

Program EVALPLOT
(Version 2021-1)

by

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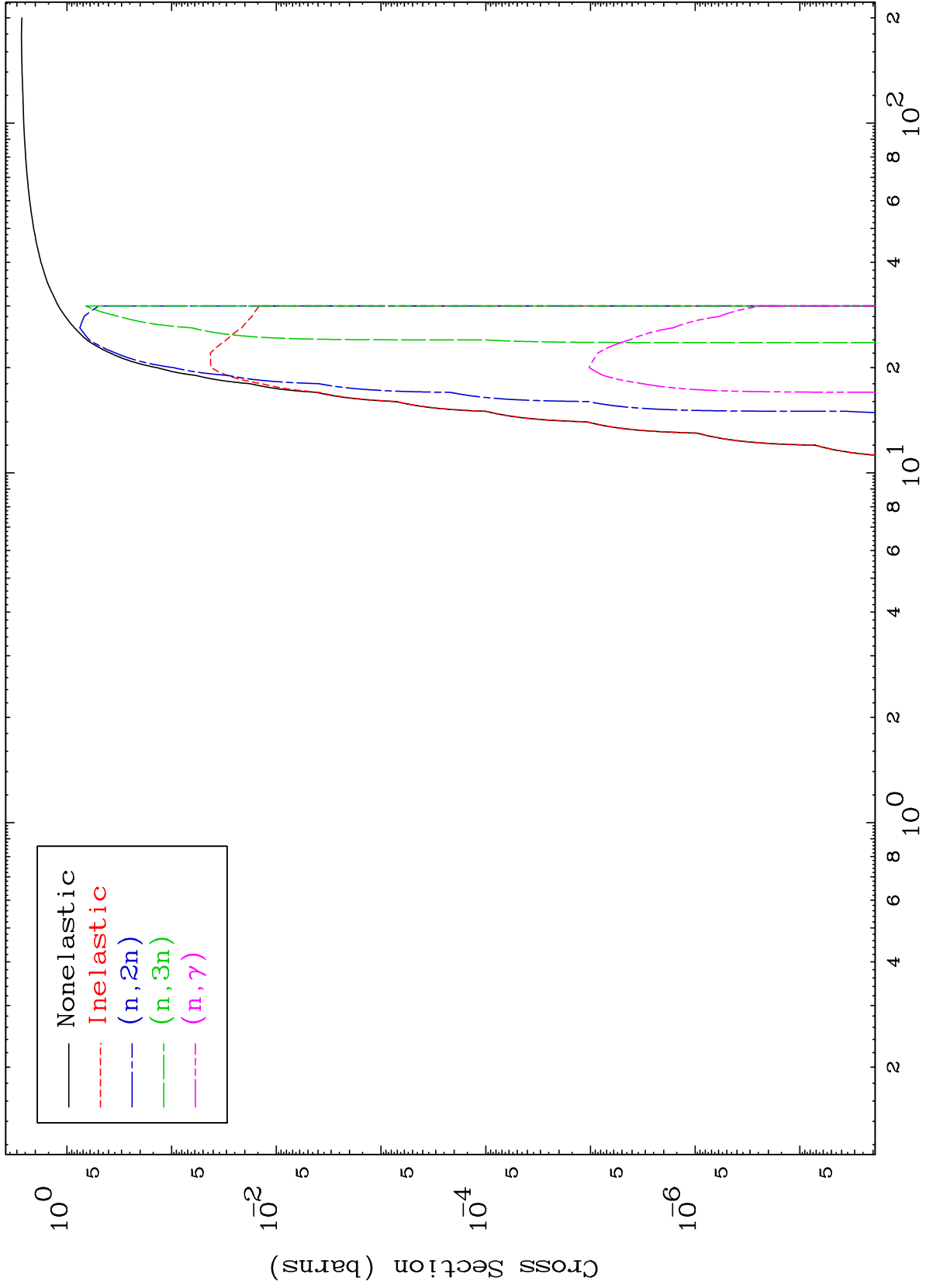
Press Mouse Button to Start

MAT 7644

α Major

76-0s-190m

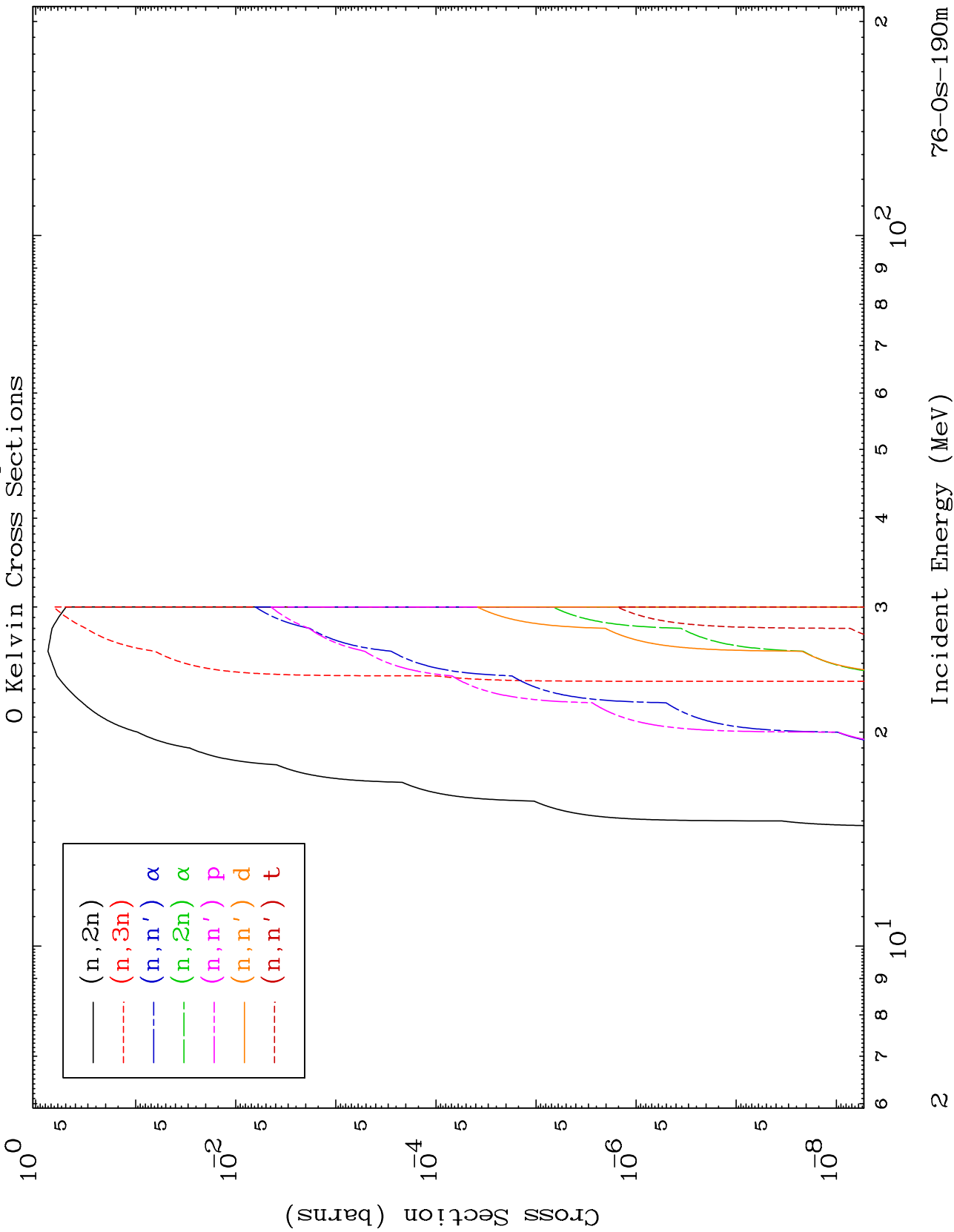
0 Kelvin Cross Sections



MAT 7644

α Neutron Absorption
0 Kelvin Cross Sections

76-0s-190m



76-0s-190m

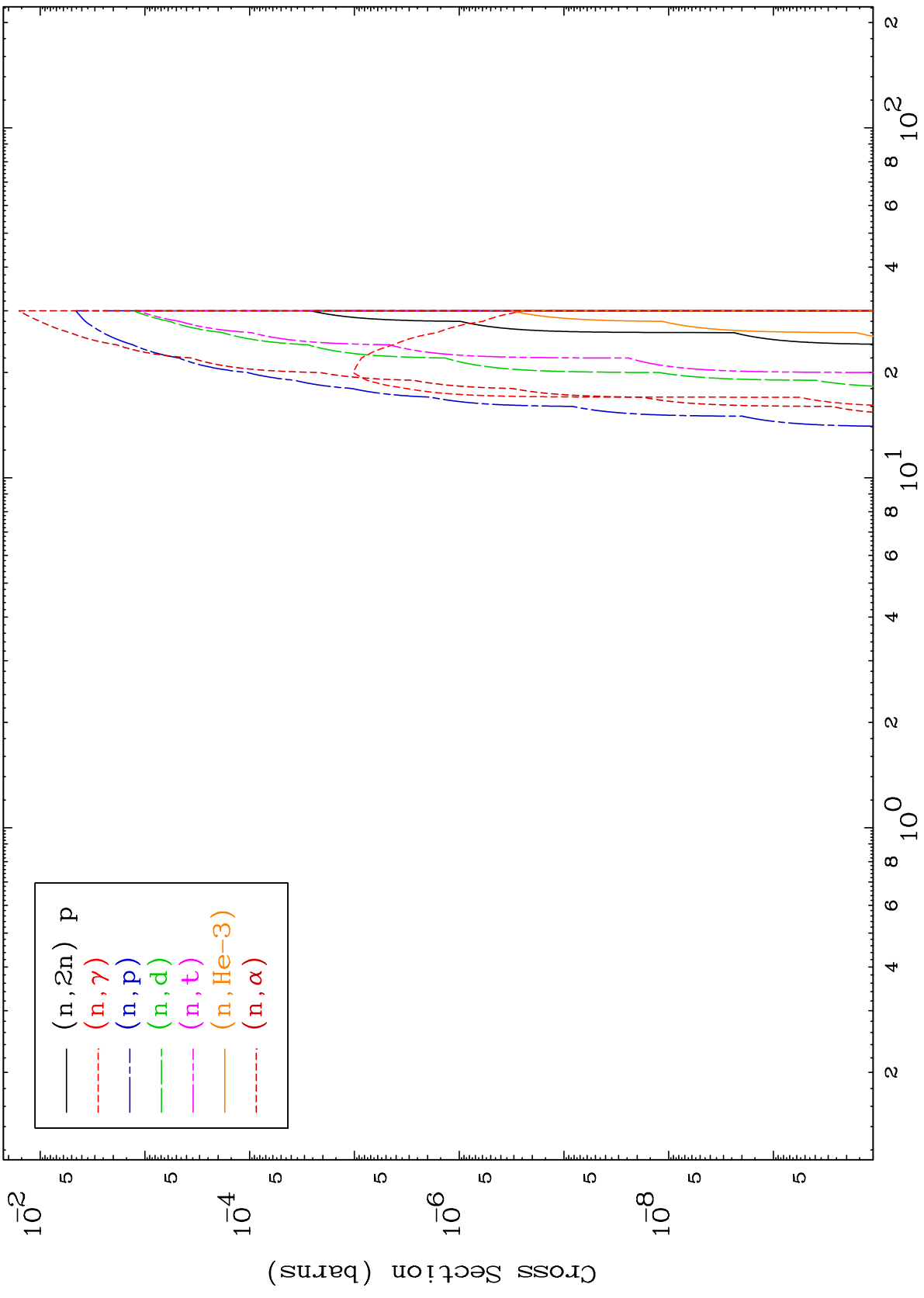
Incident Energy (MeV)

2

MAT 7644

α Neutron Absorption
0 Kelvin Cross Sections

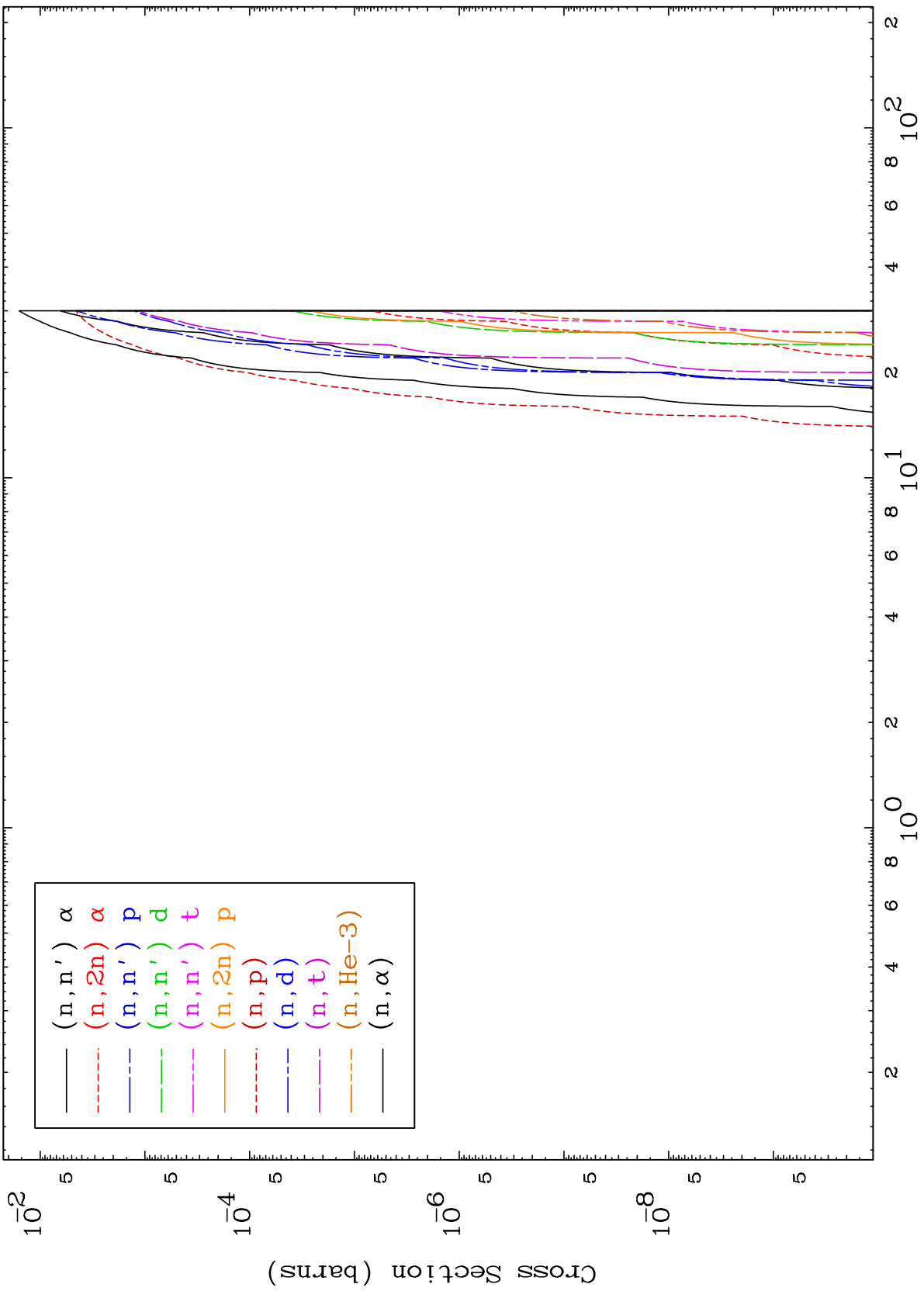
76-0s-190m



MAT 7644

α Charged Particle
0 Kelvin Cross Sections

76-0s-190m



Incident Energy (MeV)

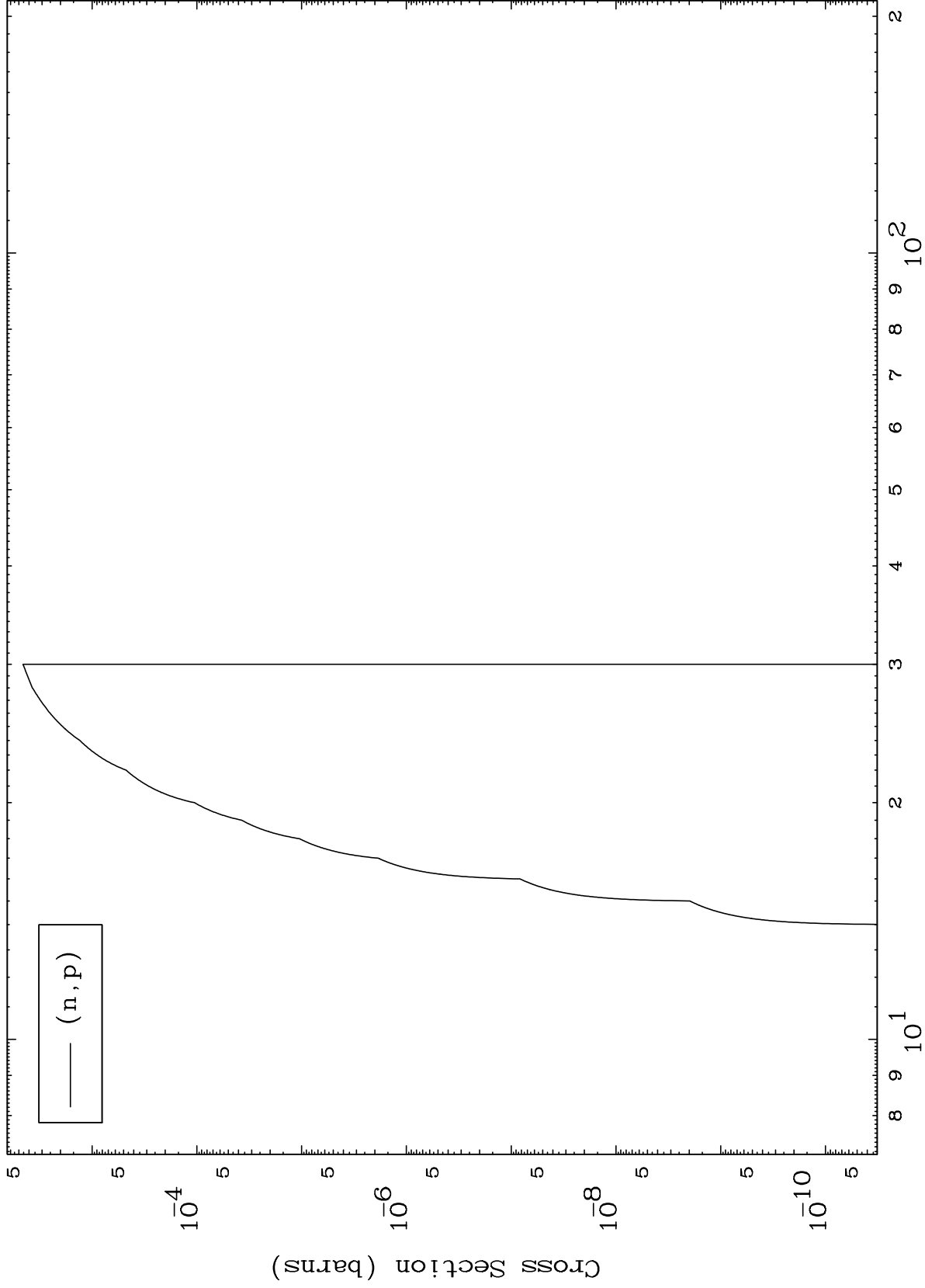
76-0s-190m

MAT 7644

(α, p) Levels

76-0s-190m

0 Kelvin Cross Sections



Incident Energy (MeV)

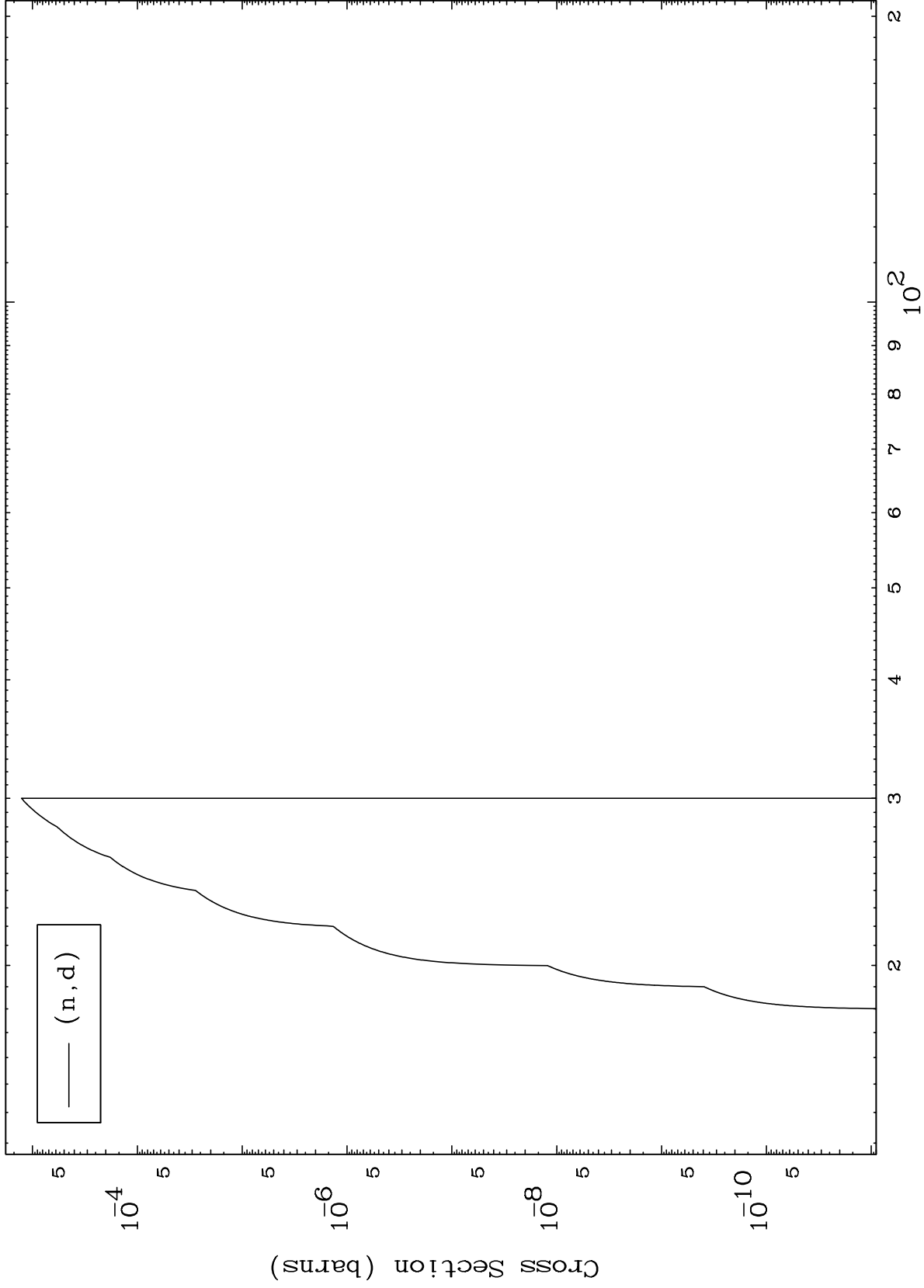
76-0s-190m

5

MAT 7644

(α, d) Levels
0 Kelvin Cross Sections

76-0s-190m



6

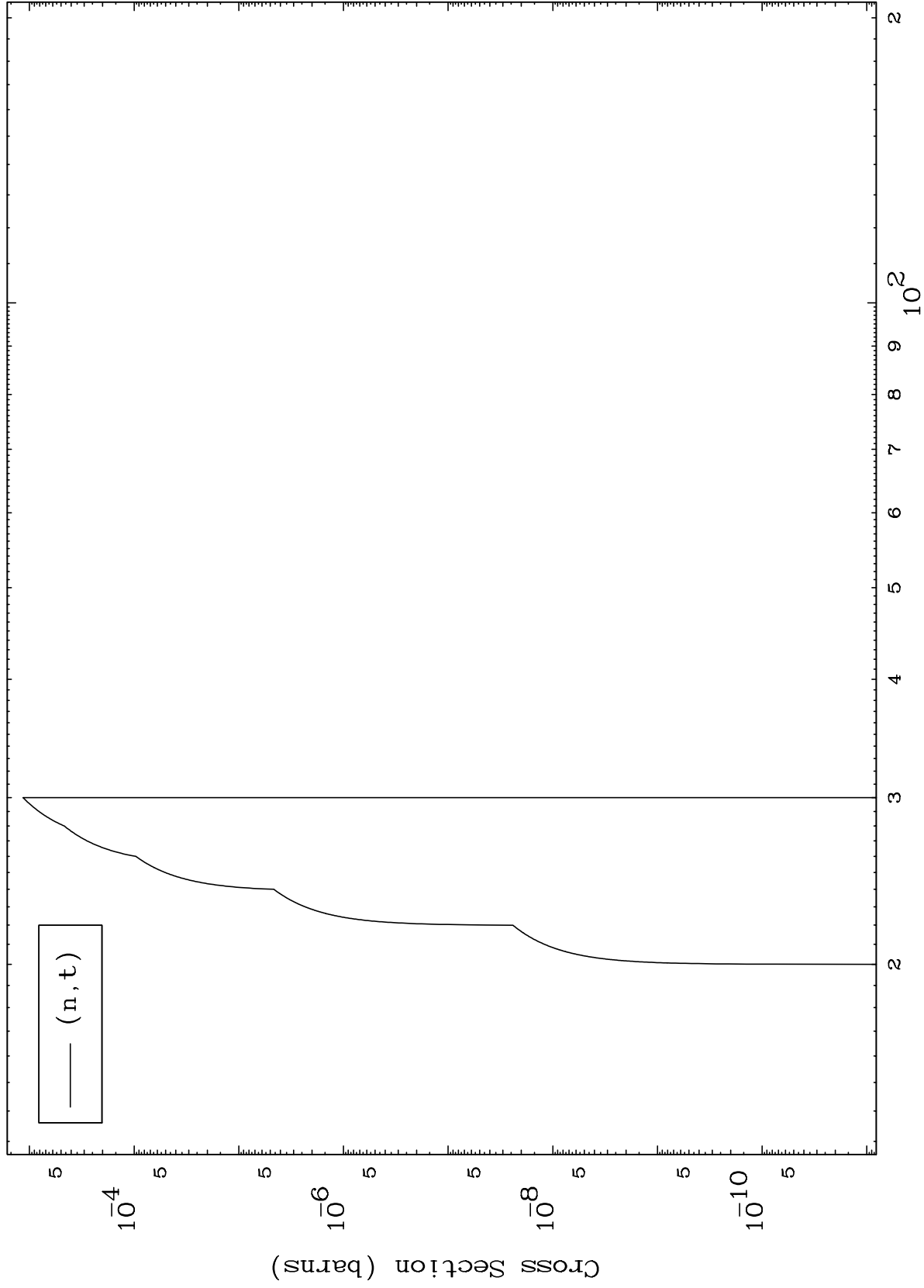
Incident Energy (MeV)

76-0s-190m

MAT 7644

(α, t) Levels
0 Kelvin Cross Sections

76-0s-190m



7

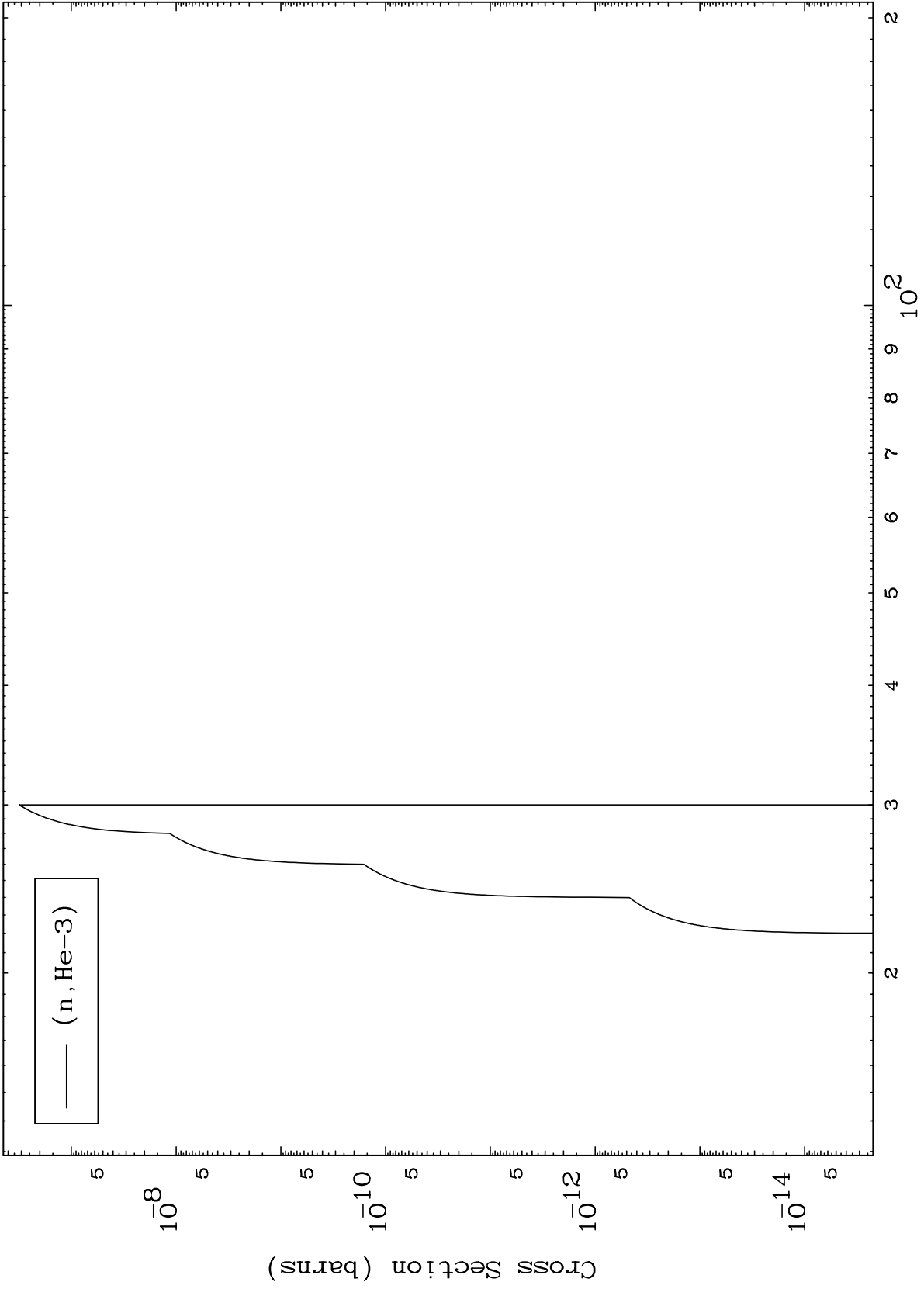
Incident Energy (MeV)

76-0s-190m

MAT 7644

($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

76-0s-190m

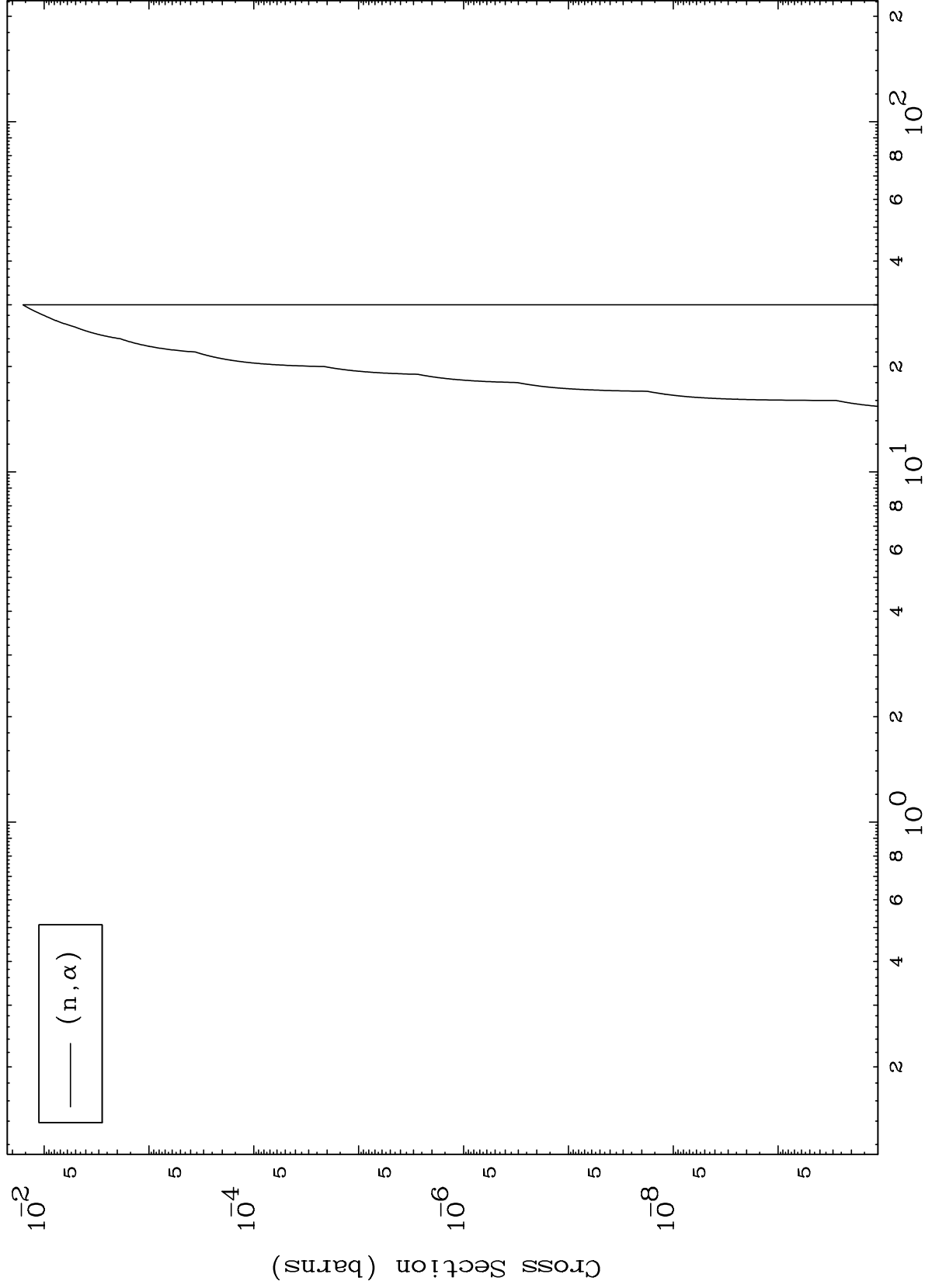


MAT 7644

(α, α) Levels

76-0s-190m

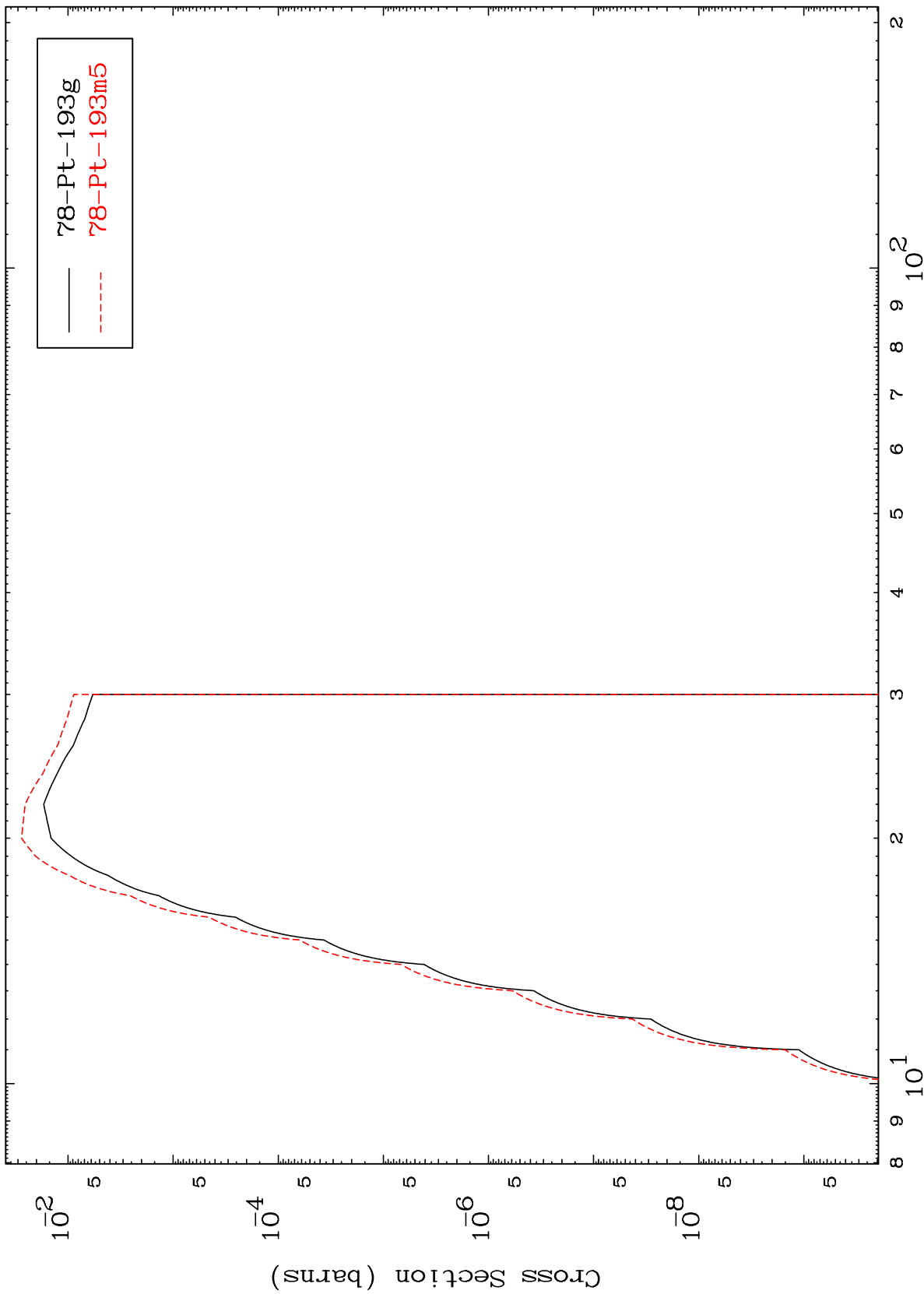
0 Kelvin Cross Sections



MAT 7644

76-0s-190m

Inelastic
Radionuclide Production Cross Section



76-0s-190m

Incident Energy (MeV)

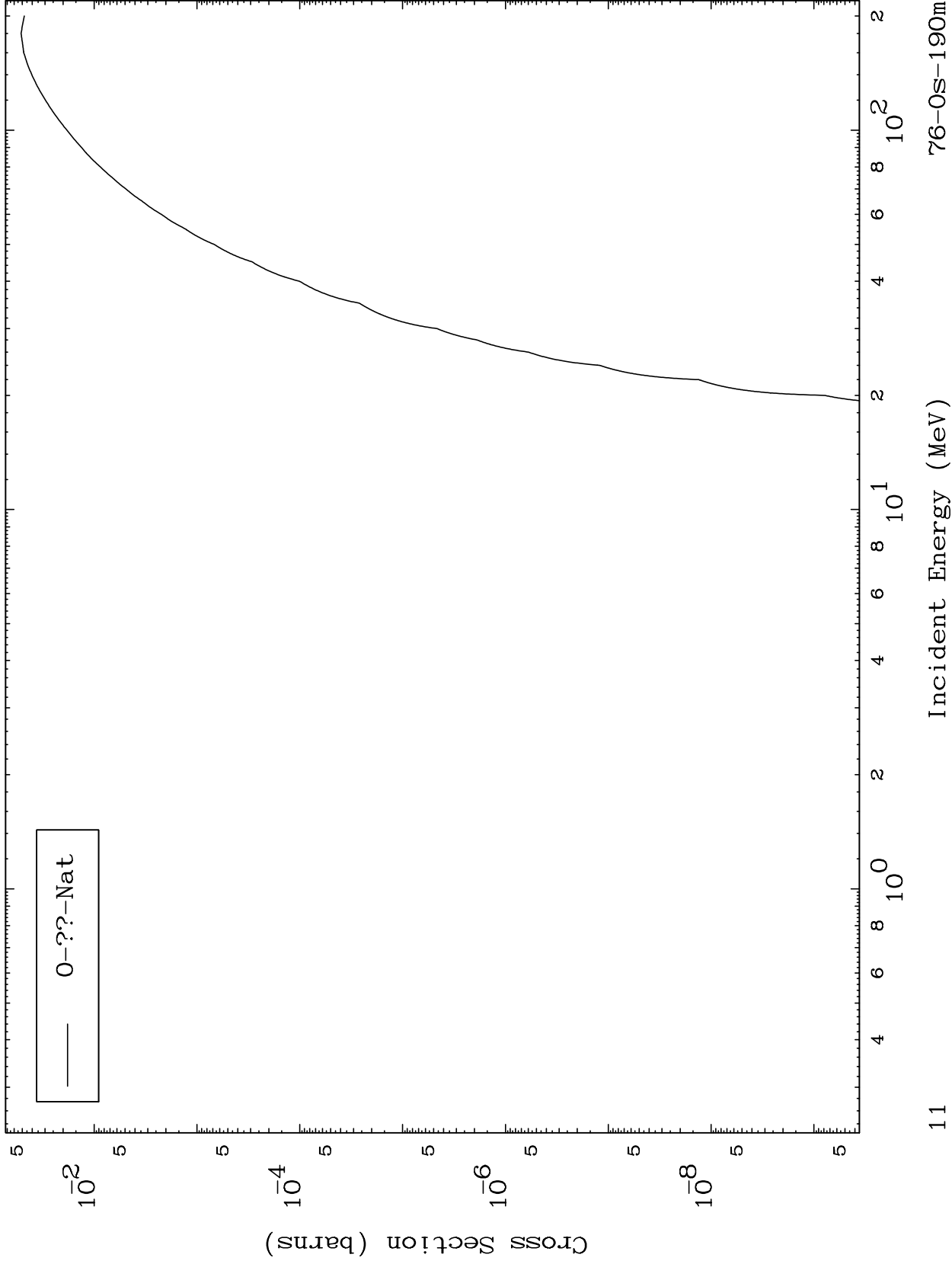
10

MAT 7644

Fission

76-0s-190m

Radionuclide Production Cross Section

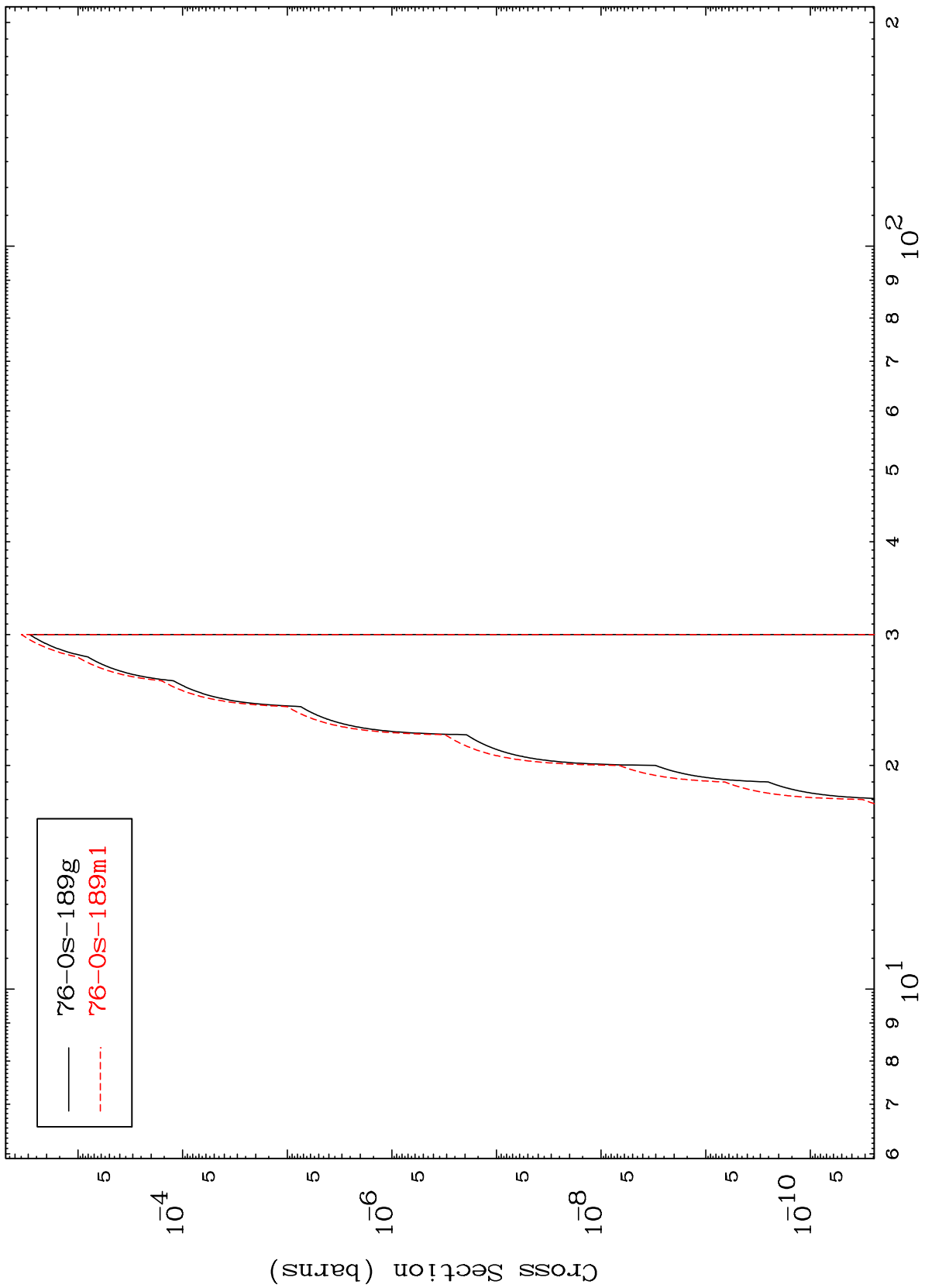


MAT 7644

(n,n') α

76-0s-190m

Radionuclide Production Cross Section

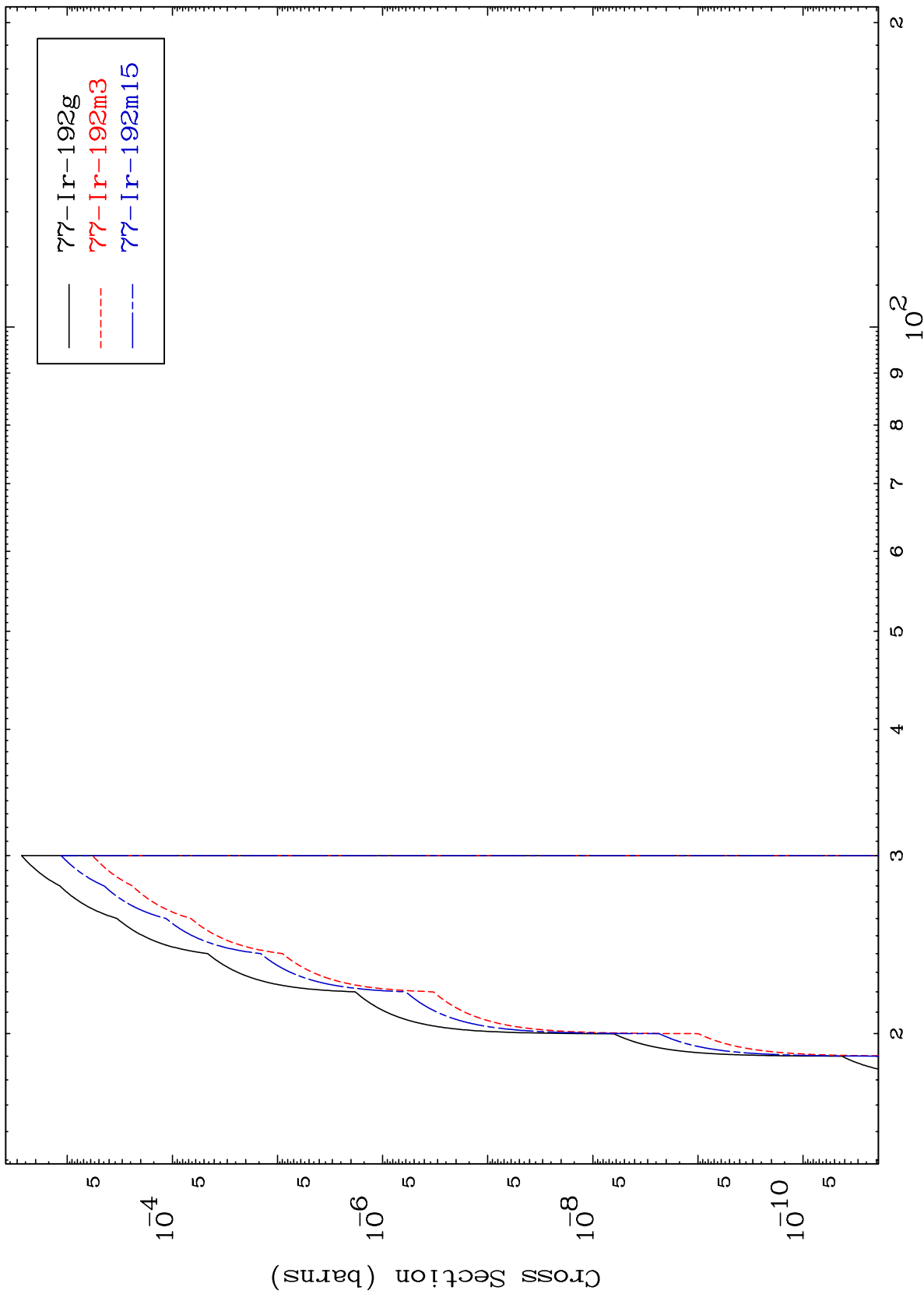


MAT 7644

(n,n') p

76-0s-190m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

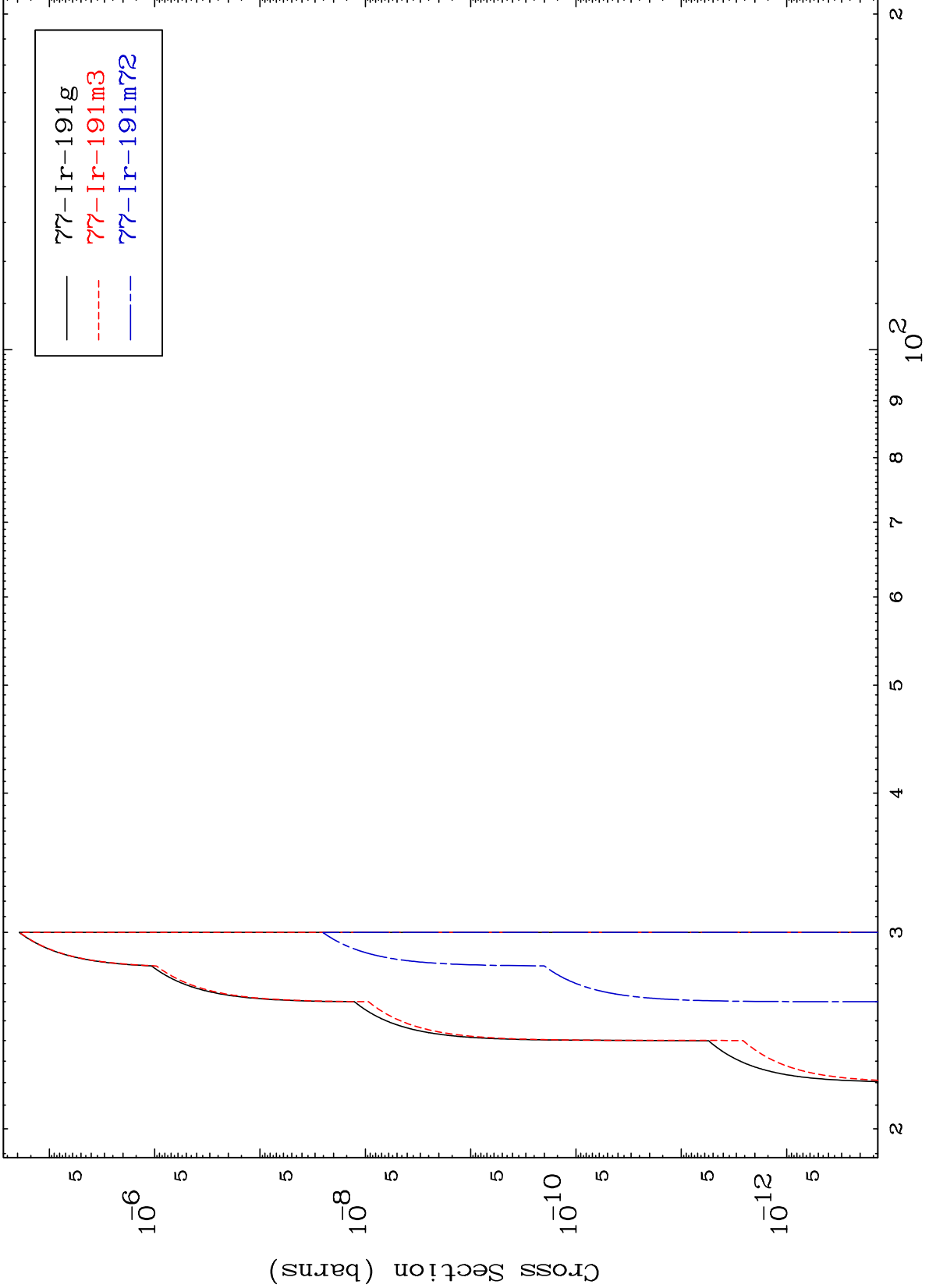
76-0s-190m

MAT 7644

(n,n') d

76-0s-190m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

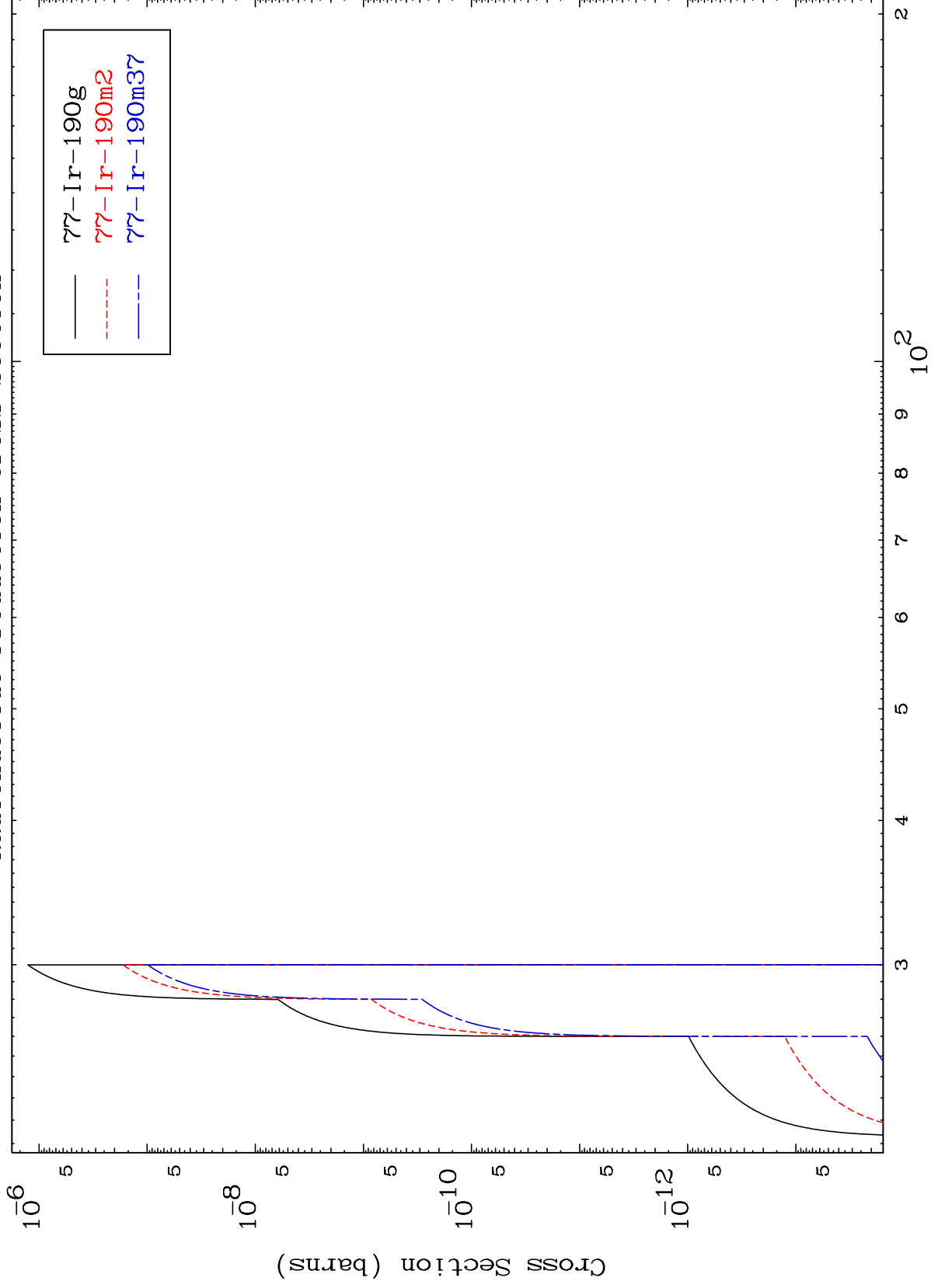
76-0s-190m

MAT 7644

(n,n') t

76-0s-190m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

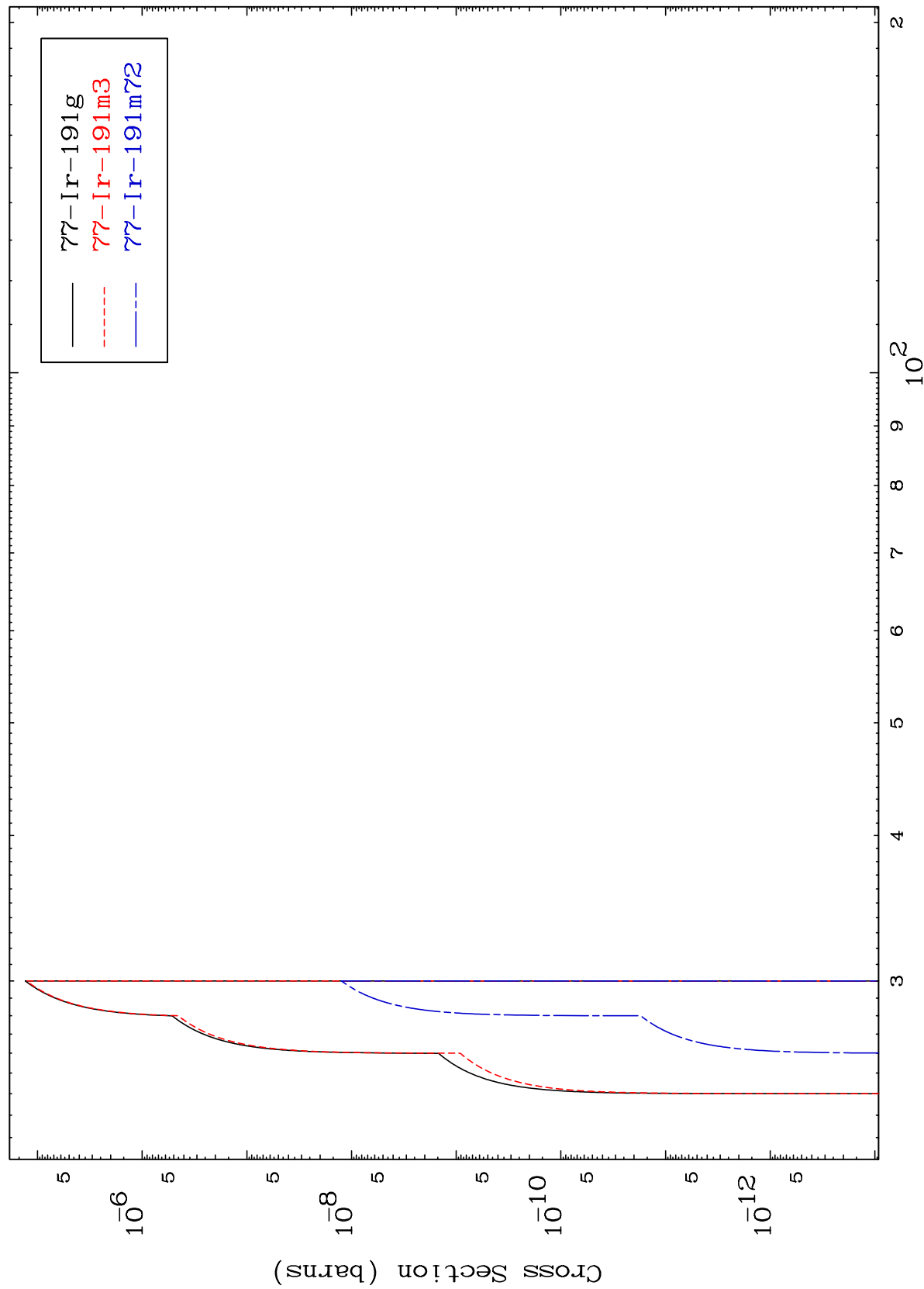
76-0s-190m

MAT 7644

(n,2n) p

76-0s-190m

Radionuclide Production Cross Section



16

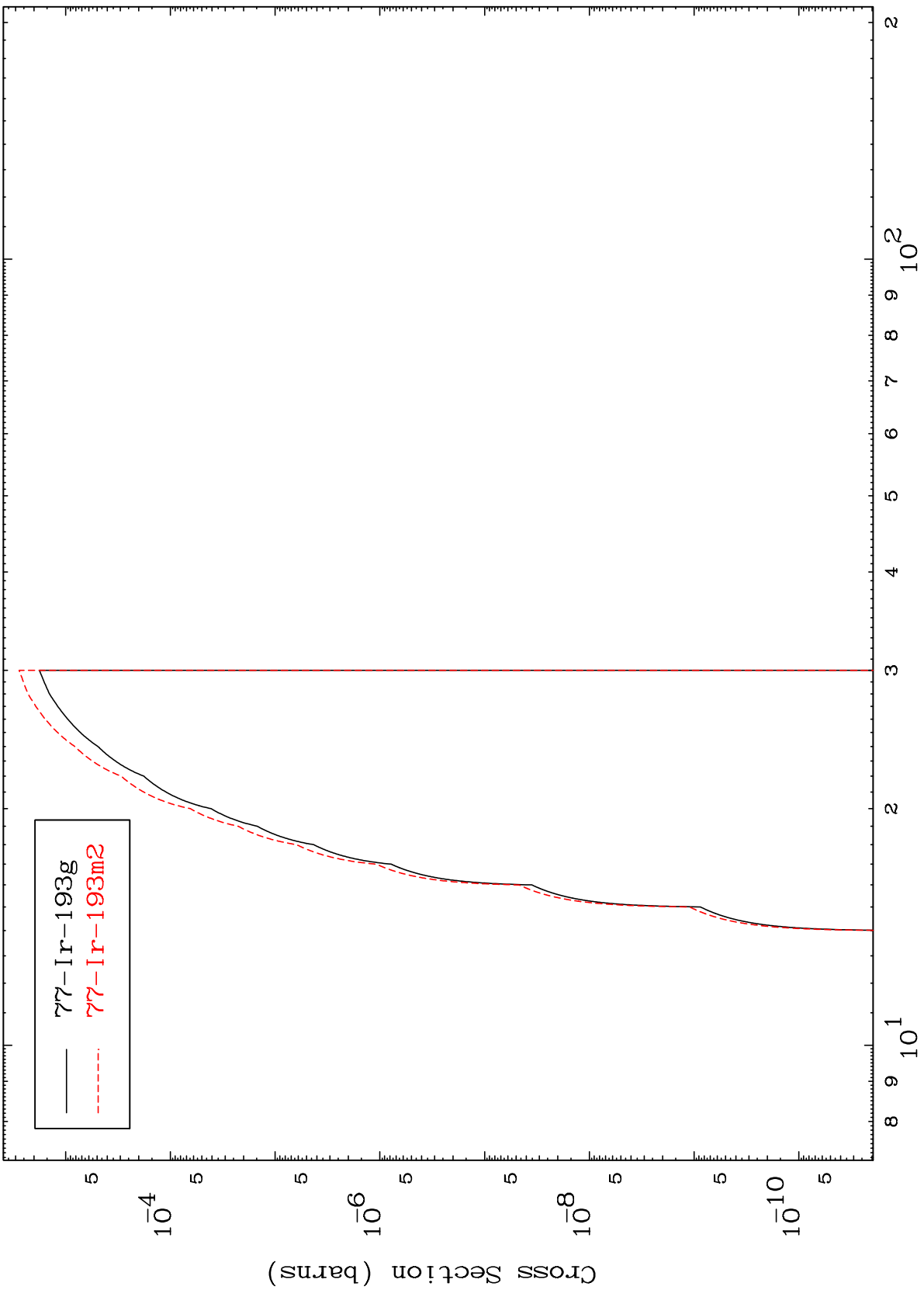
Incident Energy (MeV)

76-0s-190m

MAT 7644

76-0s-190m

Radionuclide Production Cross Section (n,p)



17

Incident Energy (MeV)

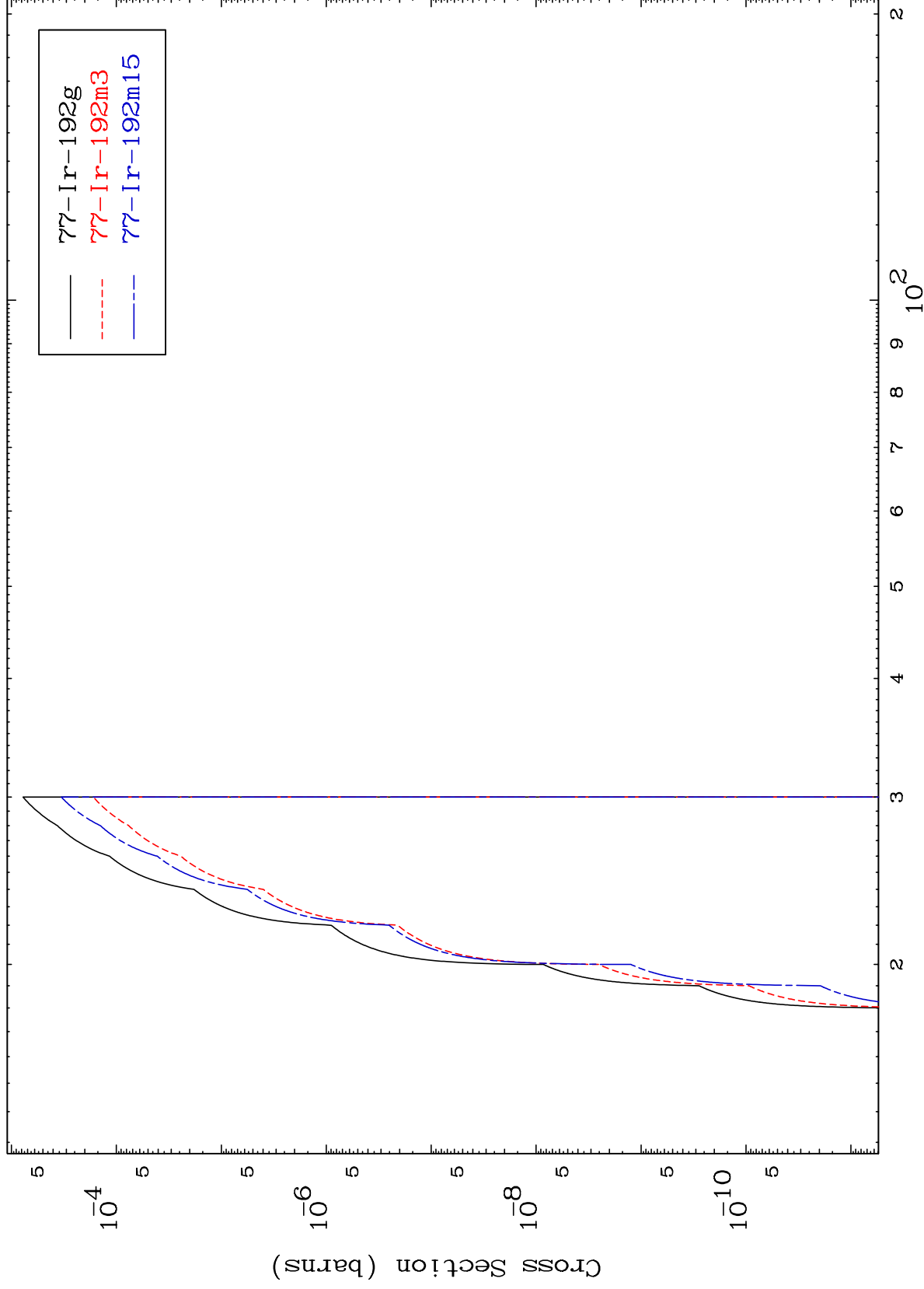
76-0s-190m

MAT 7644

(n,d)

76-0s-190m

Radionuclide Production Cross Section



18

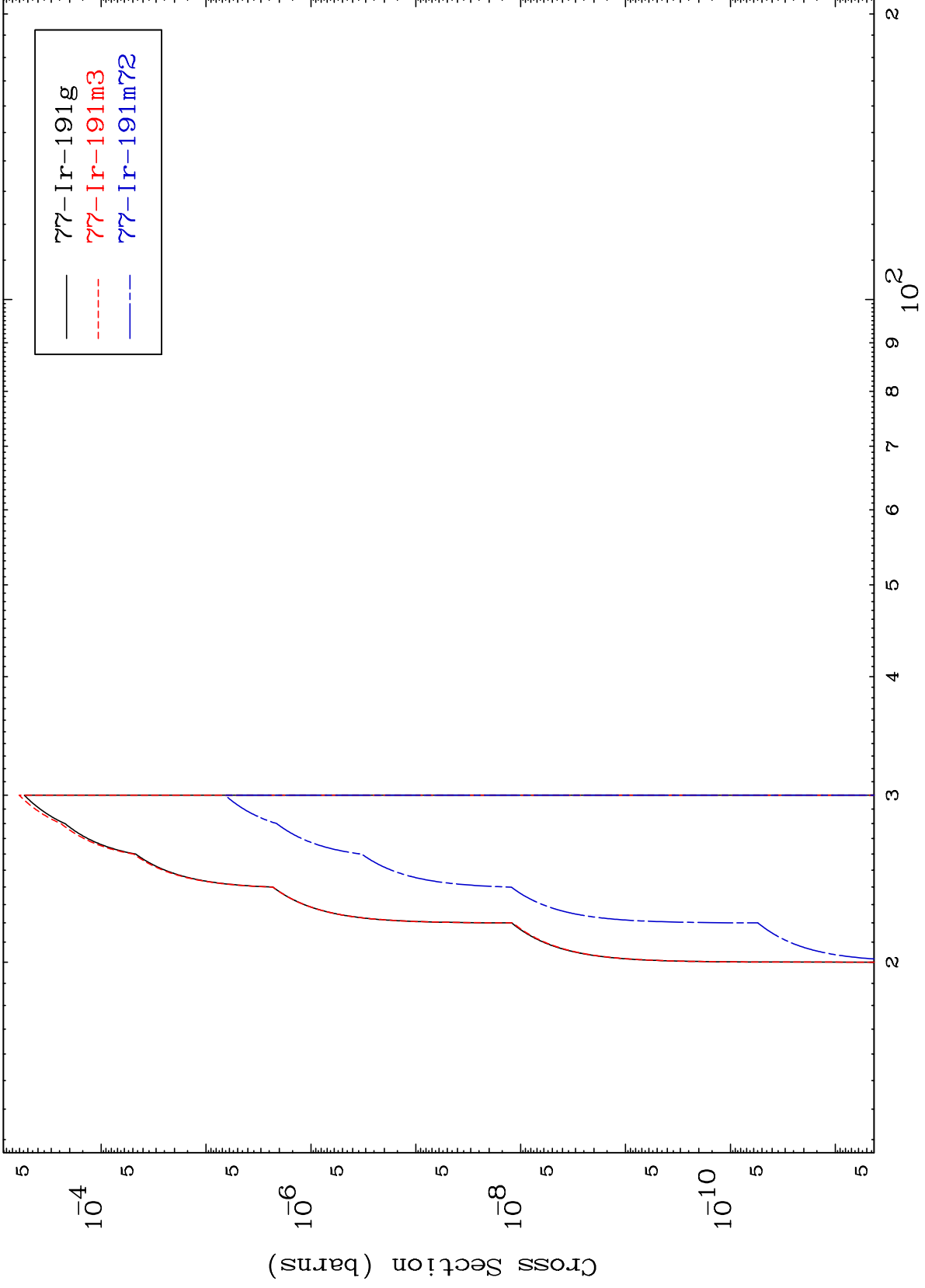
Incident Energy (MeV)

76-0s-190m

MAT 7644

76-0s-190m

(n,t)
Radionuclide Production Cross Section



19

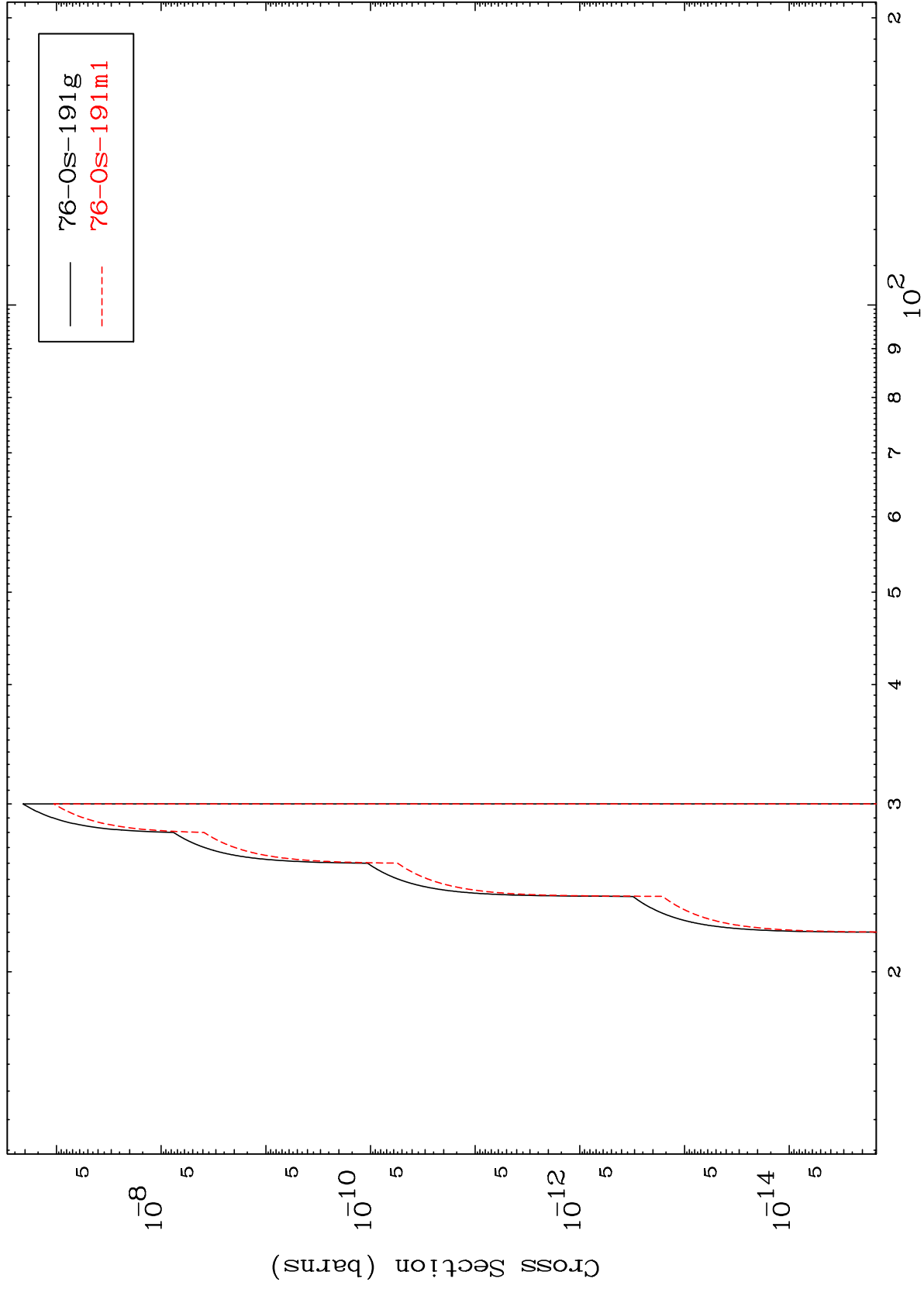
76-0s-190m

Incident Energy (MeV)

MAT 7644

76-0s-190m

(n,He-3)
Radionuclide Production Cross Section



20

76-0s-190m

Incident Energy (MeV)

MAT 7644

(n, α)

76-0s-190m

