

Program EVALPLOT  
(Version 2021-1)

by

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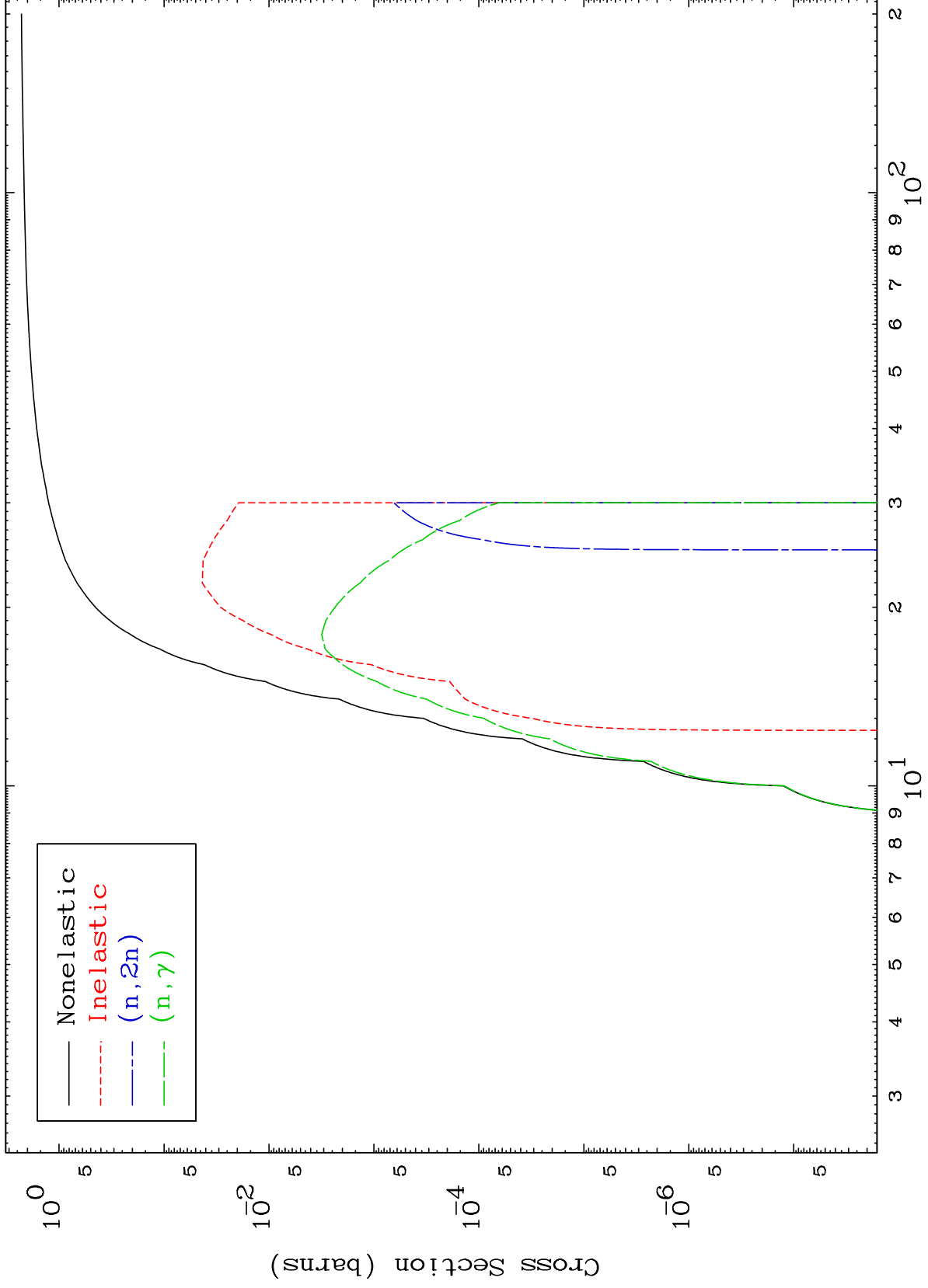
Press Mouse Button to Start

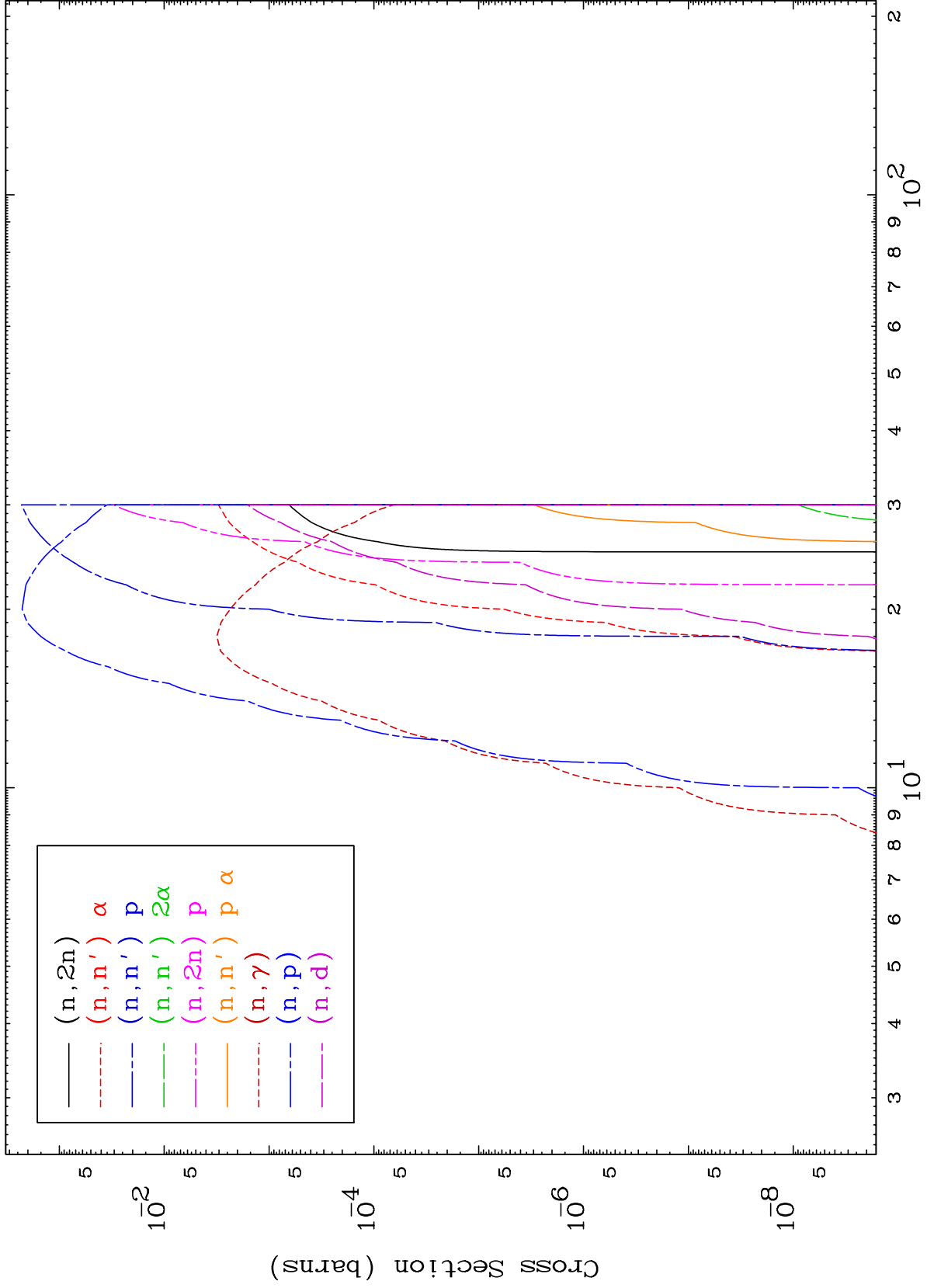
MAT 5992

0 Kelvin Cross Sections

$\alpha$  Major

60-Nd-131

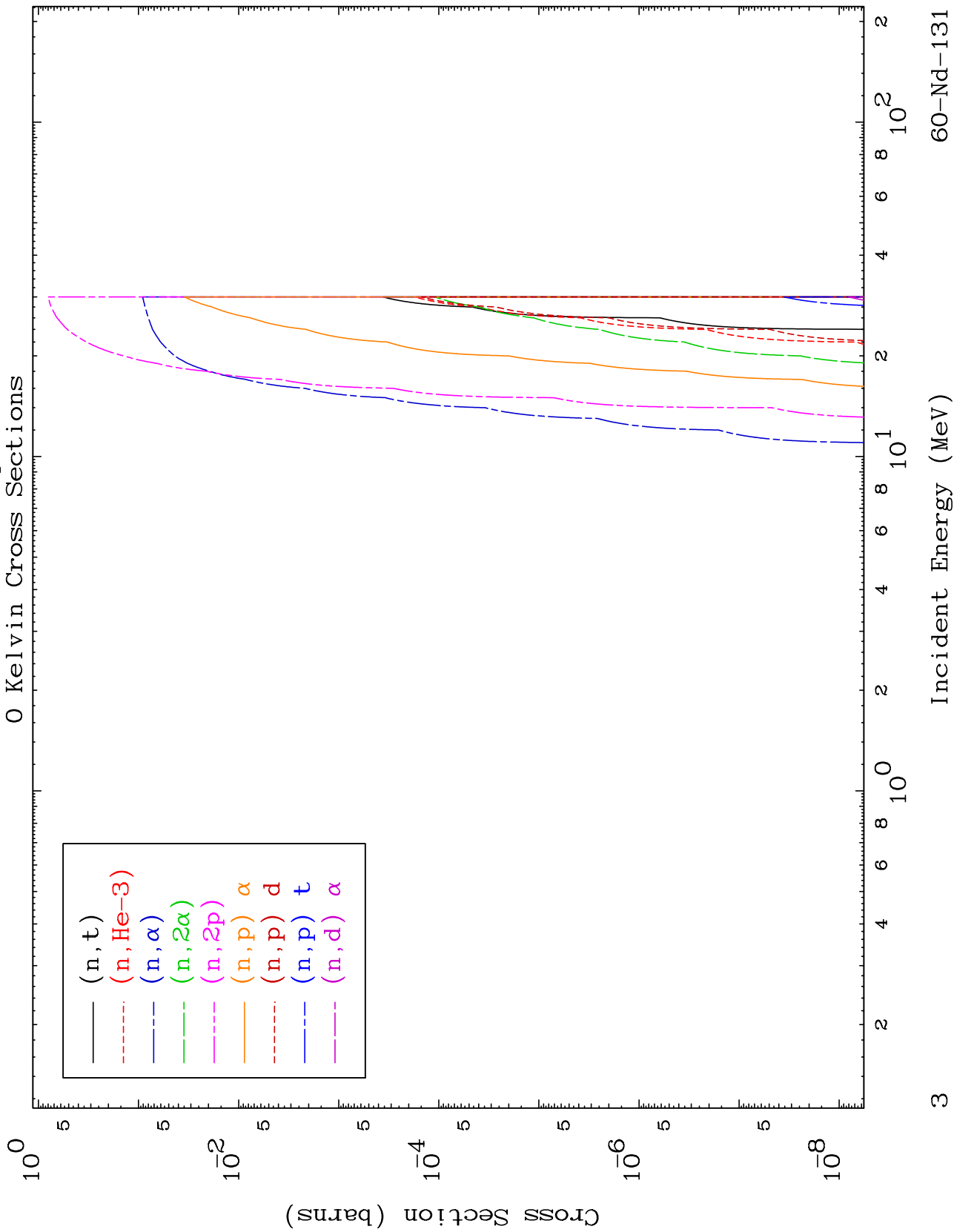


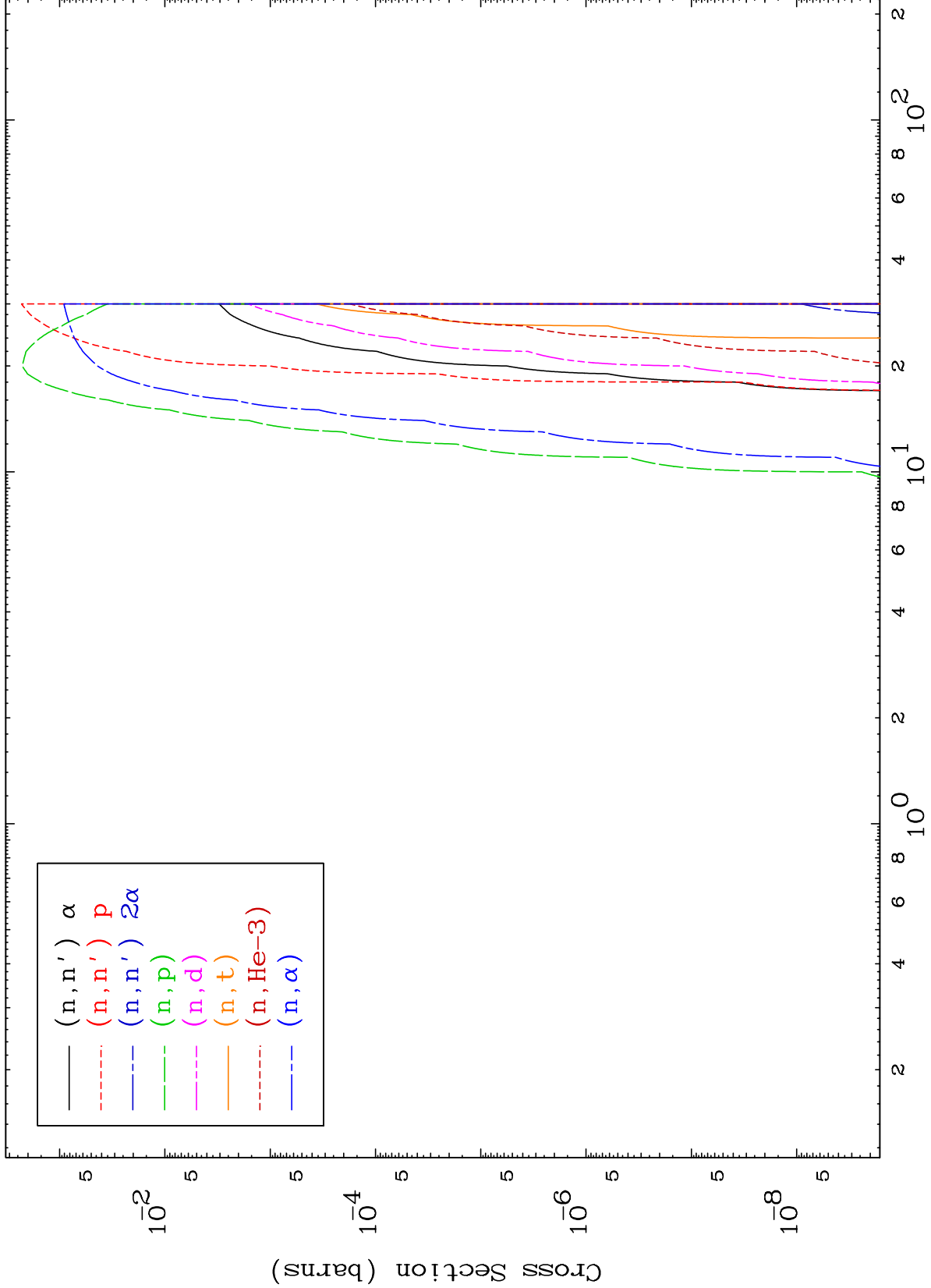


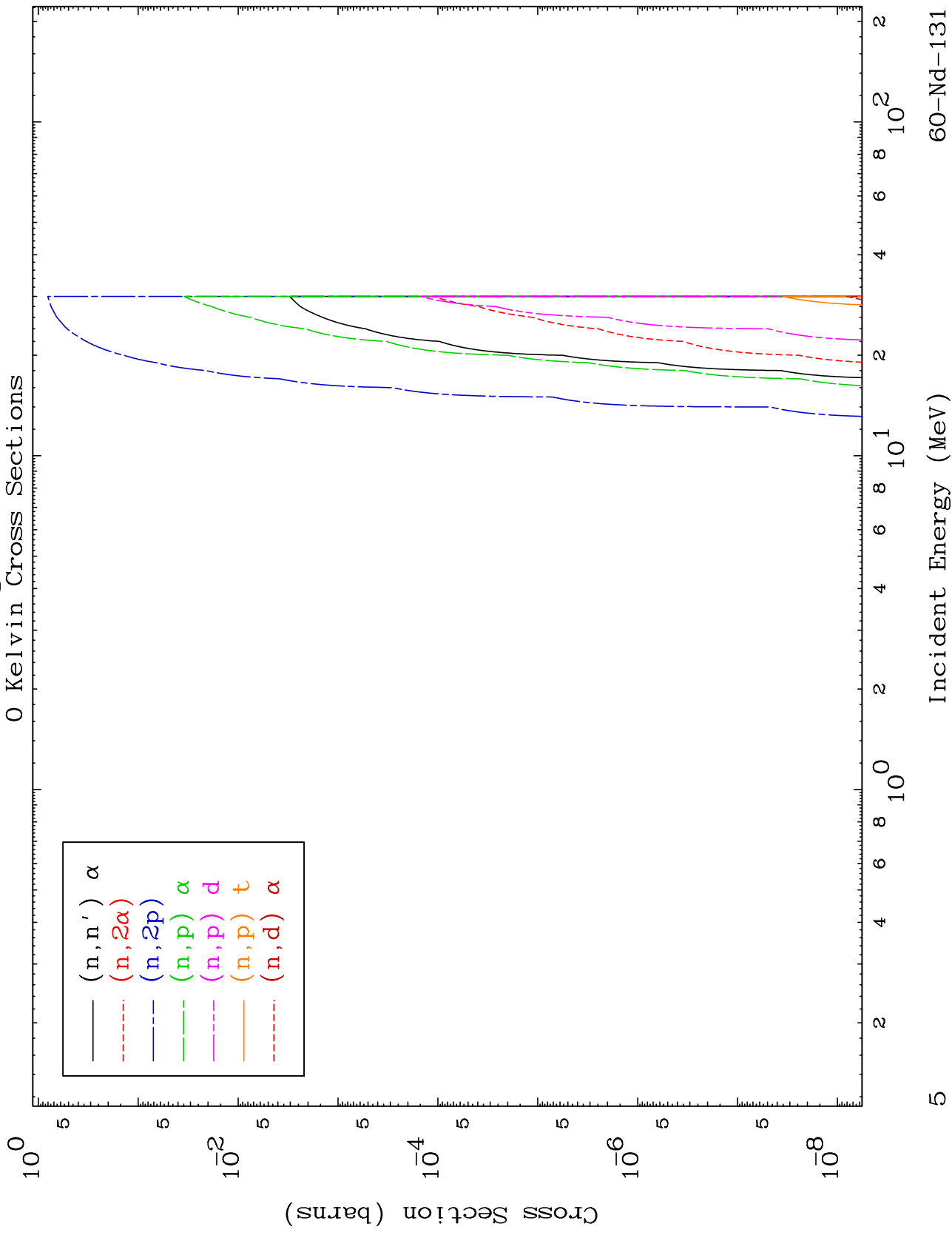
MAT 5992

$\alpha$  Neutron Absorption

60-Nd-131





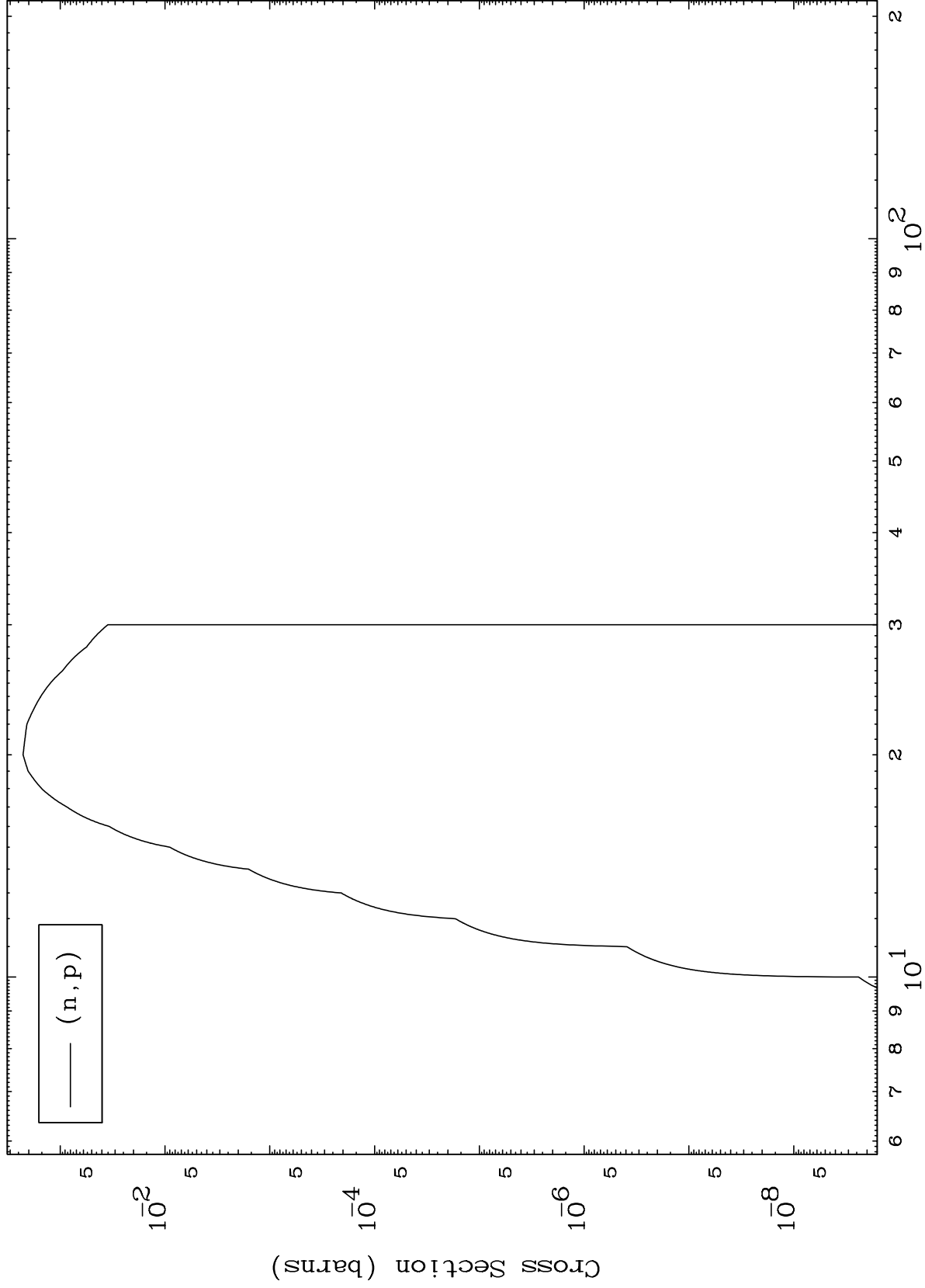


MAT 5992

( $\alpha, p$ ) Levels

$^{60}\text{Nd}-131$

0 Kelvin Cross Sections



6

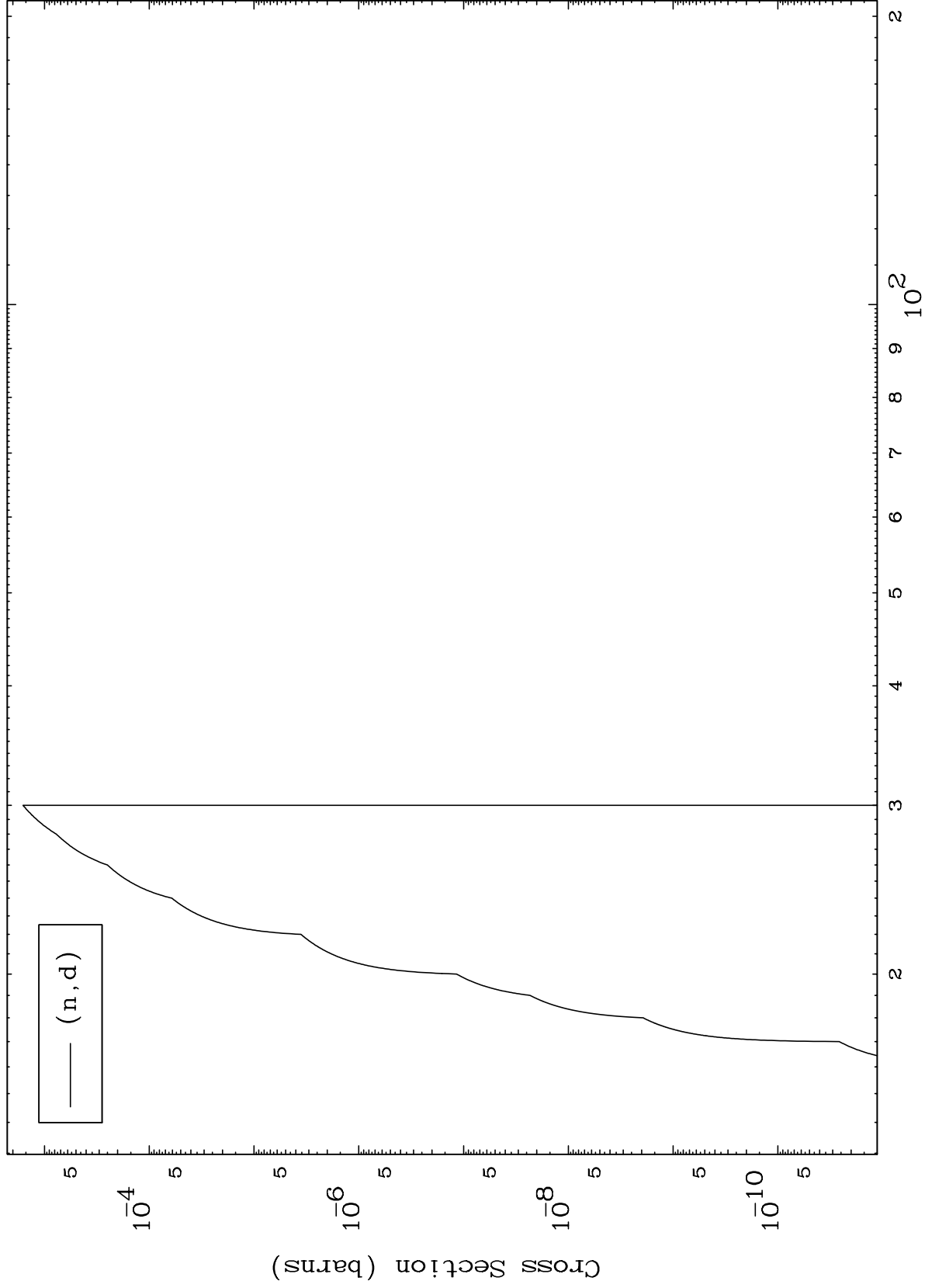
Incident Energy (MeV)

$^{60}\text{Nd}-131$

MAT 5992

60-Nd-131

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



60-Nd-131

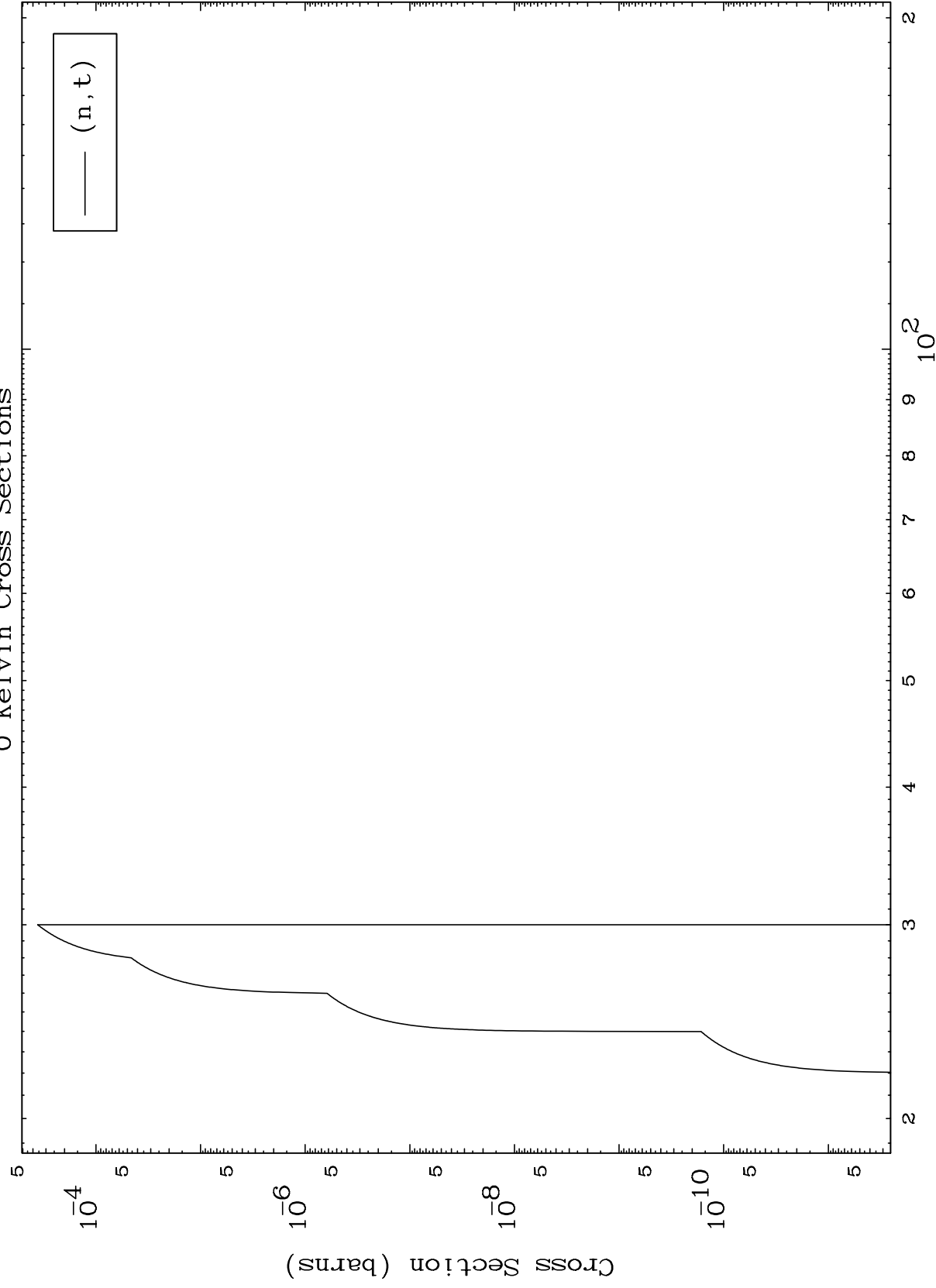
Incident Energy (MeV)

7

MAT 5992

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

60-Nd-131



8

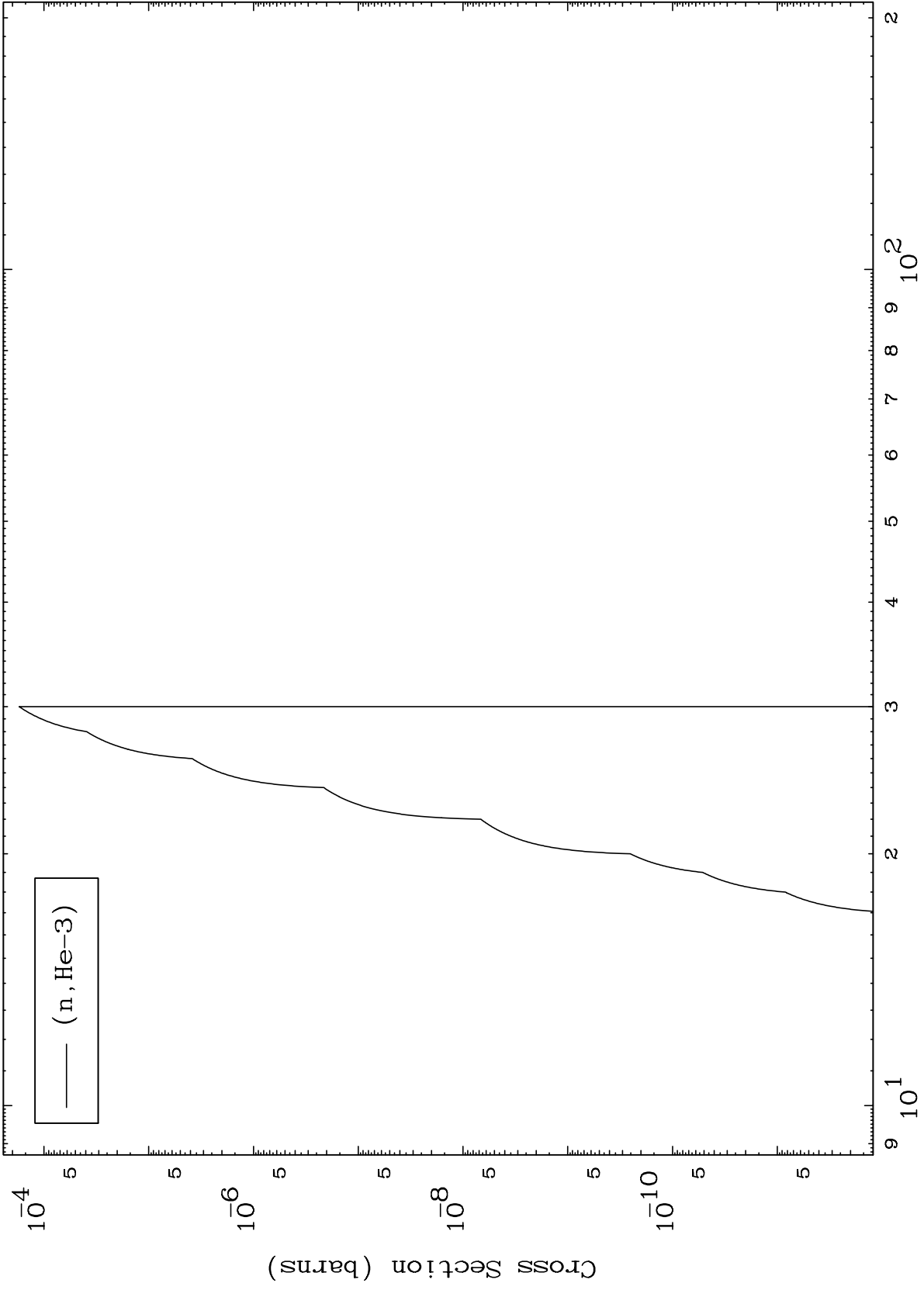
Incident Energy (MeV)

60-Nd-131

MAT 5992

60-Nd-131

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



60-Nd-131

Incident Energy (MeV)

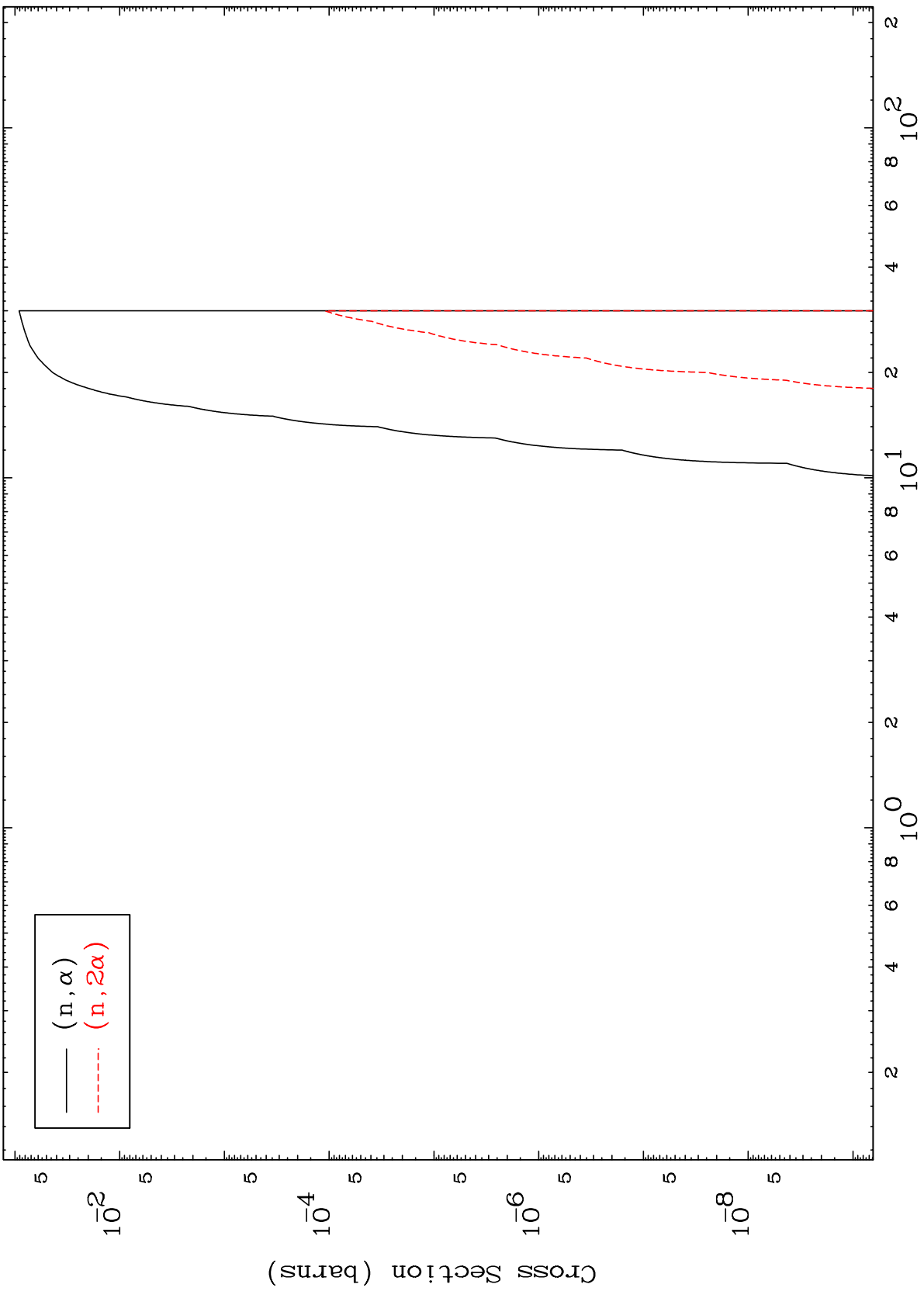
9

MAT 5992

( $\alpha, \alpha$ ) Levels

60-Nd-131

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

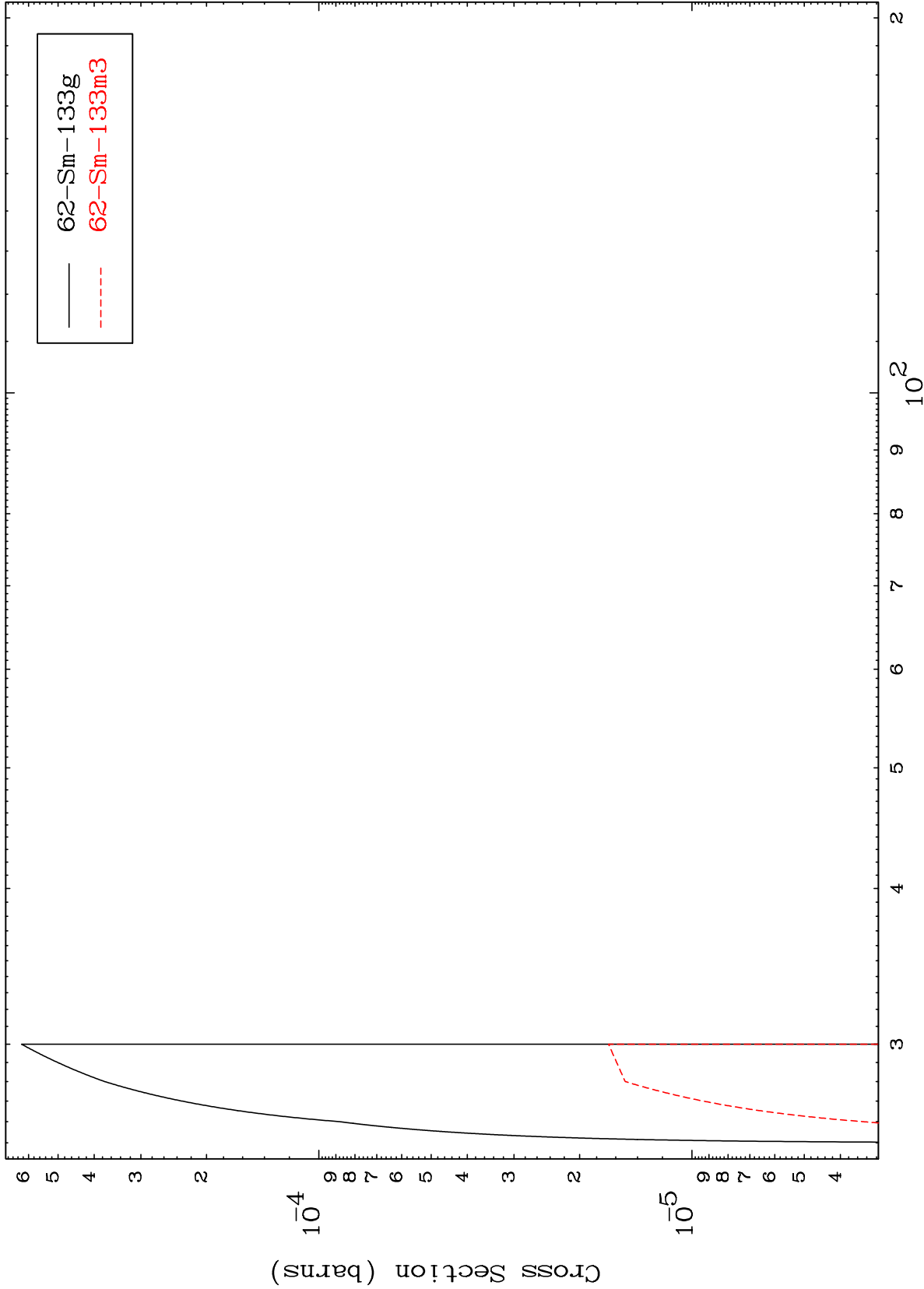
Incident Energy (MeV)

60-Nd-131

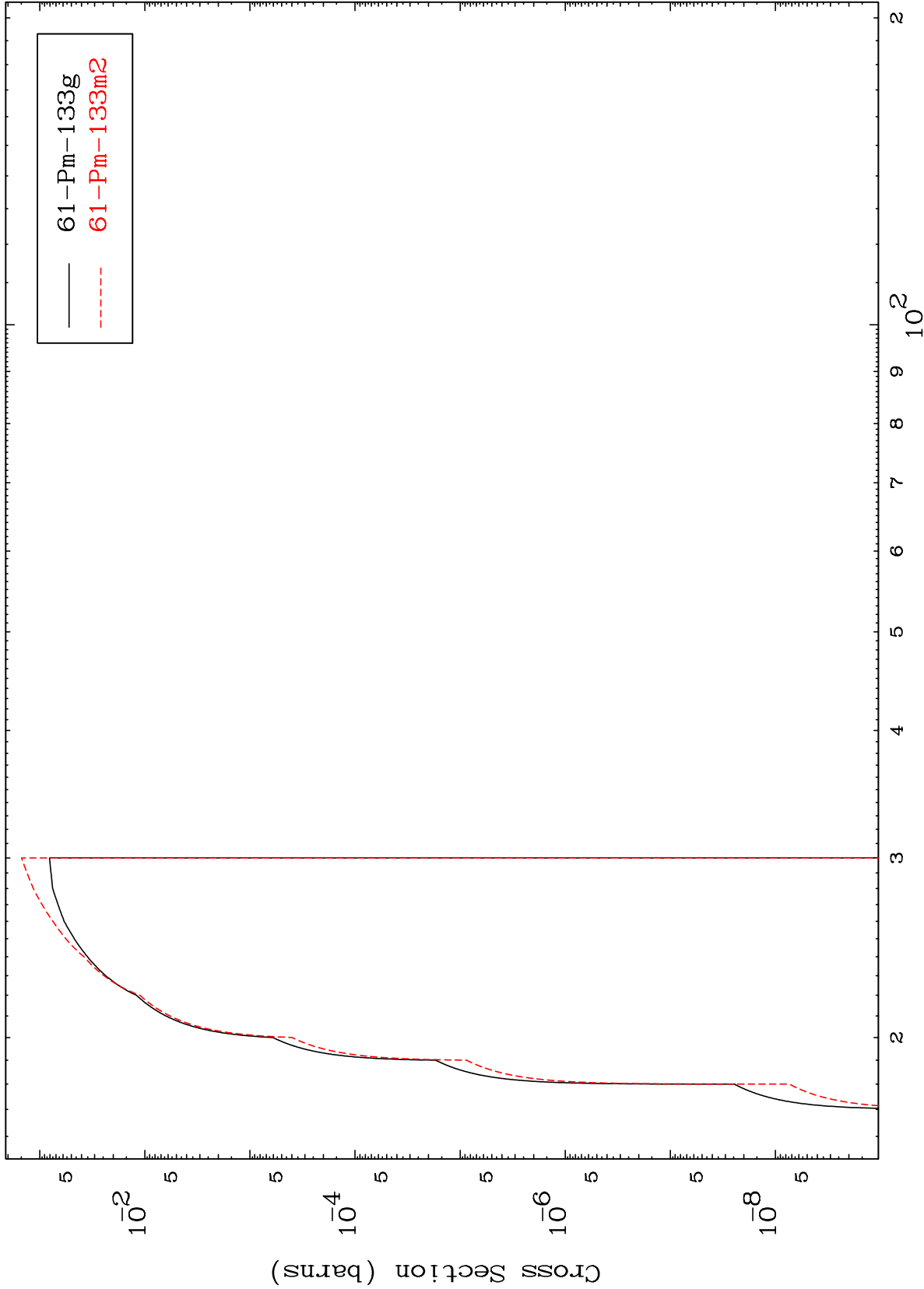
MAT 5992

60-Nd-131

(n,2n)  
Radionuclide Production Cross Section



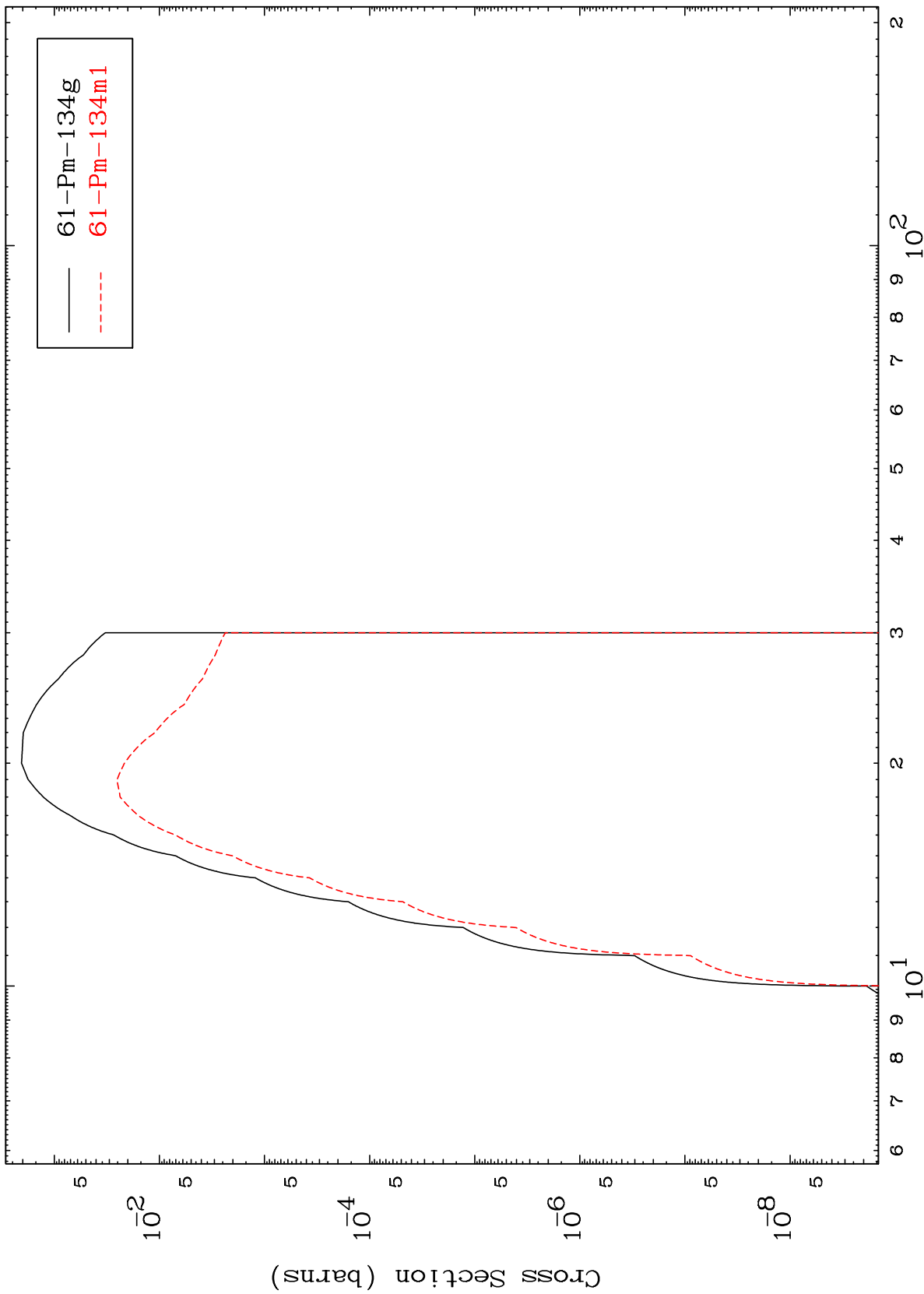
(n,n') p  
Radionuclide Production Cross Section



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<sup>60</sup>Nd-131

(n,p)  
Radionuclide Production Cross Section



13

Incident Energy (MeV)

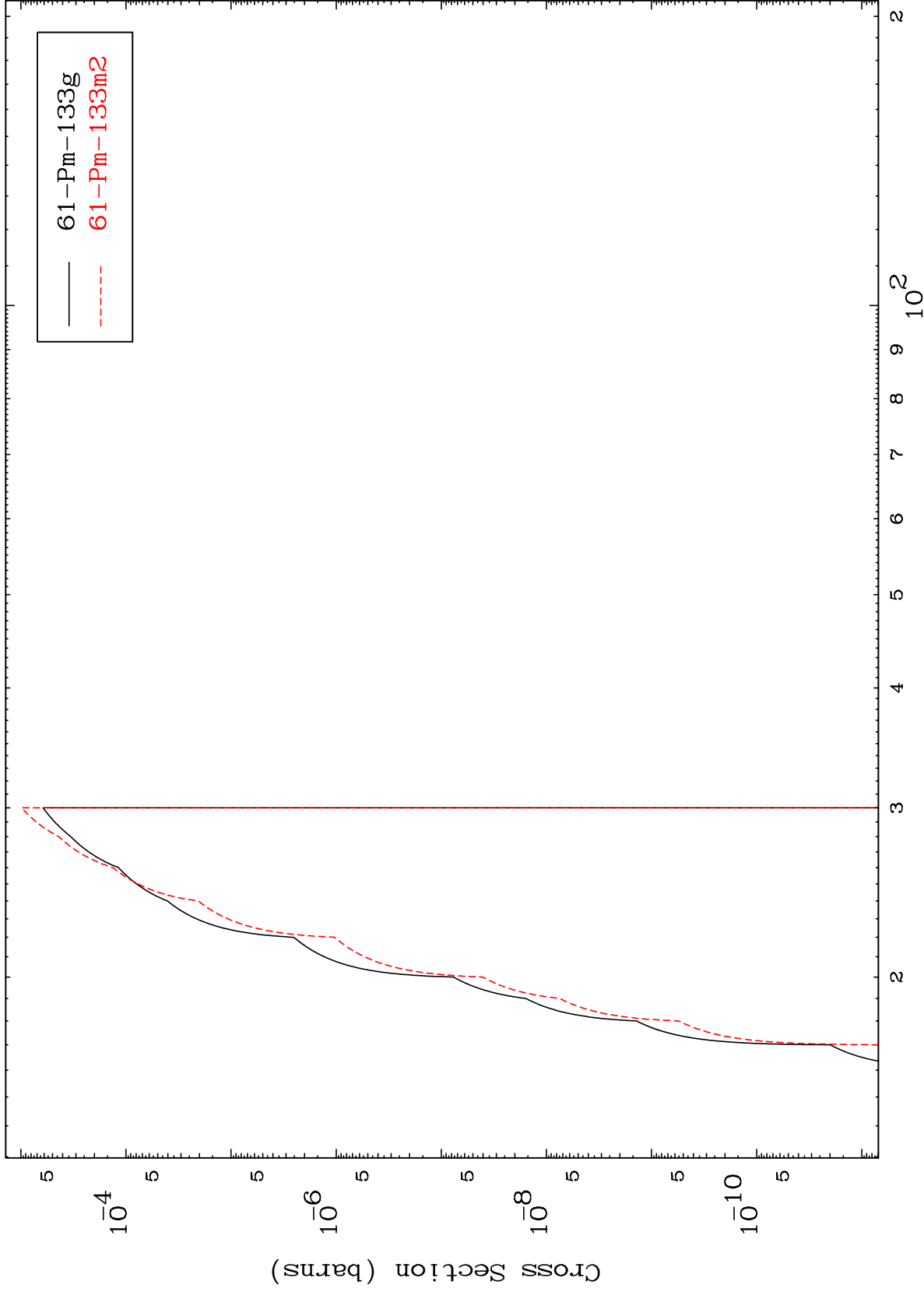
<sup>60</sup>Nd-131

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(n,d)

60-Nd-131

Radionuclide Production Cross Section



14

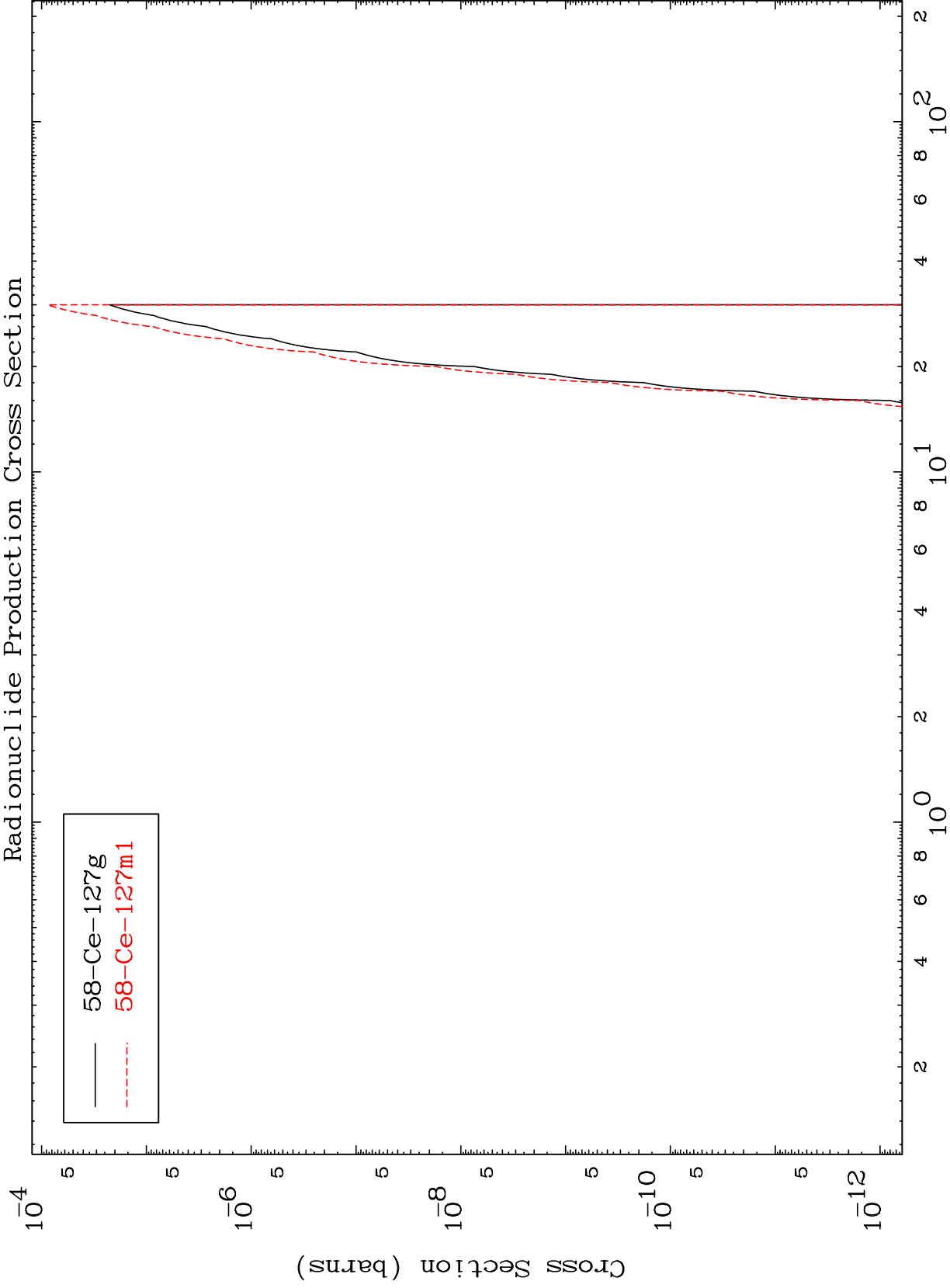
Incident Energy (MeV)

60-Nd-131

MAT 5992

60-Nd-131

Radionuclide Production Cross Section  
(n,2α)



60-Nd-131

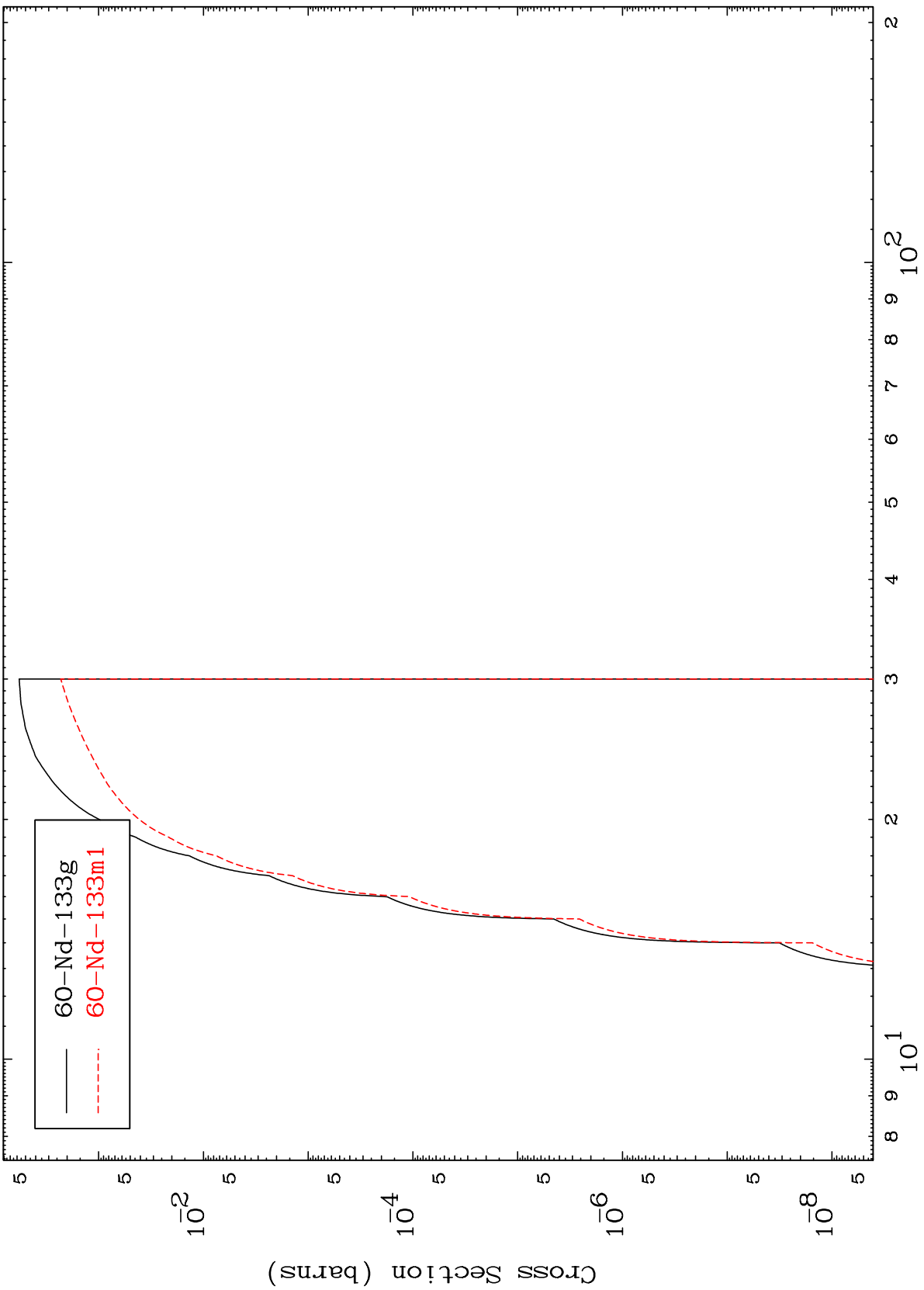
Incident Energy (MeV)

15

MAT 5992

60-Nd-131

(n,2p)  
Radionuclide Production Cross Section



60-Nd-131

Incident Energy (MeV)

16