

Program EVALPLOT  
(Version 2021-1)

by

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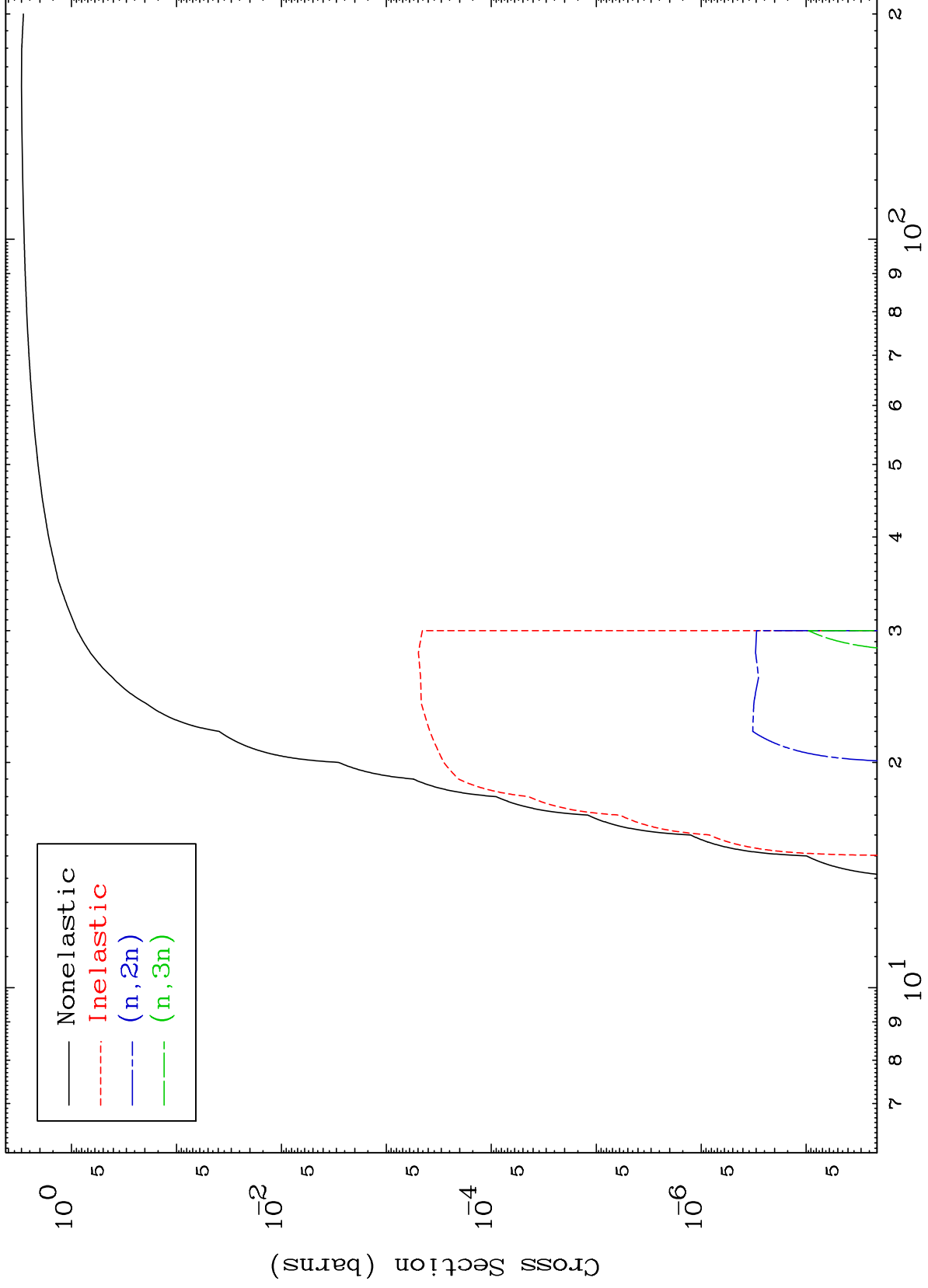
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9434

0 Kelvin Cross Sections

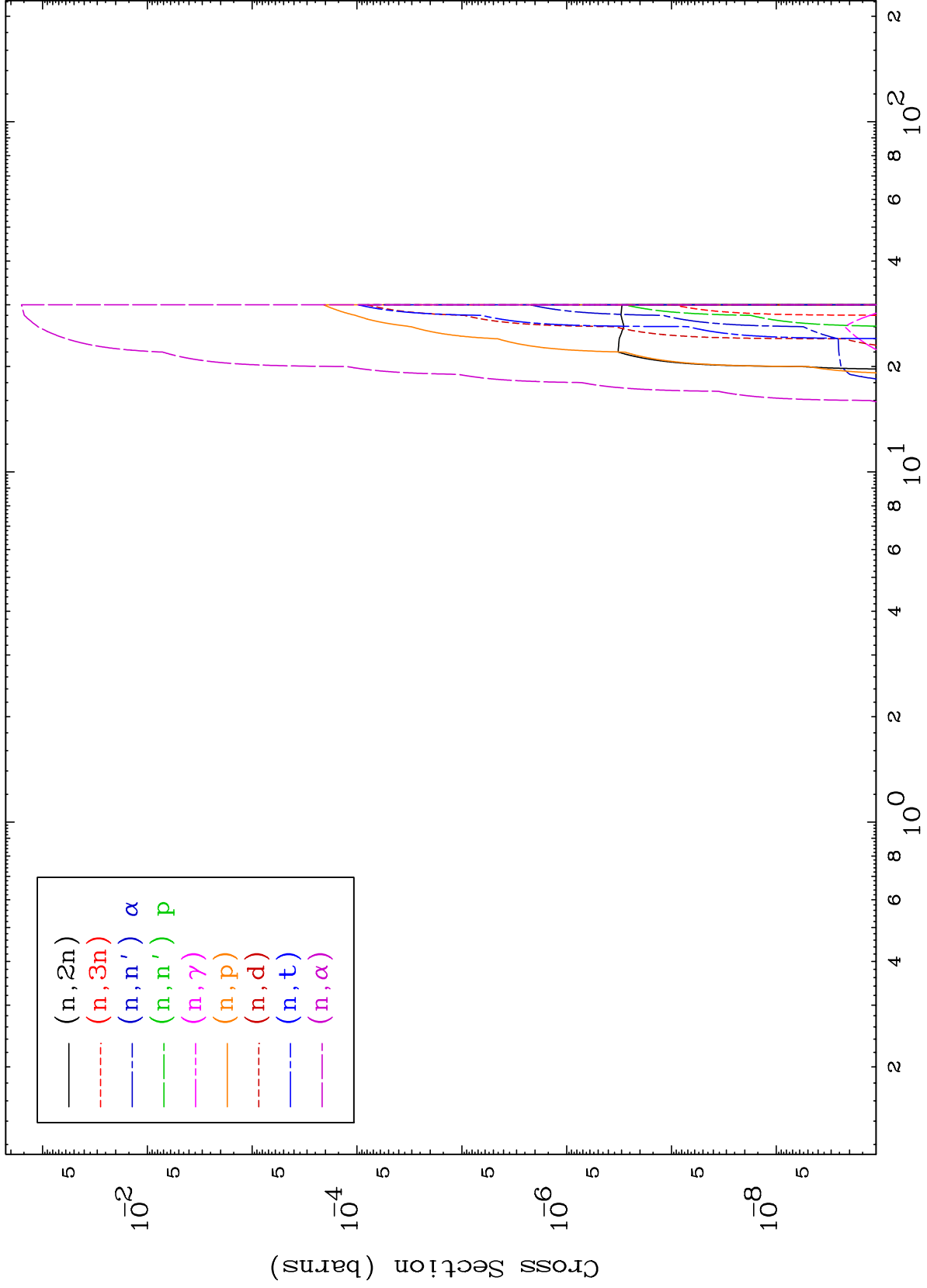
94-Pu-238

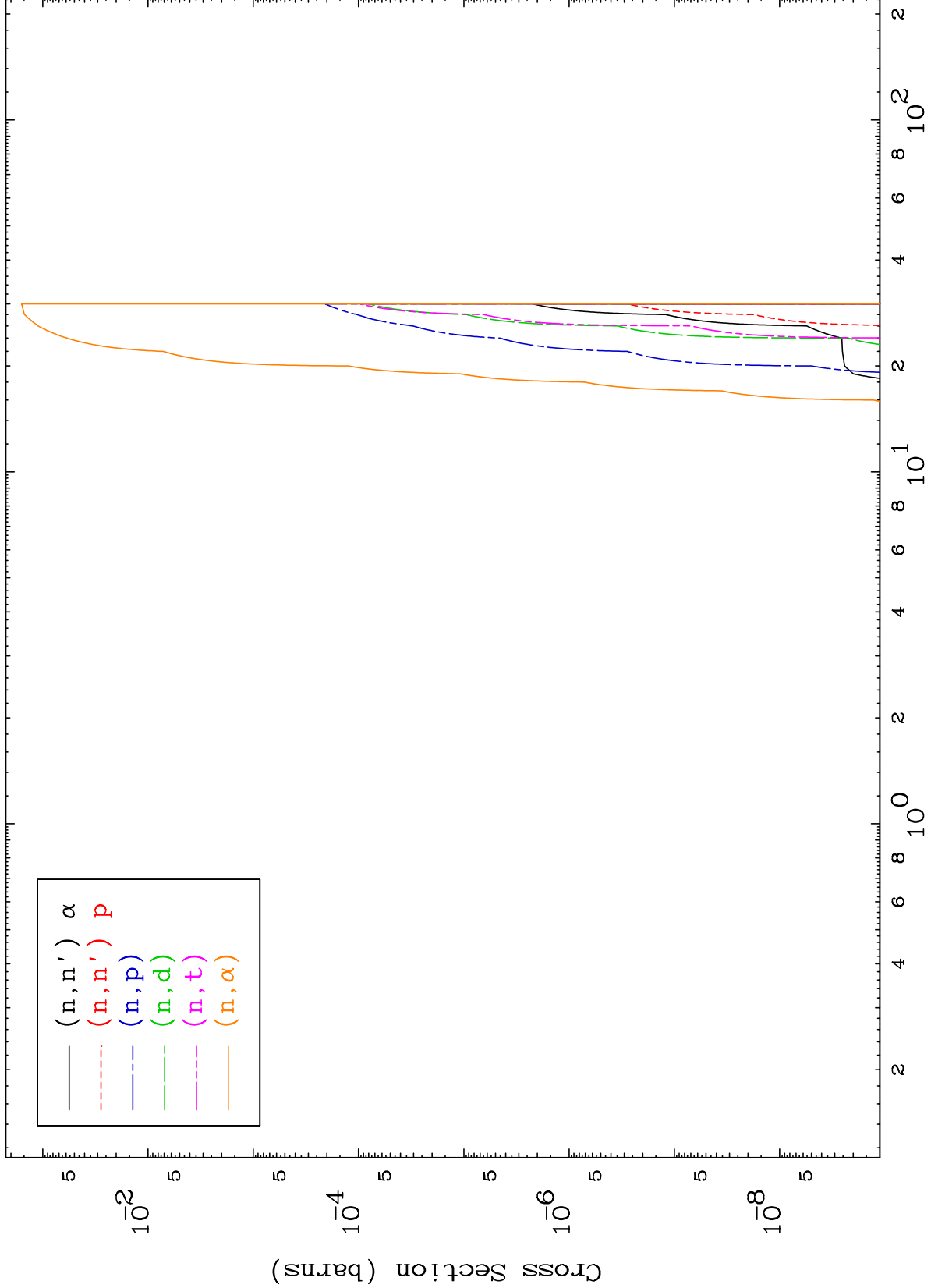


MAT 9434

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

94-Pu-238

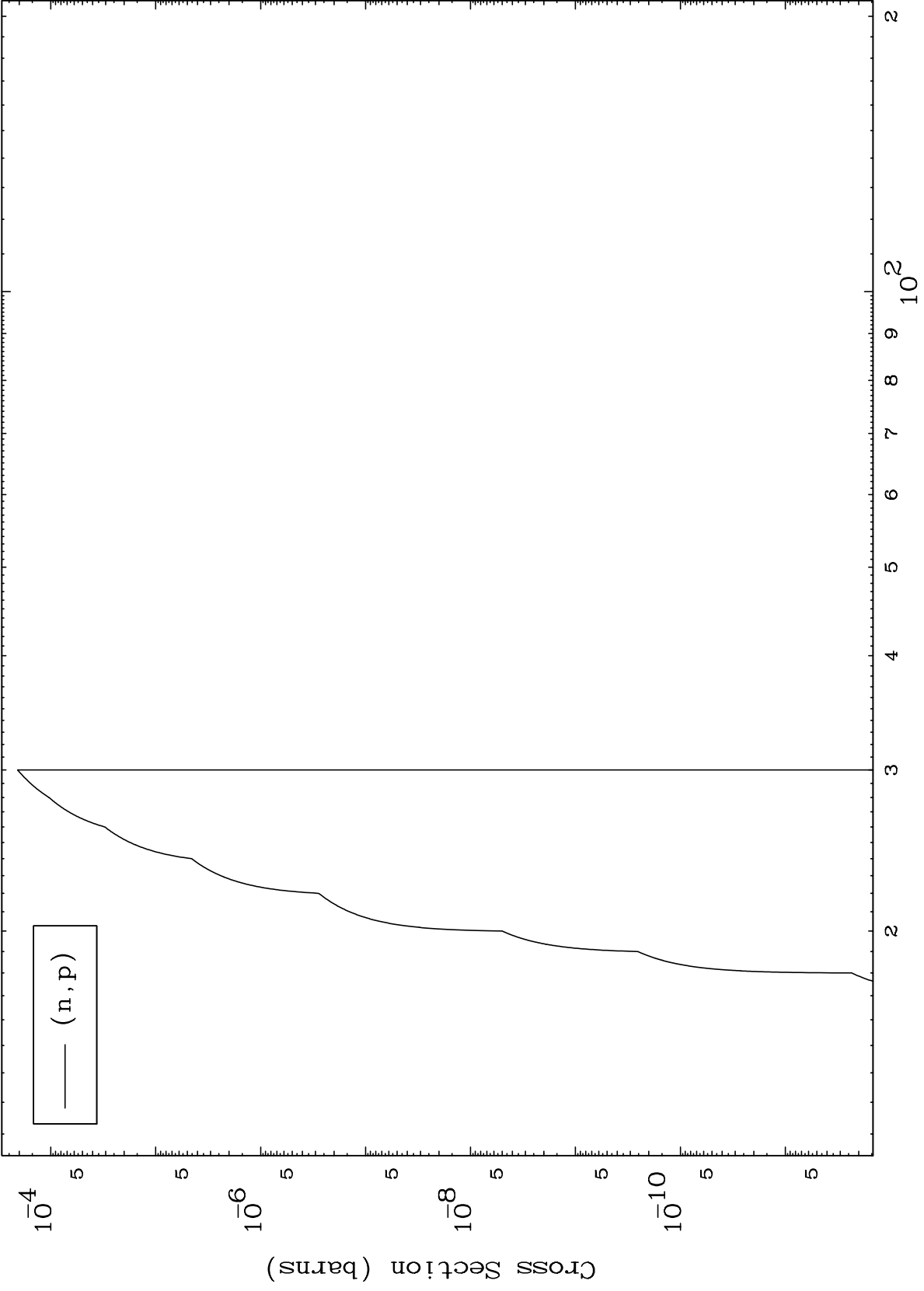




MAT 9434

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

94-Pu-238



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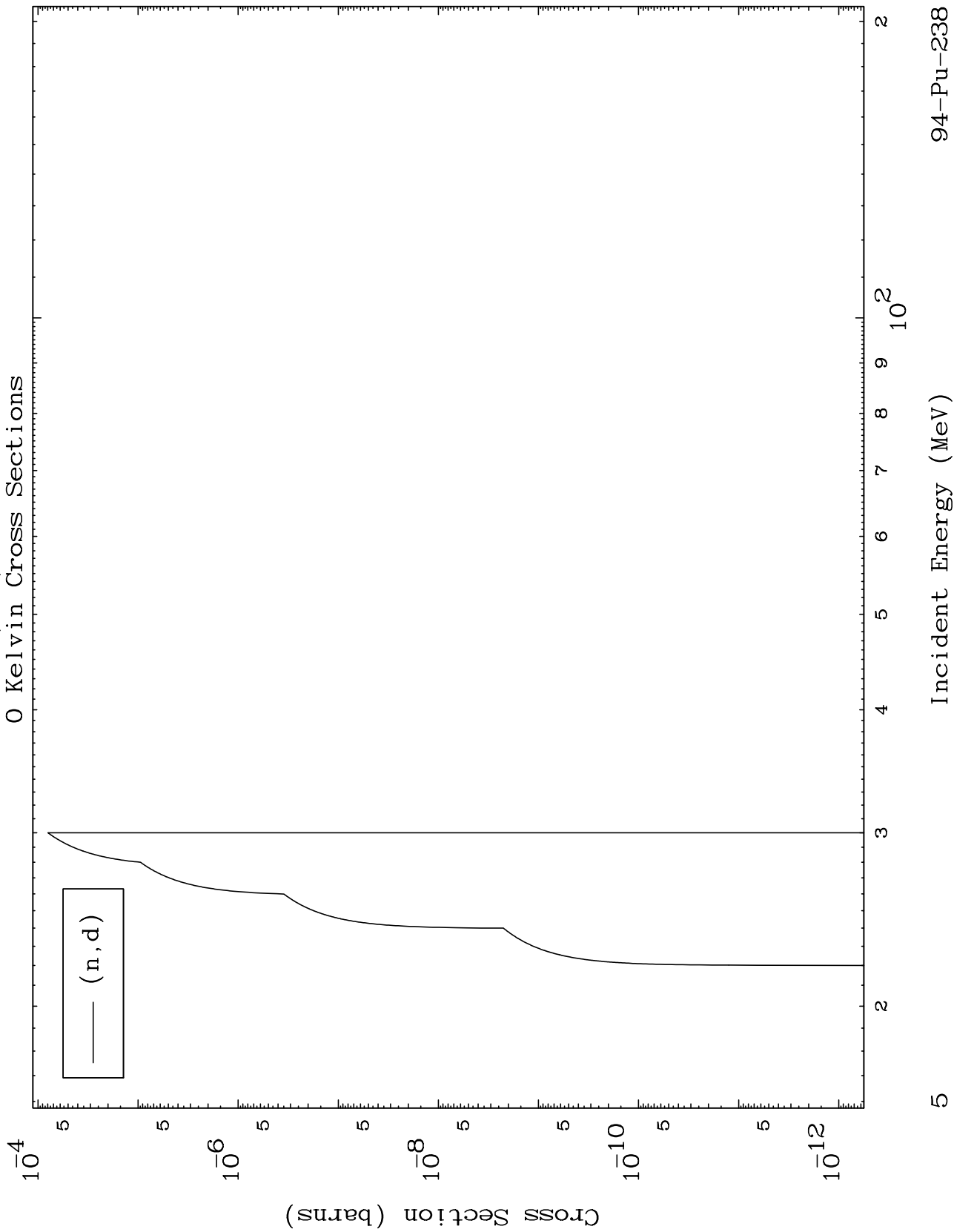
Incident Energy (MeV)

94-Pu-238

MAT 9434

( $\alpha, d$ ) Levels

94-Pu-238

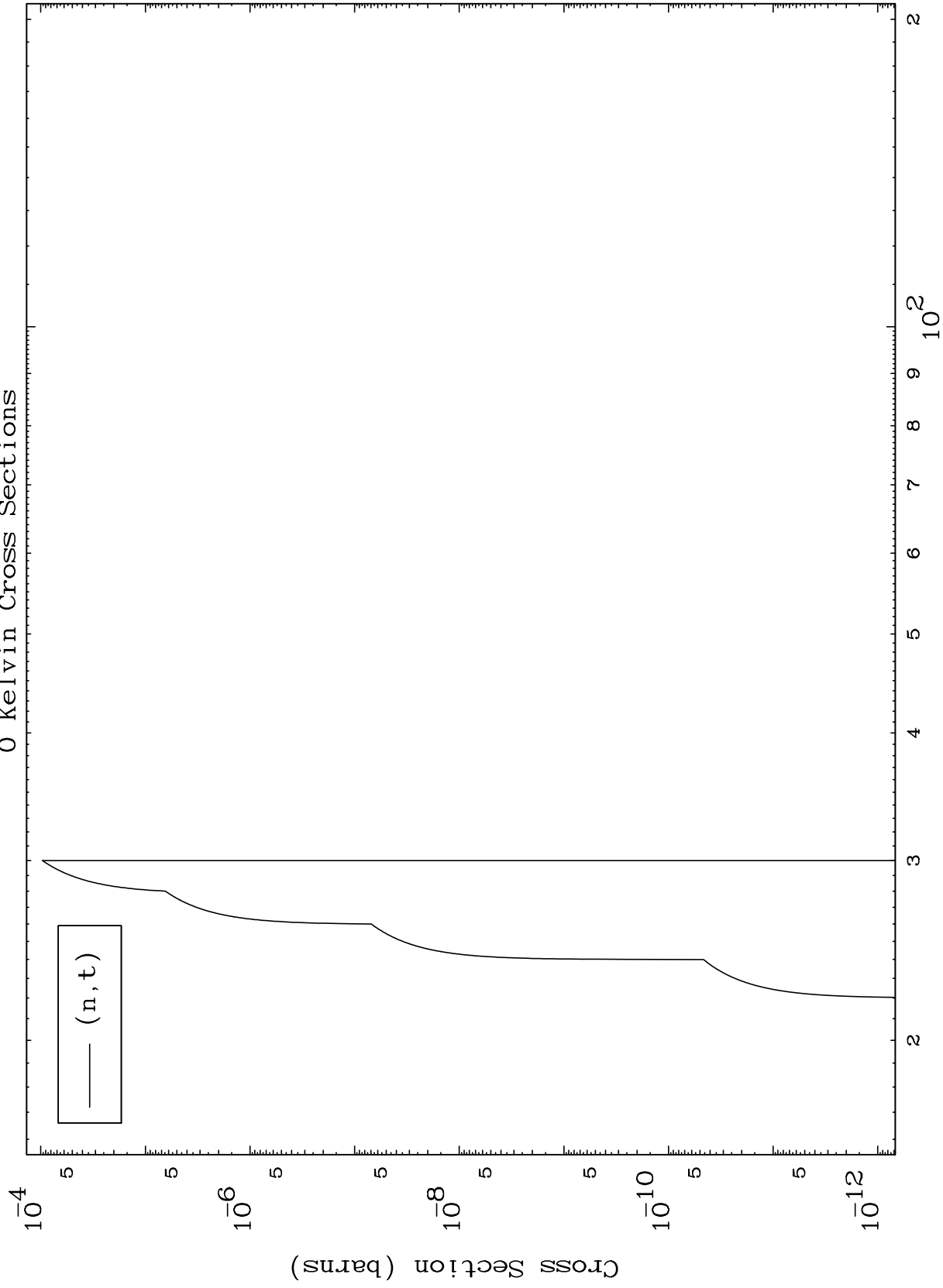


5

MAT 9434

94-Pu-238

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



94-Pu-238

Incident Energy (MeV)

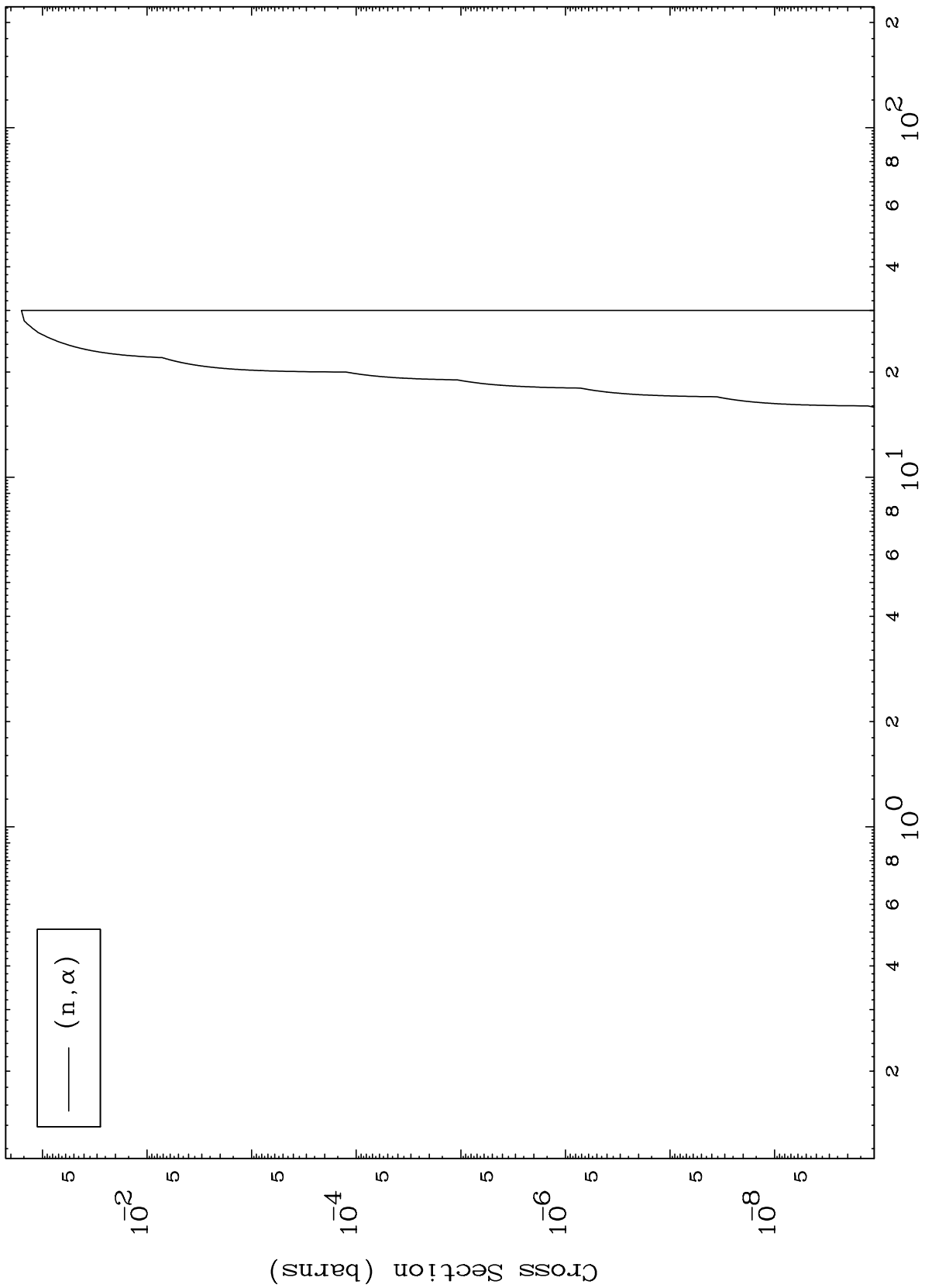
6

MAT 9434

( $\alpha, \alpha$ ) Levels

94-Pu-238

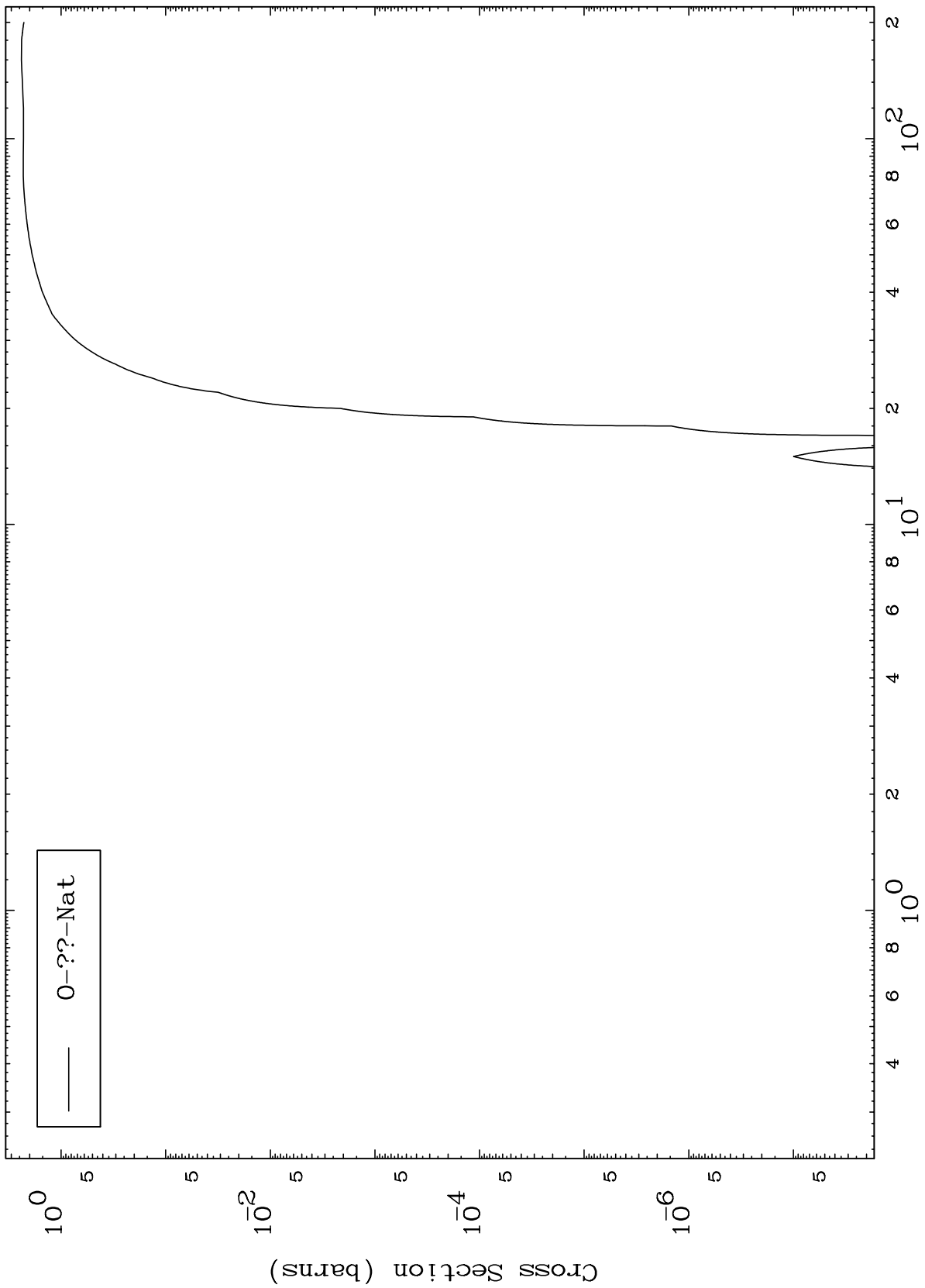
0 Kelvin Cross Sections



MAT 9434

94-Pu-238

### Fission Radionuclide Production Cross Section

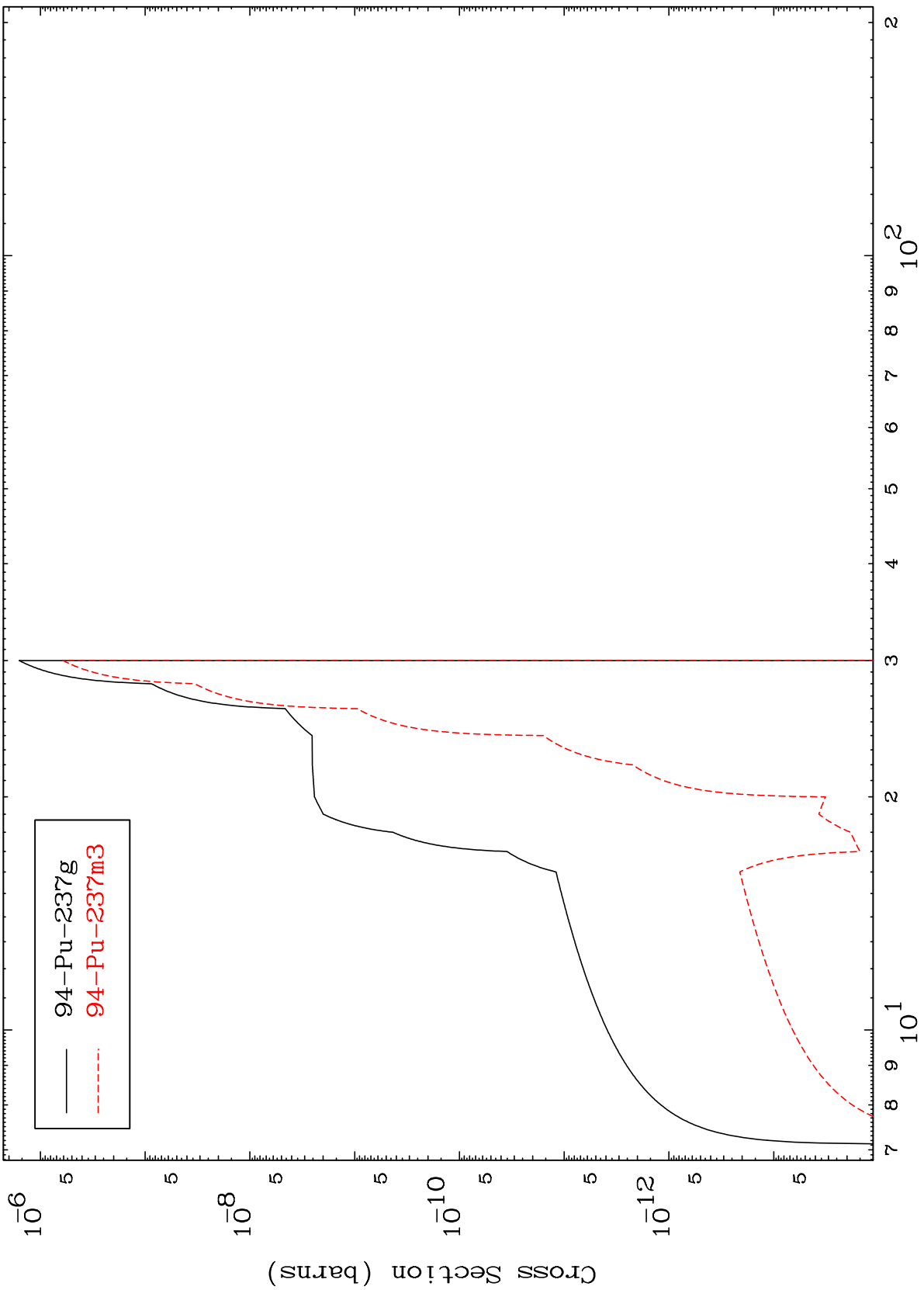


MAT 9434

(n,n')  $\alpha$

94-Pu-238

Radionuclide Production Cross Section



9

Incident Energy (MeV)

94-Pu-238