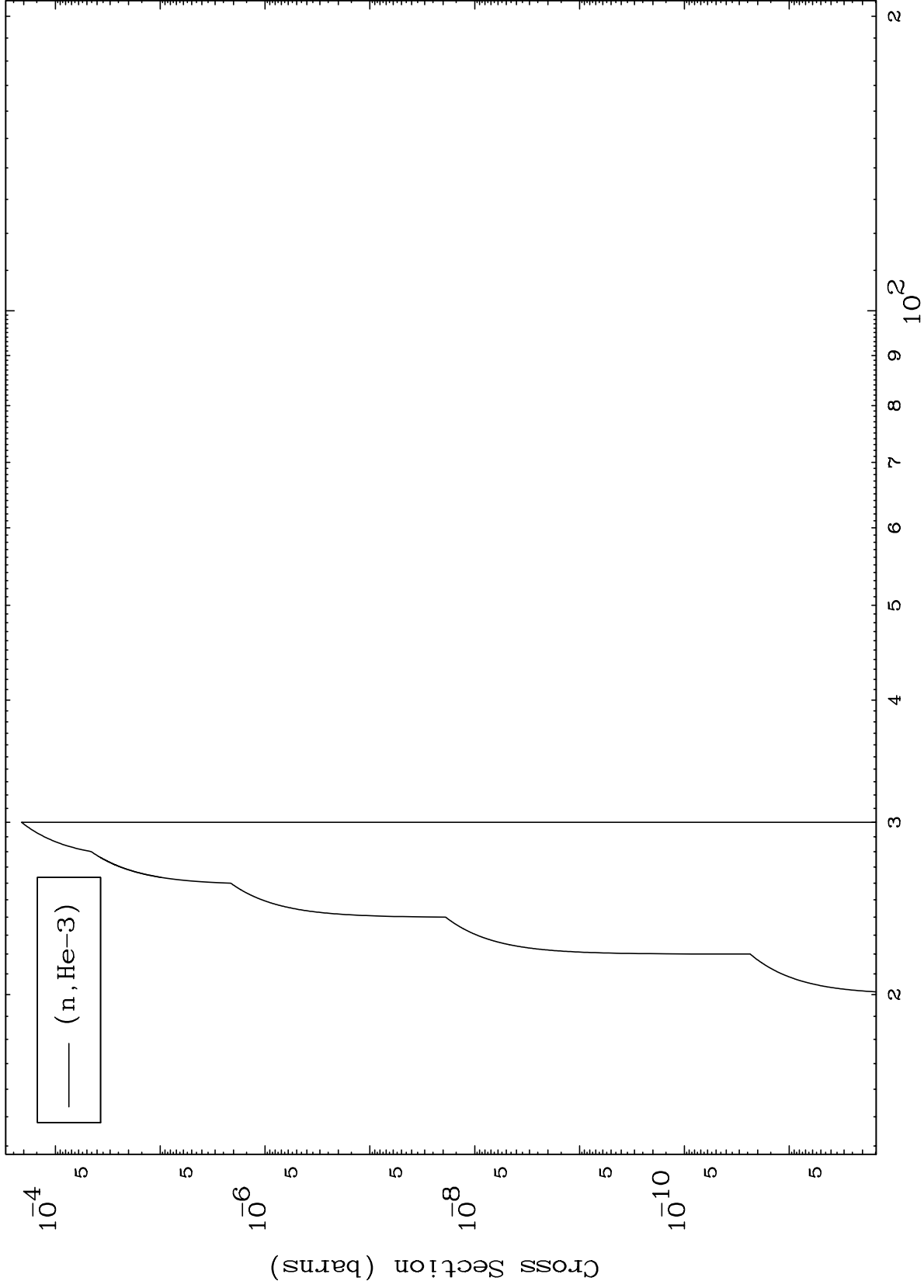
41-Nb-98



MAT 4140

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

41-Nb-98



10

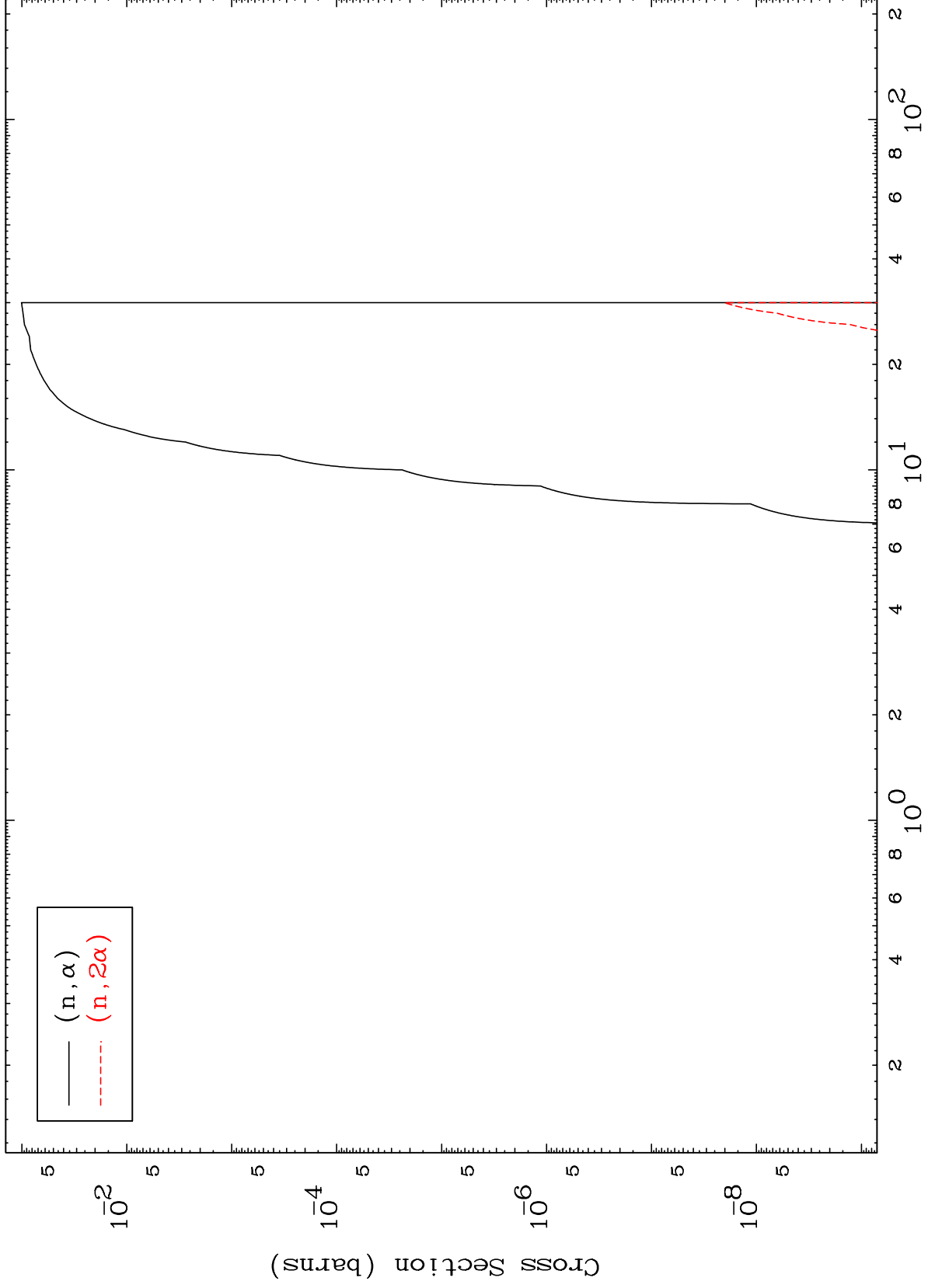
Incident Energy (MeV)

41-Nb-98

MAT 4140

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

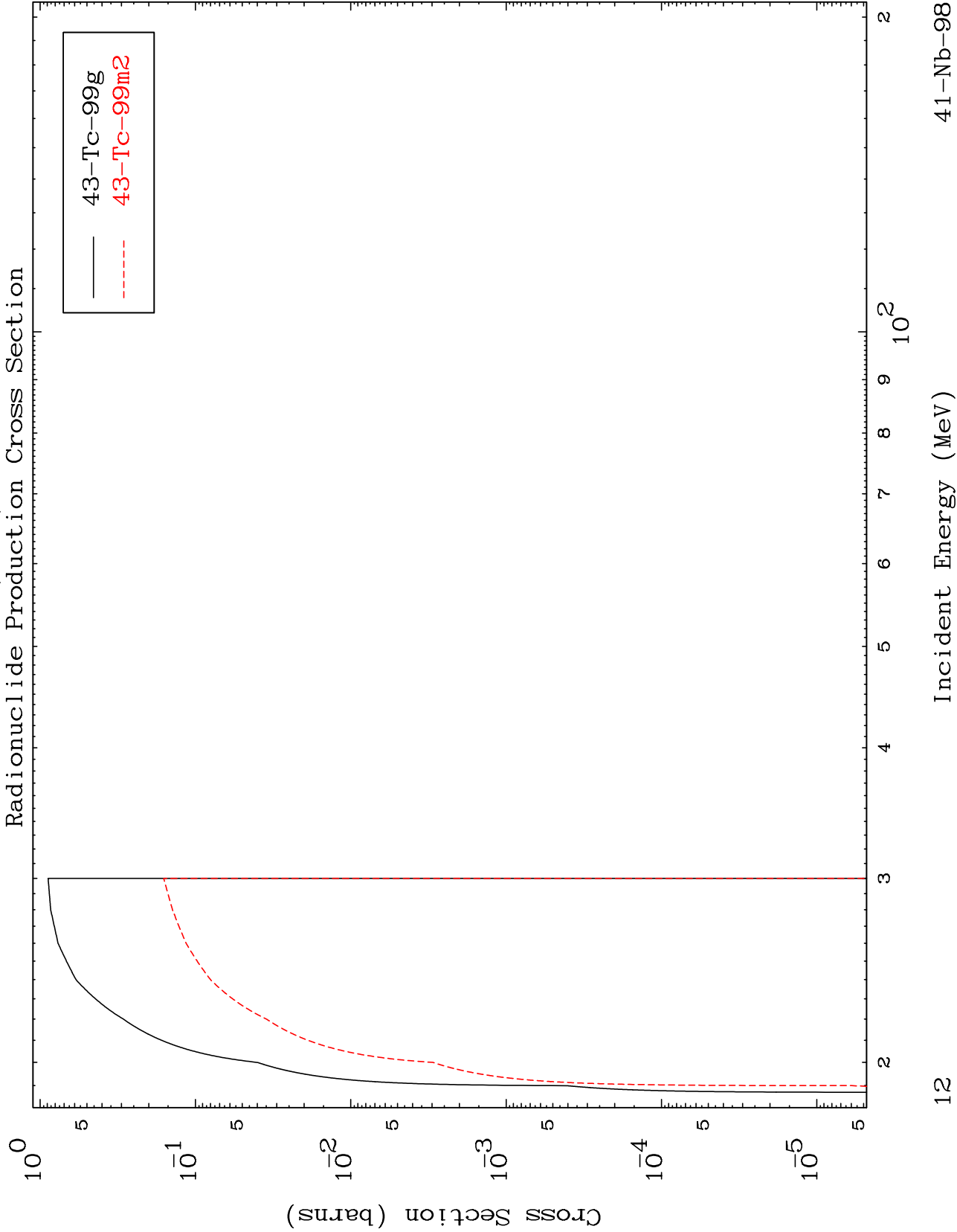
41-Nb-98



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(n,3n)

41-Nb-98

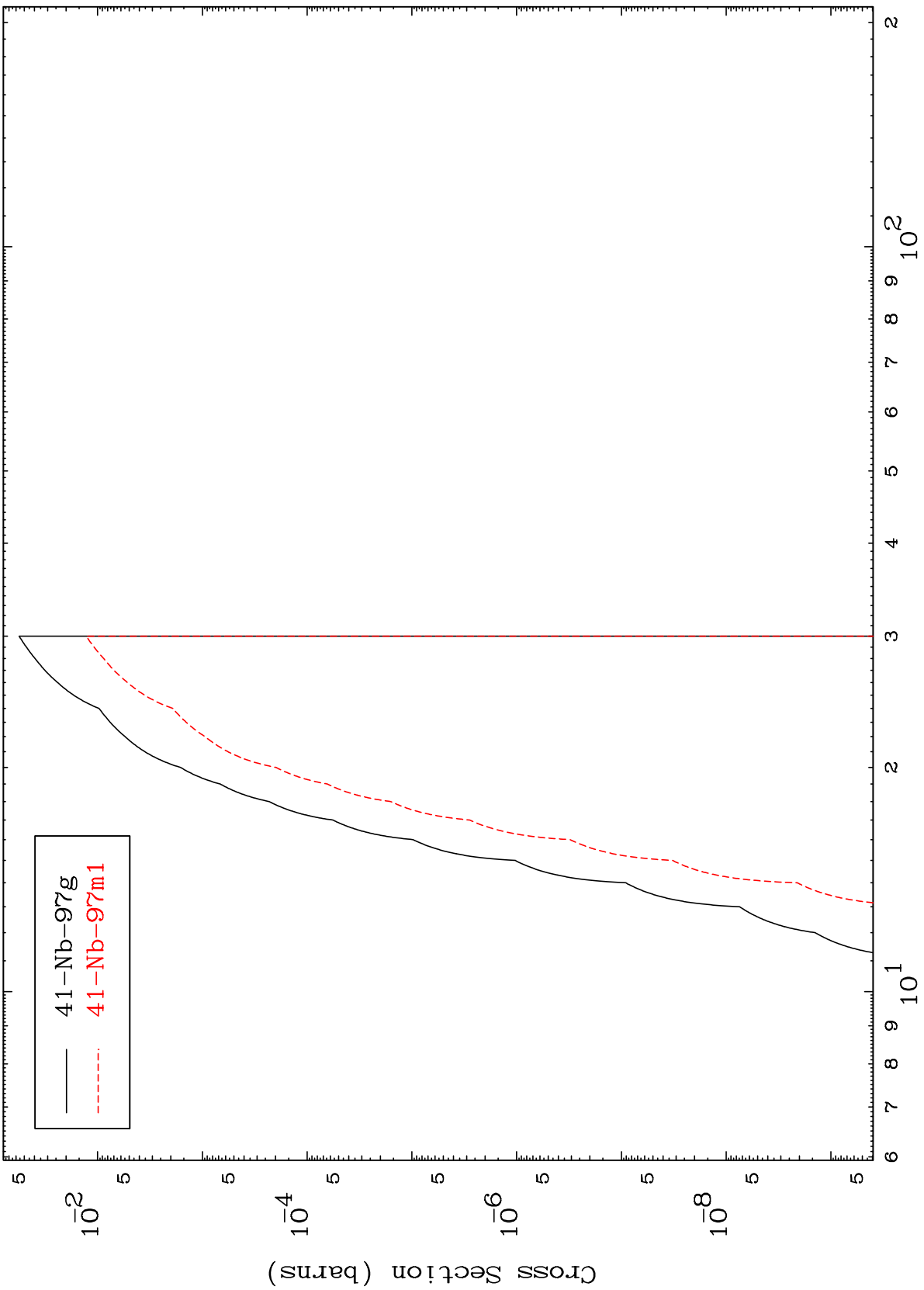


MAT 4140

$(n, n') \alpha$

41-Nb-98

Radionuclide Production Cross Section



13

Incident Energy (MeV)

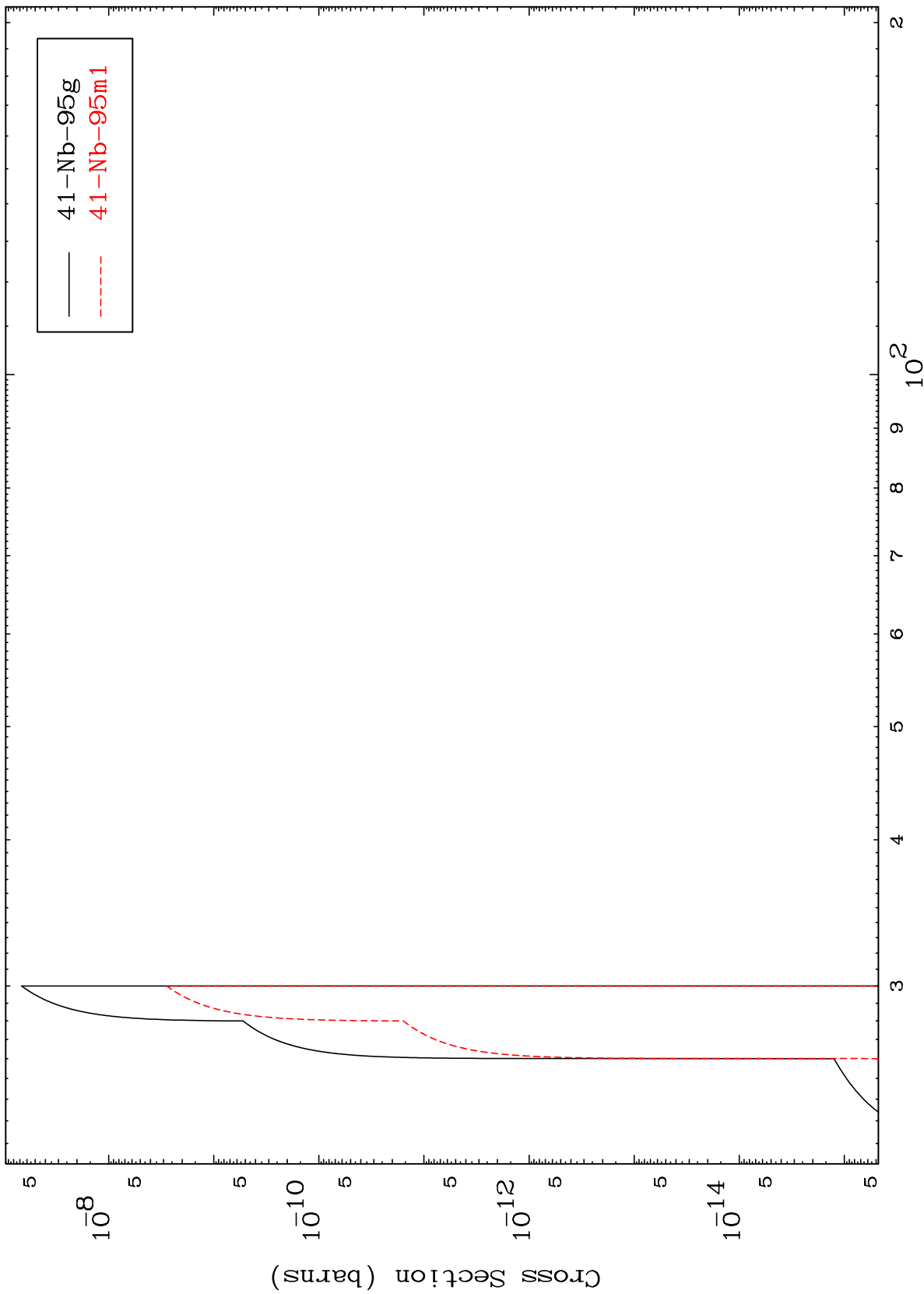
41-Nb-98

MAT 4140

(n,3n)  $\alpha$

41-Nb-98

Radionuclide Production Cross Section



14

Incident Energy (MeV)

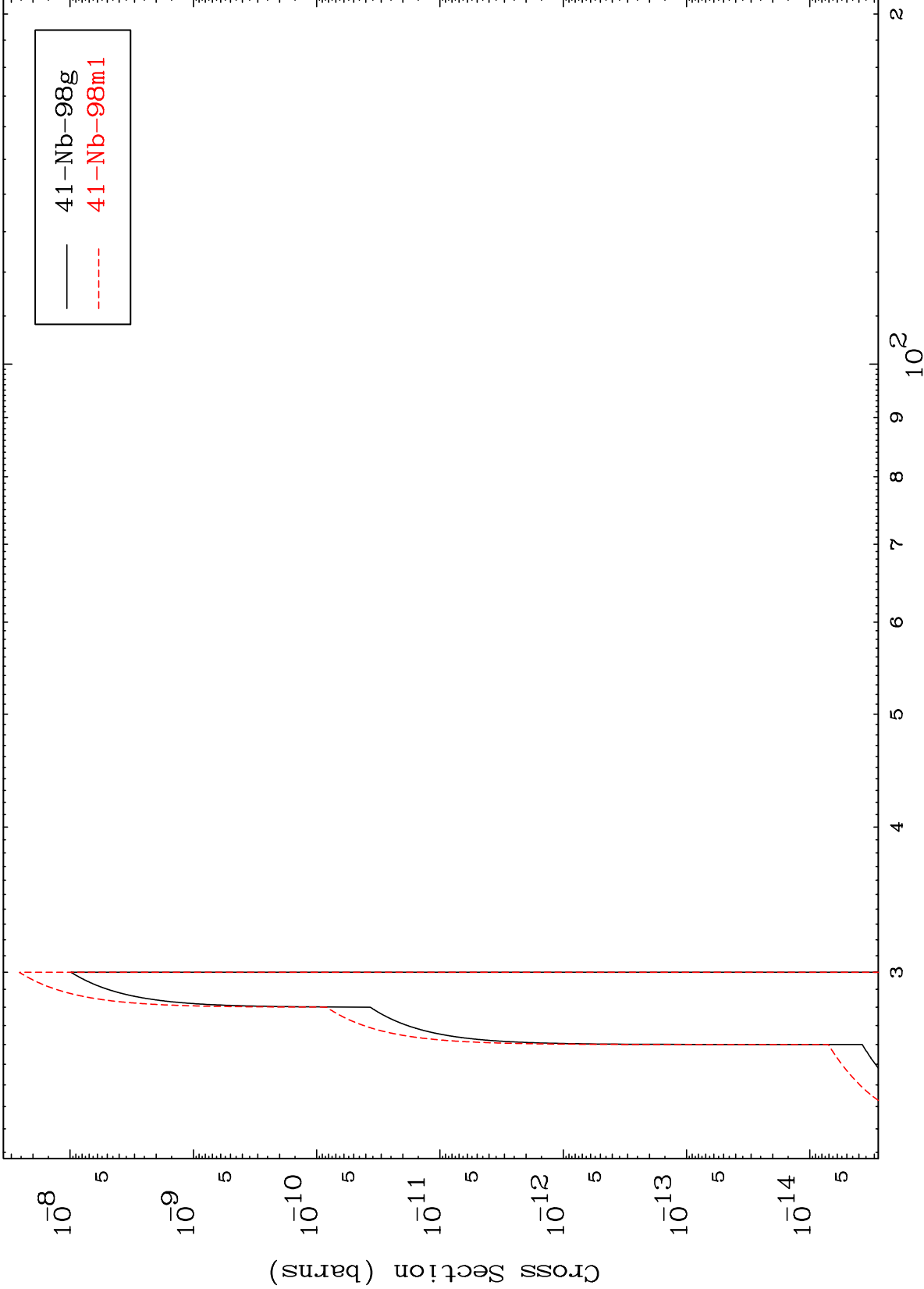
41-Nb-98

MAT 4140

(n,n') He-3

41-Nb-98

Radionuclide Production Cross Section



41-Nb-98g  
41-Nb-98m1

15

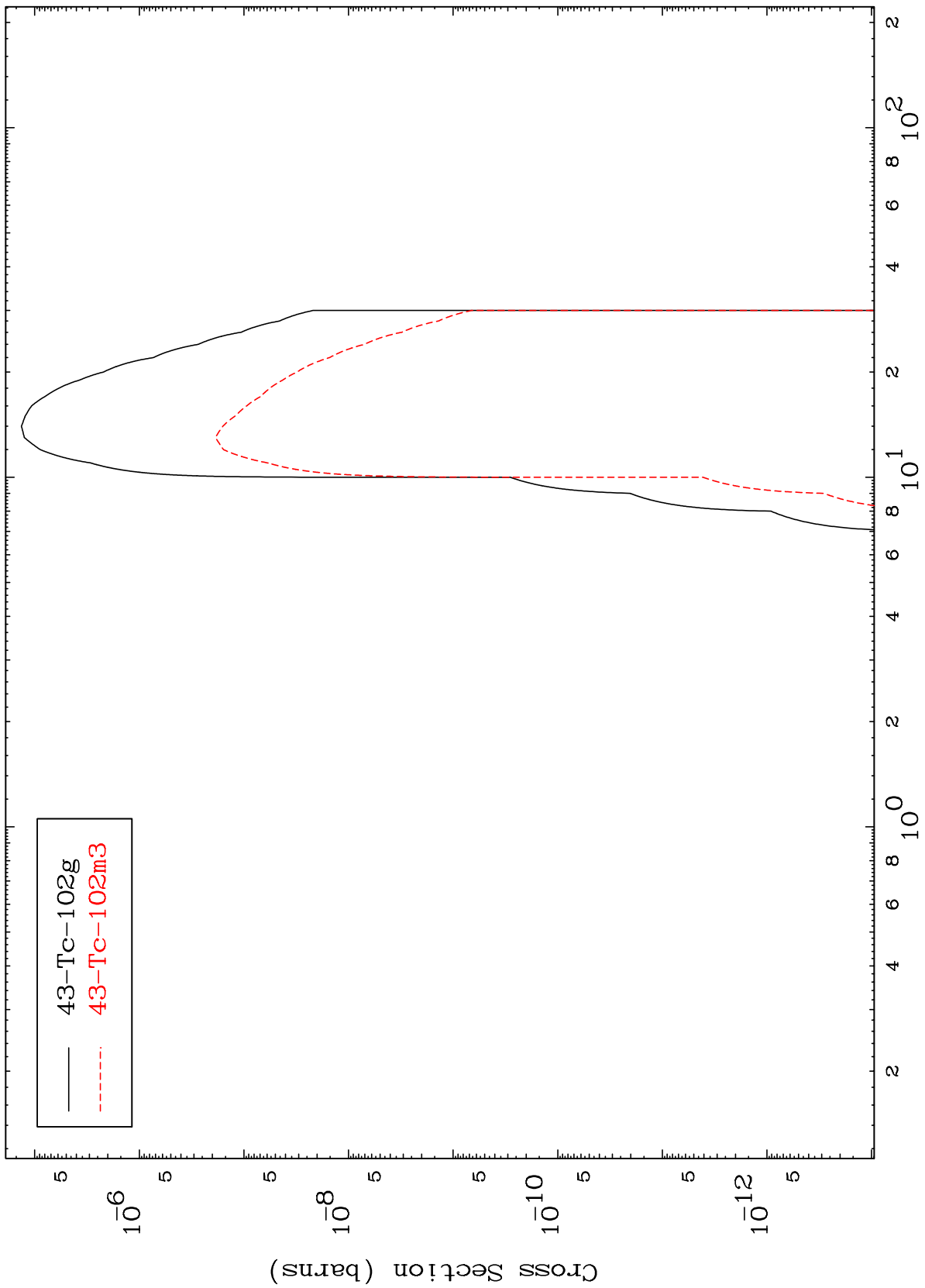
Incident Energy (MeV)

41-Nb-98

MAT 4140

41-Nb-98

Radionuclide Production Cross Section  
(n,  $\gamma$ )

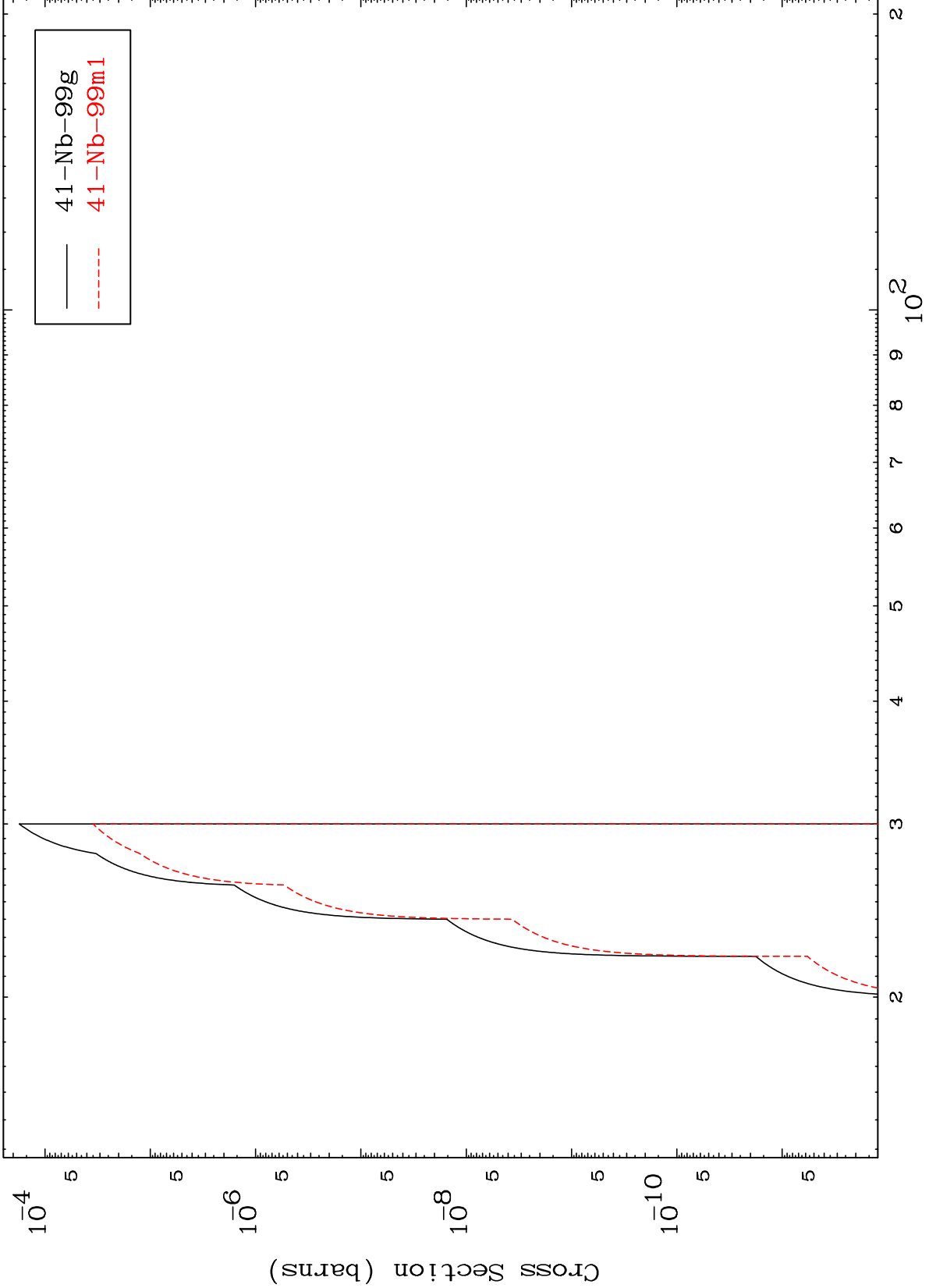


MAT 4140

(n,He-3)

41-Nb-98

Radionuclide Production Cross Section



17

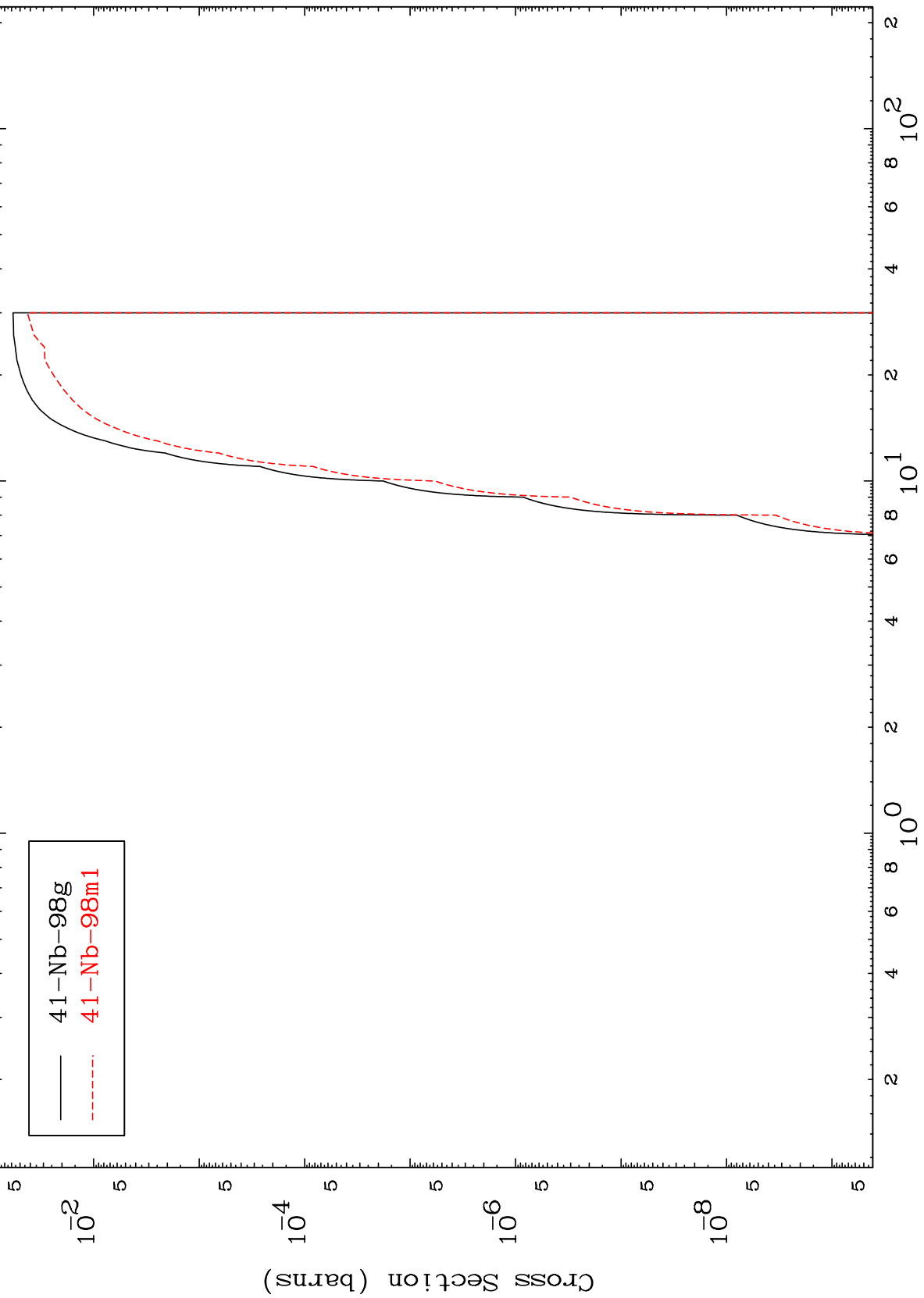
Incident Energy (MeV)

41-Nb-98

MAT 4140

41-Nb-98

Radionuclide Production Cross Section  
(n,  $\alpha$ )



— 41-Nb-98g  
- - - 41-Nb-98m1

41-Nb-98

Incident Energy (MeV)

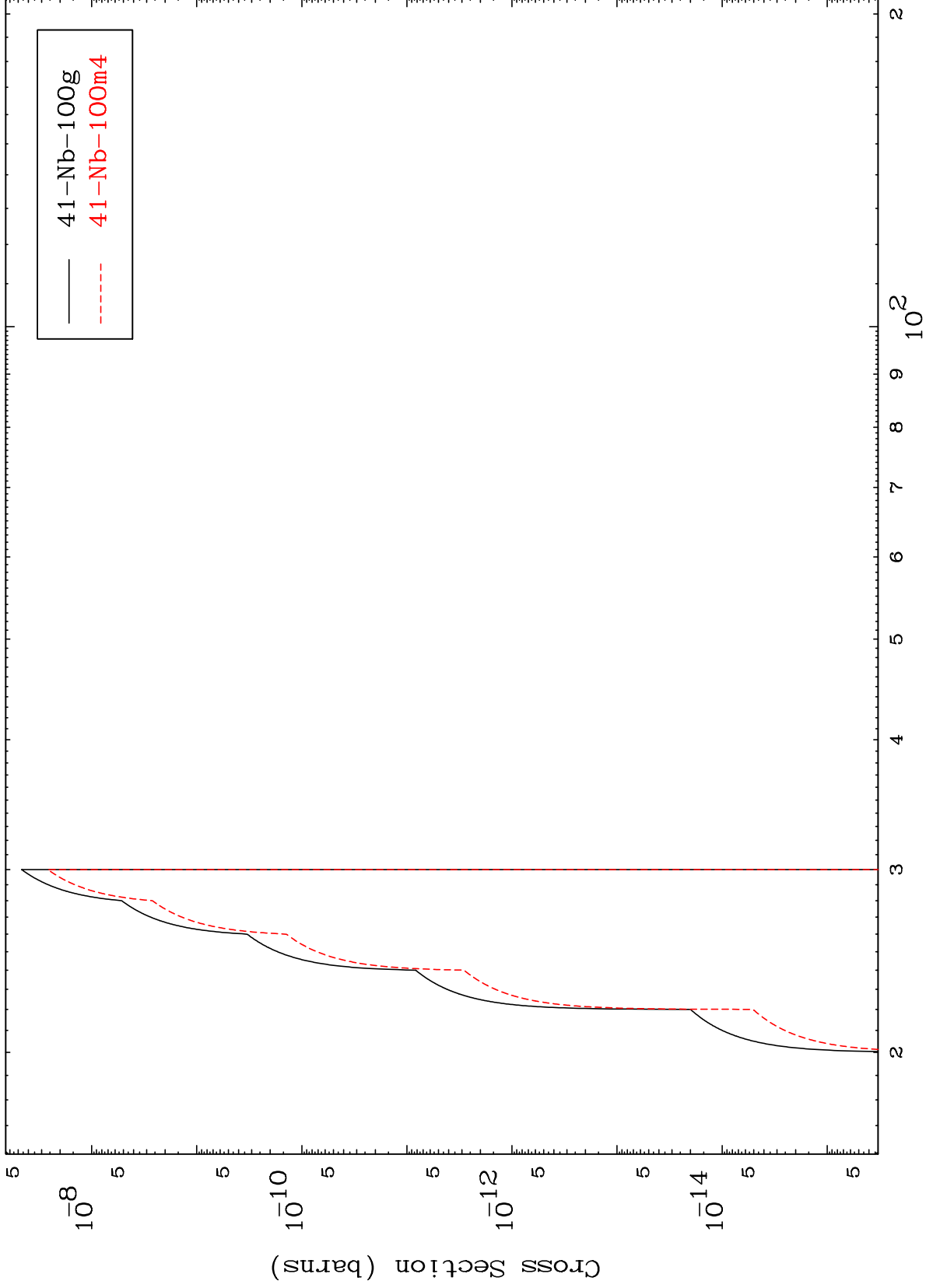
18

MAT 4140

(n,2p)

41-Nb-98

Radionuclide Production Cross Section



19

Incident Energy (MeV)

41-Nb-98