

Program EVALPLOT  
(Version 2021-1)

by

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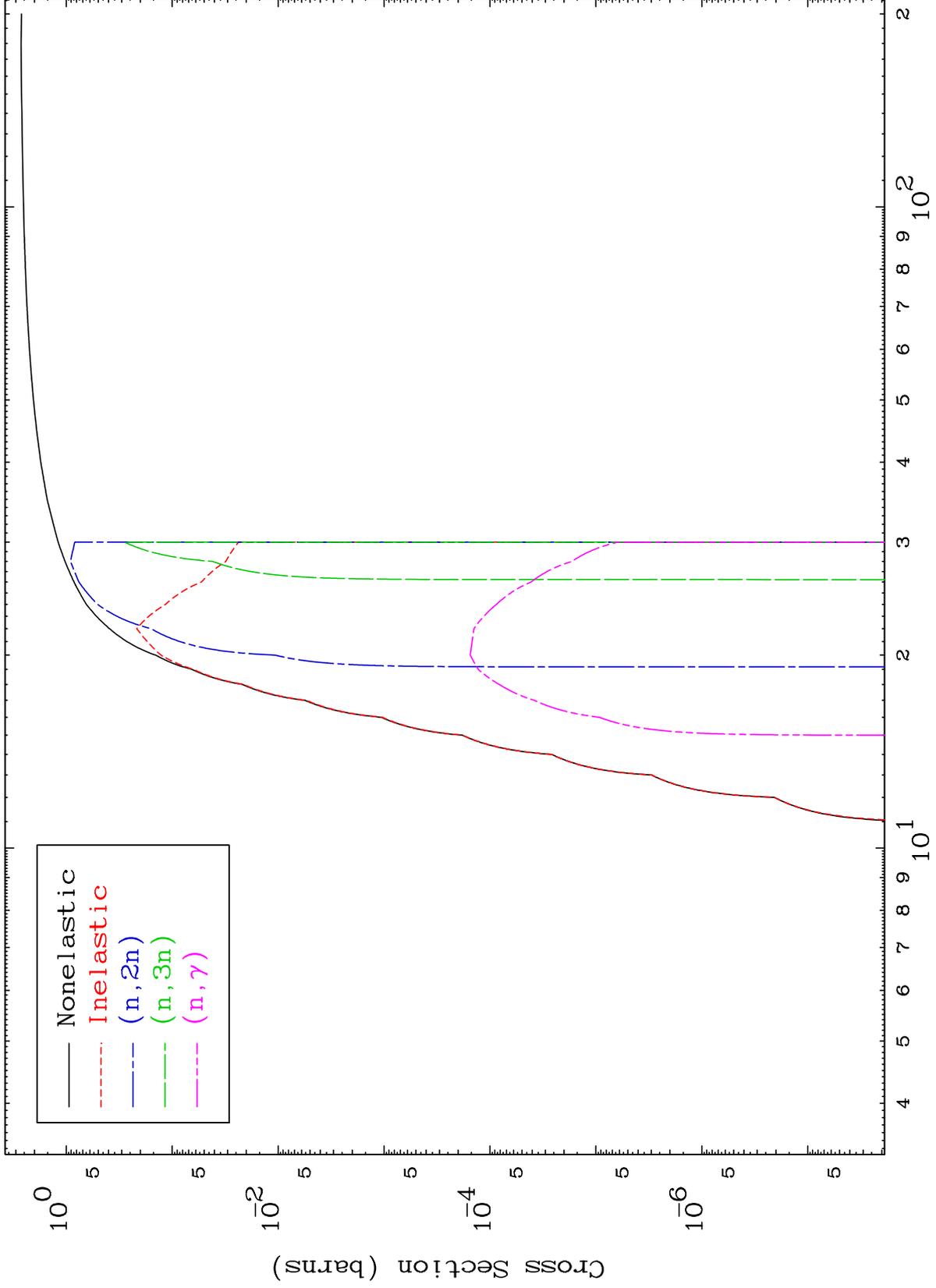
Press Mouse Button to Start

MAT 7522

$\alpha$  Major

75-Re-184

0 Kelvin Cross Sections



1

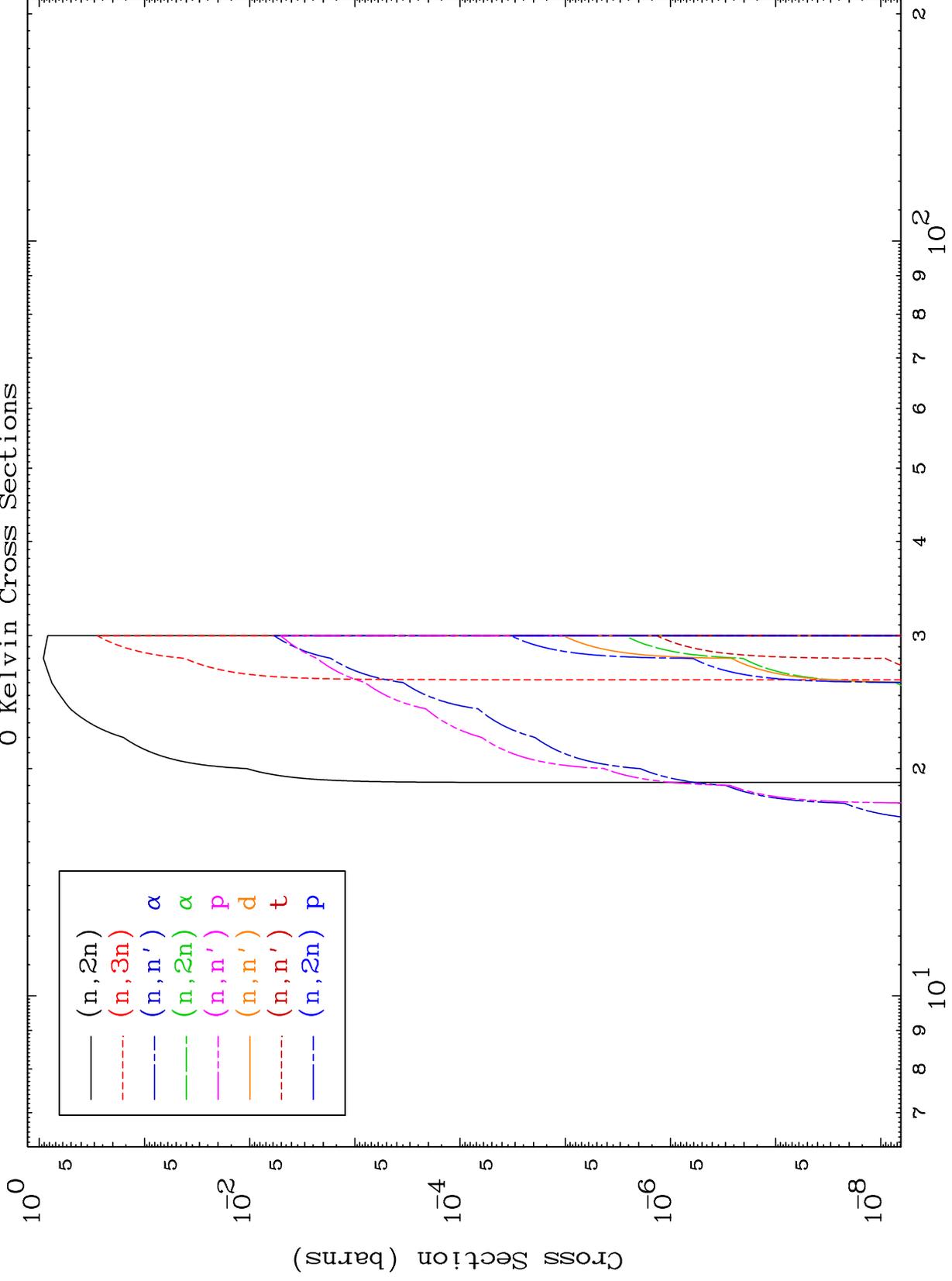
Incident Energy (MeV)

75-Re-184

MAT 7522

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

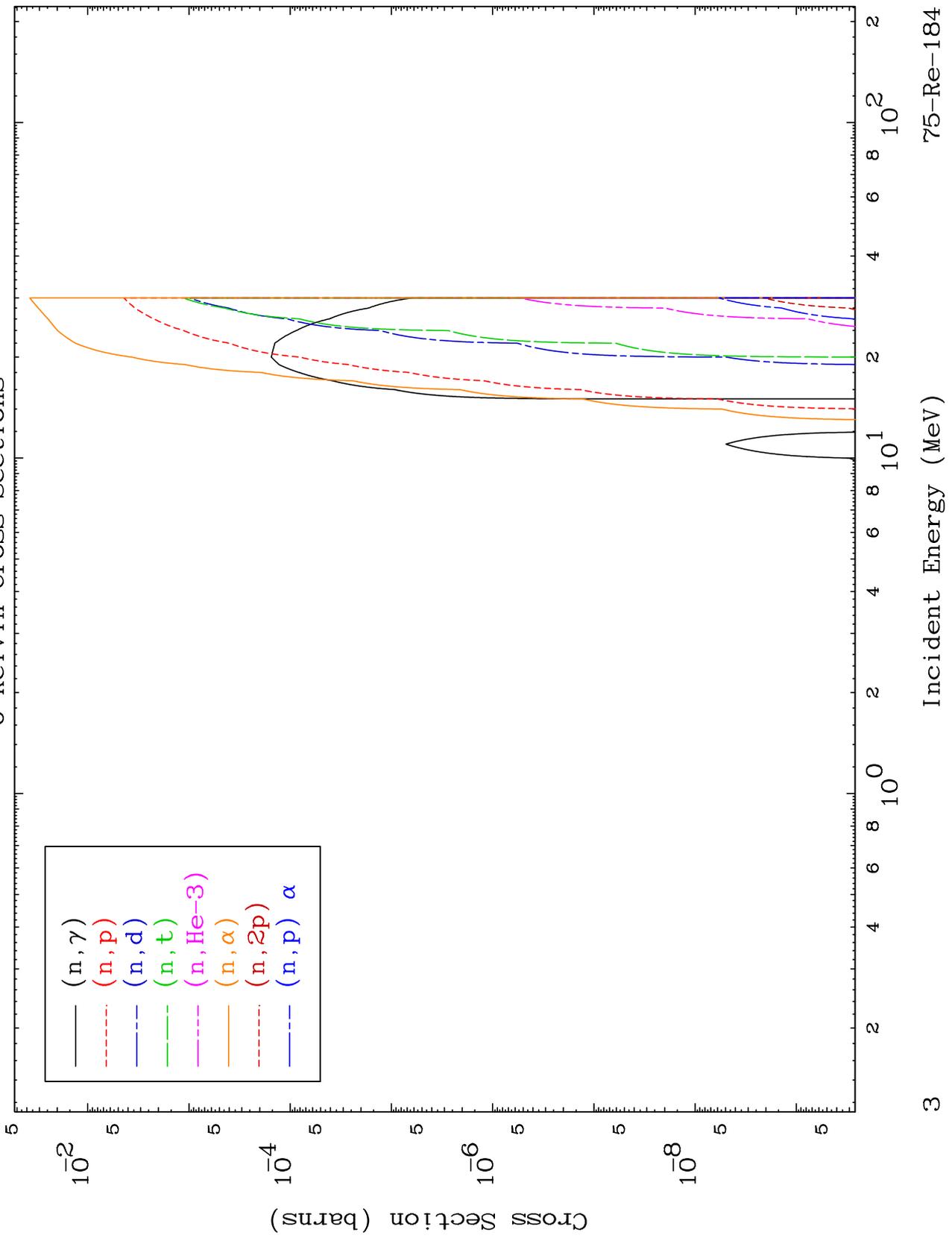
75-Re-184

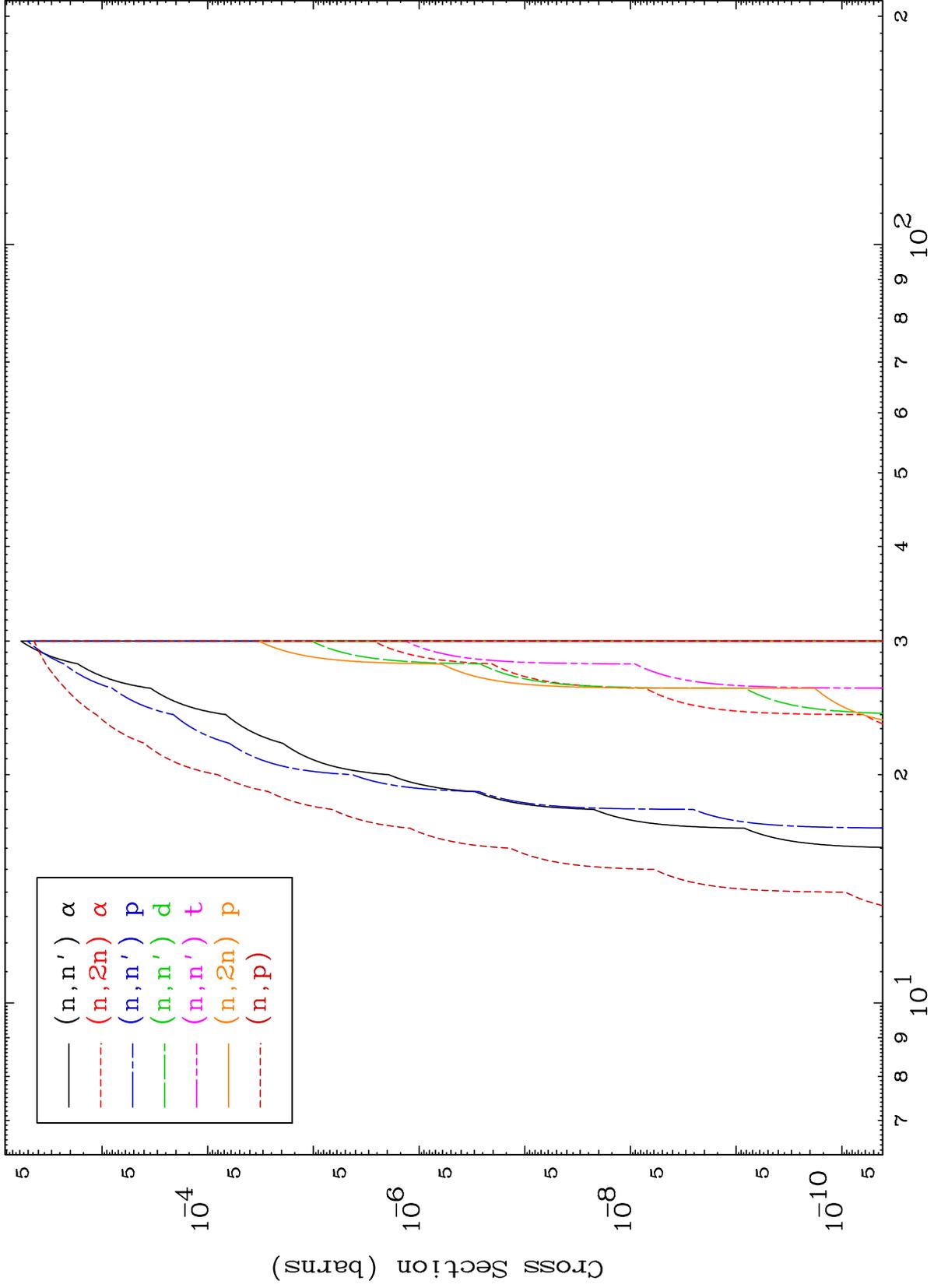


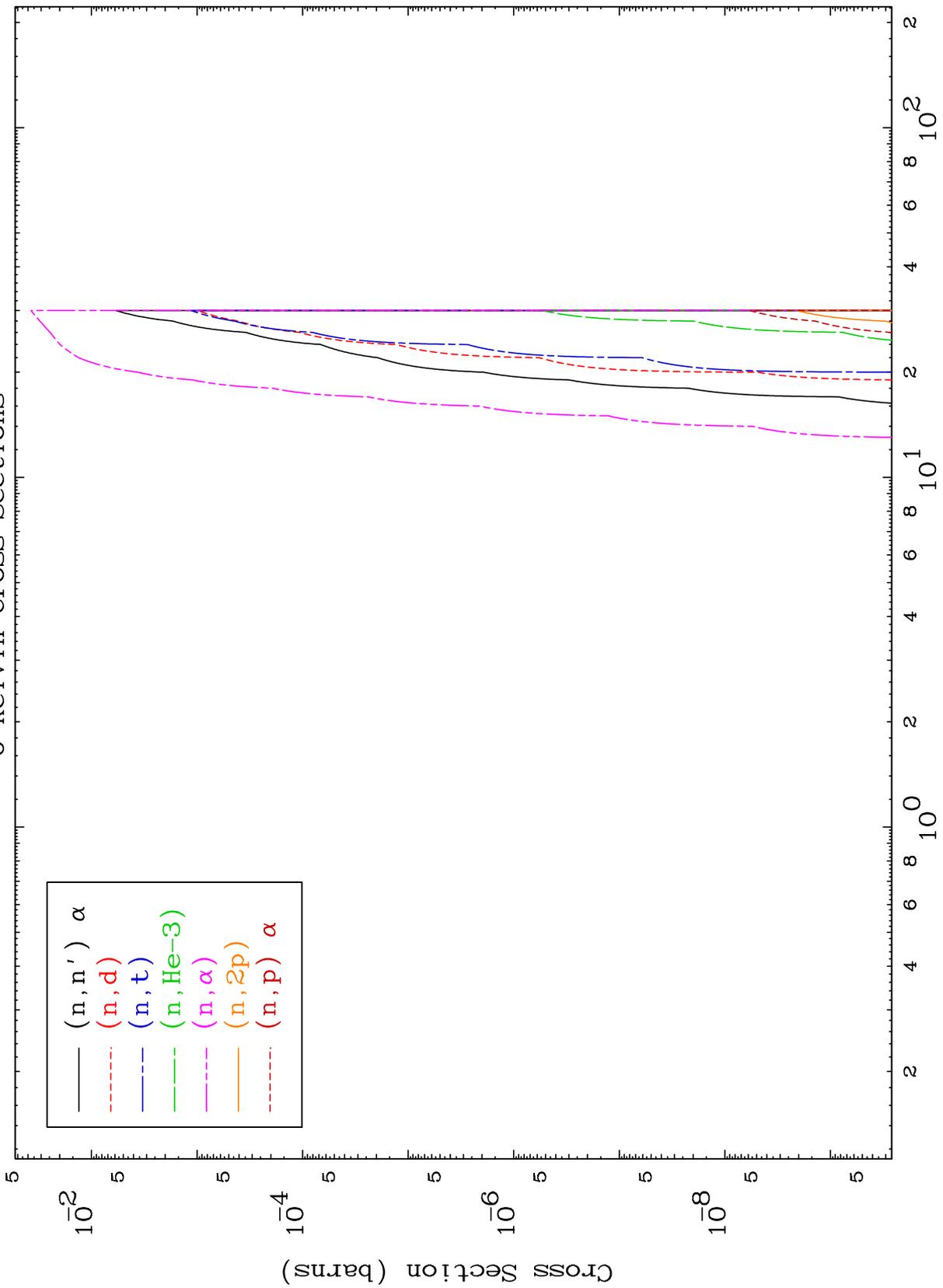
MAT 7522

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

75-Re-184



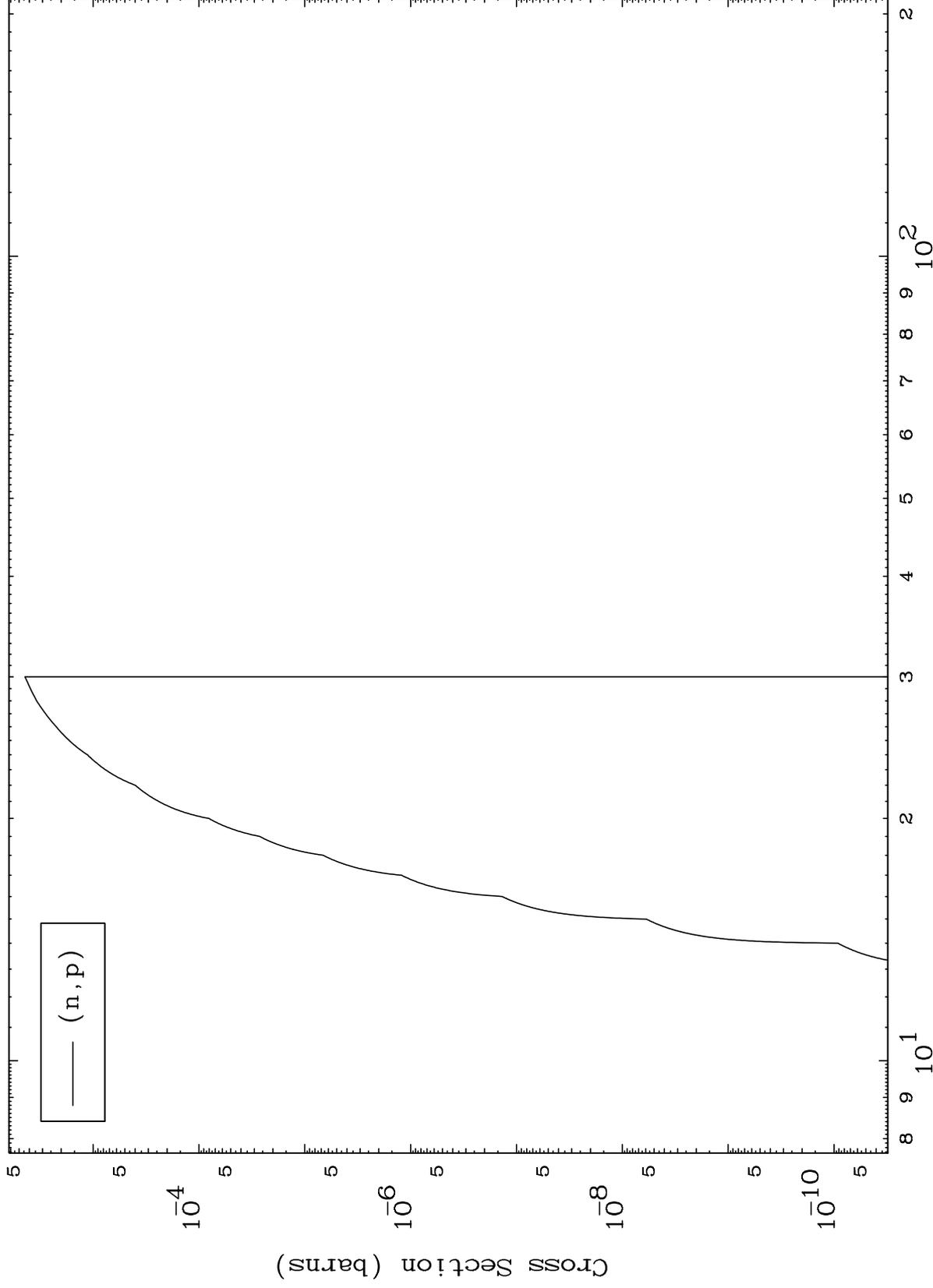




MAT 7522

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

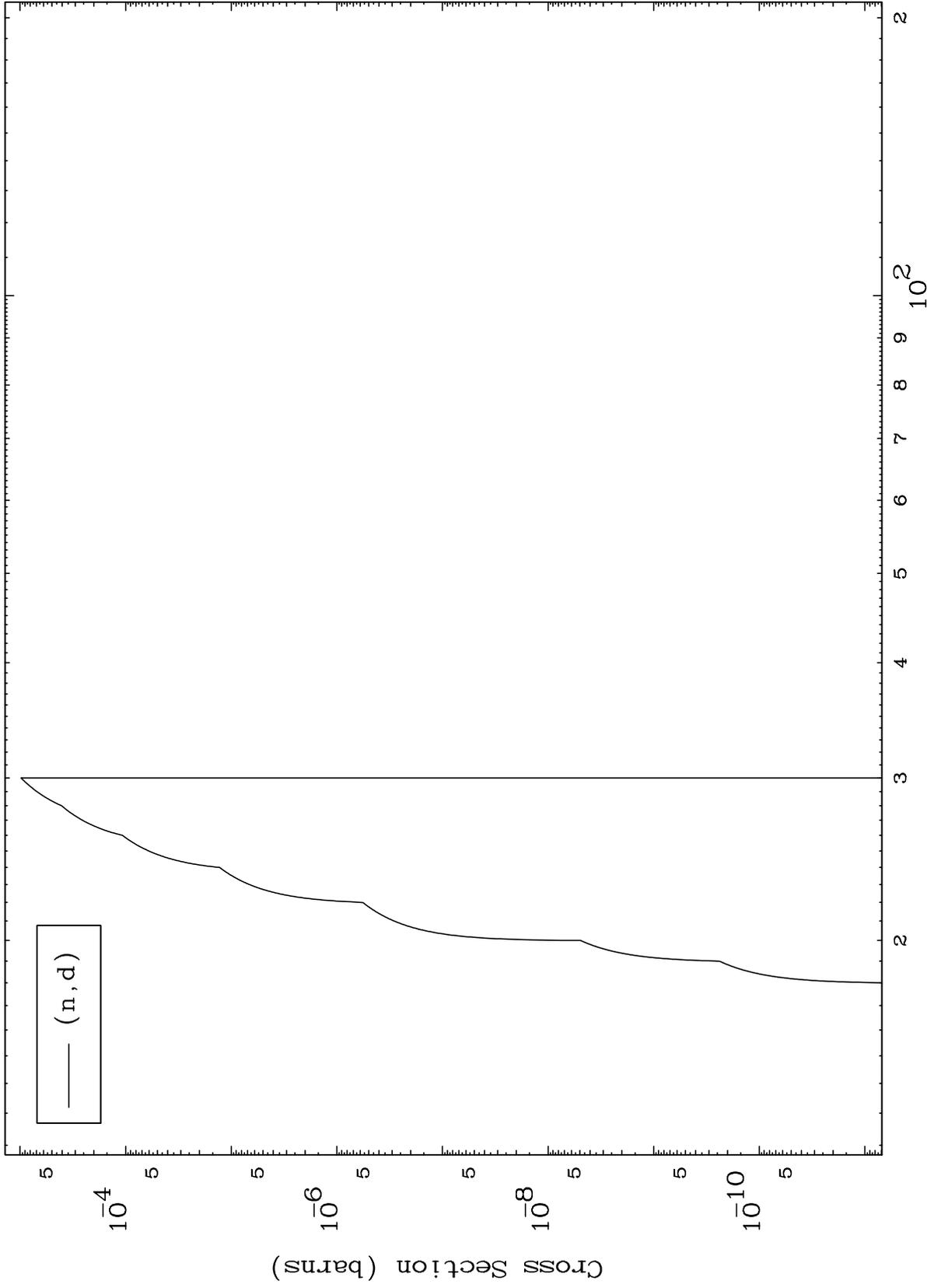
75-Re-184

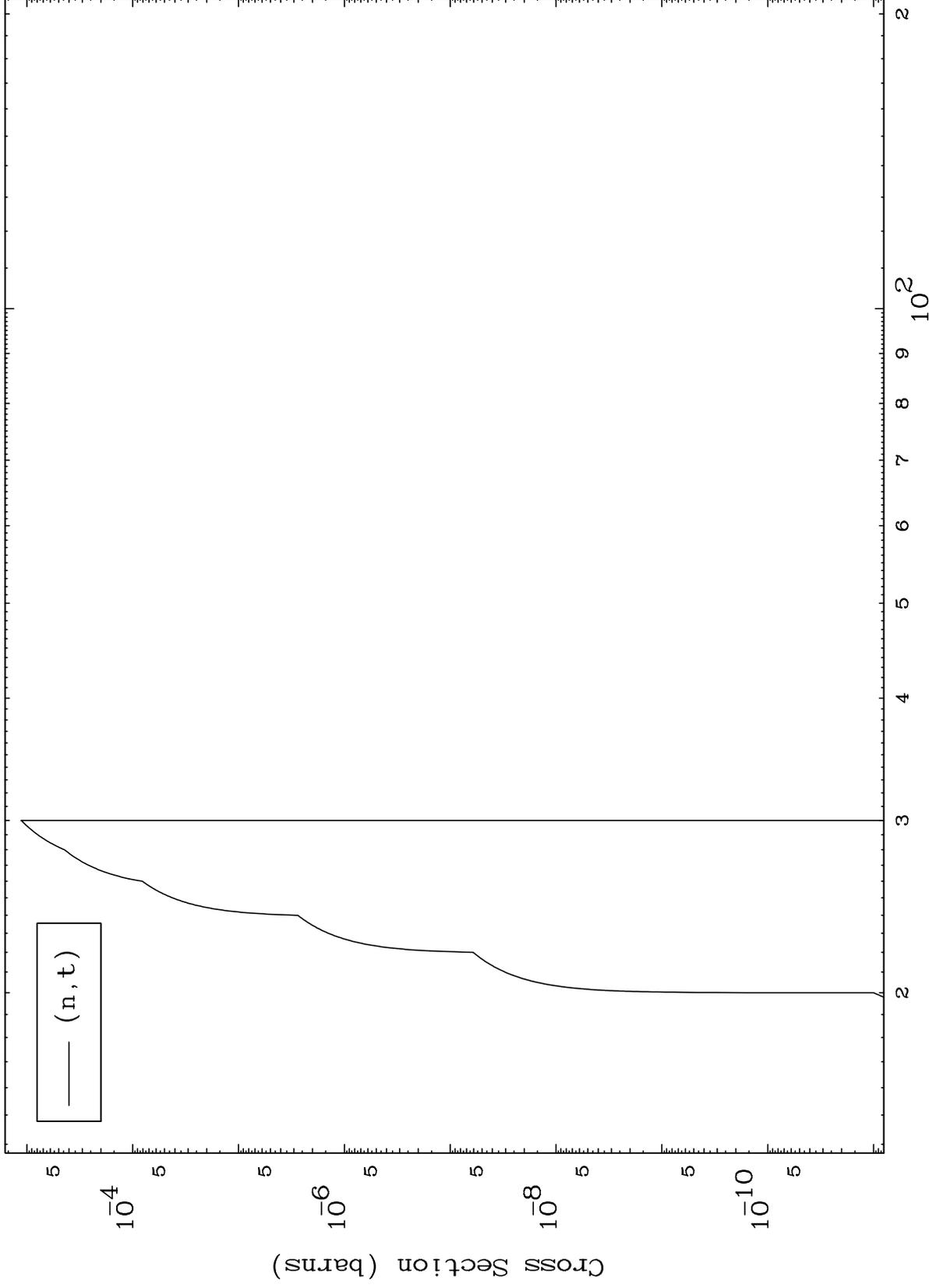


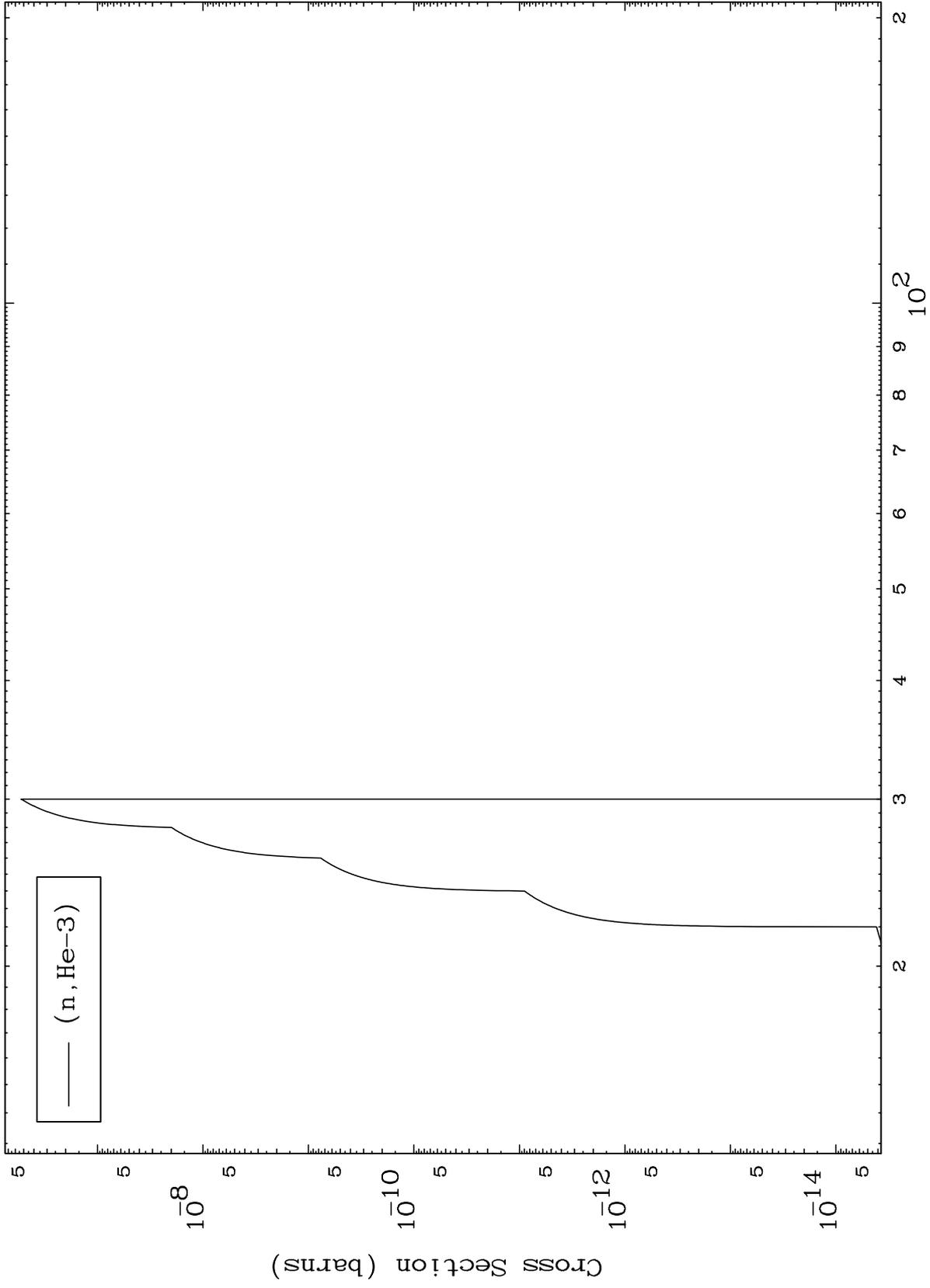
Incident Energy (MeV)

75-Re-184

6





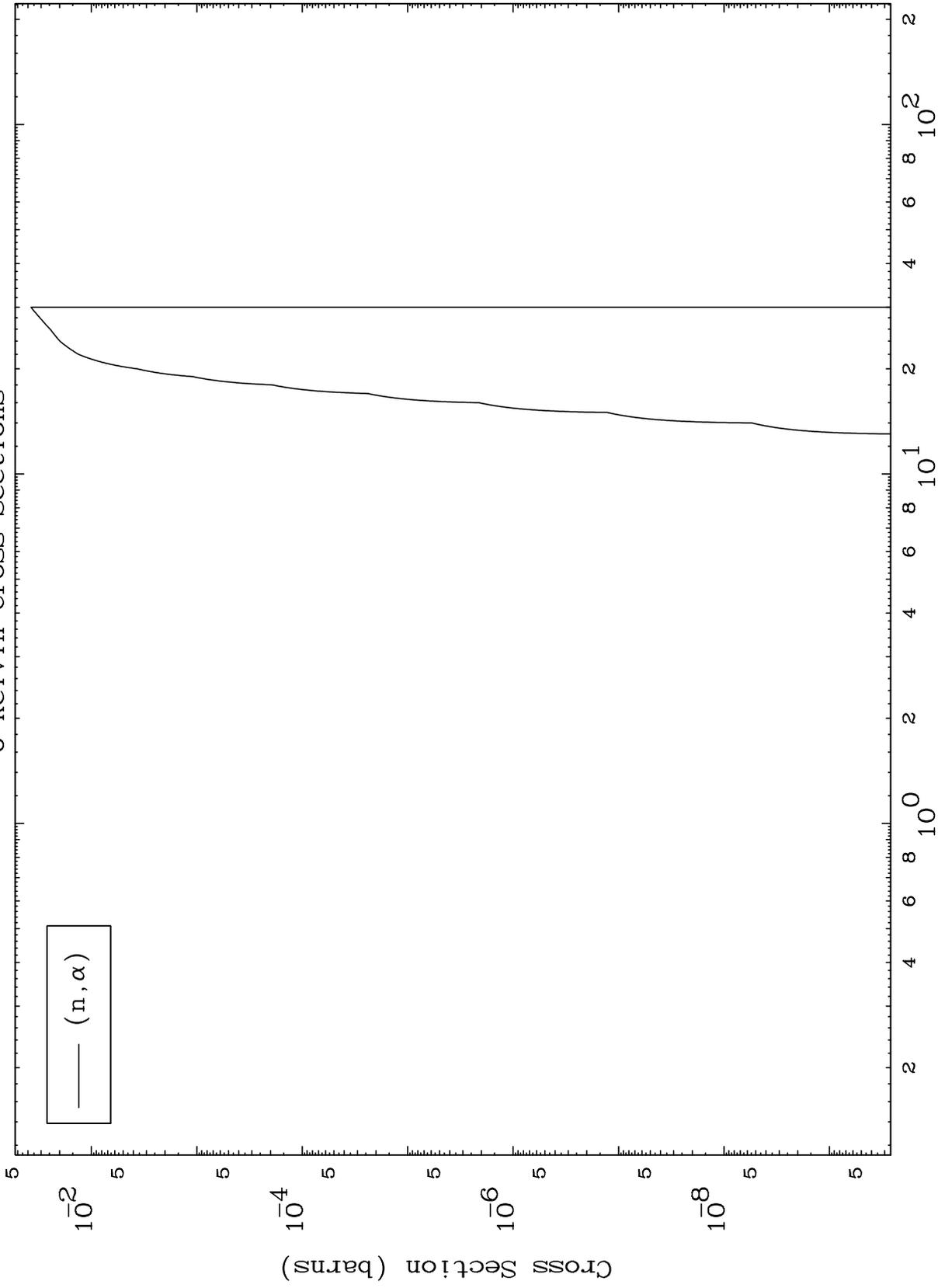


MAT 7522

( $\alpha, \alpha$ ) Levels

75-Re-184

0 Kelvin Cross Sections



10

Incident Energy (MeV)

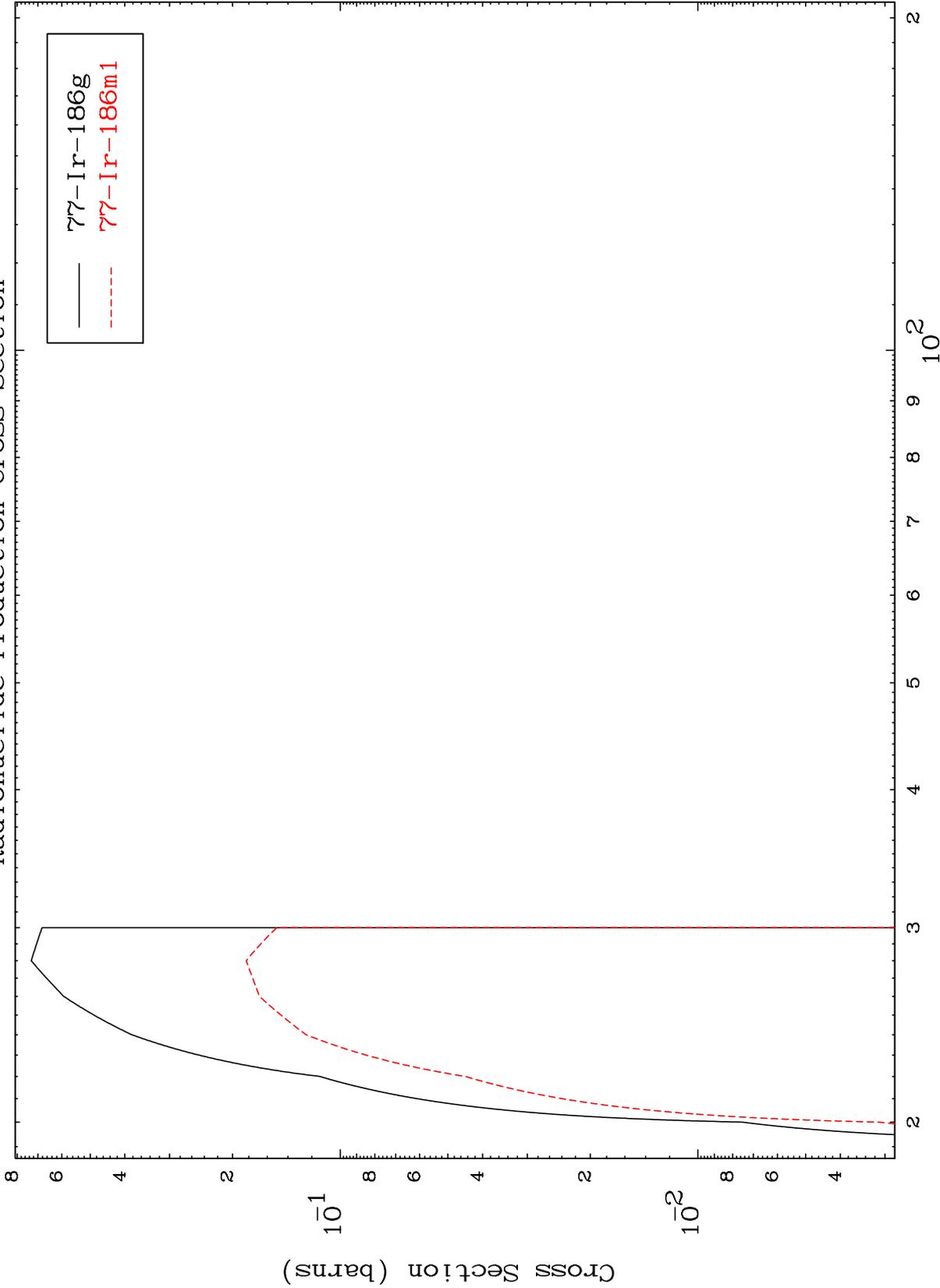
75-Re-184

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(n,2n)

75-Re-184

Radionuclide Production Cross Section



<sup>77</sup>Ir-186g  
<sup>77</sup>Ir-186m1

11

Incident Energy (MeV)

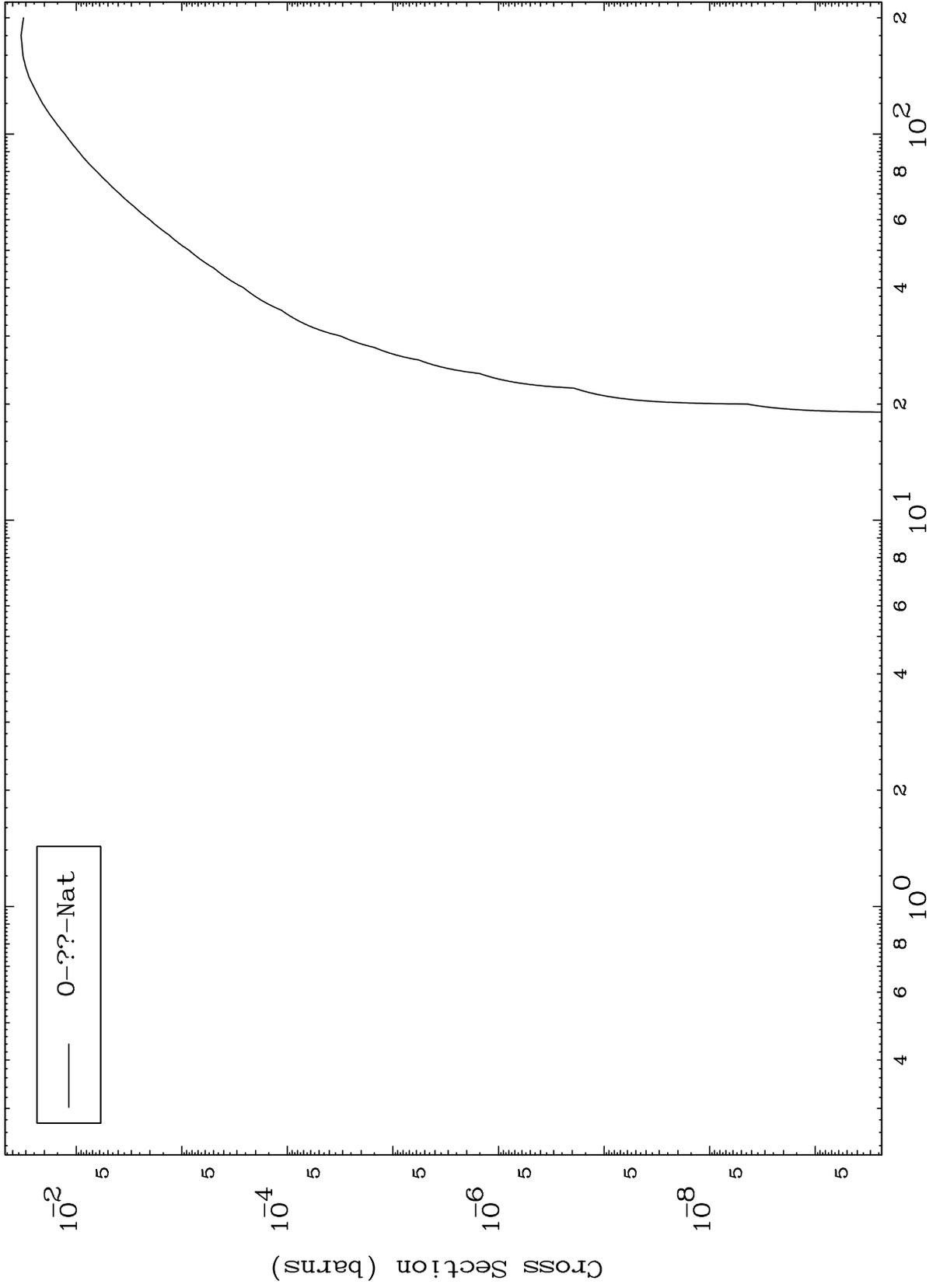
75-Re-184

MAT 7522

Fission

75-Re-184

Radionuclide Production Cross Section



12

Incident Energy (MeV)

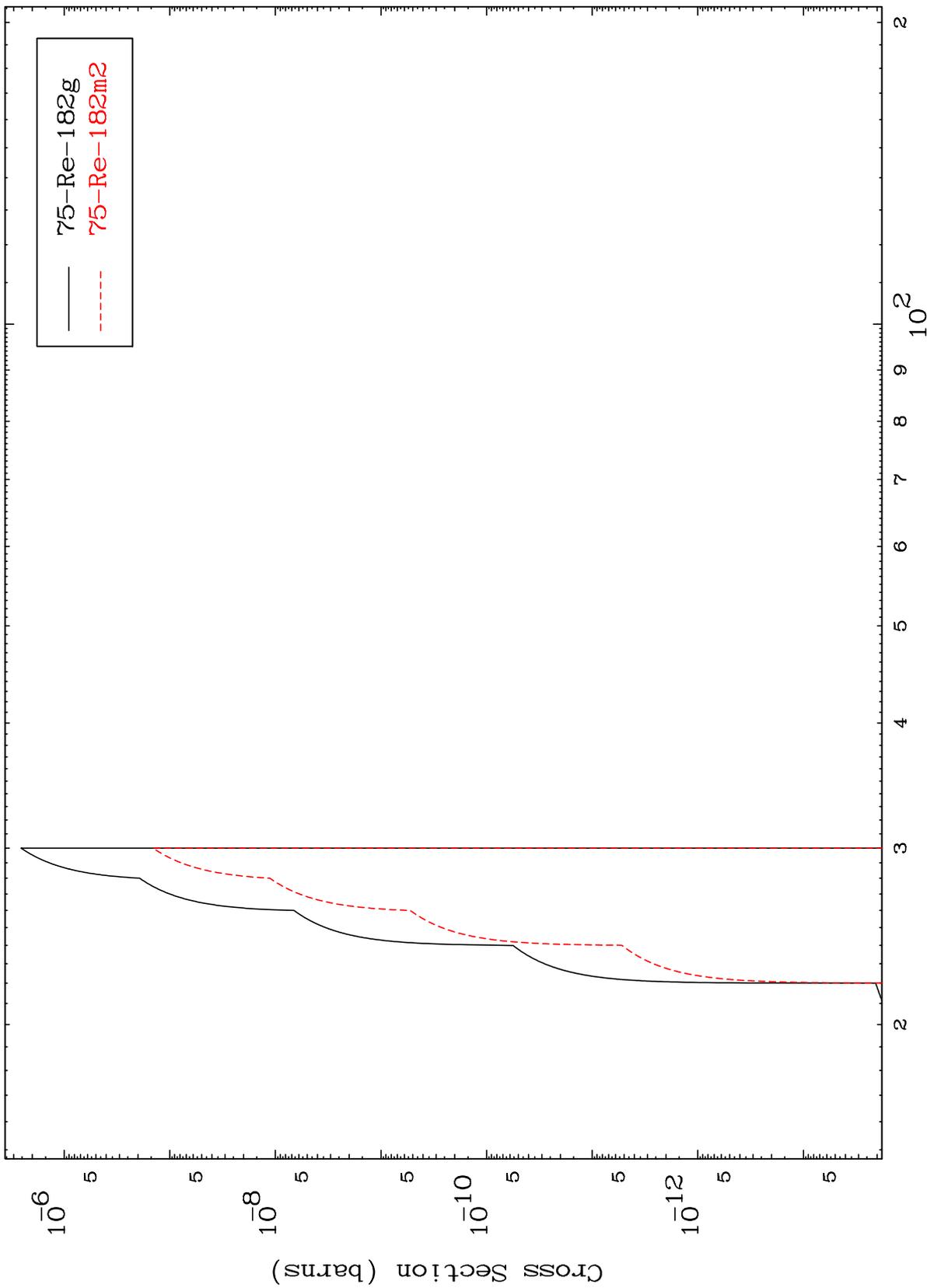
75-Re-184

MAT 7522

(n,2n)  $\alpha$

75-Re-184

Radionuclide Production Cross Section



13

Incident Energy (MeV)

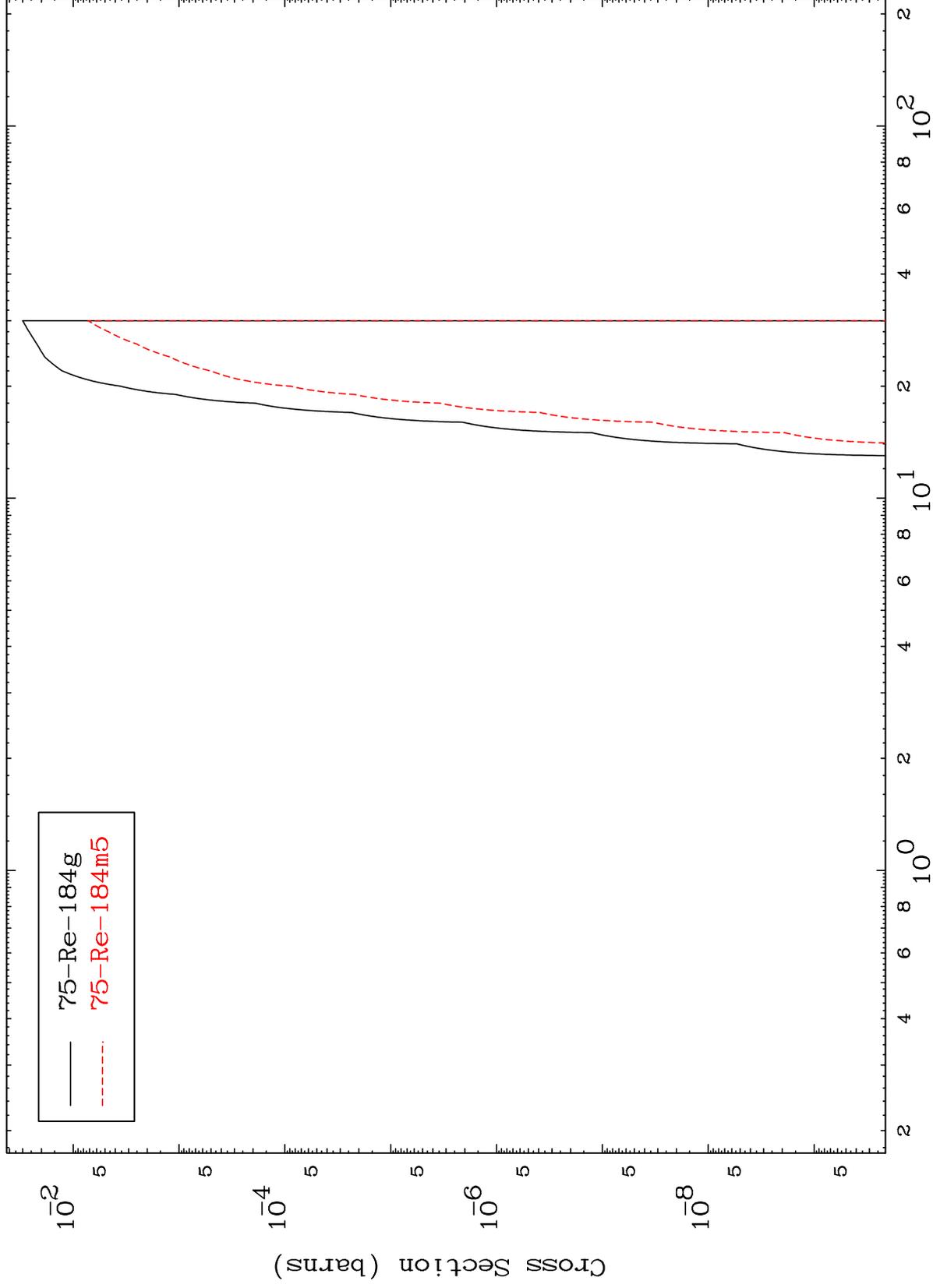
75-Re-184

MAT 7522

(n,  $\alpha$ )

75-Re-184

Radionuclide Production Cross Section



14

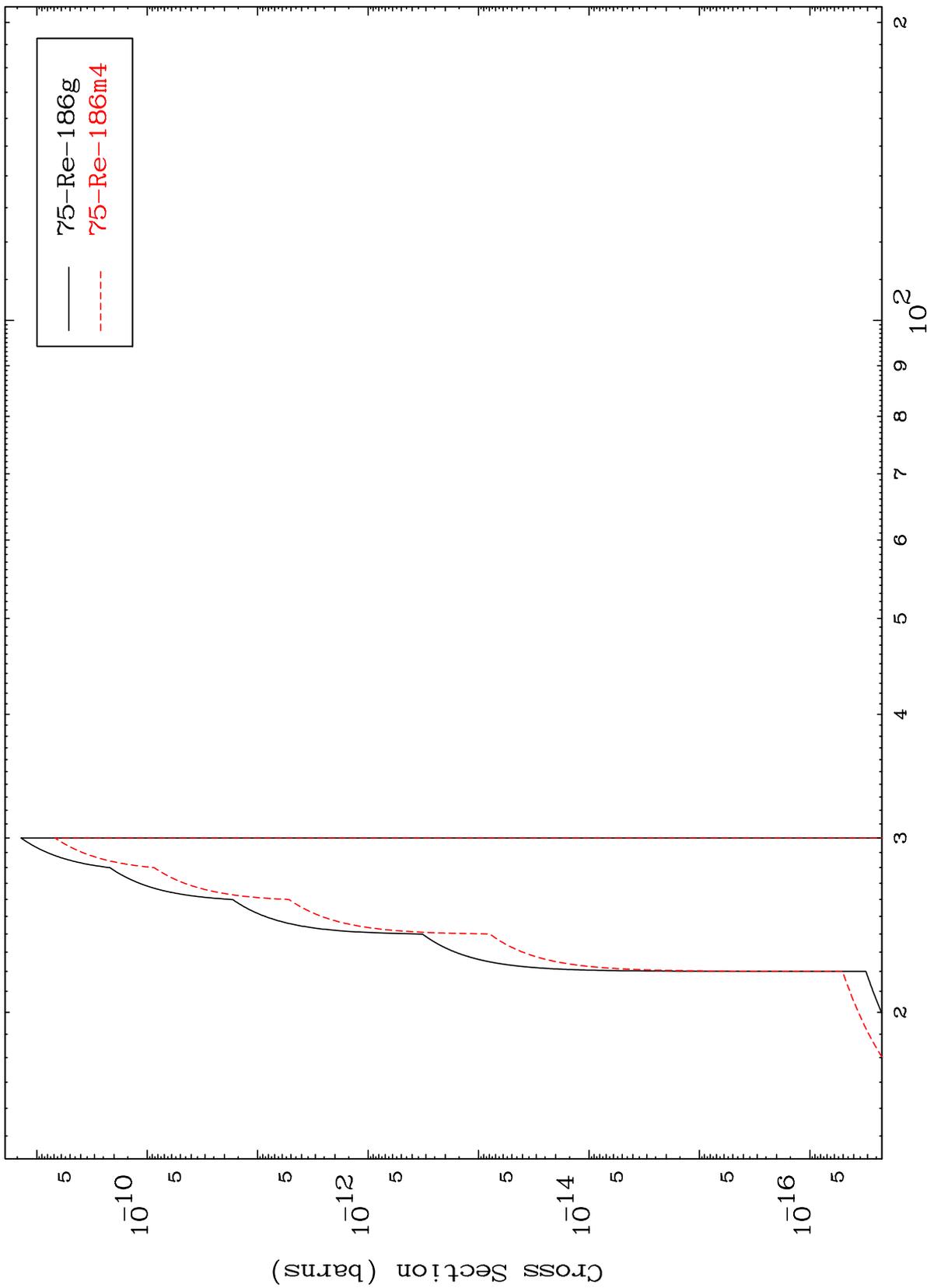
Incident Energy (MeV)

75-Re-184

MAT 7522

<sup>75</sup>Re-184

(n,2p)  
Radionuclide Production Cross Section



15

<sup>75</sup>Re-184

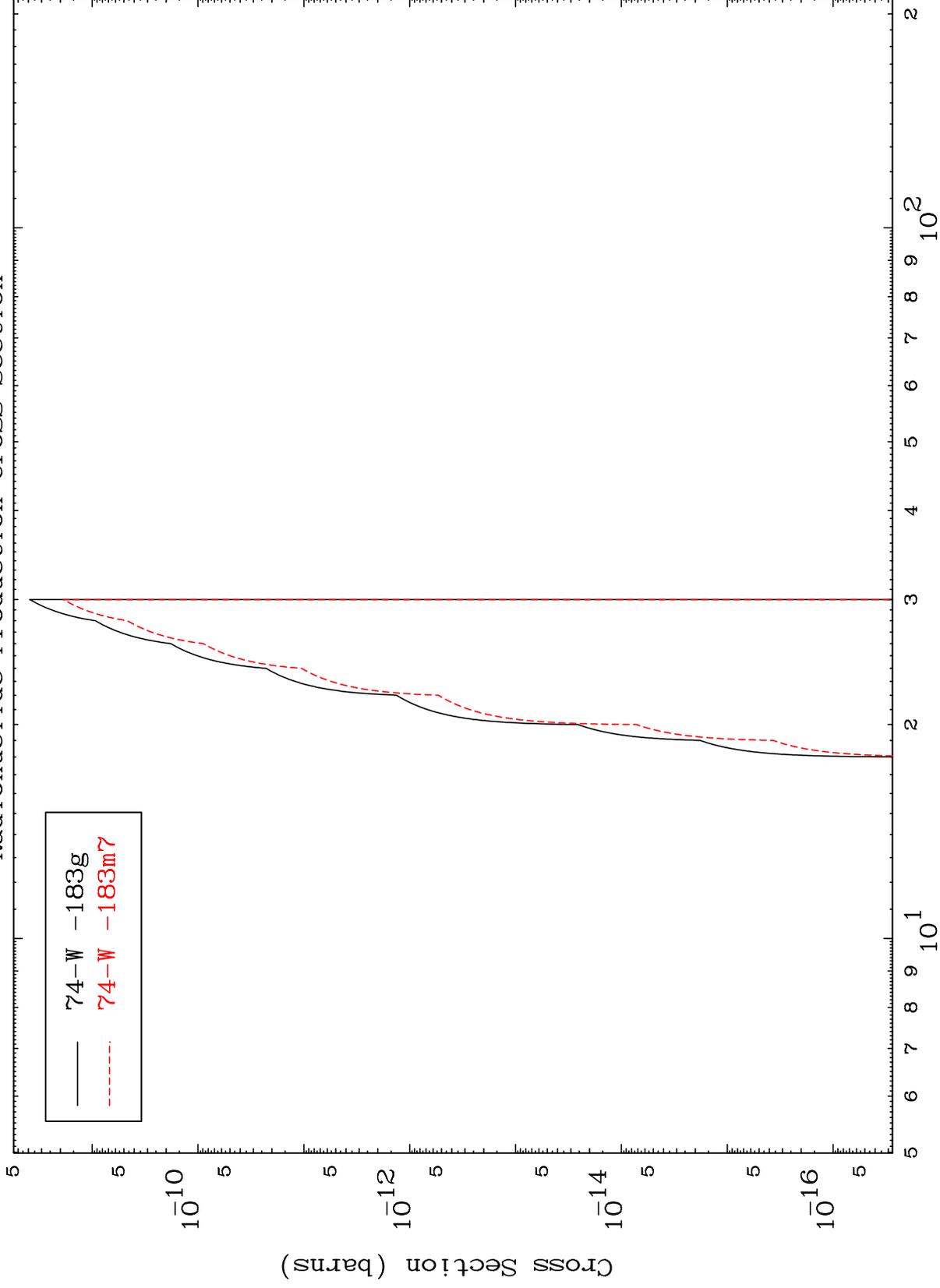
Incident Energy (MeV)

MAT 7522

(n,p)  $\alpha$

<sup>75</sup>Re-<sup>184</sup>

Radionuclide Production Cross Section



16

Incident Energy (MeV)

<sup>75</sup>Re-<sup>184</sup>