

Program EVALPLOT  
(Version 2021-1)

by

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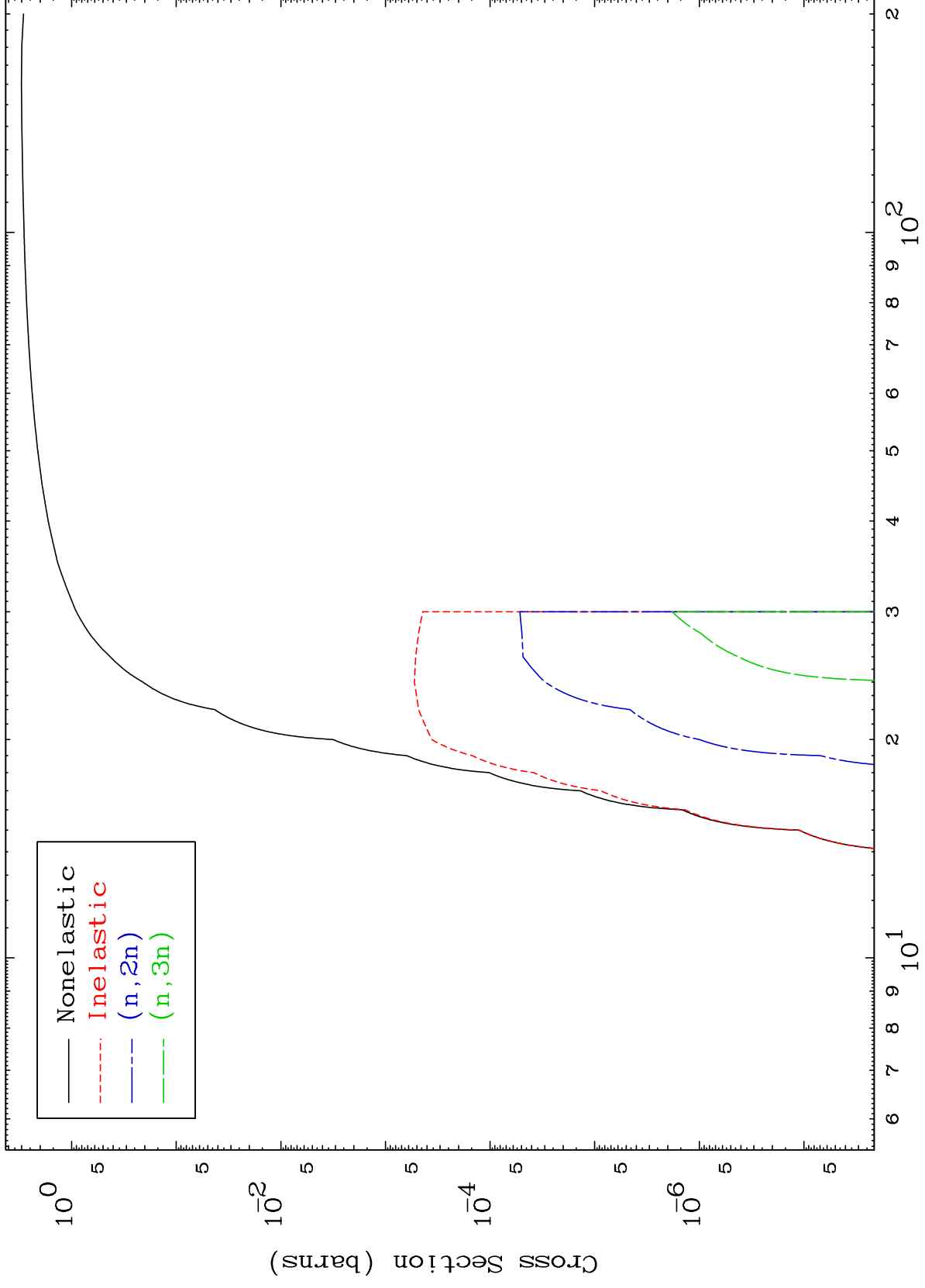
Press Mouse Button to Start

MAT 9443

0 Kelvin

$\alpha$  Major Cross Sections

94-Pu-241



1

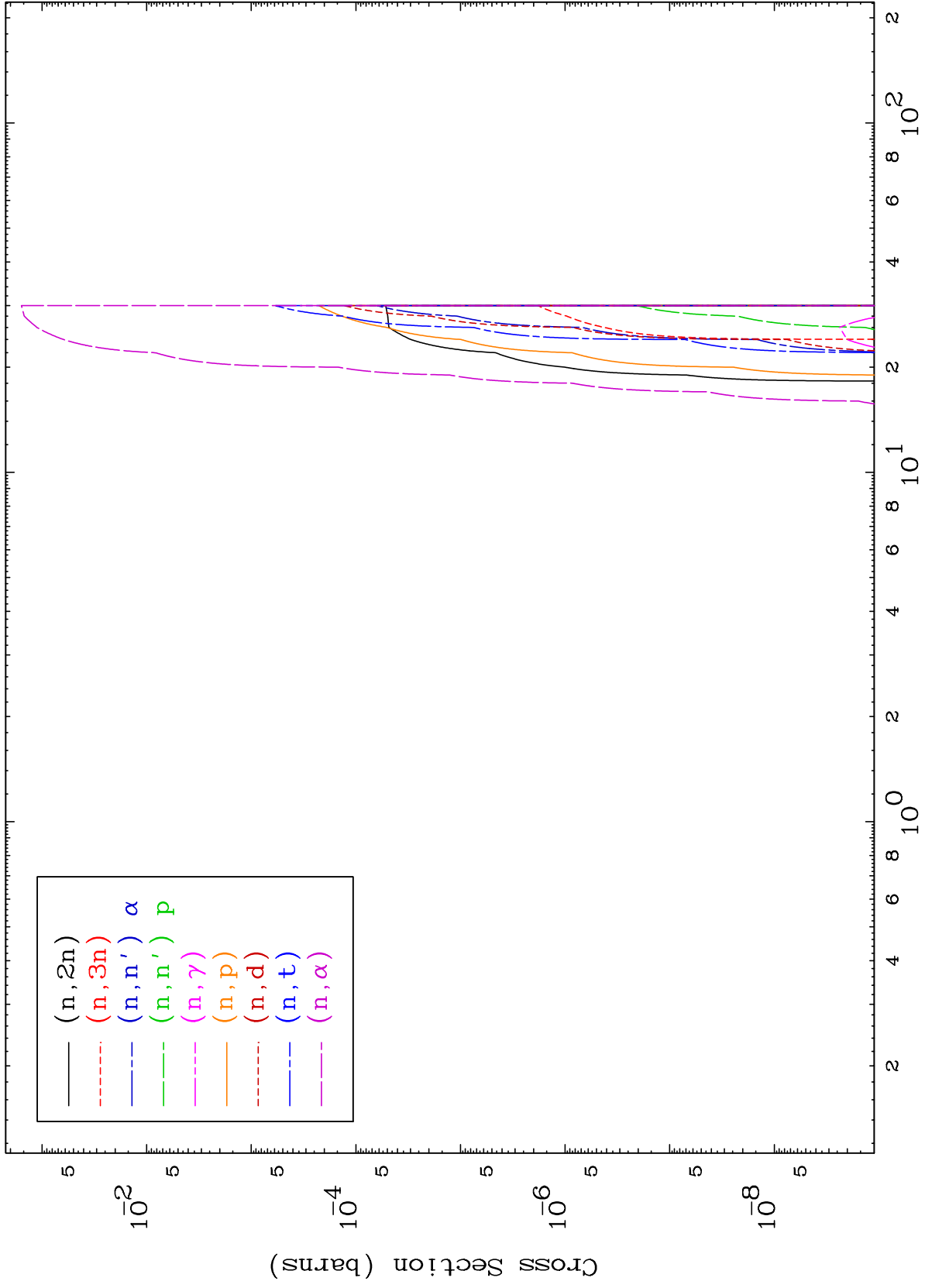
Incident Energy (MeV)

94-Pu-241

MAT 9443

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

94-Pu-241

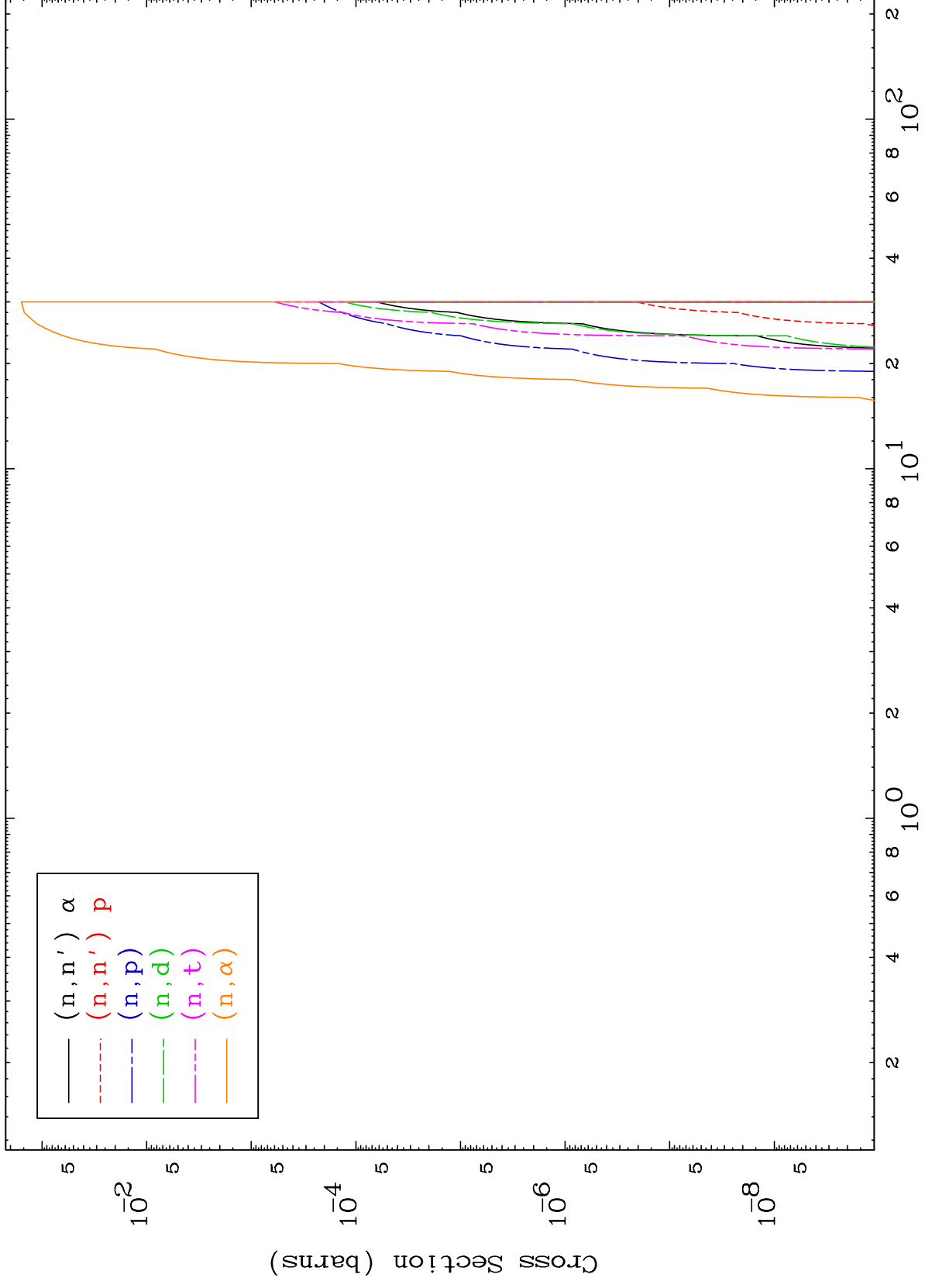


MAT 9443

$\alpha$  Charged Particle

94-Pu-241

0 Kelvin Cross Sections



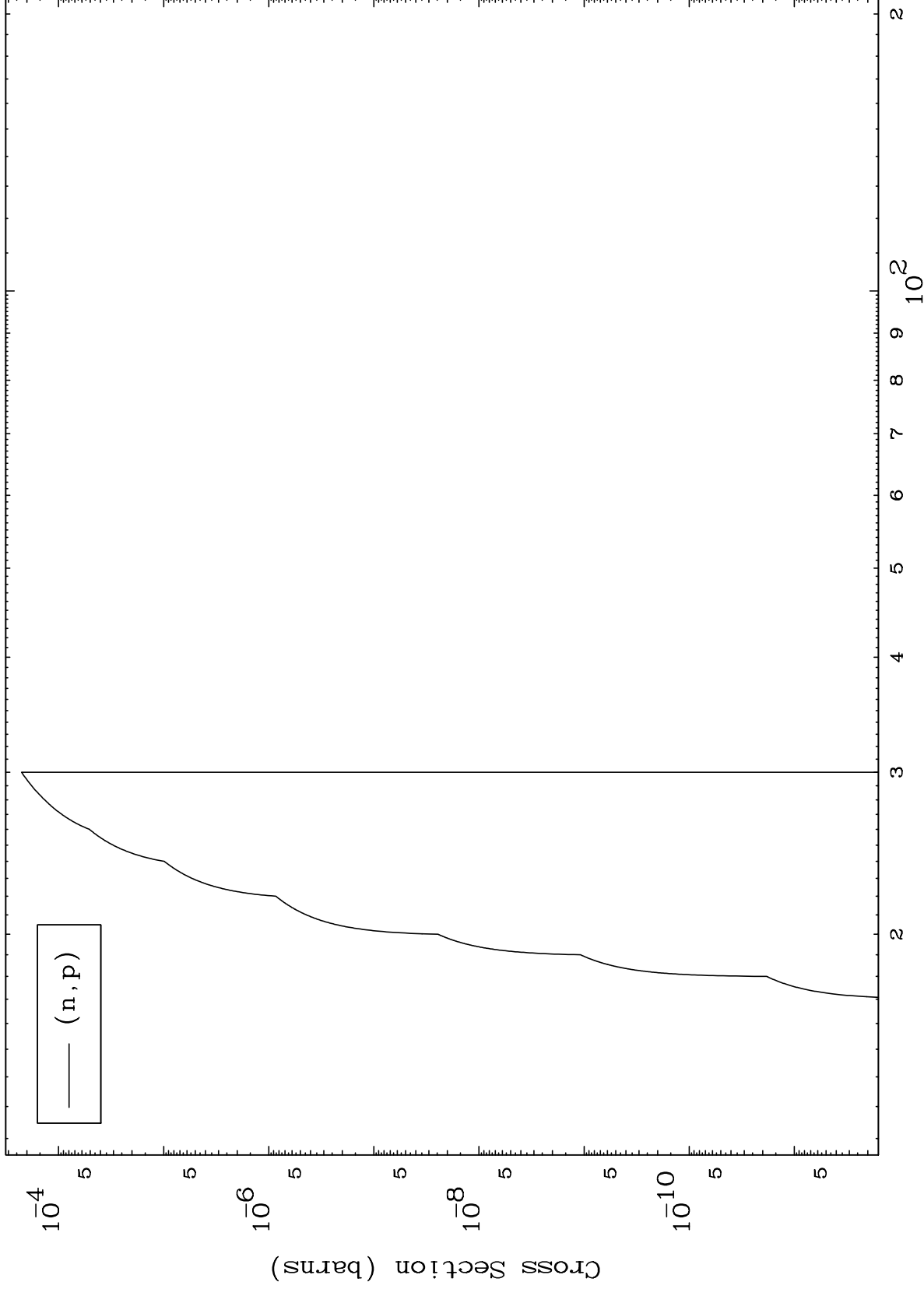
94-Pu-241

Incident Energy (MeV)

MAT 9443

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

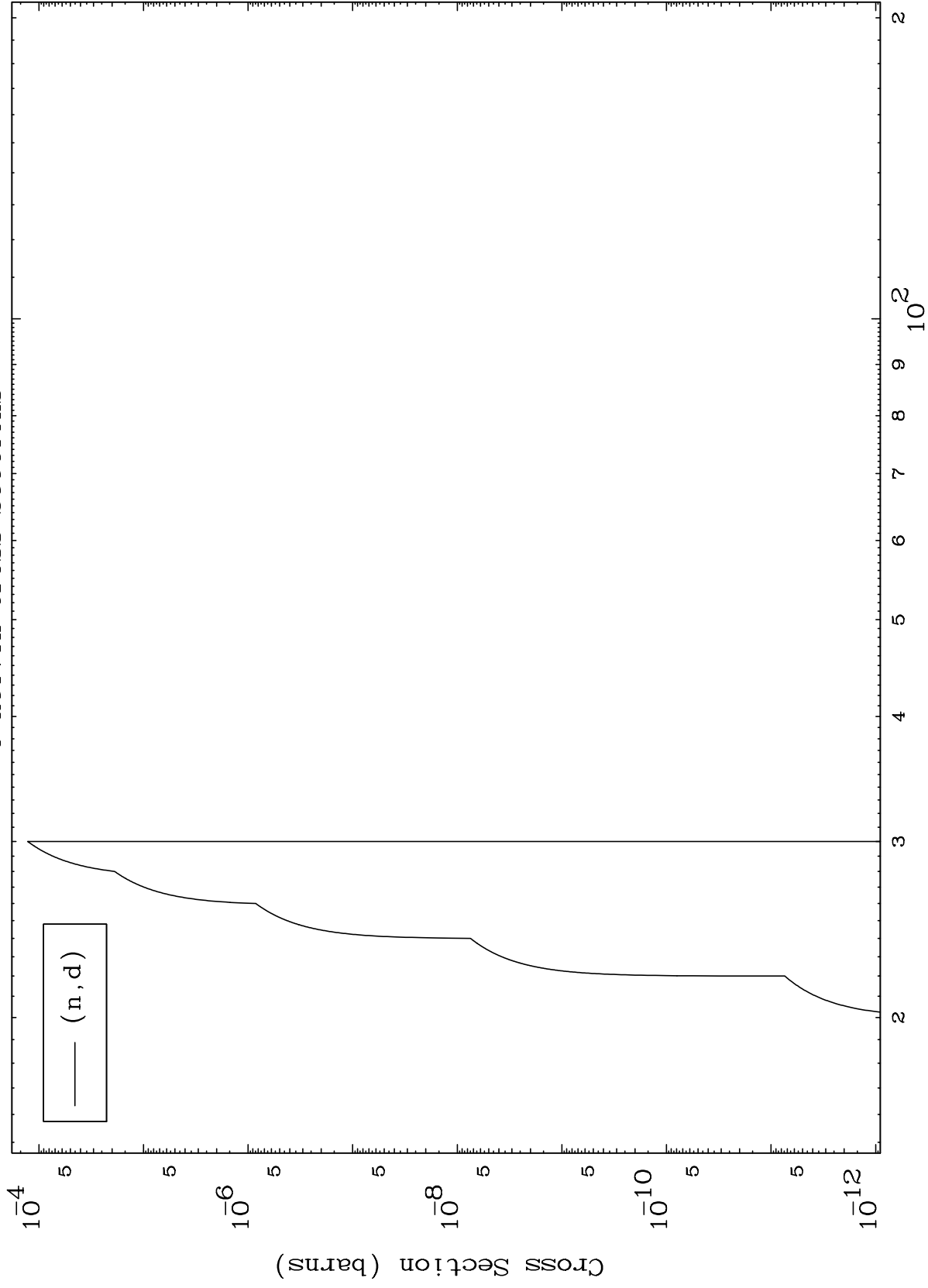
94-Pu-241



MAT 9443

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

94-Pu-241



5

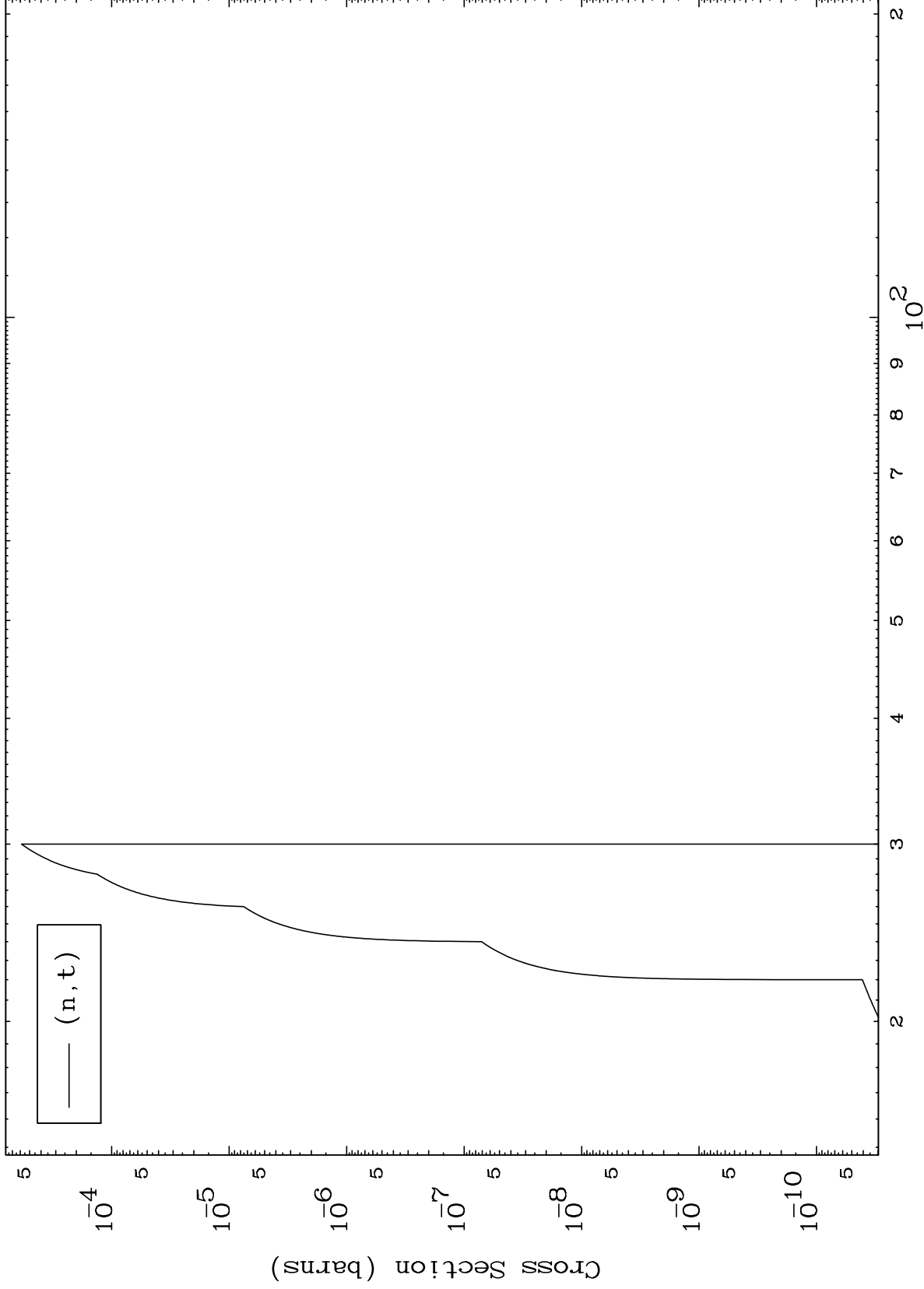
Incident Energy (MeV)

94-Pu-241

MAT 9443

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

94-Pu-241



6

Incident Energy (MeV)

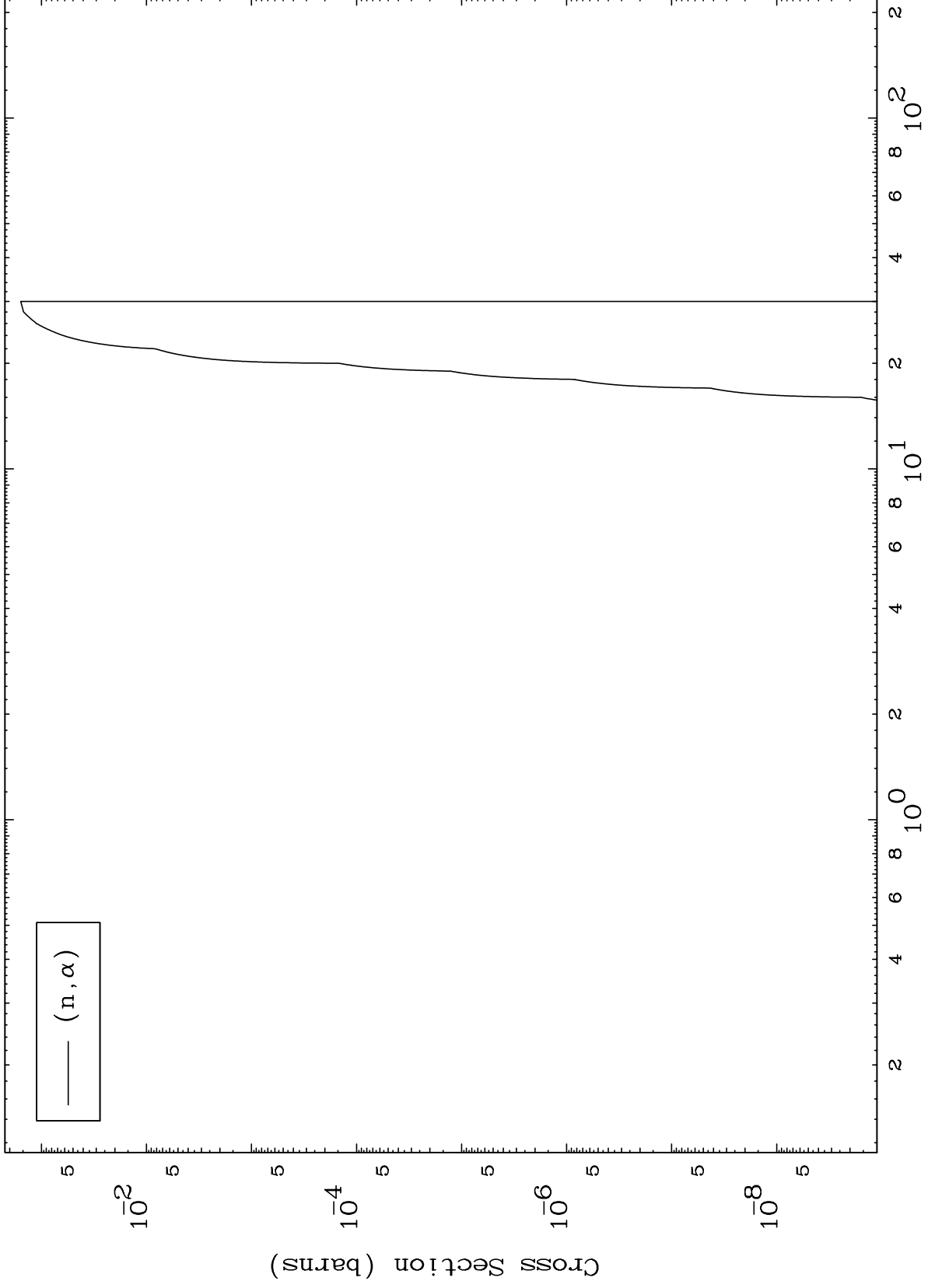
94-Pu-241

MAT 9443

( $\alpha, \alpha$ ) Levels

94-Pu-241

0 Kelvin Cross Sections



7

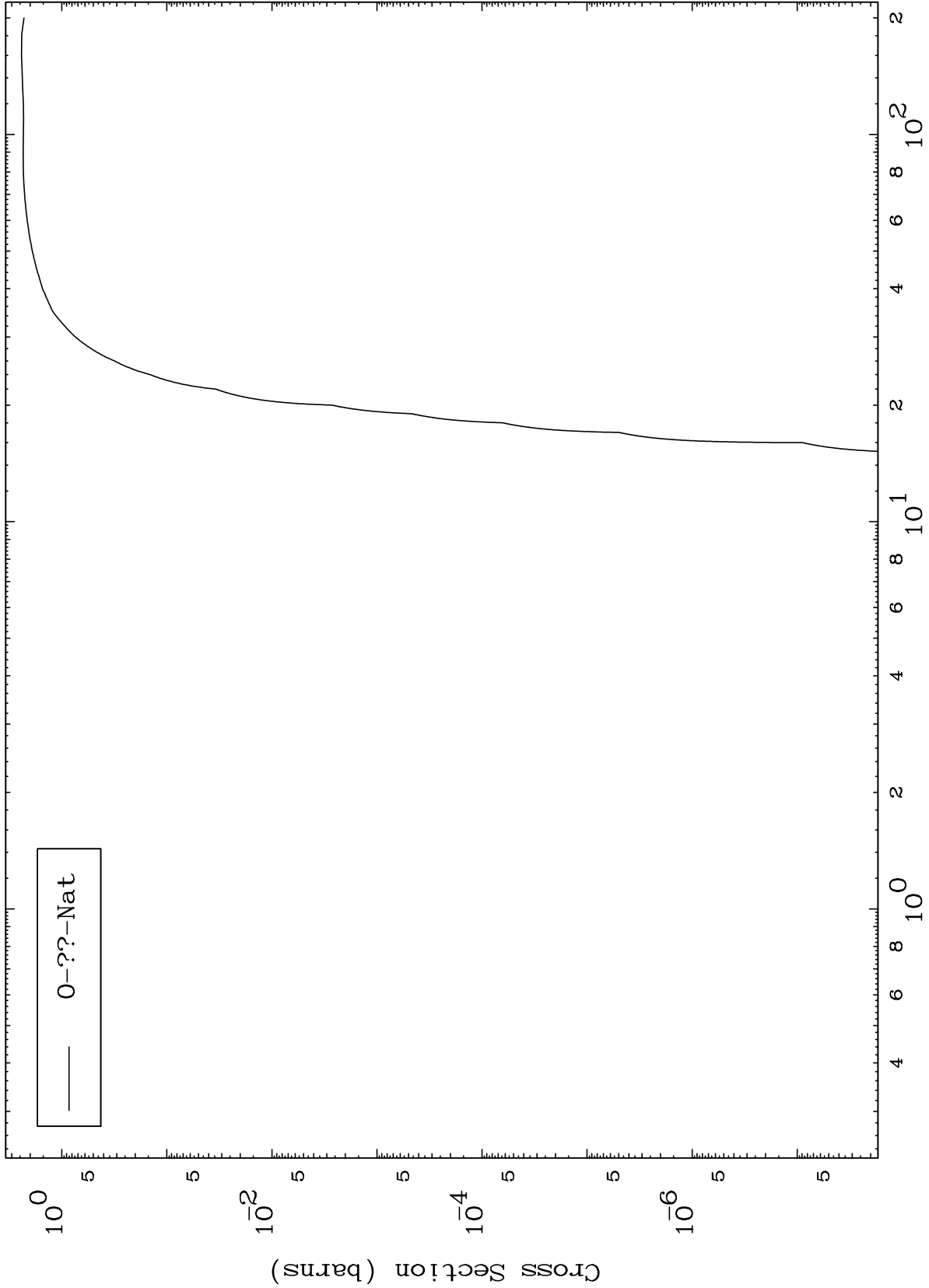
Incident Energy (MeV)

94-Pu-241

MAT 9443

94-Pu-241

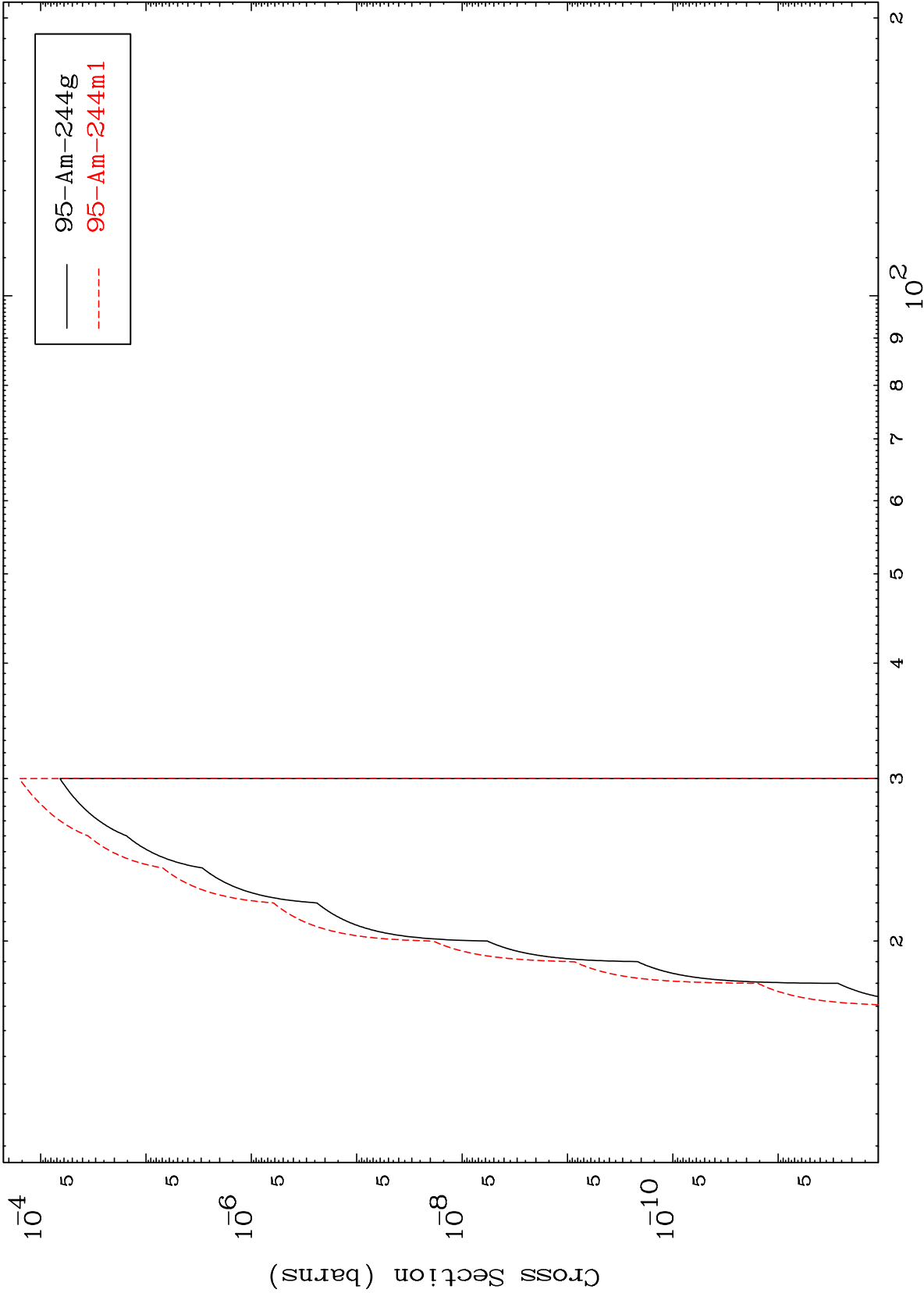
Fission  
Radionuclide Production Cross Section



MAT 9443

94-Pu-241

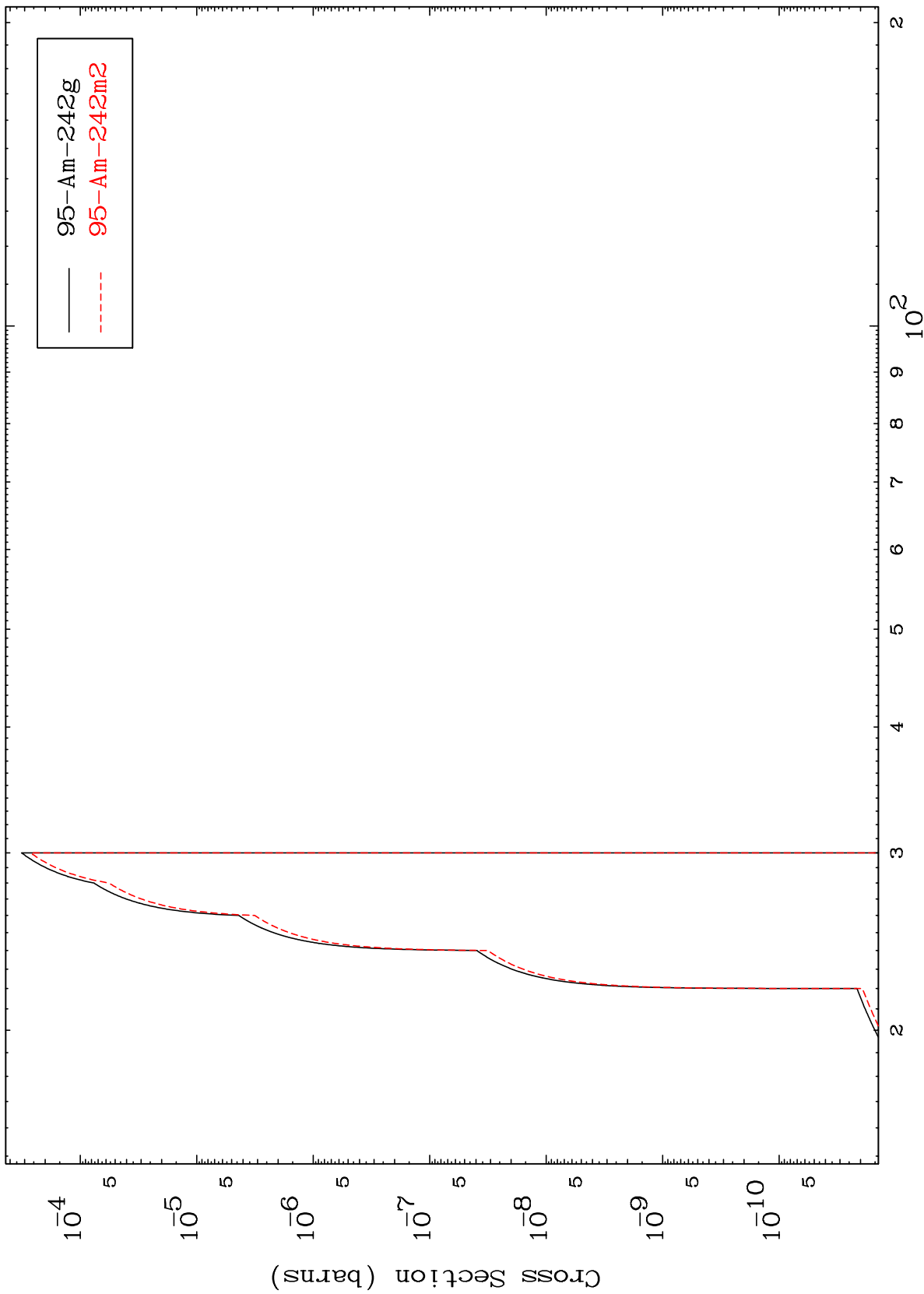
(n,p)  
Radionuclide Production Cross Section



MAT 9443

94-Pu-241

(n, t)  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

94-Pu-241