

Program EVALPLOT
(Version 2021-1)

by

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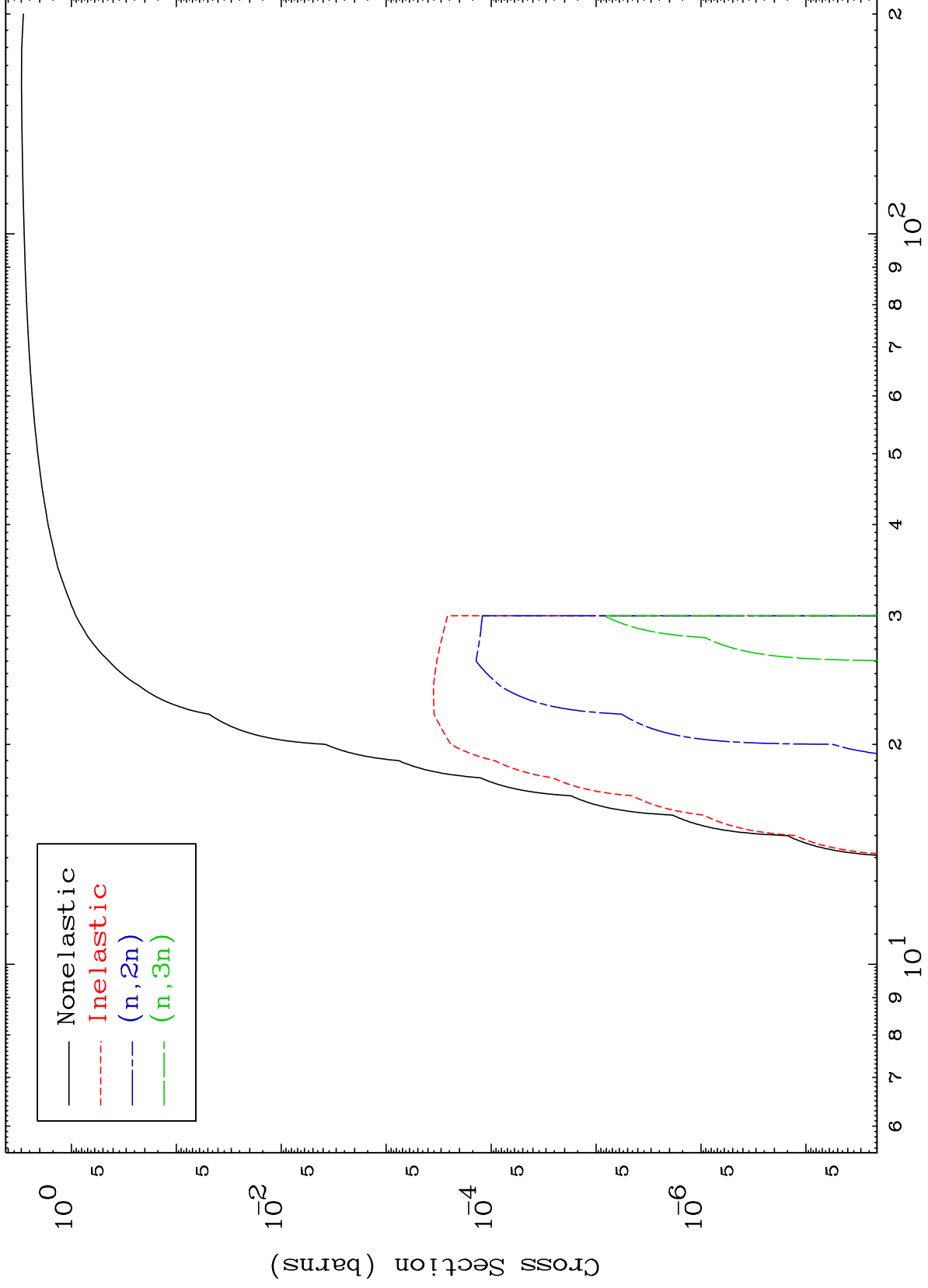
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Press Mouse Button to Start

MAT 9343

0 Kelvin Cross Sections

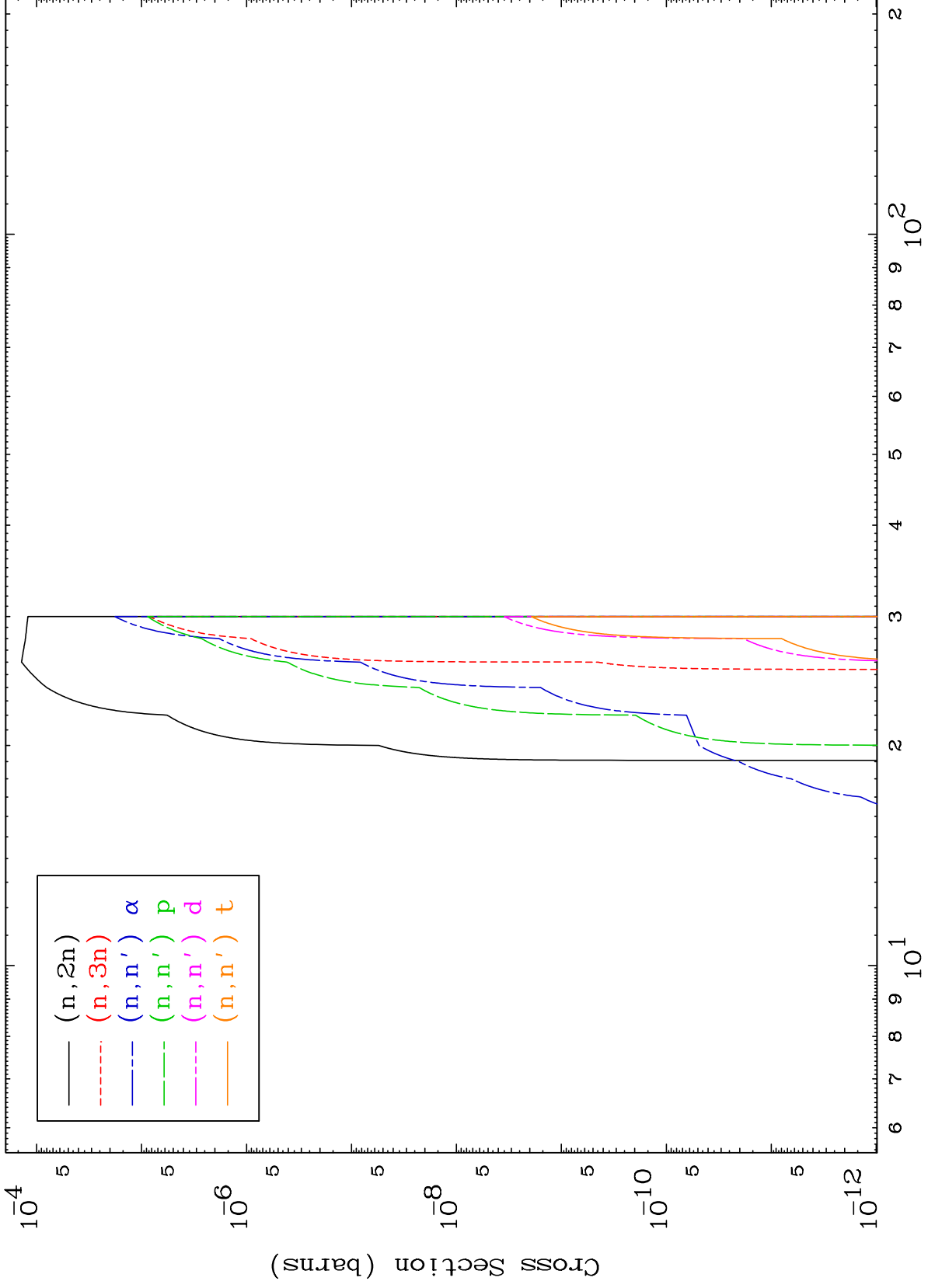
93-Np-236



1

Incident Energy (MeV)

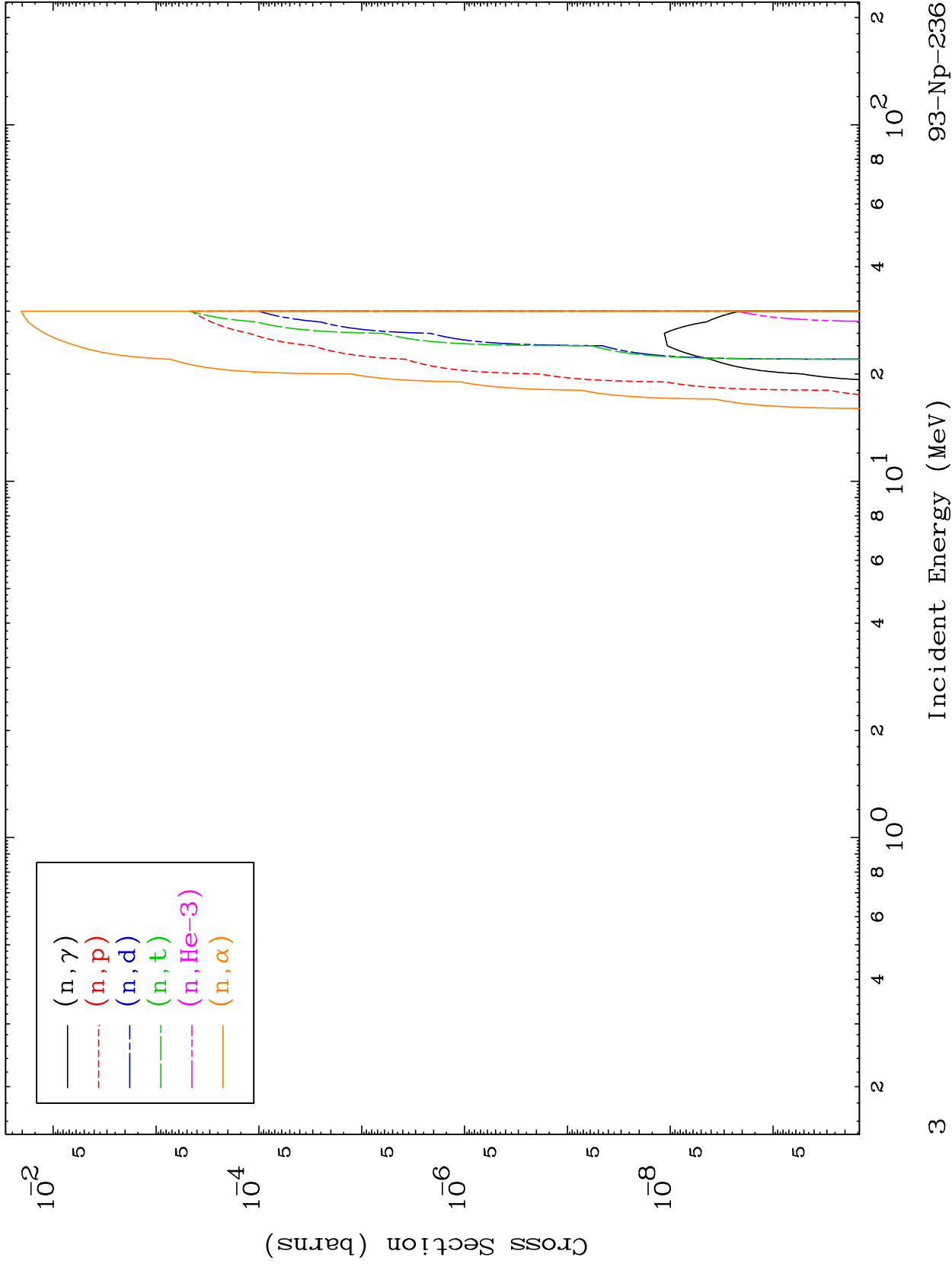
93-Np-236



MAT 9343

α Neutron Absorption
0 Kelvin Cross Sections

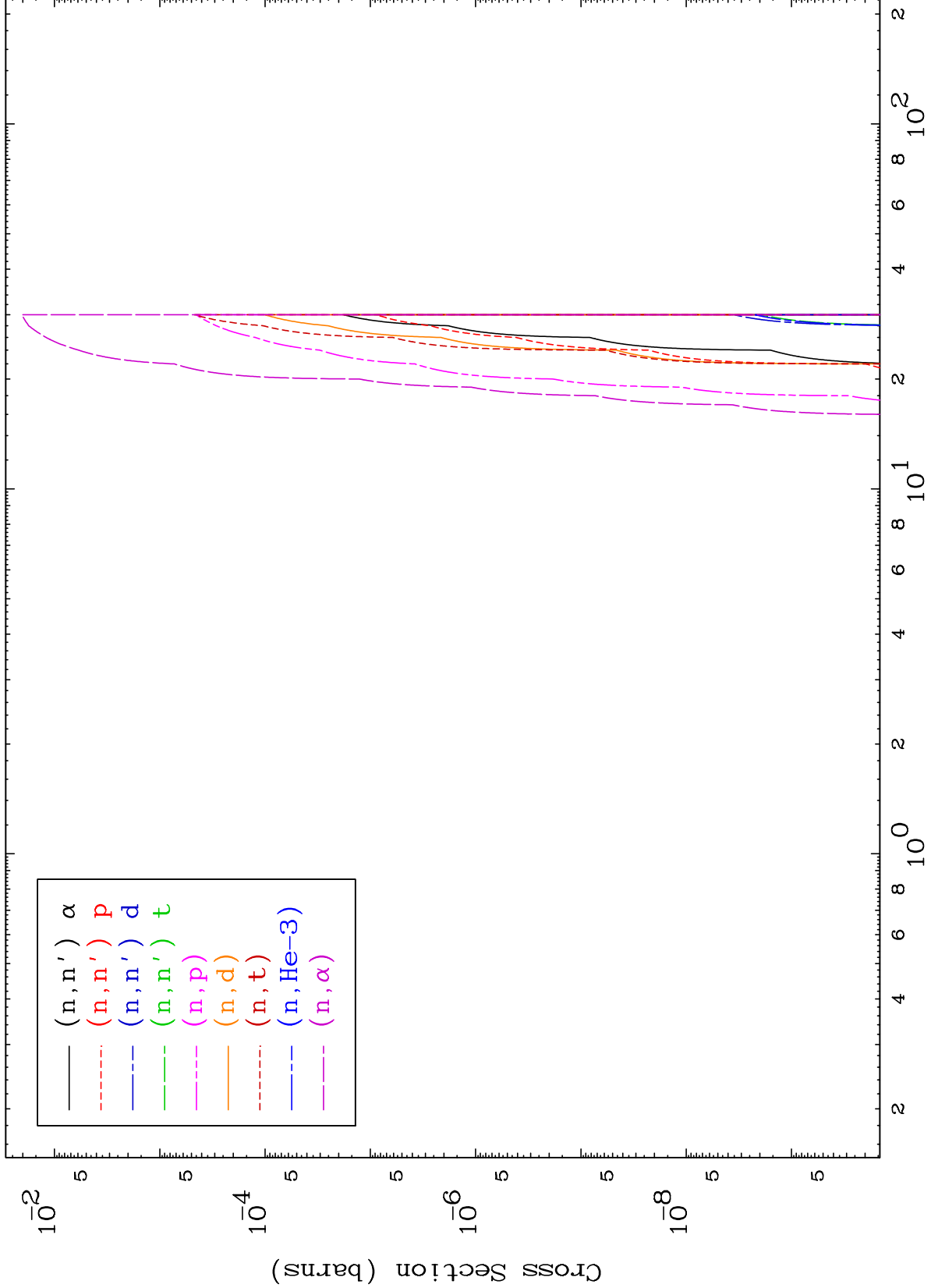
93-Np-236



MAT 9343

α Charged Particle
0 Kelvin Cross Sections

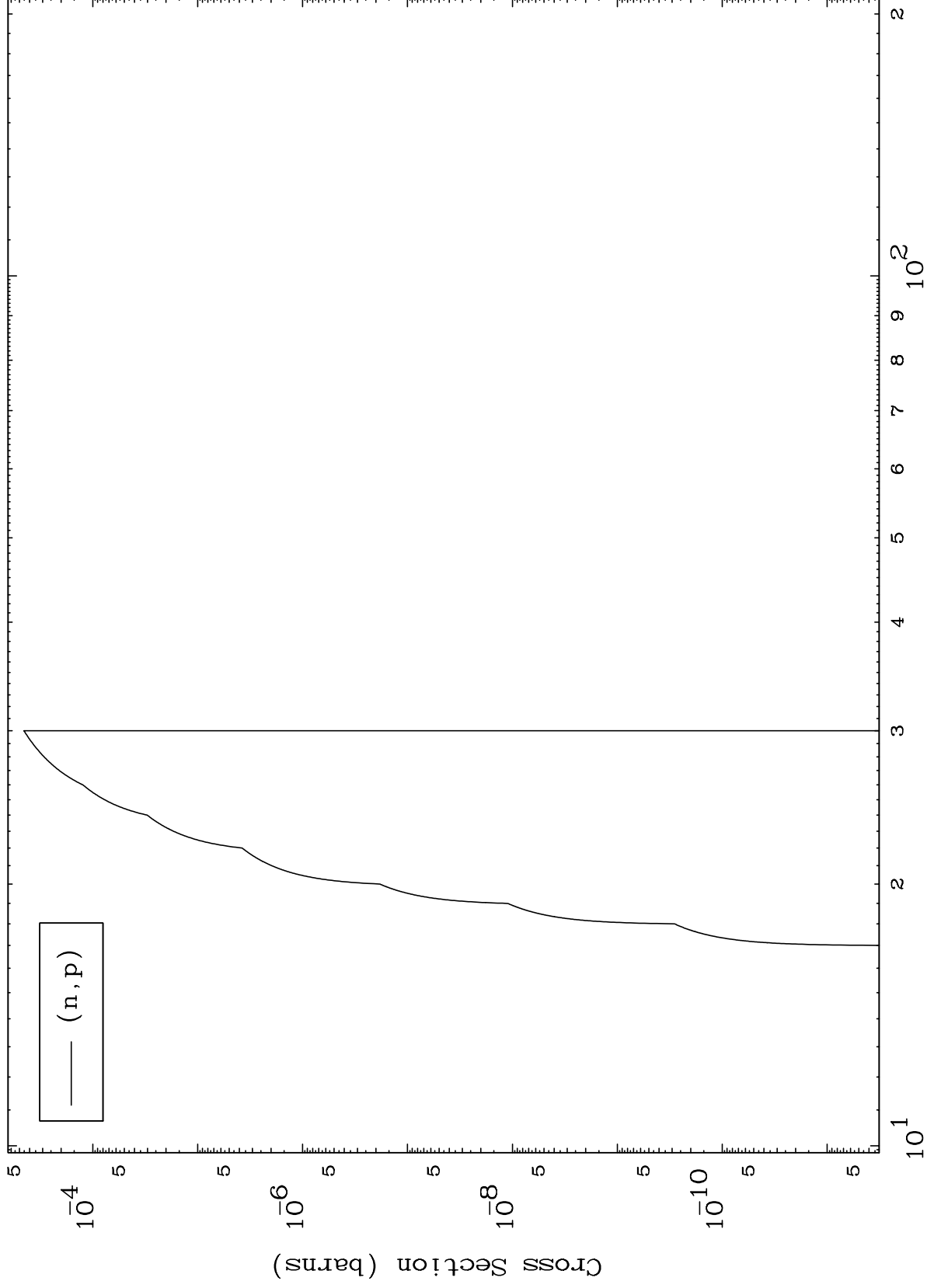
93-Np-236



MAT 9343

(α, p) Levels
0 Kelvin Cross Sections

93-Np-236



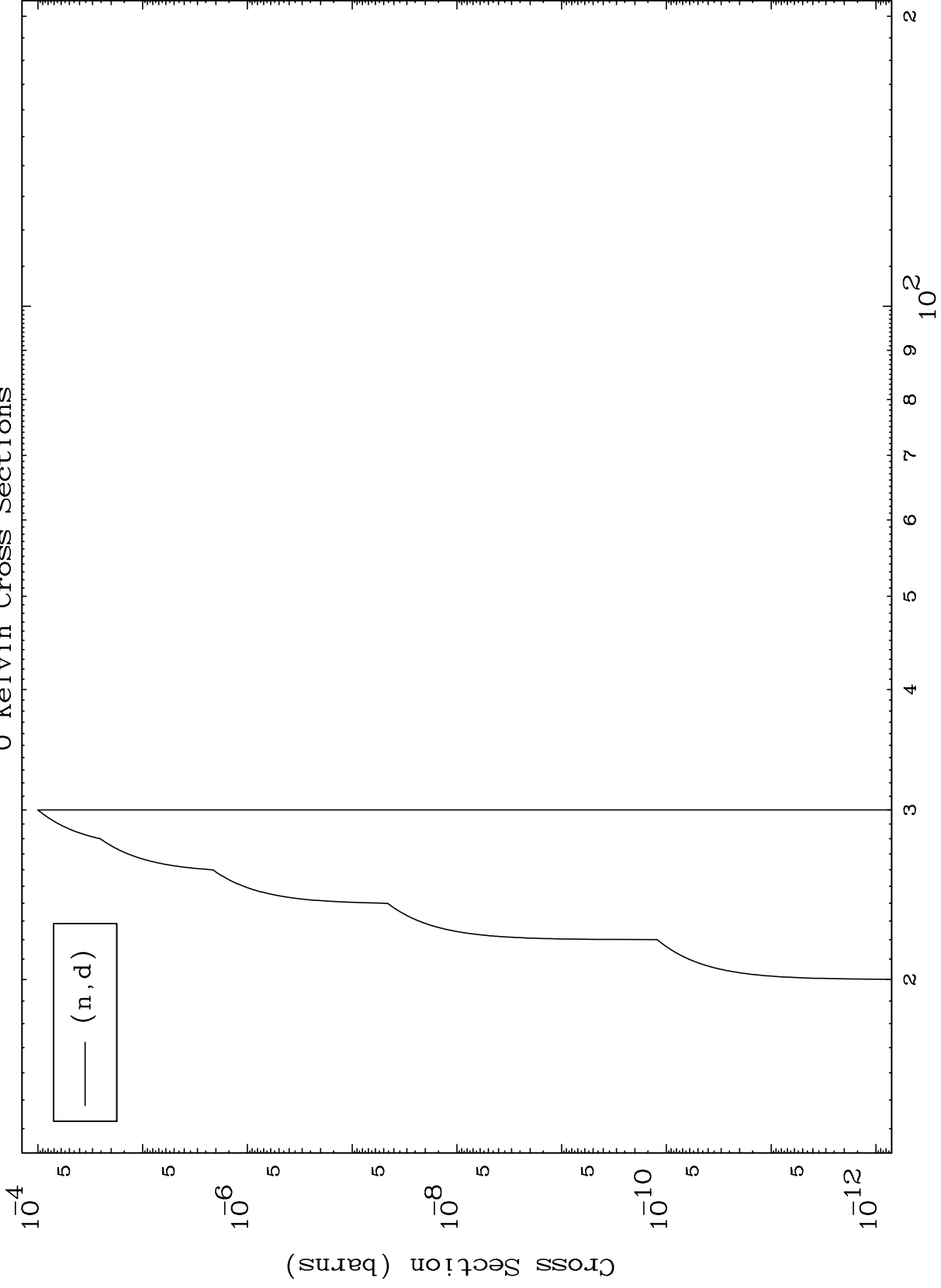
Incident Energy (MeV)

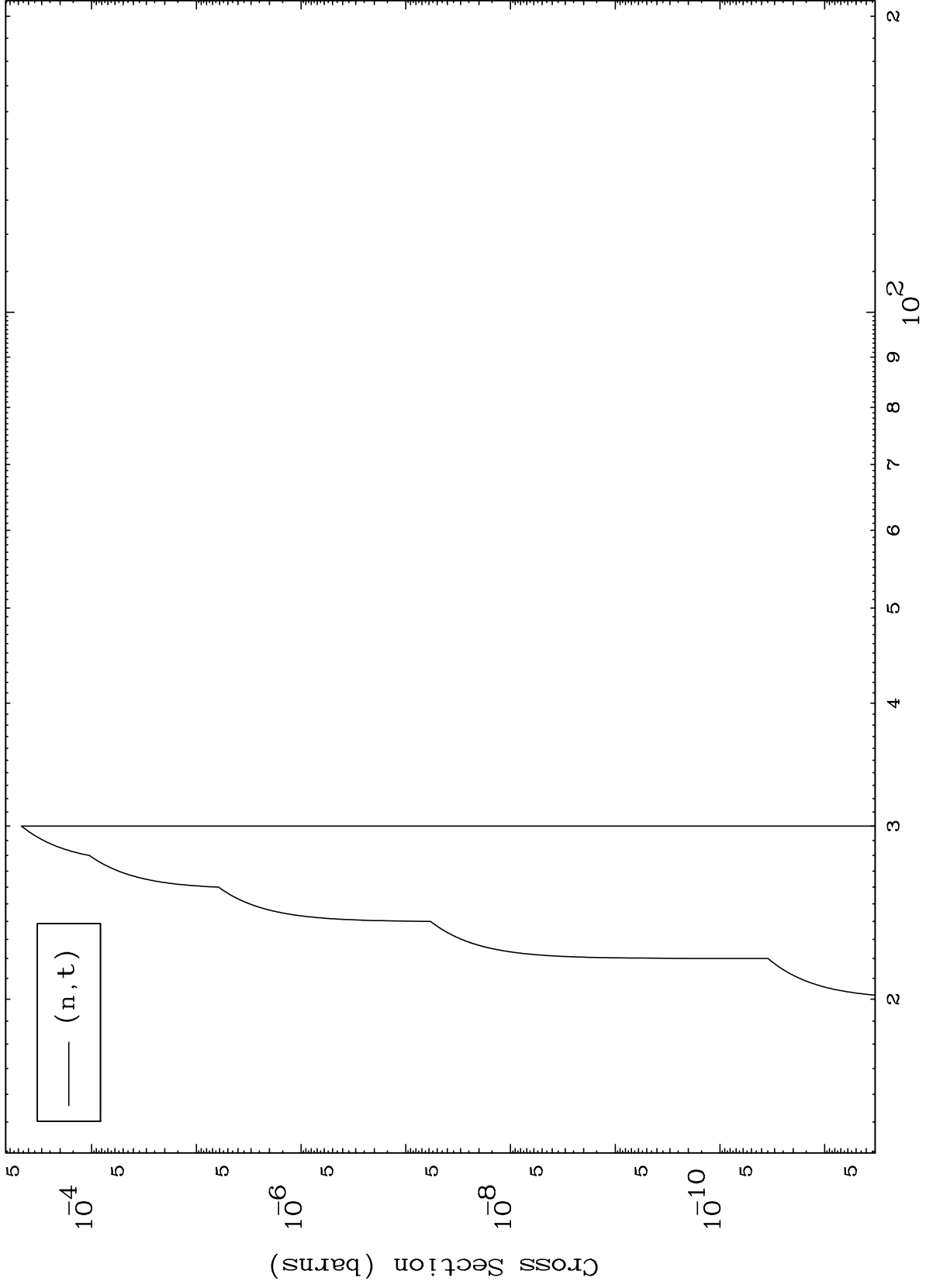
93-Np-236

MAT 9343

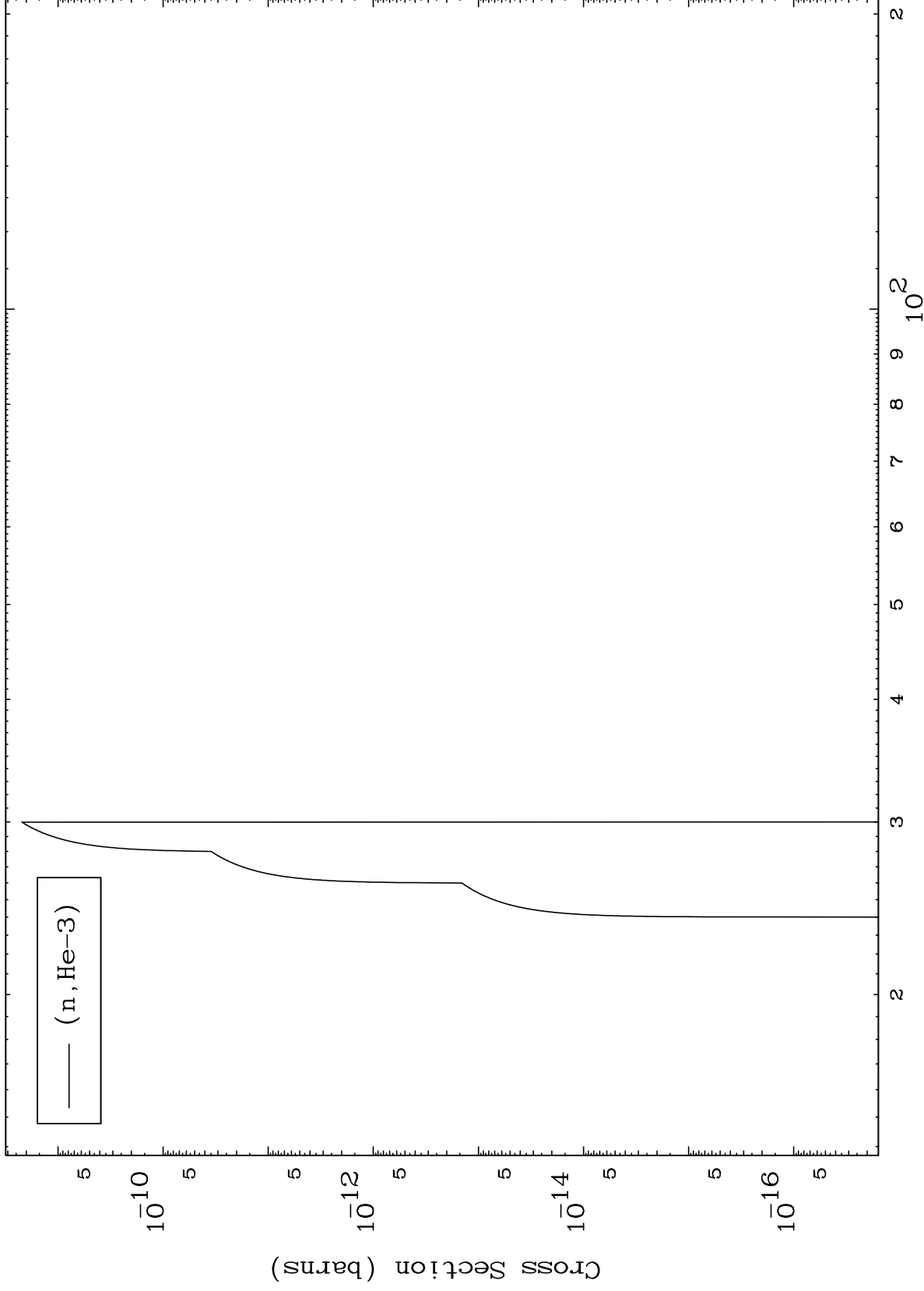
(α, d) Levels
0 Kelvin Cross Sections

93-Np-236





0 Kelvin Cross Sections

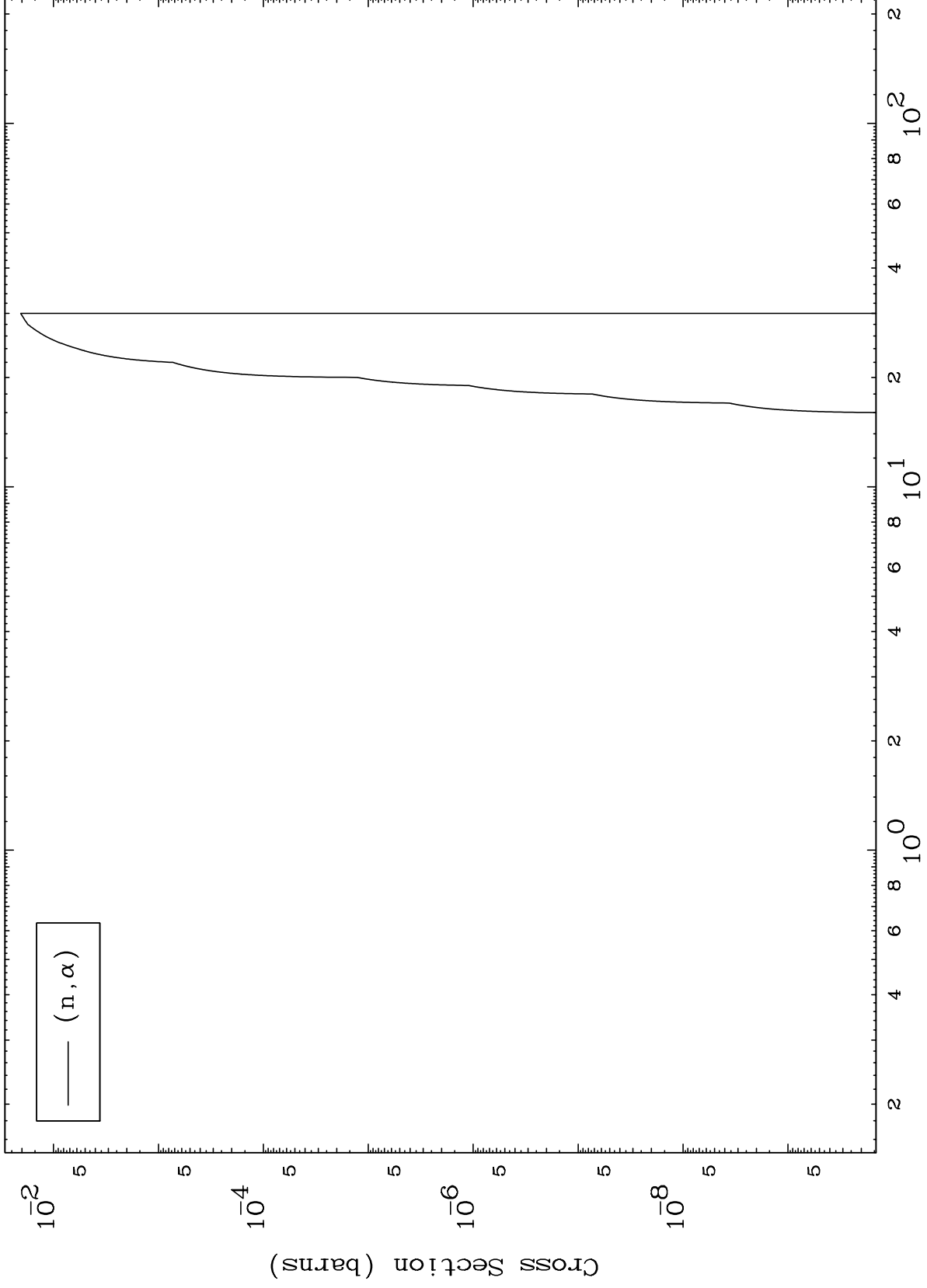


MAT 9343

(α, α) Levels

93-Np-236

0 Kelvin Cross Sections

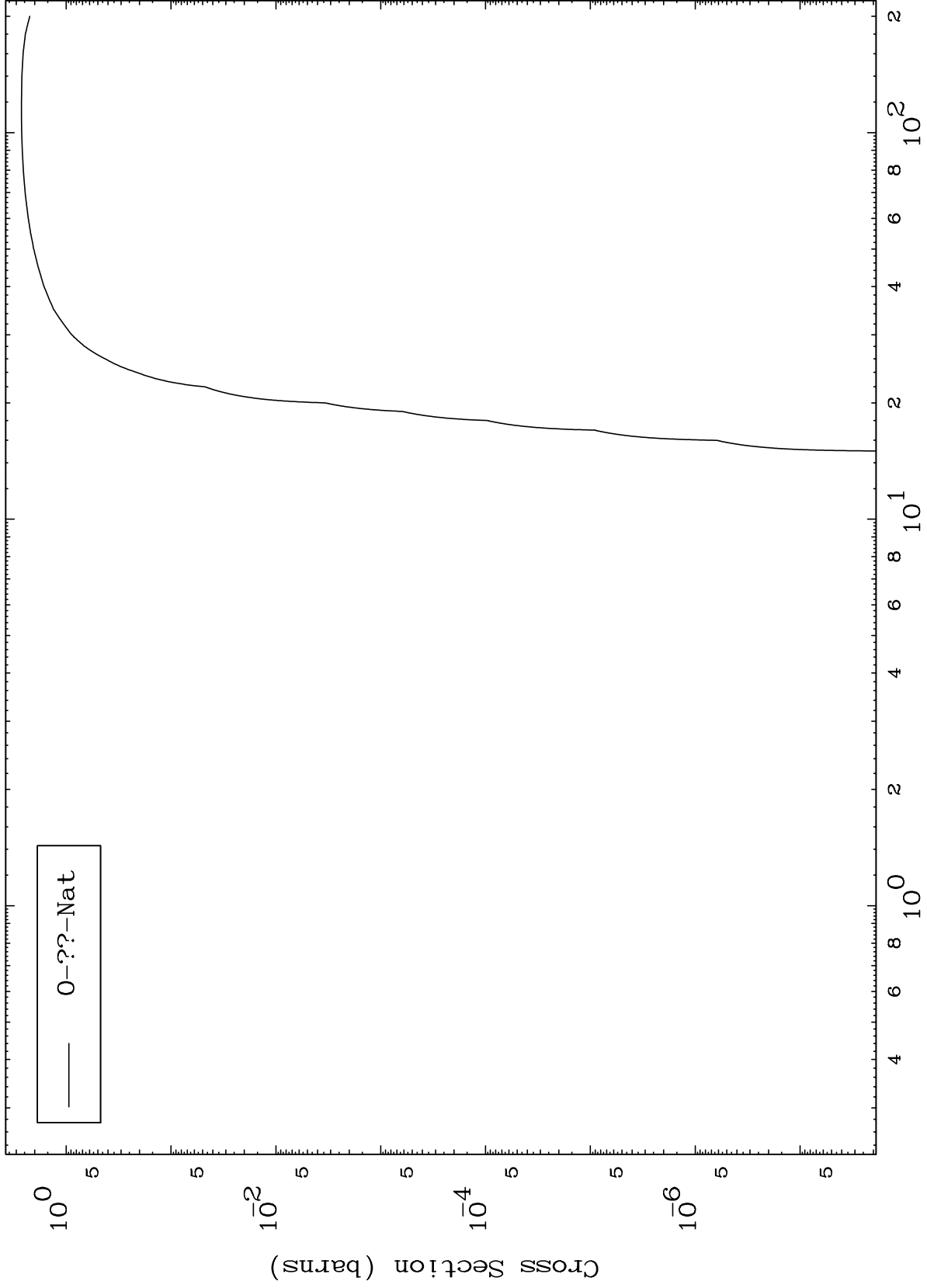


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Fission

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Radionuclide Production Cross Section

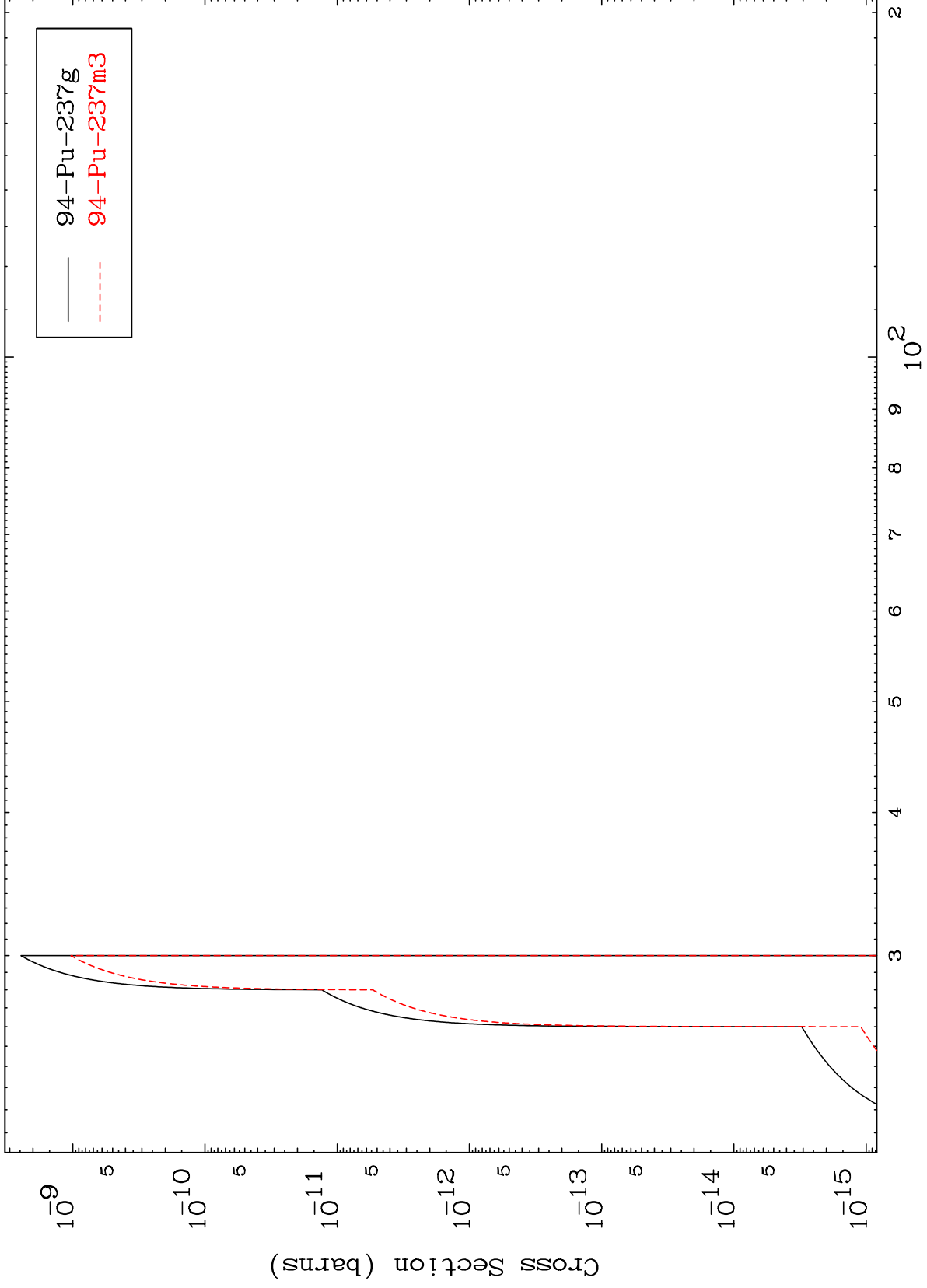


10

Incident Energy (MeV)

93-Np-236

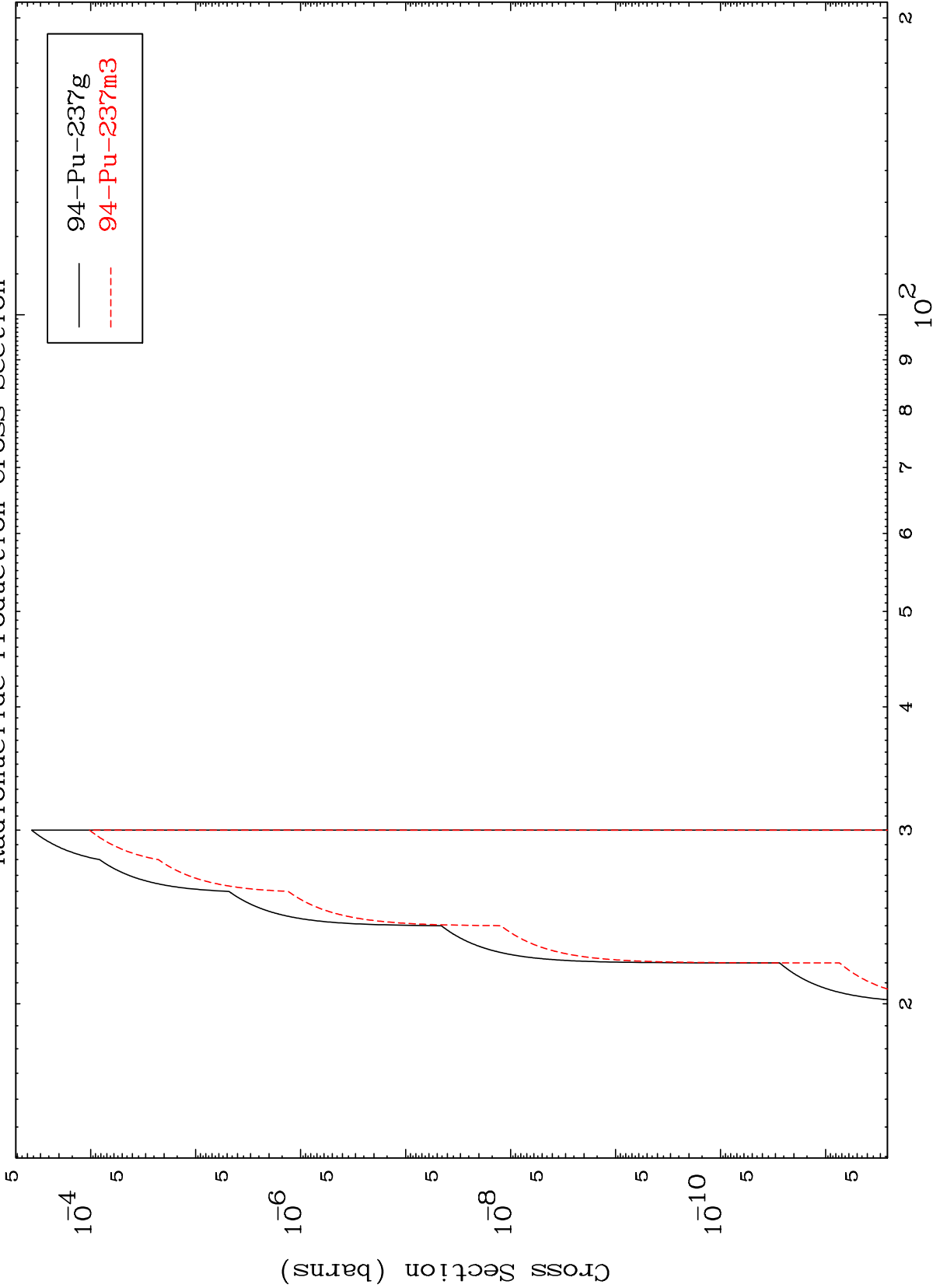
Radionuclide Production Cross Section



MAT 9343

93-Np-236

(n, t)
Radionuclide Production Cross Section



94-Pu-237g
94-Pu-237m3

12

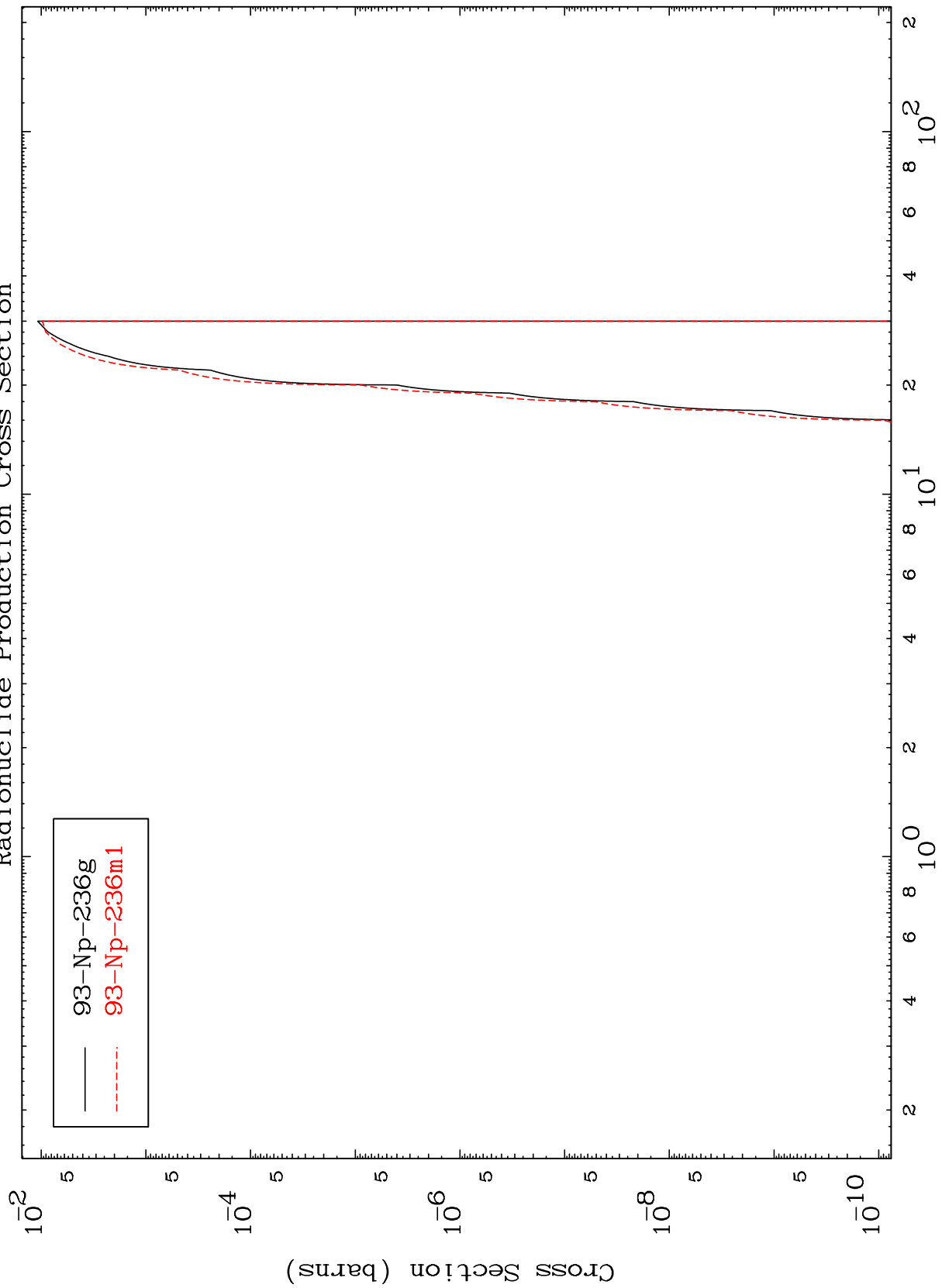
93-Np-236

Incident Energy (MeV)

MAT 9343

93-Np-236

(n, α)
Radionuclide Production Cross Section



13

Incident Energy (MeV)

93-Np-236