

Program EVALPLOT
(Version 2021-1)

by

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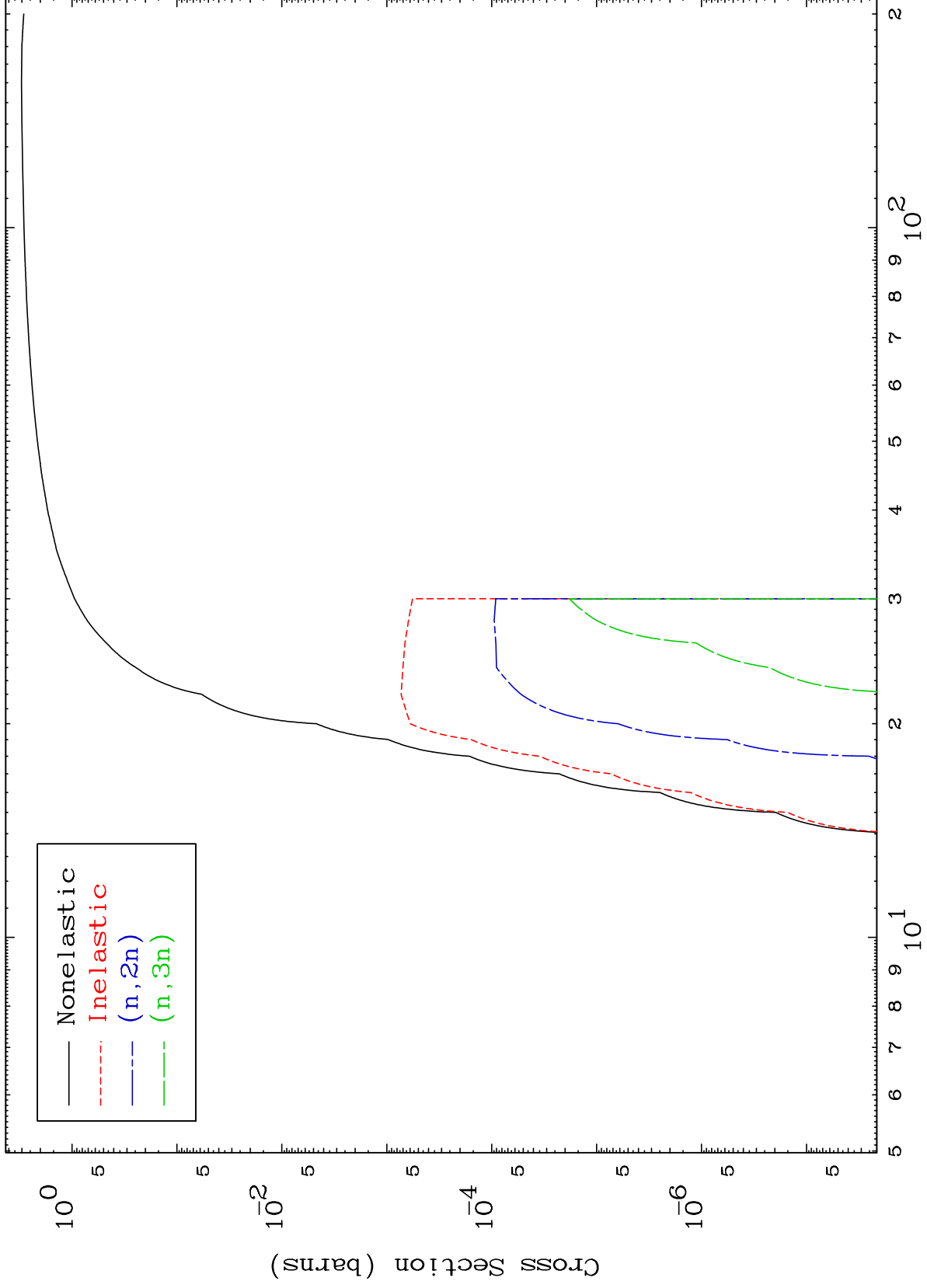
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Press Mouse Button to Start

MAT 9361

0 Kelvin Cross Sections

93-Np-242



1

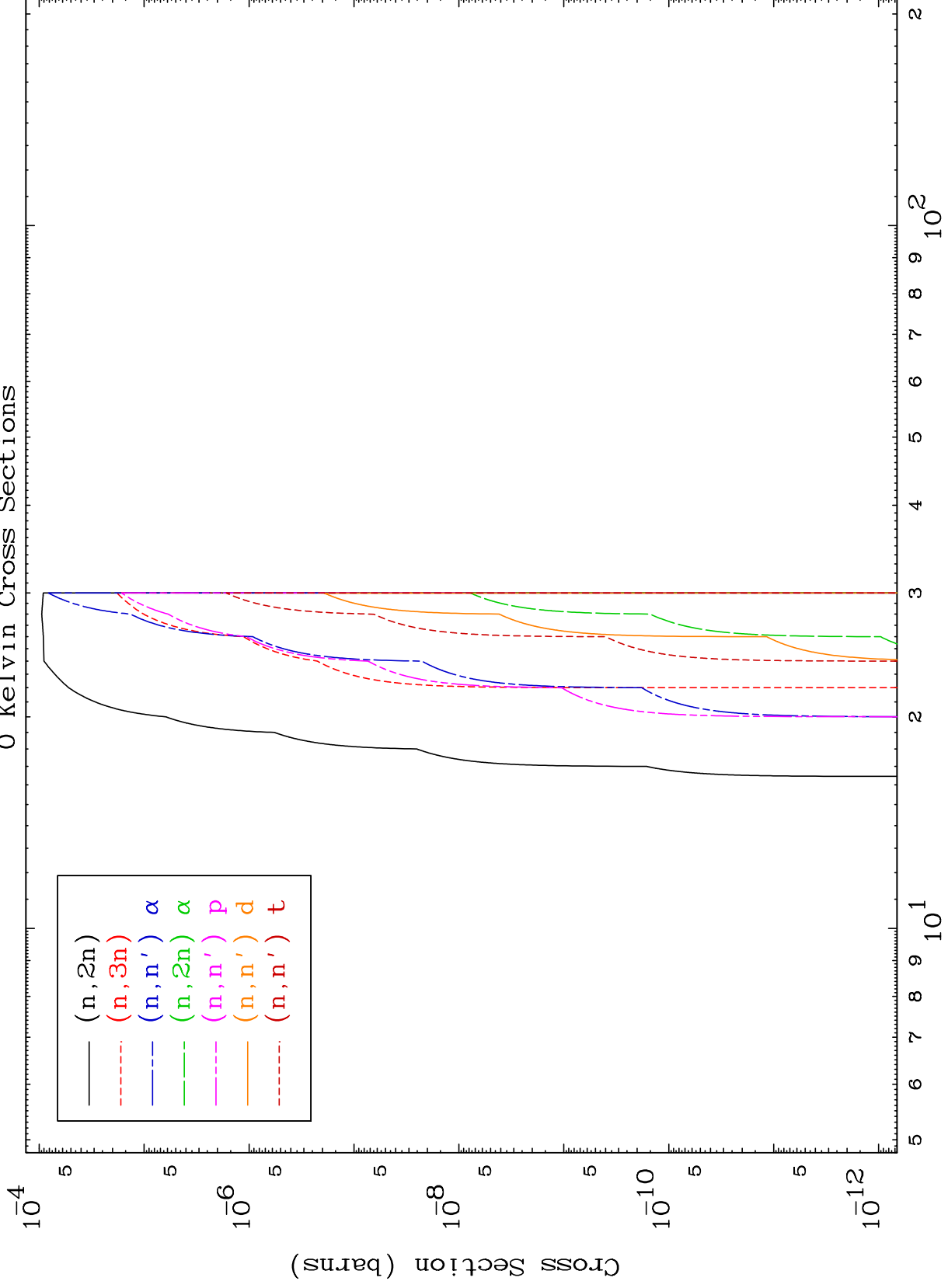
Incident Energy (MeV)

93-Np-242

MAT 9361

α Neutron Absorption
0 Kelvin Cross Sections

93-Np-242



2

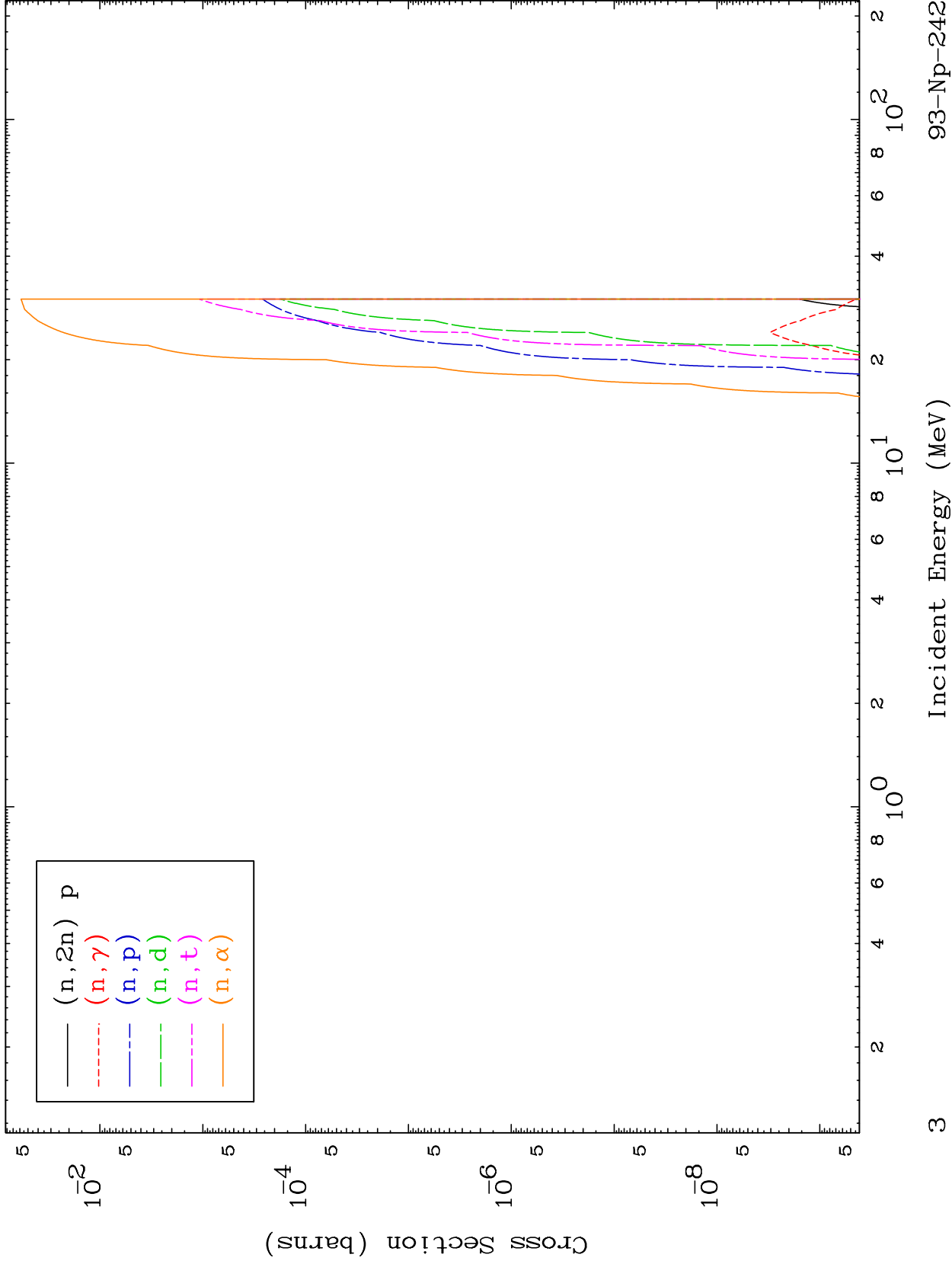
Incident Energy (MeV)

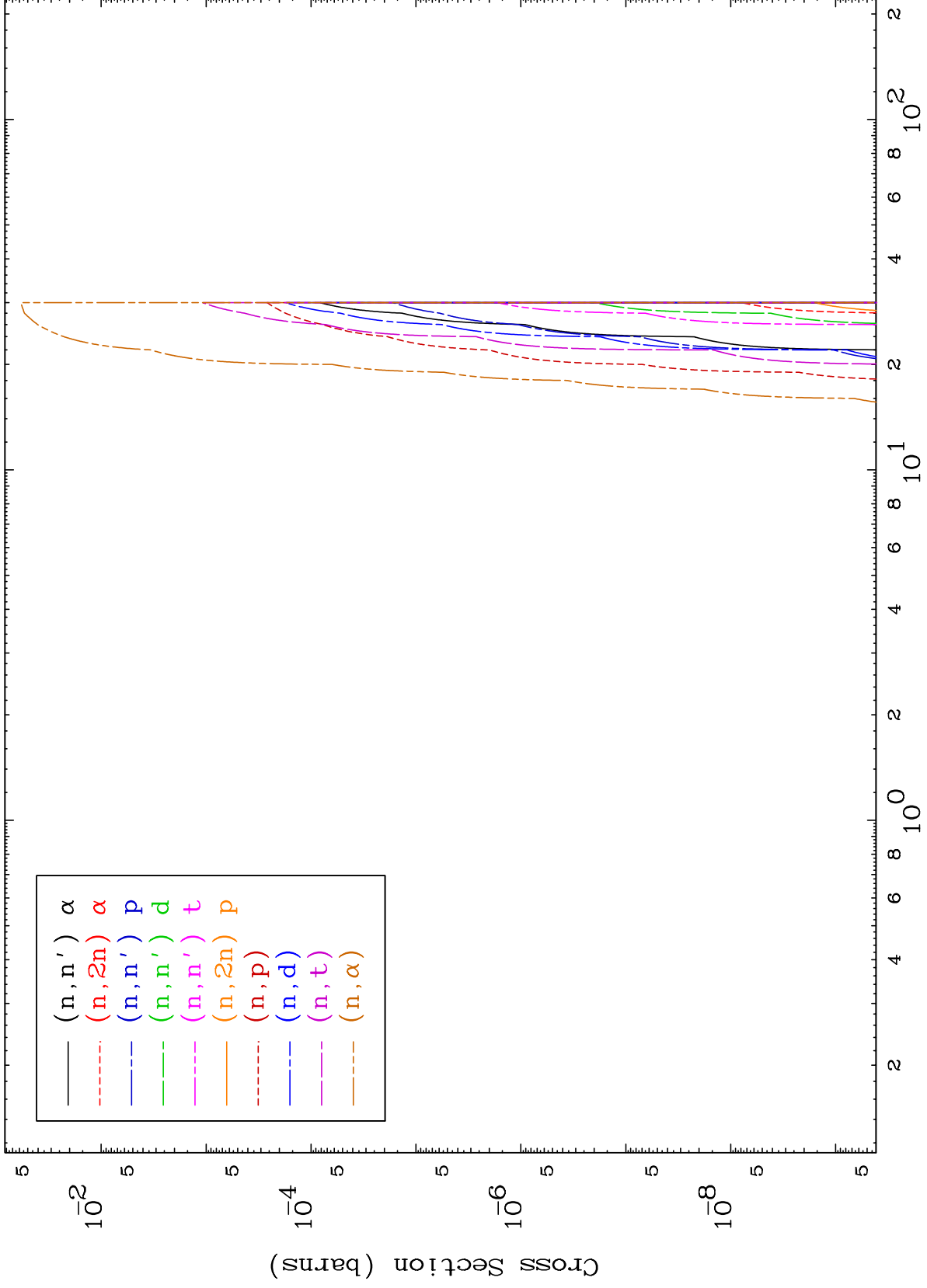
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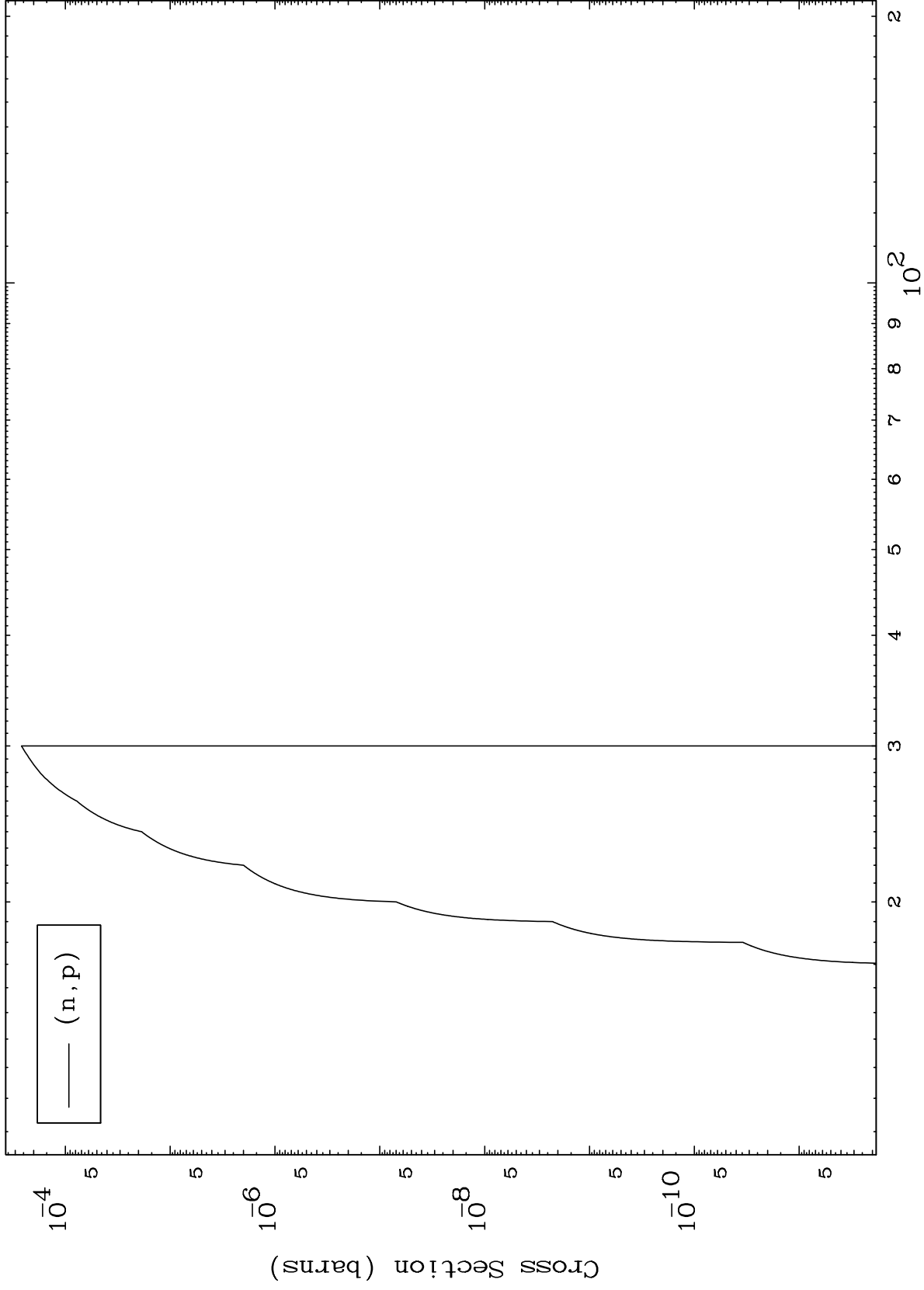
MAT 9361

α Neutron Absorption
0 Kelvin Cross Sections

93-Np-242



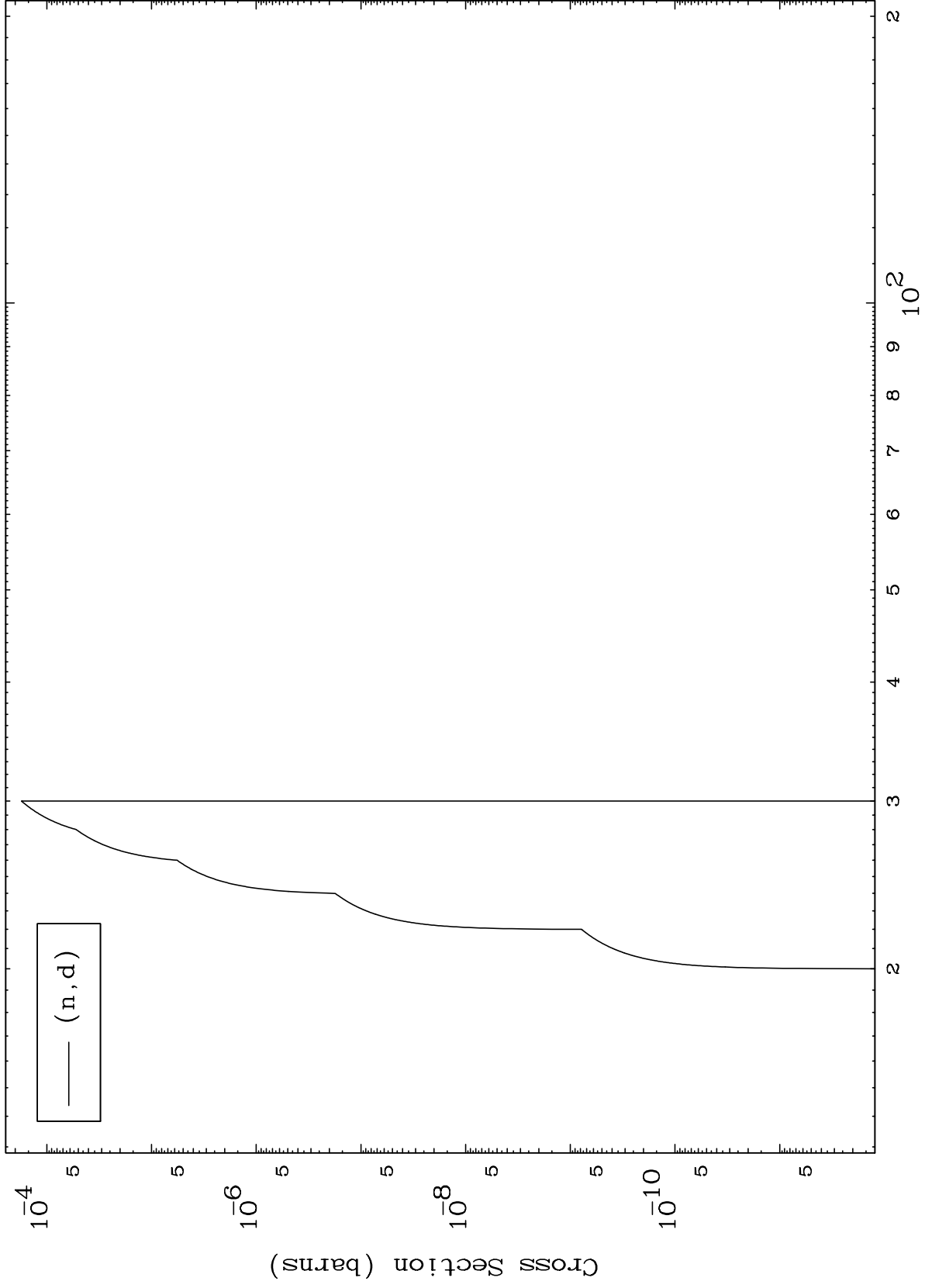




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(α, d) Levels
0 Kelvin Cross Sections

93-Np-242



6

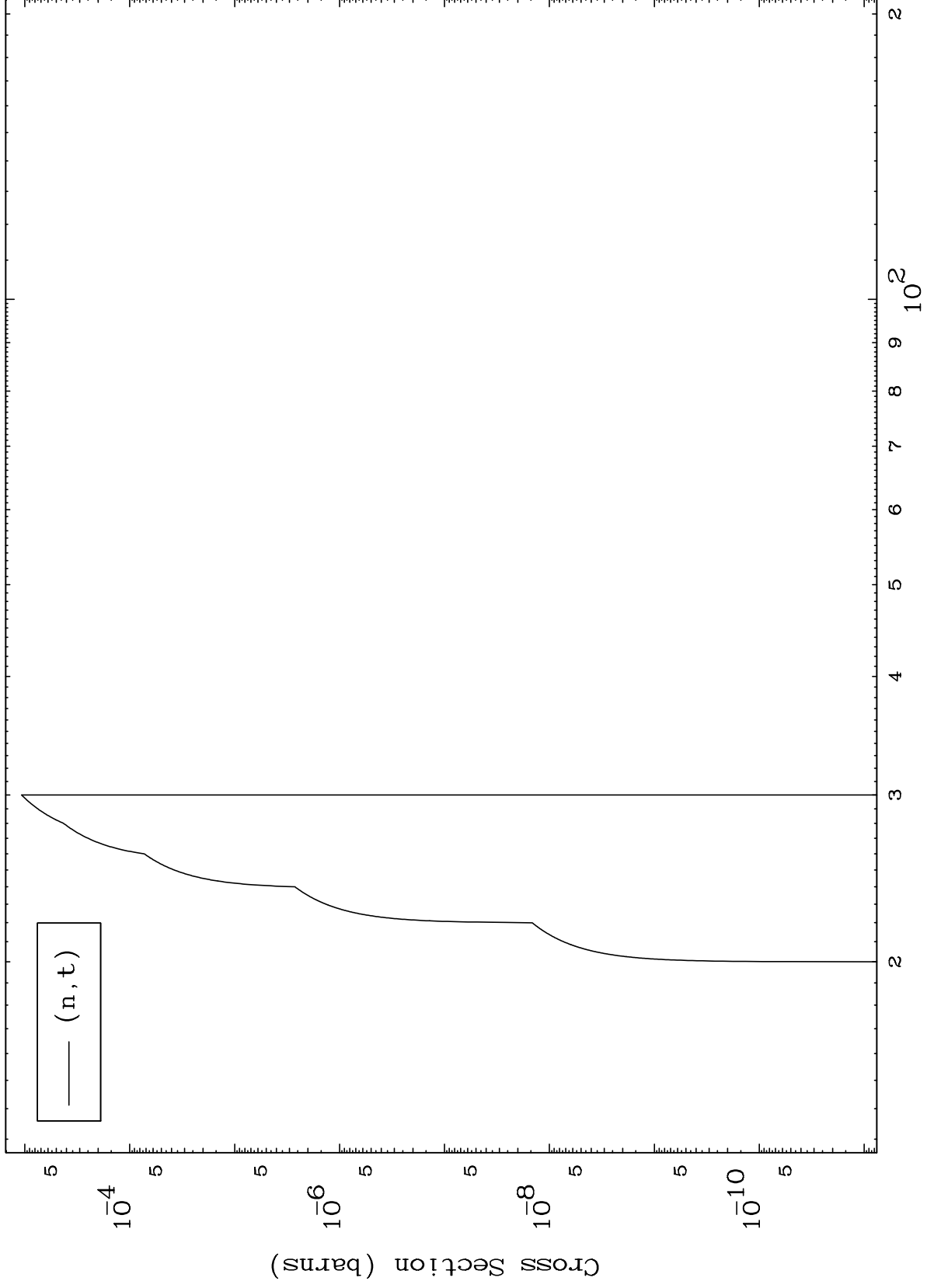
Incident Energy (MeV)

93-Np-242

MAT 9361

(α, t) Levels
0 Kelvin Cross Sections

93-Np-242

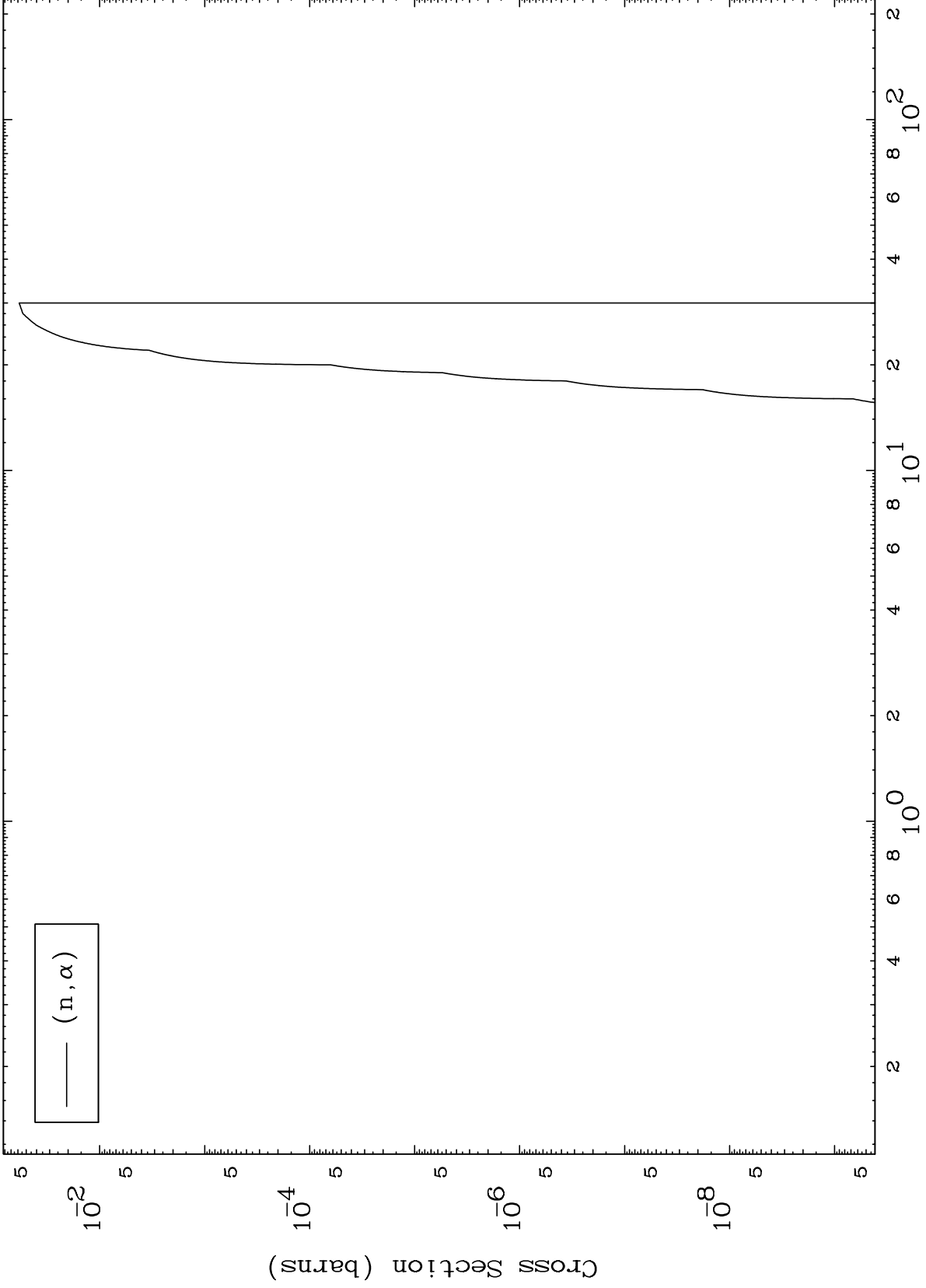


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(α, α) Levels

93-Np-242

0 Kelvin Cross Sections

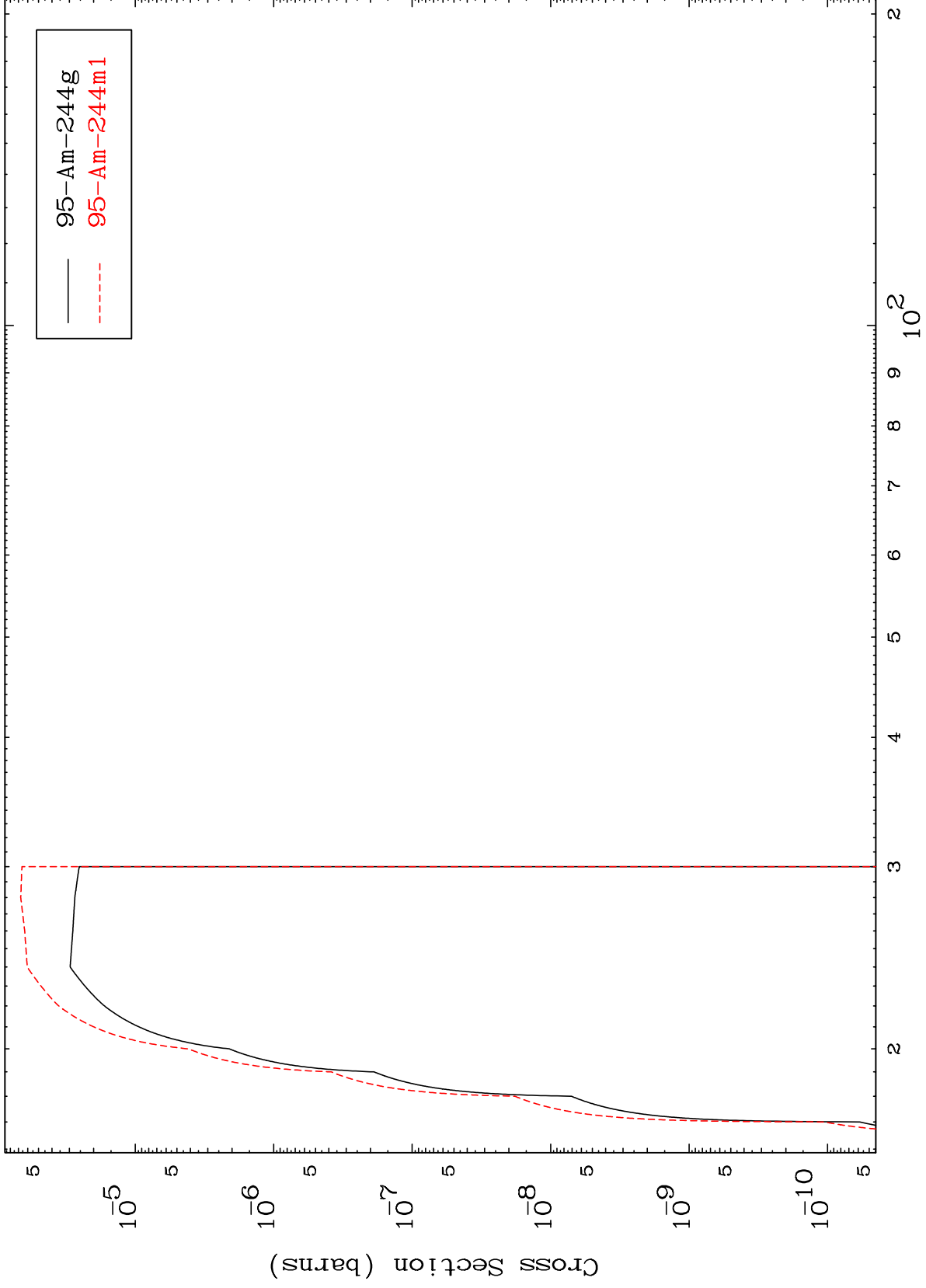


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(n,2n)

93-Np-242

Radionuclide Production Cross Section



9

Incident Energy (MeV)

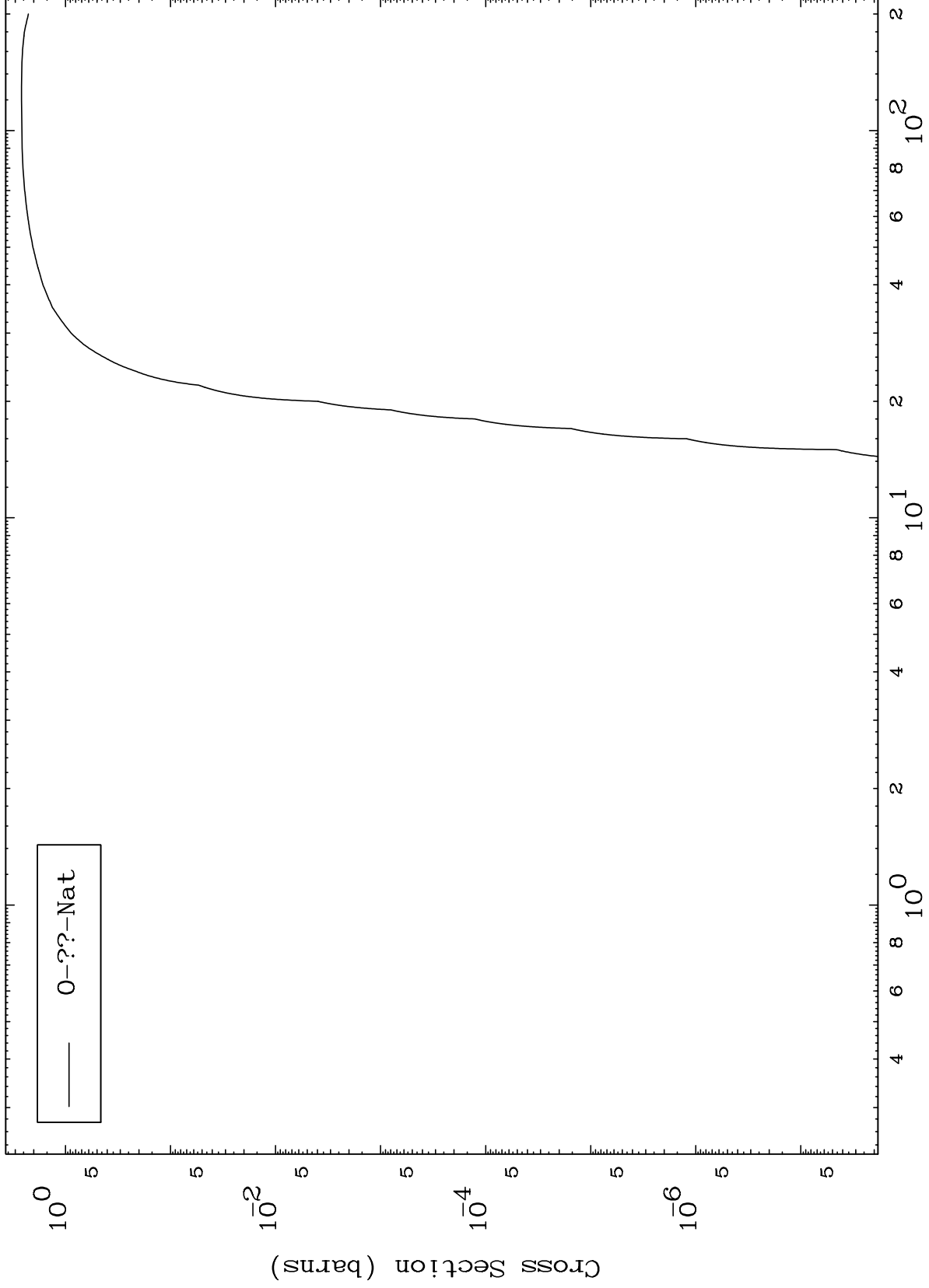
93-Np-242

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Fission

93-Np-242

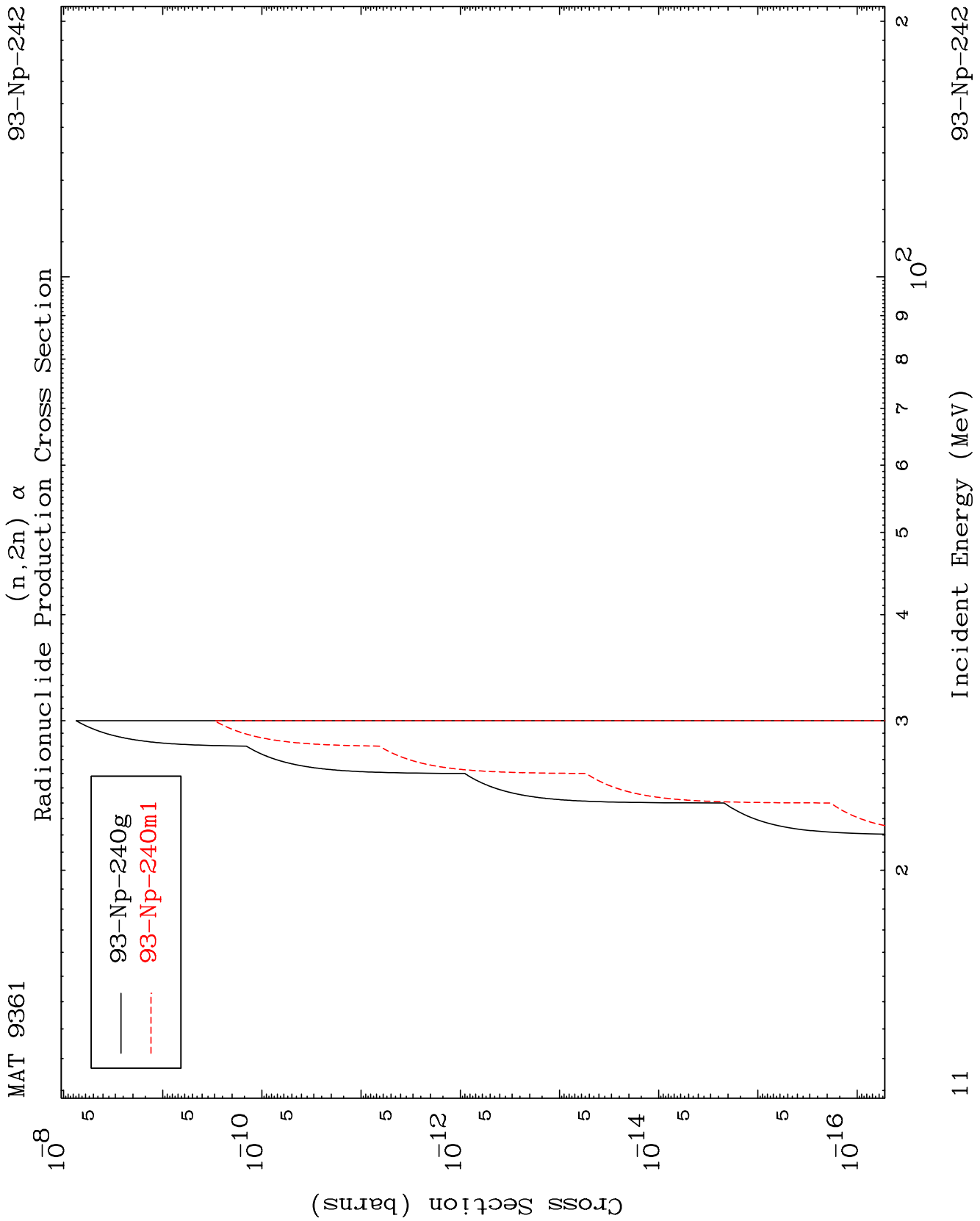
Radionuclide Production Cross Section



10

Incident Energy (MeV)

93-Np-242

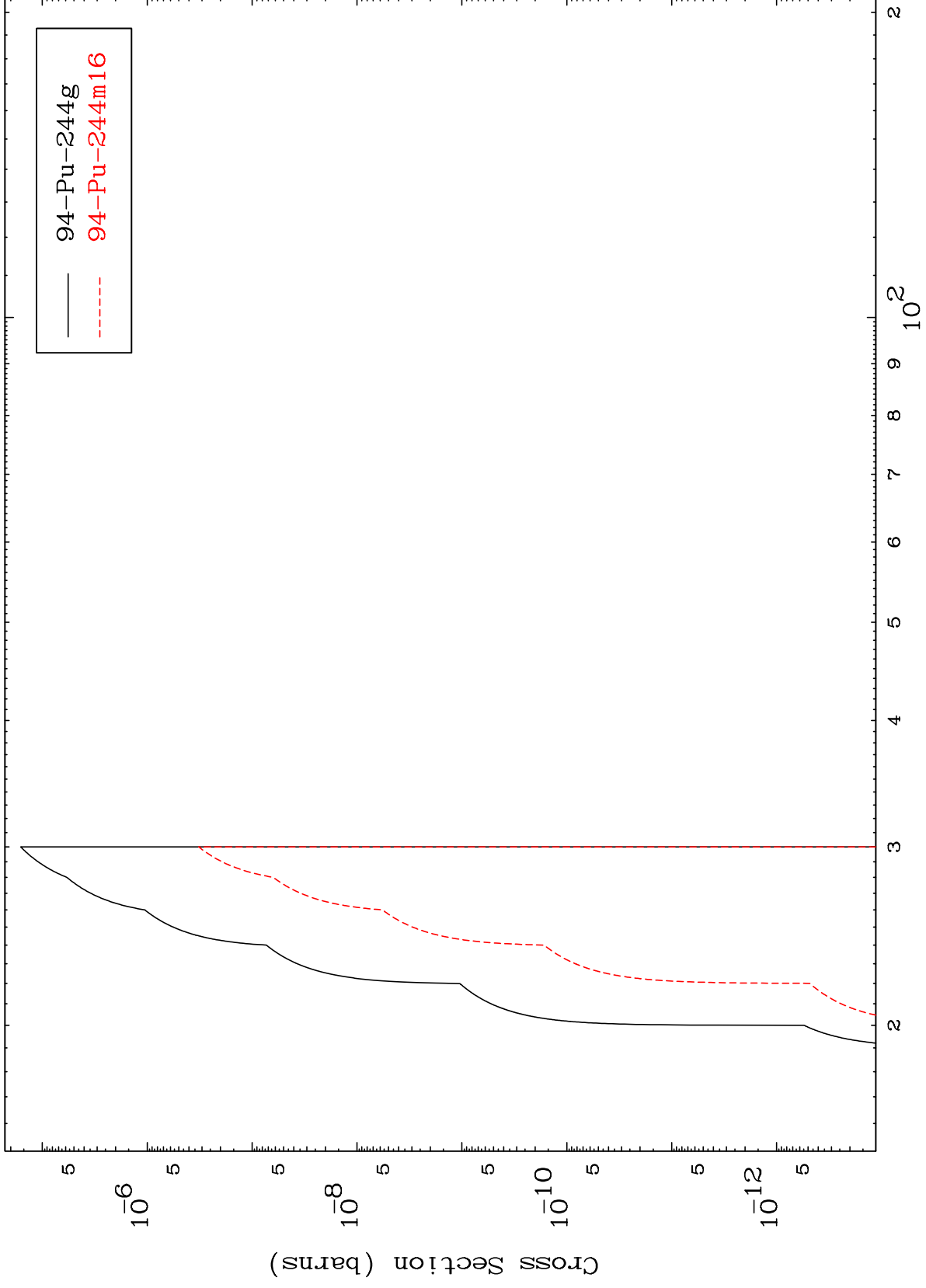


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(n,n') p

93-Np-242

Radionuclide Production Cross Section



12

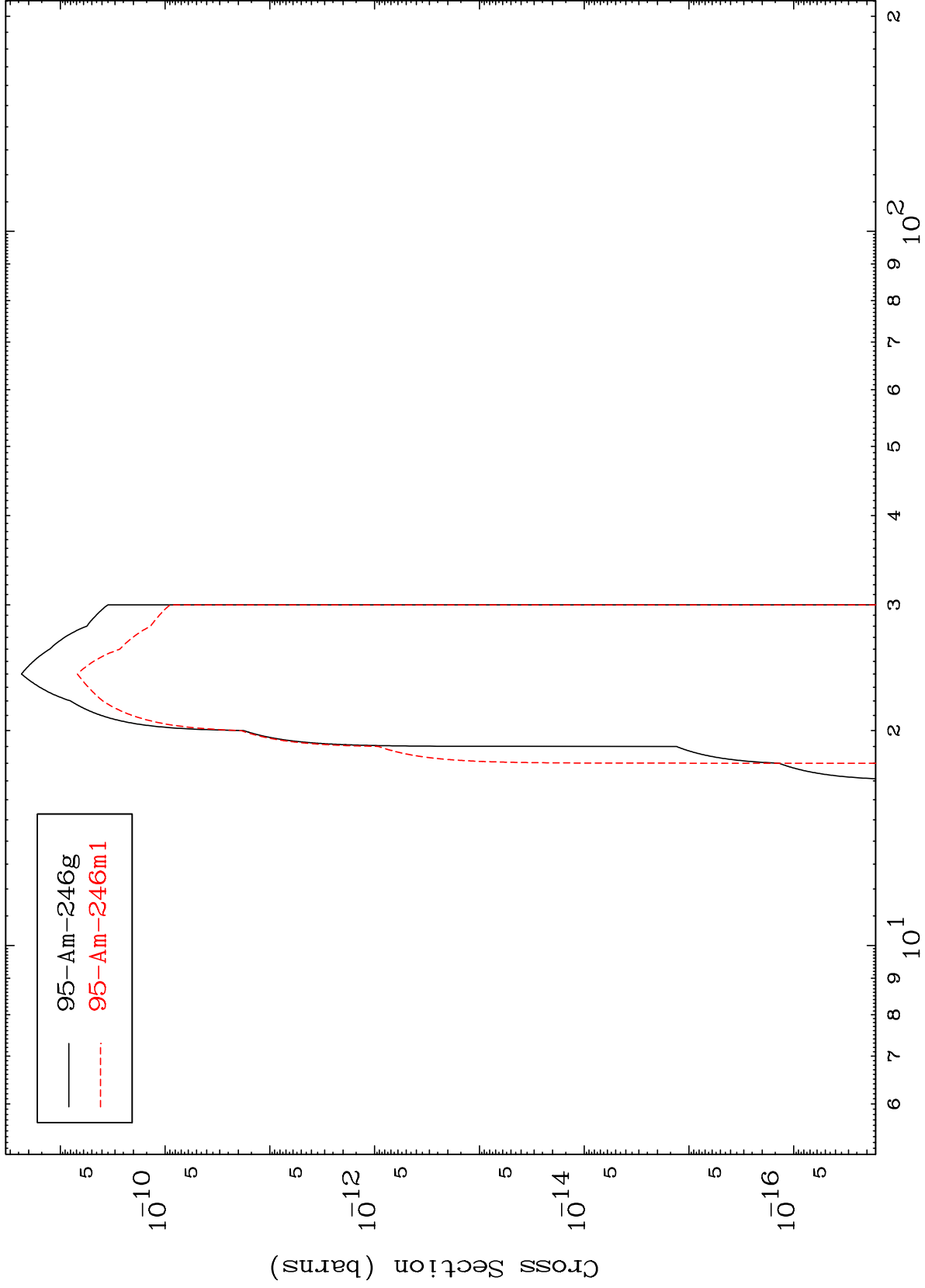
Incident Energy (MeV)

93-Np-242

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93-Np-242

(n, γ)
Radionuclide Production Cross Section



13

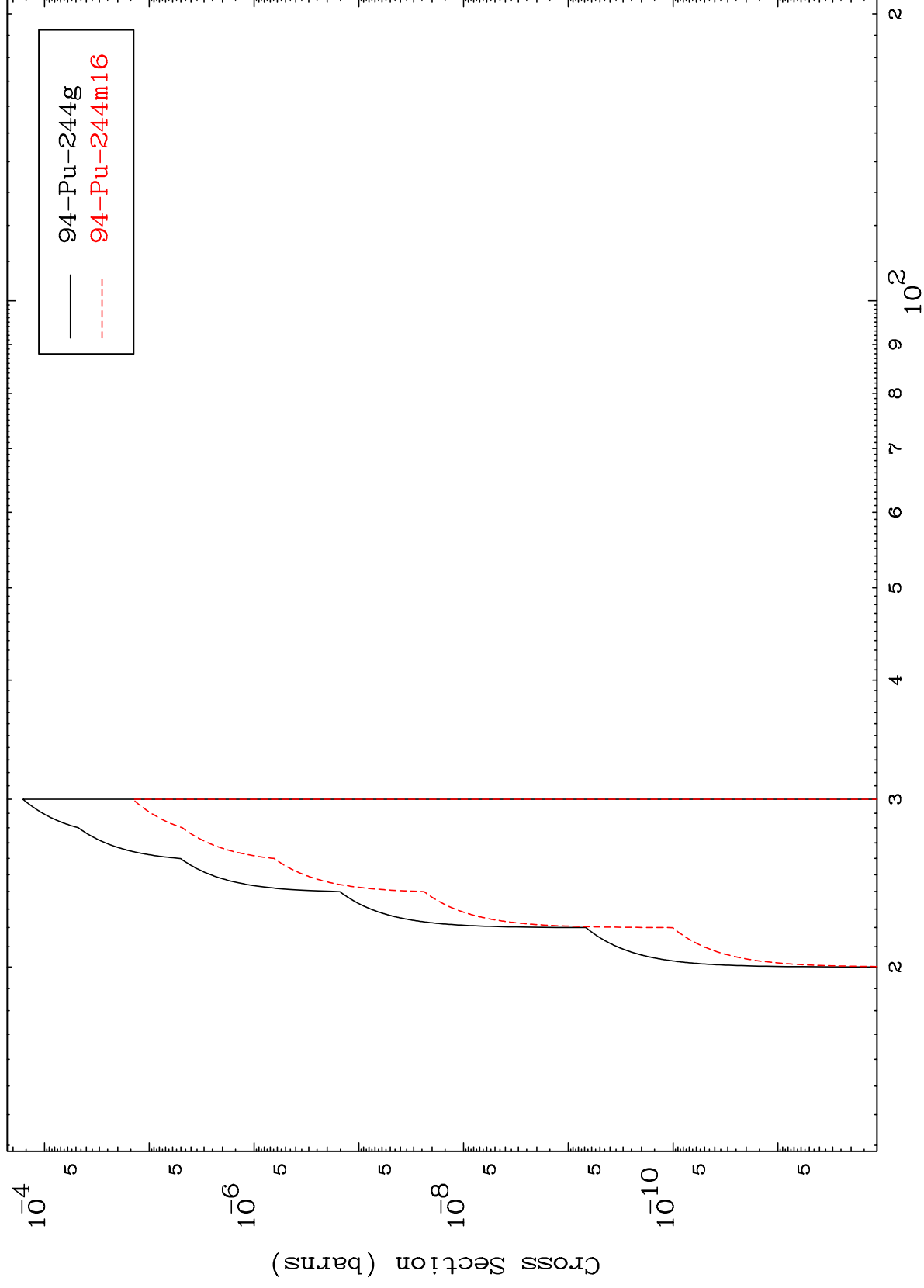
Incident Energy (MeV)

93-Np-242

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93-Np-242

(n,d)
Radionuclide Production Cross Section



14

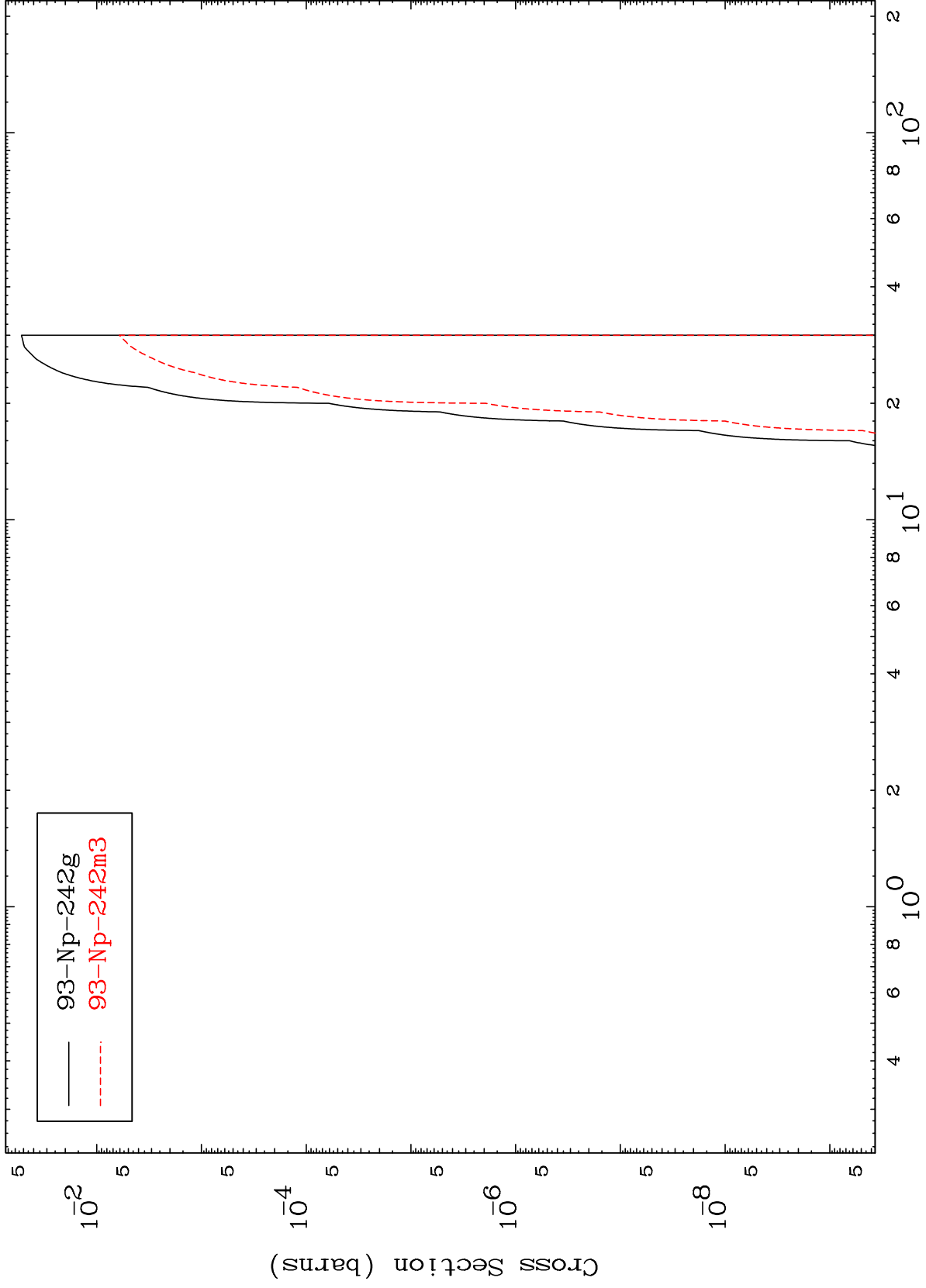
Incident Energy (MeV)

93-Np-242

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93-Np-242

(n, α)
Radionuclide Production Cross Section



15

Incident Energy (MeV)

93-Np-242