

Program EVALPLOT  
(Version 2021-1)

by

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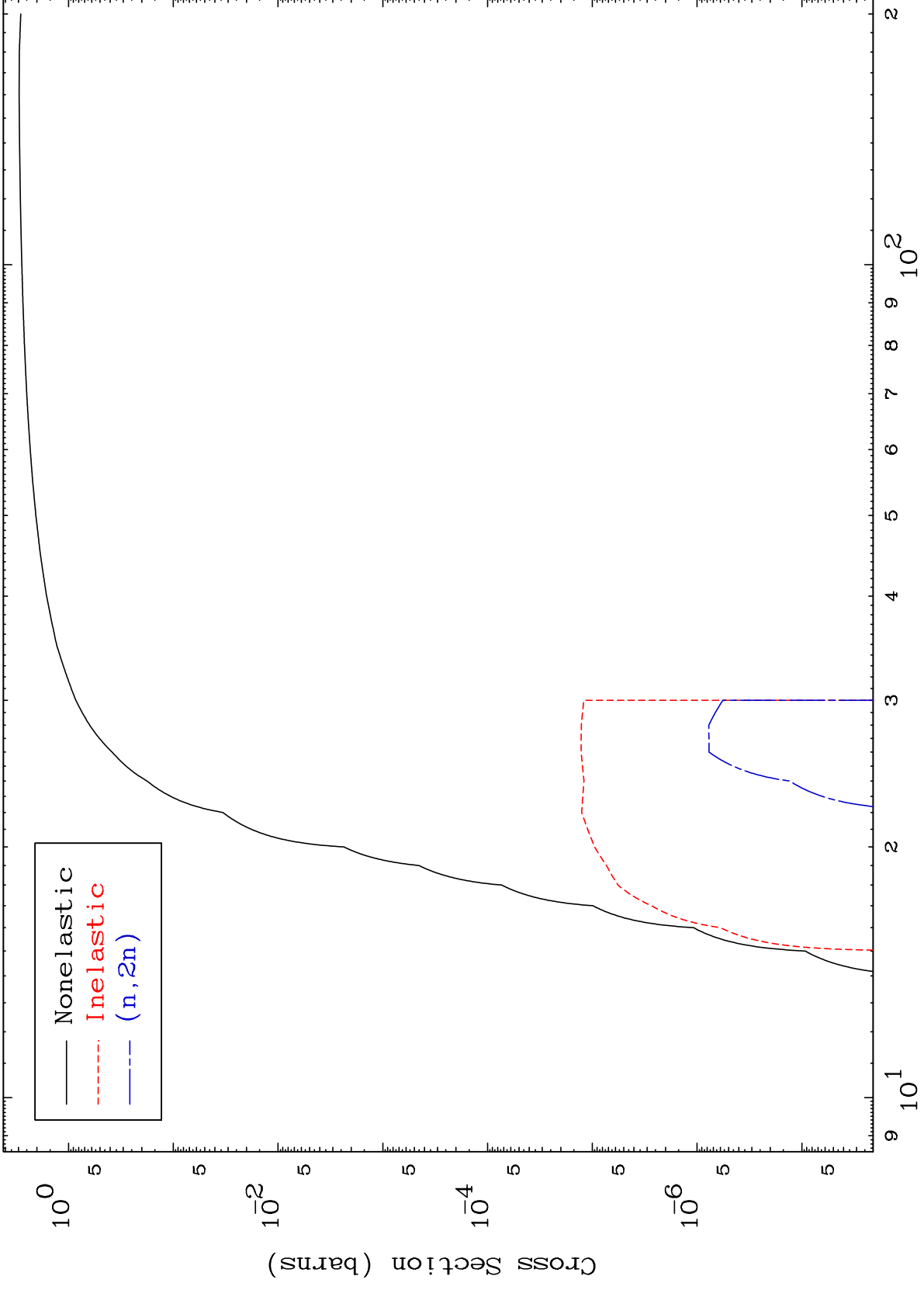
Press Mouse Button to Start

MAT 9419

$\alpha$  Major

94-Pu-233

0 Kelvin Cross Sections



94-Pu-233

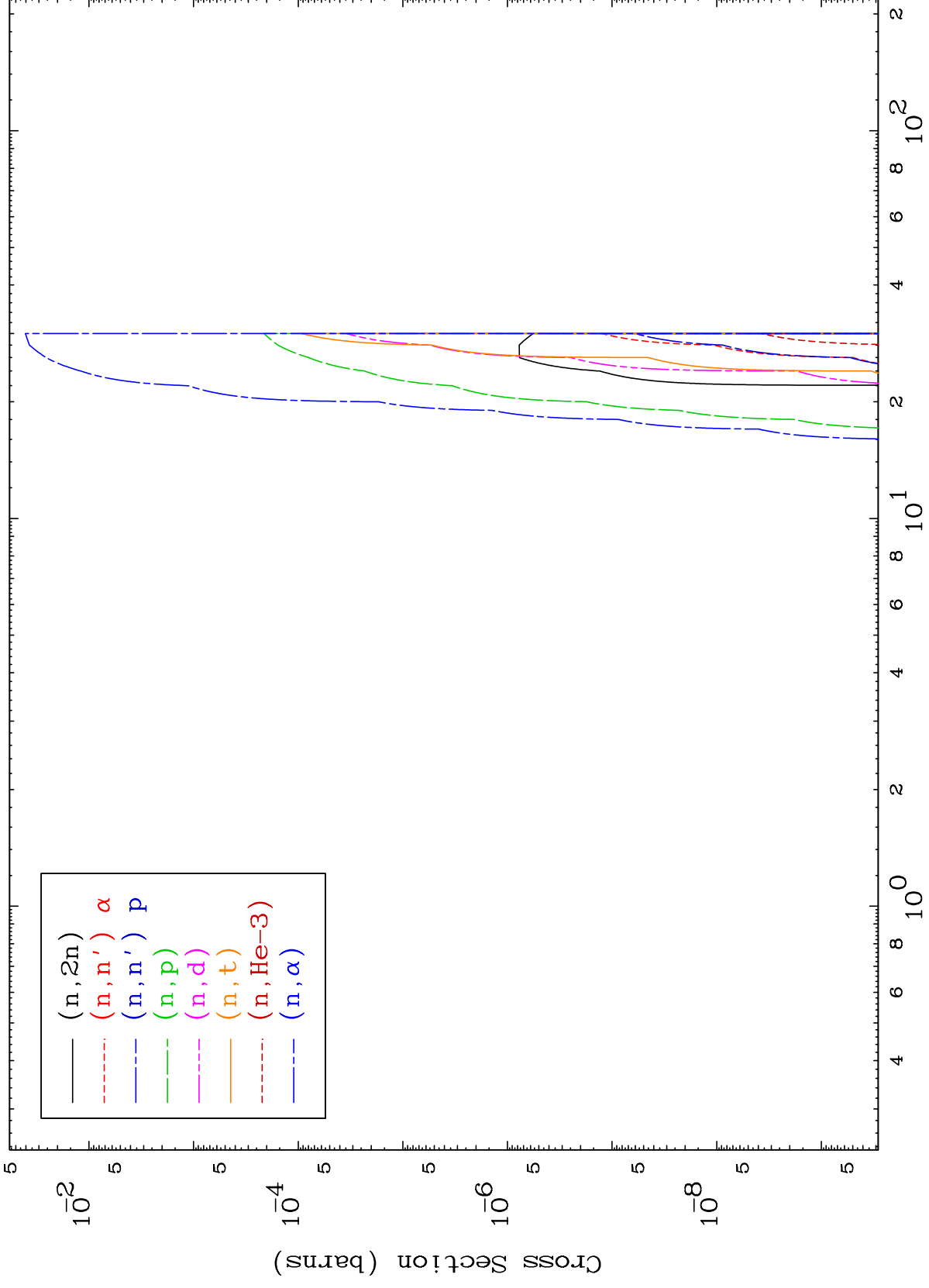
Incident Energy (MeV)

1

MAT 9419

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

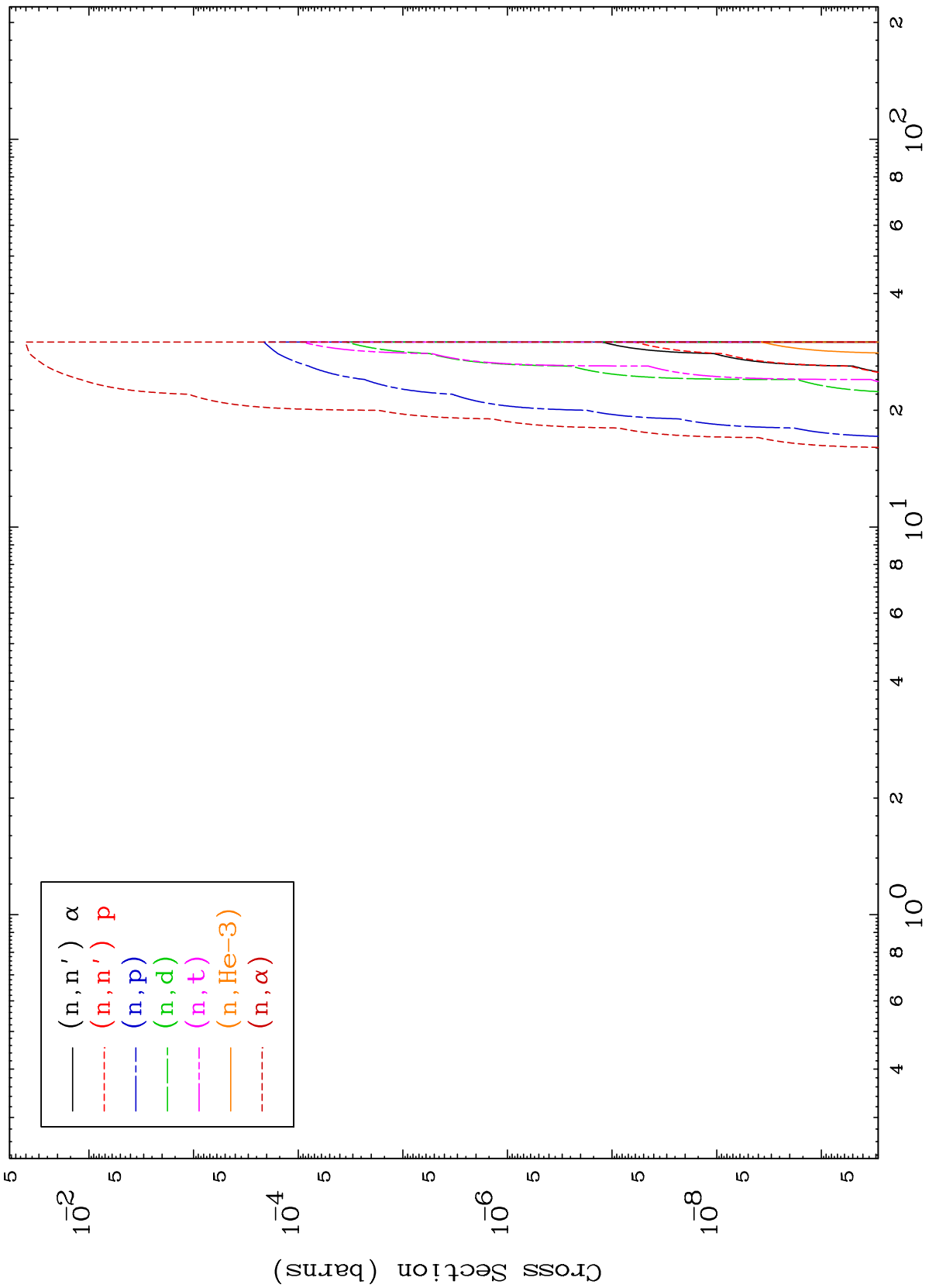
94-Pu-233



94-Pu-233

Incident Energy (MeV)

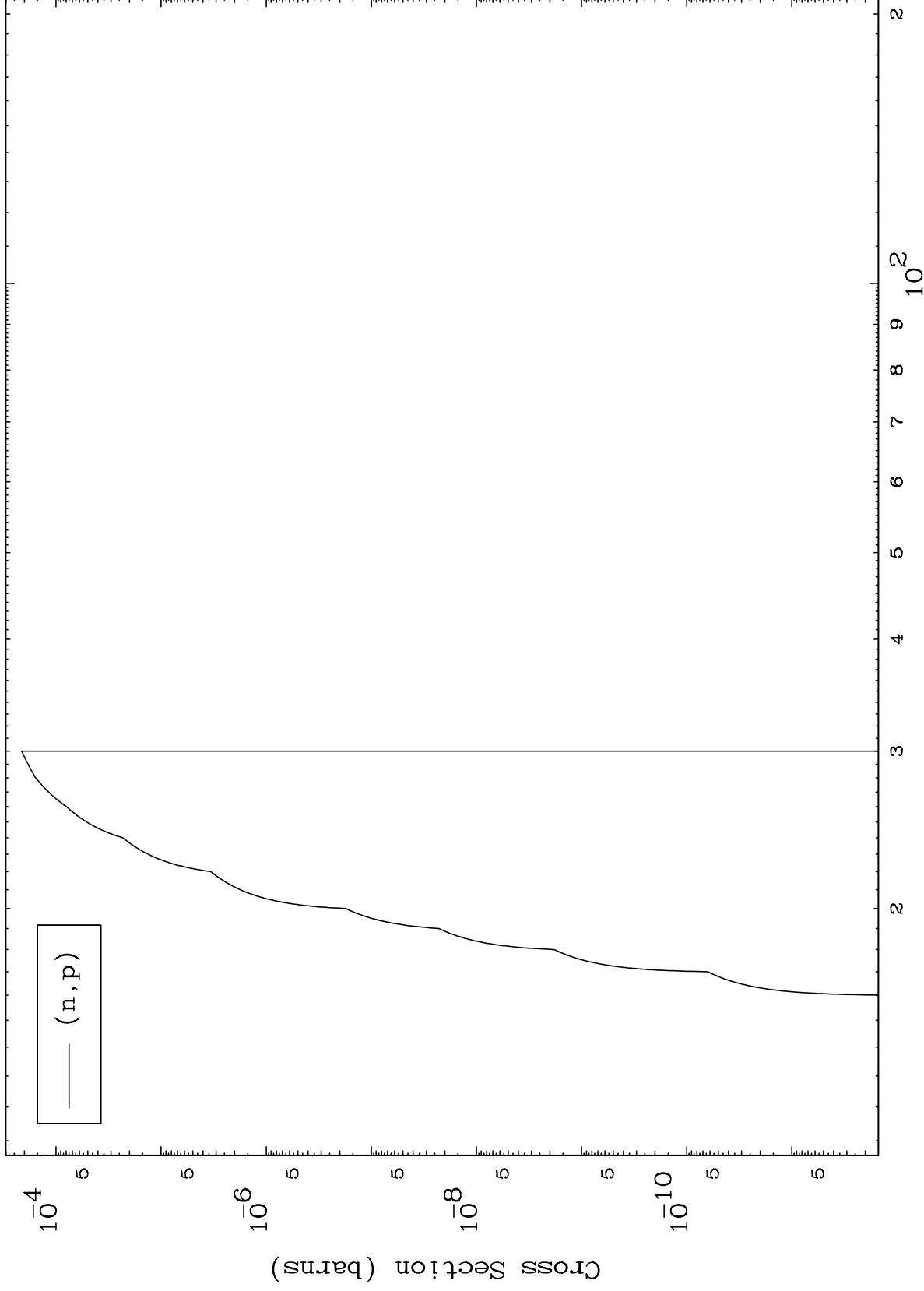
2

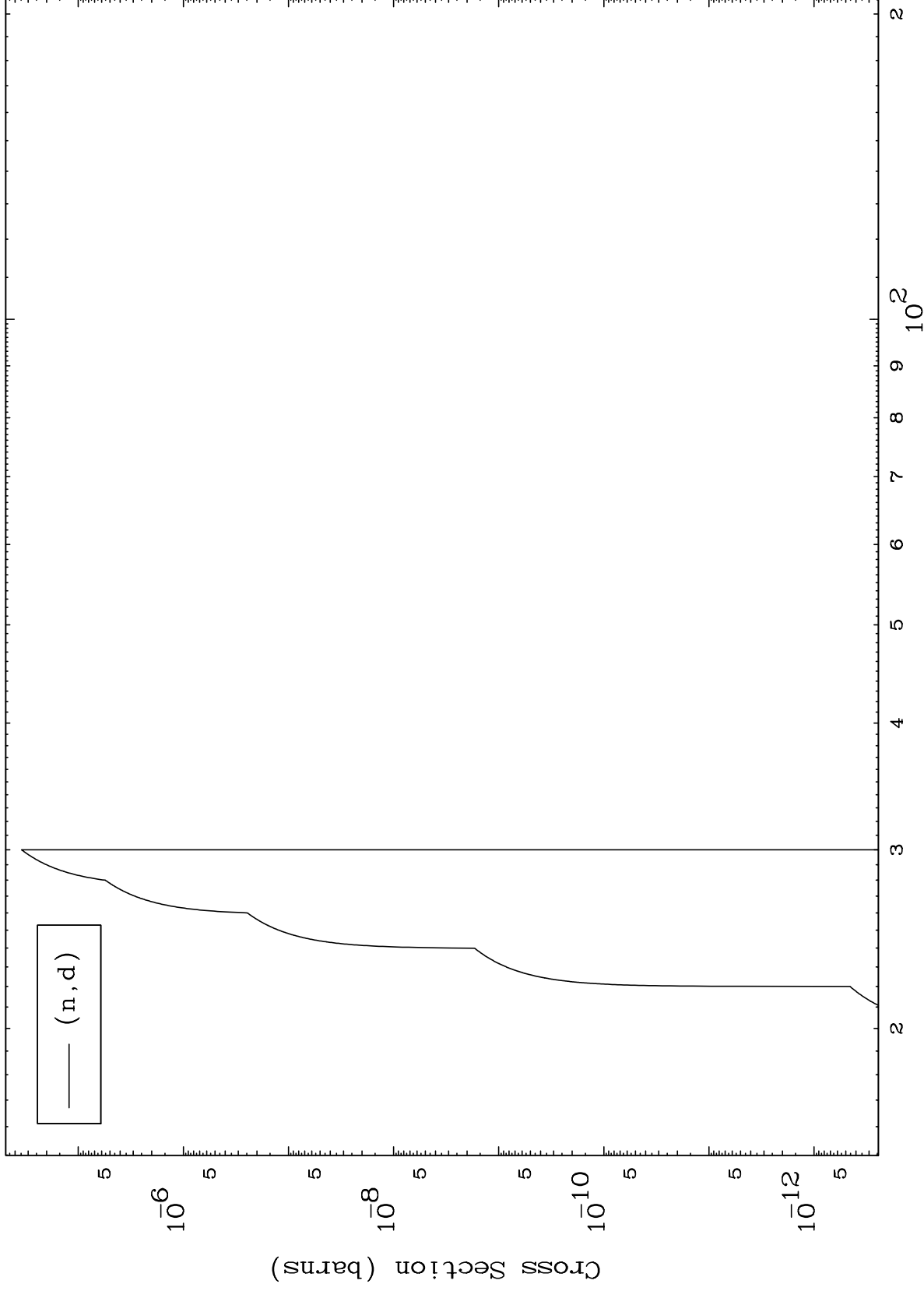


MAT 9419

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

94-Pu-233

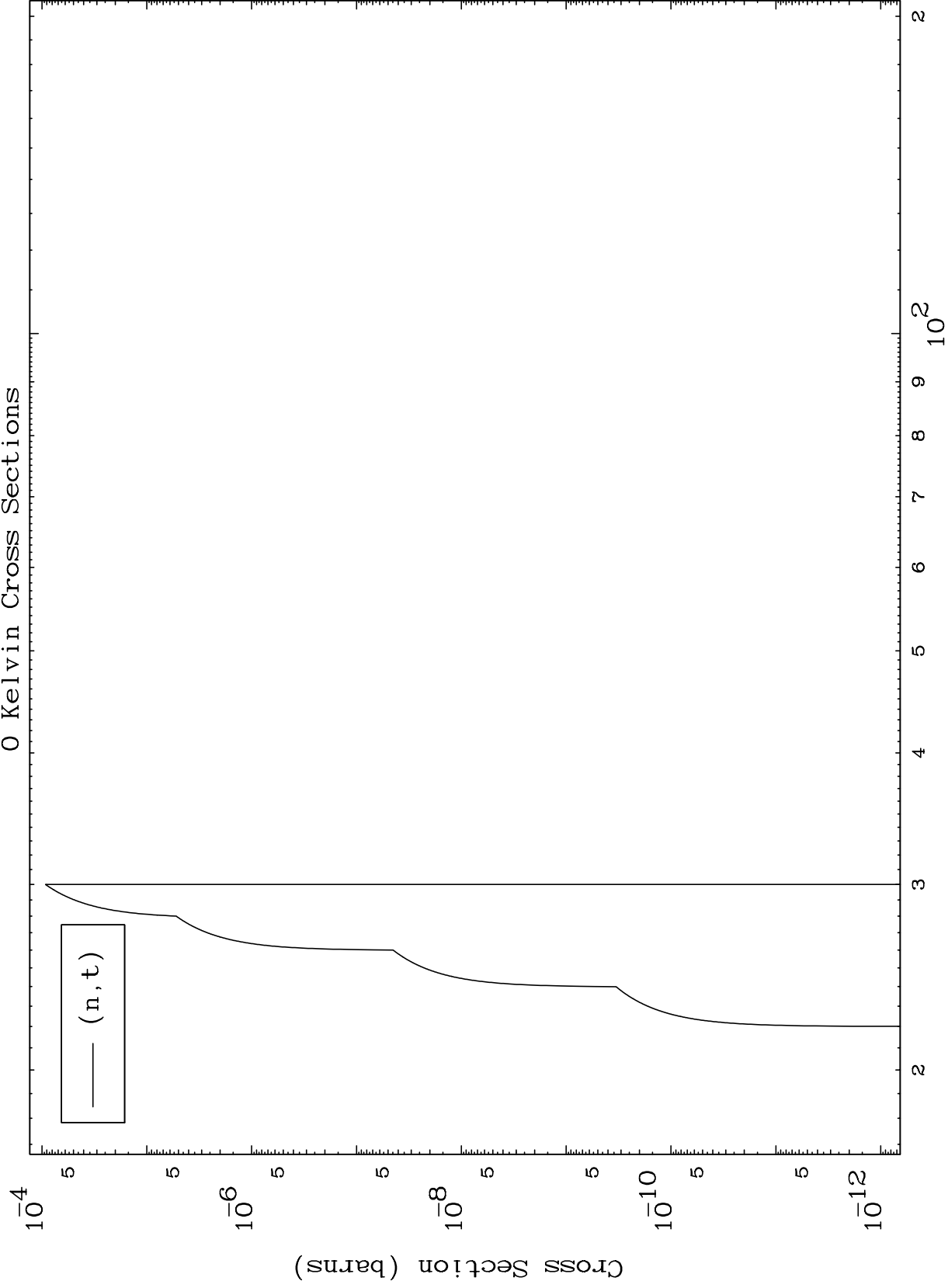




MAT 9419

94-Pu-233

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



94-Pu-233

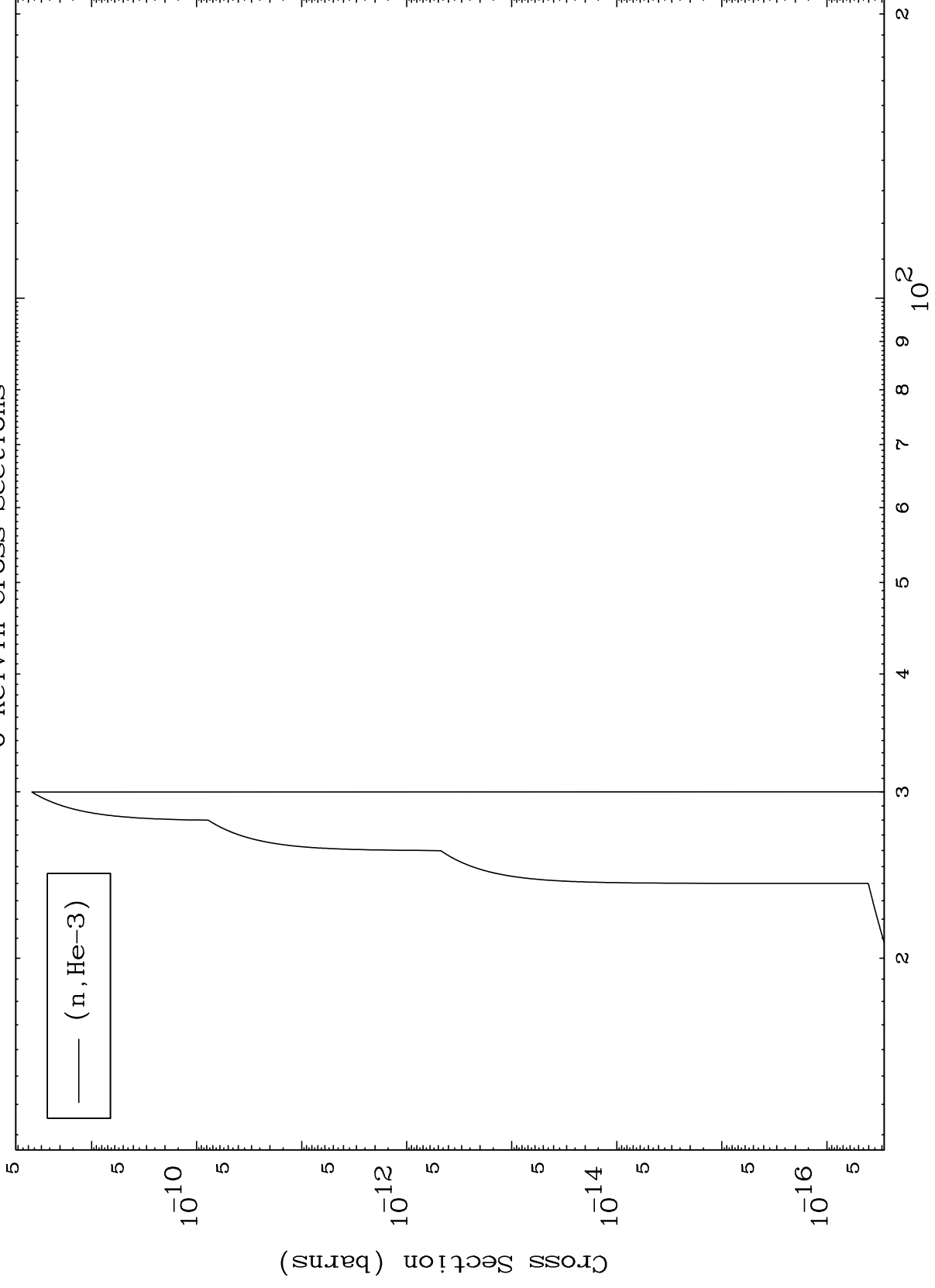
Incident Energy (MeV)

6

MAT 9419

94-Pu-233

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



94-Pu-233

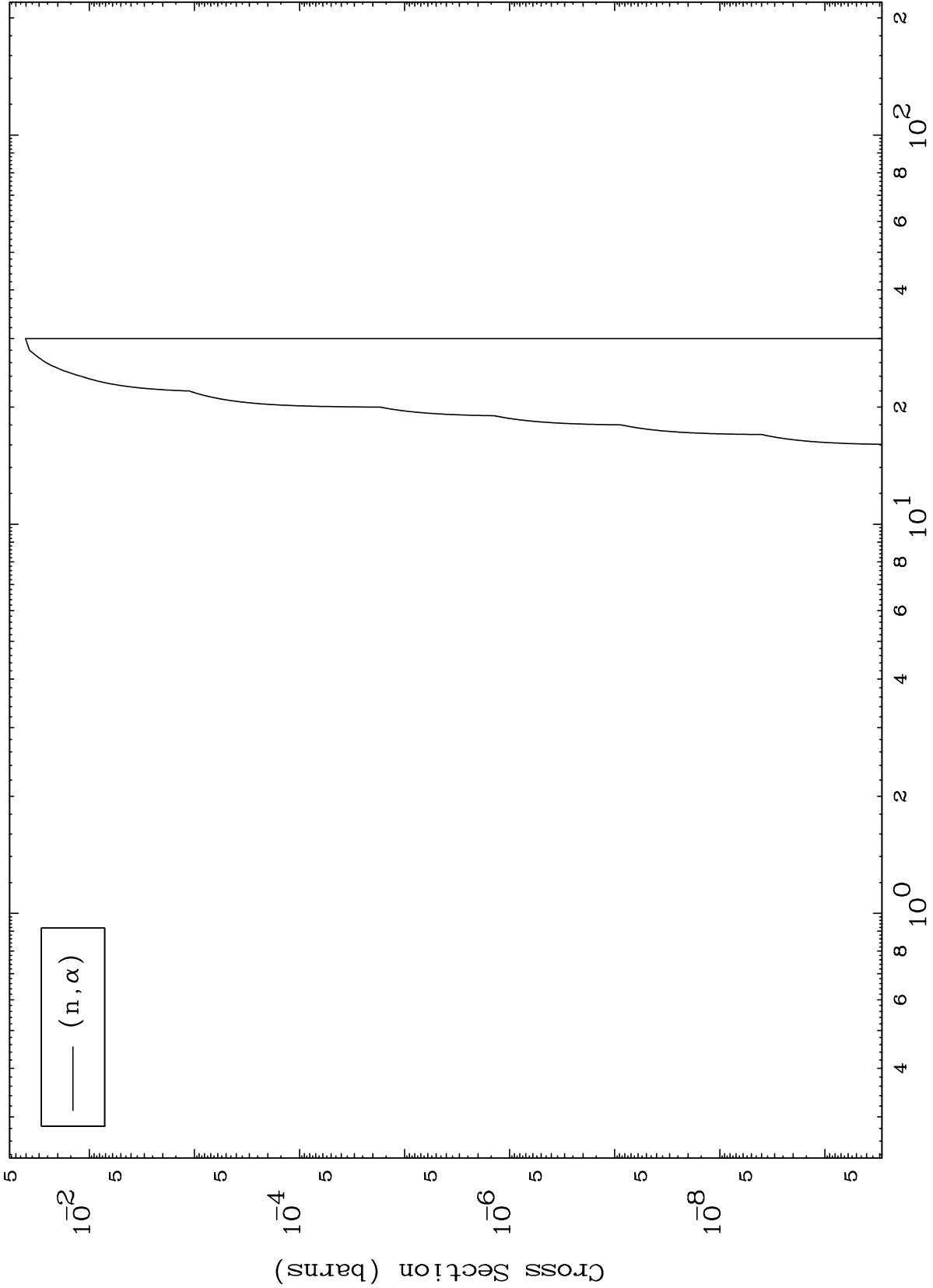
Incident Energy (MeV)

MAT 9419

( $\alpha, \alpha$ ) Levels

94-Pu-233

0 Kelvin Cross Sections

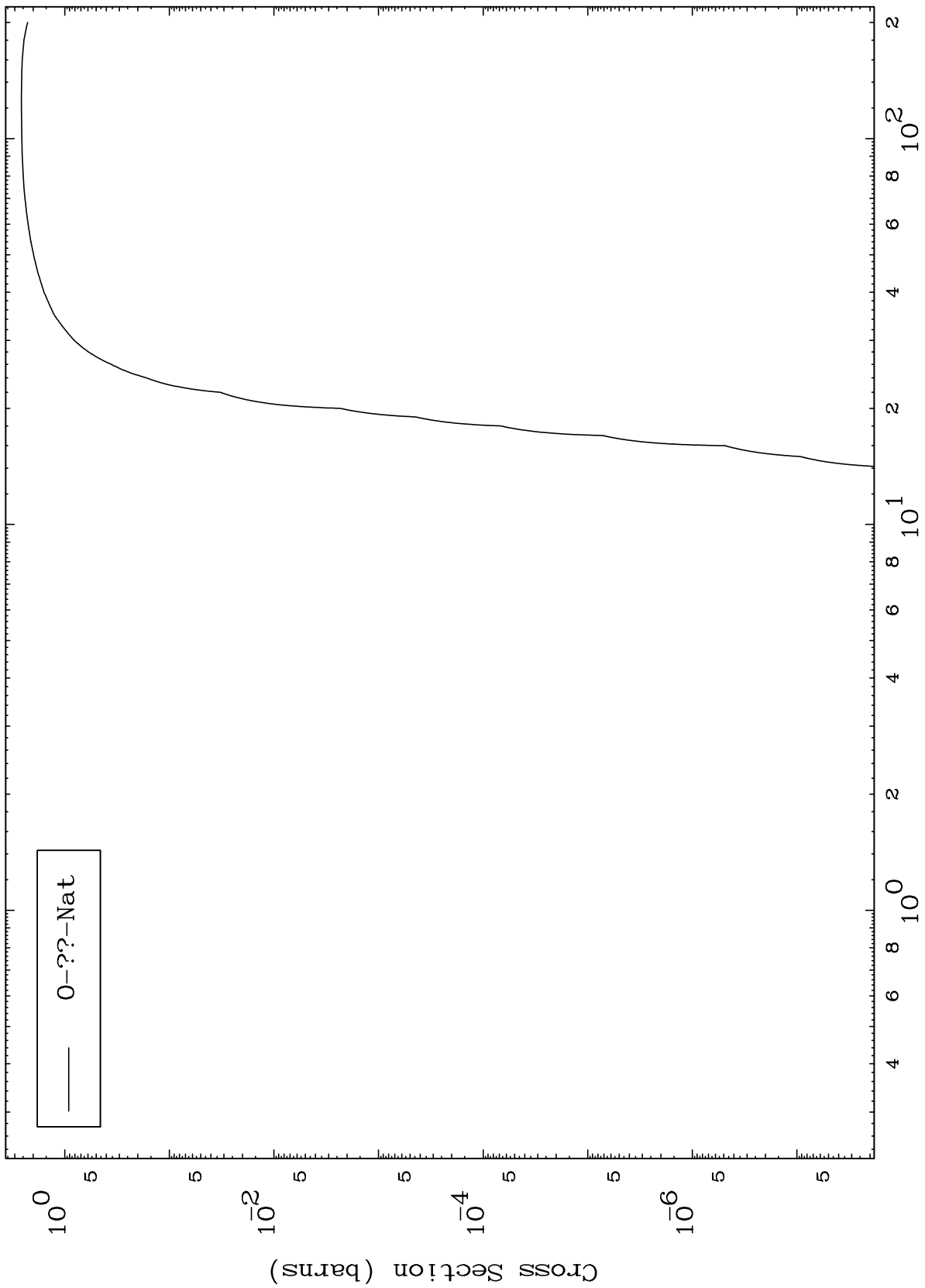


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Fission

94-Pu-233

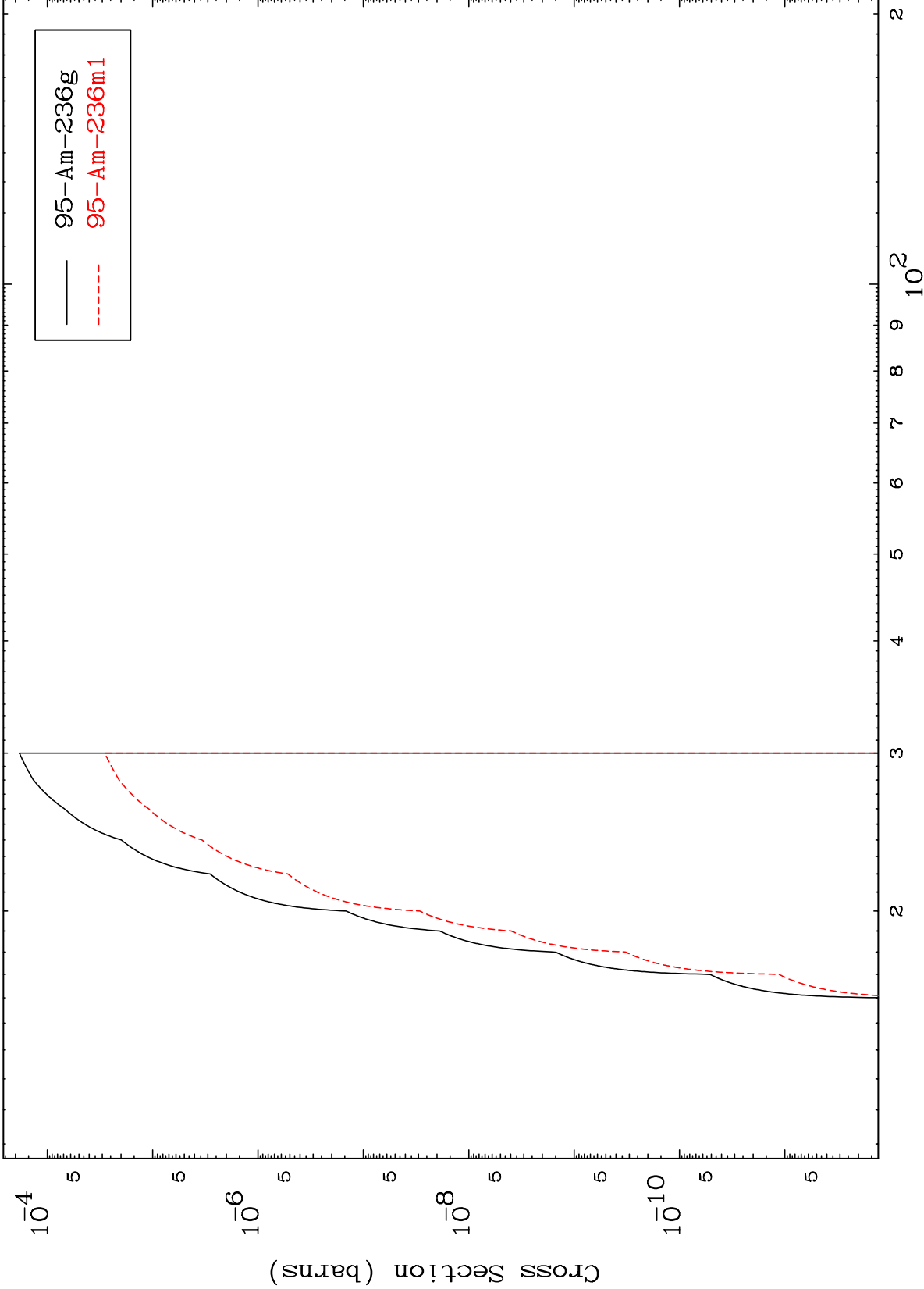
Radionuclide Production Cross Section



MAT 9419

94-Pu-233

(n,p)  
Radionuclide Production Cross Section



95-Am-236g  
95-Am-236m1

10

Incident Energy (MeV)

94-Pu-233